

UNITED STATES ATOMIC ENERGY COMMISSION

DOCKET NO. 50-298

NEBRASKA PUBLIC POWER DISTRICT

(COOPER NUCLEAR STATION)

NOTICE OF ISSUANCE OF A FACILITY OPERATING LICENSE

Notice is hereby given that the Atomic Energy Commission (the Commission) has issued Facility Operating License No. DPR-46 to the Nebraska Public Power District (the licensee) for operation of the Cooper Nuclear Station (the facility) at steady state reactor core power levels not in excess of 2381 megawatts (thermal), in accordance with the provisions of the license and the Technical Specifications. The facility is a boiling water nuclear reactor located at the licensee's site near Brownville in Nemaha County, Nebraska, and Atcheson County, Missouri.

Activities authorized under the license are being limited temporarily, by restrictions in the Technical Specifications (Appendix A) which accompany the license, to fuel loading, low power testing, and operation at one (1) percent of the rated power level of 2381 megawatts thermal until the following matters have been resolved: (a) reactor internals vibration cold flow tests are completed and evaluated; and (b) procedures for operation at above one (1) percent rated power have been completed and approved.

The Commission has made appropriate findings as required by the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and

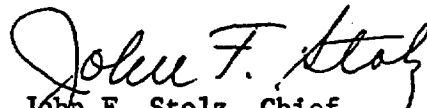
regulations in 10 CFR Chapter I, which are set forth in the license. The application for the license complies with the standards and requirements of the Act and the Commission's rules and regulations.

The license is effective as of the date of issuance and shall expire at midnight on June 4, 2008.

A copy of (1) Facility Operating License No. DPR-46, complete with Technical Specifications (Appendices "A" and "B"), (2) the licensee's Environmental Report dated September 17, 1971 and supplements thereto, (3) the report of the Advisory Committee on Reactor Safeguards, dated April 18, 1973, (4) the Directorate of Licensing's Safety Evaluation dated February 14, 1973, and Supplements 1 and 2 thereto, (5) the Final Safety Analysis Report and amendments thereto, (6) the Draft Environmental Statement dated November 1972, and (7) the Final Environmental Statement dated February 1973, are available for public inspection at the Commission's Public Document Room at 1717 H Street, N. W., Washington, D. C. 20545, and the Auburn Public Library, 1118 - 15th Street, Auburn, Nebraska 68305. A copy of the license and the Safety Evaluation may be obtained upon request addressed to the United States Atomic Energy Commission, Regulation, Washington, D. C. 20545, Attention: Deputy Director for Reactor Projects, Directorate of Licensing.

Dated at Bethesda, Maryland, this ^{18th} day of January 1974.

FOR THE ATOMIC ENERGY COMMISSION


John F. Stolz, Chief
Light Water Reactors Project
Branch 2-1
Directorate of Licensing

ENVIRONMENTAL TECHNICAL SPECIFICATIONS

APPENDIX B

TO

OPERATING LICENSE NO. DPR-46

FOR THE

COOPER NUCLEAR STATION

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

(All 84 pages of these Appendix B Technical Specifications have been deleted in their entirety by the generation of Radiological Environmental Technical Specifications (RETS) in Appendix A.)

3.0 SR APPLICABILITY

SR 3 0.3
(continued)

If the Surveillance is not performed within the delay period, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

When the Surveillance is performed within the delay period and the Surveillance is not met, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

SR 3.0.4

Entry into a MODE or other specified condition in the Applicability of an LCO shall not be made unless the LCO's Surveillances have been met within their specified Frequency. This provision shall not prevent entry into MODES or other specified conditions in the Applicability that are required to comply with Actions or that are part of a shutdown of the unit.

SR 3.0.4 is only applicable for entry into a MODE or other specified condition in the Applicability in MODES 1, 2, and 3.

Table 3.3.1.1-1 (page 1 of 3)
Reactor Protection System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION D.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1. Intermediate Range Monitors					
a. Neutron Flux — High	2	3	G	SR 3.3.1.1.1 SR 3.3.1.1.3 SR 3.3.1.1.4 SR 3.3.1.1.5 SR 3.3.1.1.6 SR 3.3.1.1.12 SR 3.3.1.1.13 SR 3.3.1.1.15	≤ 121/125 divisions of full scale
	5(a)	3	H	SR 3.3.1.1.1 SR 3.3.1.1.3 SR 3.3.1.1.4 SR 3.3.1.1.12 SR 3.3.1.1.13 SR 3.3.1.1.15	≤ 121/125 divisions of full scale
b. Inop	2	3	G	SR 3.3.1.1.3 SR 3.3.1.1.4 SR 3.3.1.1.13	NA
	5(a)	3	H	SR 3.3.1.1.3 SR 3.3.1.1.4 SR 3.3.1.1.13	NA
2. Average Power Range Monitors					
a. Neutron Flux — High (Startup)	2	2	G	SR 3.3.1.1.1 SR 3.3.1.1.3 SR 3.3.1.1.4 SR 3.3.1.1.6 SR 3.3.1.1.8 SR 3.3.1.1.10 SR 3.3.1.1.13 SR 3.3.1.1.15	≤ 14.5% RTP
b. Neutron Flux-High (Flow Biased)	1	2	F	SR 3.3.1.1.1 SR 3.3.1.1.2 SR 3.3.1.1.4 SR 3.3.1.1.7 SR 3.3.1.1.8 SR 3.3.1.1.9 SR 3.3.1.1.10 SR 3.3.1.1.12 SR 3.3.1.1.13 SR 3.3.1.1.15	≤ 0.66 W + 71.5% RTP ^(b)

(continued)

(a) With any control rod withdrawn from a core cell containing one or more fuel assemblies.

(b) [0.66 W + 71.5% - 0.66 ΔW] RTP when reset for single loop operation per LCO 3.4.1, "Recirculation Loops Operating."

Primary Containment Isolation Instrumentation
3.3.6.1

Table 3.3.6.1-1 (page 3 of 3)
Primary Containment Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
6. RHR Shutdown Cooling System Isolation					
a. Reactor Pressure - High	1,2,3	1	F	SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.6	≤ 72 psig
b. Reactor Vessel Water Level - Low (Level 3)	3,4,5	4(b)	I	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.4 SR 3.3.6.1.6	≥ 3 inches

(b) Only one trip system is required in MODES 4 and 5 when RHR Shutdown Cooling System integrity maintained.

June 20, 1986

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Docket No. 50-298

DISTRIBUTION:

Mr. J. M. Pilant, Technical
Staff Manager
Nuclear Power Group
Nebraska Public Power District
Post Office Box 499
Columbus, Nebraska 68601

Docket File TBarnhart (4)
NRC PDR WJones
Local PDR DVassallo
SNorris ACRS (10)
WOLong OPA
OELD LFMB
LJHarmon Gray File
ELJordan JPartlow
BGrimes NThompson
EButcher

Dear Mr. Pilant:

SUBJECT: TECHNICAL SPECIFICATIONS INSERT PAGES

Re: Cooper Nuclear Station

On December 24, 1984, the Commission issued Amendment 89 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. That amendment consisted of changes to the Technical Specifications in response to your application dated March 7, 1984, as supplemented by letters dated April 10, 1984, July 19, 1984, and November 15, 1984.

Amendment 89 revises the Technical Specifications to incorporate Radiological Effluent Technical Specifications (RETS). Certain pages included with Amendment 89 were intended for insertion in the Technical Specifications on their effective date of July 1, 1986. However, since December 24, 1984, other amendments to the Technical Specifications have been issued which affect some of the pages. In order to properly update your copy of the Technical Specifications you should destroy those pages which were forwarded by the December 24, 1984 amendment for insertion on July 31, 1986, and instead, insert the enclosed pages on July 1, 1986.

Insertion of the enclosed pages also corrects errors and omissions in recent amendments. The enclosed list identifies the affected pages.

Sincerely,

/s/

William O. Long, Project Manager
BWR Project Directorate No. 2
Division of BWR Licensing

Enclosures:
As stated

cc w/encls (See next page)

OFC : DBL:PD#2	: DBL:PD#2	:	:	:	:	:
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Docket No. 50-298

Nebraska Public Power District
ATTN: Mr. J. M. Pilant, Manager
Licensing and Quality Assurance
P. O. Box 499
Columbus, Nebraska 68601

Gentlemen:

The Atomic Energy Commission has issued Facility Operating License No. DPR-46 (copy enclosed) authorizing operation of the Cooper Nuclear Station at steady state reactor core power levels not in excess of 2381 megawatts thermal in accordance with the Technical Specifications, Appendices A and B, attached thereto. However, activities authorized under License No. DPR-46 are limited temporarily, by restrictions in the Technical Specifications (Appendix A) which accompany the license, to fuel loading, low-power testing, and operation at one (1) percent of the rated power level of 2381 megawatts thermal until the following matters have been resolved: (a) reactor internals vibration cold flow tests are completed and evaluated against the James A. Fitzpatrick Nuclear Power Plant (Docket No. 50-333) which has been designated as the prototype for reactor internals vibration testing; and (b) procedures for operation at above one (1) percent rated power have been completed and approved. A related notice, which is being forwarded to the Office of the Federal Register for filing and publication, is enclosed for your information.

Two signed originals of Amendment No. 4 to Indemnity Agreement No. B-57, which covers the activities authorized under License No. DPR-46, are enclosed for your review and acceptance. Please sign and return one copy of Amendment No. 4 to this office.

Sincerely,

~~Original Signed by~~

Voss A. Moore, Assistant Director
for Light Water Reactors, Group 2
Directorate of Licensing

Enclosures: See page 2

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OFFICE >						
SURNAME >						
DATE >						

February 12, 2002

Mr. David L. Wilson
Vice President of Nuclear Energy
Nebraska Public Power District
P. O. Box 98
Brownville, NE 68321

**SUBJECT: COOPER NUCLEAR STATION - CORRECTION OF TYPOGRAPHICAL ERROR
IN AMENDMENT NO. 178**

Dear Mr. Swailes:

This is in response to the letter of December 14, 2001 (NLS2001110), from David L. Wilson, Vice President of Nuclear Energy, which identified an error on a Technical Specification (TS) page in Amendment No. 178, which was issued July 31, 1998. The error is that an incorrect surveillance requirement (SR) was listed for Function 6.a on page 3.3-53 of TS Table 3.3.6.1-1. This was discussed by phone with your staff on February 6, 2002.

The reference to SR 3.3.6.1.5 in Table 3.3.6.1-1 for Function 6.a is incorrect and should be SR 3.3.6.1.6. SR 3.3.6.1.5 concerns calibration of radiation detectors which duplicates SR 3.3.6.1.4 listed for Function 6.a on page 3.3-53. The correct surveillance SR 3.3.6.1.6 requires a logic system functional test. The NRC staff has verified that the error was not submitted as part of the conversion to the Improved Technical Specifications that were issued in Amendment No. 178, and SR 3.3.6.1.6 was submitted for Function 6.a in the licensee's letters for the amendment. Therefore, issuing incorrect TS page 3.3-53 in Amendment No. 178 was an inadvertent error. Based on this, the staff is issuing the enclosed corrected page for page 3.3-53 of the TSs.

Sincerely,

/RA by Robert A. Gramm for/

Jack N. Donohew, Senior Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosure: Corrected TS Page

cc w/encl: See next page

May 9, 2002

Mr. David L. Wilson
Vice President of Nuclear Energy
Nebraska Public Power District
P. O. Box 98
Brownville, NE 68321

**SUBJECT: COOPER NUCLEAR STATION - ISSUANCE OF AMENDMENT REGARDING
CORRECTIONS OF ERRORS IN TECHNICAL SPECIFICATIONS
(TAC NO. MB2004)**

Dear Mr. Wilson:

The Commission has issued the enclosed Amendment No. 191 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station (CNS). The amendment consists of changes to Technical Specification Table 3.3.1.1-1, (1) to correct typographic errors, and (2) to delete the fixed portion of the allowable value in TS Table 3.3.1.1-1, function 2.b, in response to Nebraska Public Power District (the licensee) application dated May 9, 2001, as supplemented by e-mail dated February 28, 2002.

The licensee's application of May 9, 2001, erroneously described change (2) as correction of an error, when it was a request for change. By an electronic mail dated February 28, 2002, the licensee provided a correction to its May 9, 2001, request. The NRC staff has reviewed the revised change (2), and finds it acceptable.

Additionally, the licensee has provided revised Technical Specification Bases pages for incorporation in the CNS Bases document. The Nuclear Regulatory Commission staff noted an error in the Technical Specification Bases pages and discussed it with the licensee by telephone on November 28, 2001. The licensee provided corrected Bases pages by electronic mail dated November 28, 2001. The corrected Bases pages are included in this amendment.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Mohan C. Thadani, Senior Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosures: 1. Amendment No. 191 to DPR-46
2. Safety Evaluation

cc w/encls: See next page

March 6, 2003

Mr. Clay C. Warren
Vice President of Nuclear Energy
Nebraska Public Power District
P. O. Box 98
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - ISSUANCE OF AMENDMENT ON MISSED SURVEILLANCES (TAC NO. MB6547)

Dear Mr. Warren:

The Commission has issued the enclosed Amendment No. 197 to Facility Operating License No. DPR-46 for the Cooper Nuclear Station. The amendment consists of changes to the Technical Specifications in response to your application dated September 26, 2002.

The amendment revises Surveillance Requirement (SR) 3.0.3 to extend the delay period, before entering a Limiting Condition for Operation, following a missed surveillance. The delay period is extended from the current limit of "... up to 24 hours or up to the limit of the specified Frequency, whichever is less" to "...up to 24 hours or up to the limit of the specified Frequency, whichever is greater." In addition, the following requirement is added to SR 3.0.3: "A risk evaluation shall be performed for any Surveillance delayed greater than 24 hours and the risk impact shall be managed."

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Mohan C. Thadani, Senior Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosures: 1. Amendment No. 197 to DPR-46
2. Safety Evaluation

cc w/encls: See next page