

June 1, 2004

LICENSEE: Indiana Michigan Power Company  
FACILITY: Donald C. Cook Nuclear Plant, Units 1 and 2  
SUBJECT: SUMMARY OF TELEPHONE CONFERENCE ON MAY 6, 2004, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION (NRC) AND INDIANA MICHIGAN POWER COMPANY (I&M) REPRESENTATIVES CONCERNING DRAFT REQUESTS FOR ADDITIONAL INFORMATION ON DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2, LICENSE RENEWAL APPLICATION (TAC NOS. MC1202 AND MC1203)

The U.S. Nuclear Regulatory Commission staff (the staff) and representatives of Indiana Michigan Power Company (the applicant) held a telephone conference call on May 6, 2004, to discuss a draft request for additional information (D-RAI) concerning the Donald C. Cook Nuclear Plant (CNP) license renewal application (LRA).

The conference call was useful in clarifying the intent of the staff's question. On the basis of the discussions, the applicant was able to better understand the staff's D-RAI. No staff decisions were made during the meeting.

Enclosure 1 provides a listing of the telephone conference participants. Enclosure 2 contains the D-RAI discussed with the applicant, including a brief description on the status of the item. The applicant has had an opportunity to review and comment on this summary.

***/RA/***

Jonathan Rowley, Project Manager  
License Renewal Section A  
License Renewal and Environmental Impacts Program  
Division of Regulatory Improvement Programs  
Office of Nuclear Reactor Regulation

Docket Nos.: 50-315 and 50-316

Enclosures: As stated

cc w/enclosures: See next page

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DRAFT REQUESTS FOR ADDITIONAL INFORMATION MAY 6, 2004**

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**DRAFT REQUEST FOR ADDITIONAL INFORMATION (D-RAI) DISCUSSED FOR  
DONALD C. COOK (CNP), UNITS 1 AND 2, LICENSE RENEWAL  
DURING MAY 6, 2004 TELEPHONE CONFERENCE**

**Donald C. Cook LRA Section 4.3, "METAL FATIGUE"**

**D-RAI 4.3.3-1**

Section 4.3.3 of the LRA discusses I&M's evaluation of the impact of the reactor water environment on the fatigue life of components. The discussion references the fatigue sensitive component locations for an early vintage Westinghouse plant identified in NUREG/CR-6260, "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components." The LRA indicates that the design usage factors provided in Table 5-98 of NUREG/CR-6260 were used for the evaluation of the charging nozzle, safety injection nozzle and RHR tee. The design usage factors were based on an evaluation of the Turkey Point facility, including a plant specific evaluation of the RHR piping and detailed finite element analyses of the charging and safety injection nozzles. Discuss the applicability of these analyses to the D. C. Cook facility. The discussion should include a comparison of piping sizes and thicknesses, including the design of the thermal sleeves between D. C. Cook and Turkey Point. The discussion should also include a comparison of the number and type of design transients cycles between D. C. Cook and Turkey Point.

**Status:** I&M was requested to consider the safety evaluations for Surry and North Anna to aid in the response to this D-RAI. I&M indicated that the question was clear.

Donald C. Cook Nuclear Plant, Units 1 and 2

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