



April 29, 2004

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Subject:

Emergency Operating Procedures

R.E. Ginna Nuclear Power Plant

Docket No. 50-244

As requested, enclosed are Ginna Station Emergency Operating Procedures.

Very truly yours,

X

JAW/jdw

xc:

U.S. Nuclear Regulatory Commission

Region I

475 Allendale Road

King of Prussia, PA 19406-1415

Ginna USNRC Senior Resident Inspector

Enclosure(s):

ATT Index E Index ATT-16.0, Rev 12 E-3, Rev 37

An equal opportunity employer

A002



NPSP0200 WRIGHTJ

Ginna Nuclear Power Plant PROCEDURE INDEX

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5 YEARS ONLY: INPUT PARAMETERS: TYPE: **PRATT** STATUS VALUE(S): EF, QU PRATT **EOP ATTACHMENTS** PROCEDURE **EFFECT** LAST NEXT ST REVIEW NUMBER PROCEDURE TITLE REV DATE REVIEW 盱 ATT-1.0 ATTACHMENT AT POWER CCW ALIGNMENT 02/12/2008 003 02/12/2003 02/12/2003 F ATT-1.1 ATTACHMENT NORMAL CCW FLOW 000 05/18/2000 05/18/2000 05/18/2005 ATT-2.1 ATTACHMENT MIN SW 005 02/03/2003 02/03/2008 FF 02/01/2001 ATT-2.2 ATTACHMENT SW ISOLATION 800 03/06/2002 03/27/2003 03/27/2008 Œ ATT-2.3 ATTACHMENT SW LOADS IN CNMT 004 03/06/2002 04/06/2004 04/06/2009 盱 F ATT-2.4 ATTACHMENT NO SW PUMPS 002 05/30/2003 10/31/2001 10/31/2006 F ATT-2.5 ATTACHMENT SPLIT SW HEADERS 000 06/26/2002 06/26/2002 06/26/2007 Œ ATT-3.0 007 ATTACHMENT CVCVI 02/17/2004 02/17/2004 02/17/2009 ATT-3.1 02/17/2004 F ATTACHMENT CNMT CLOSURE 005 02/17/2004 02/17/2009 F ATT-4.0 ATTACHMENT CNMT RECIRC FANS 003 07/26/1994 03/27/2003 03/27/2008 ATT-5.0 04/06/2009 F ATTACHMENT COND TO S/G 006 10/10/2003 04/06/2004 ATT-5.1 ATTACHMENT SAFW : 800 05/30/2002 04/06/2004 04/06/2009 Œ ATT-5.2 ATTACHMENT FIRE WATER COOLING TO TDAFW PUMP 003 01/14/1999 01/28/2004 01/28/2009 F ATT-6.0 ATTACHMENT COND VACUUM 003 12/18/1996 02/03/2003 02/03/2008 Æ ATT-7.0 ATTACHMENT CR EVAC 006 03/06/2002 02/03/2003 02/03/2008 F ATT-8.0 ATTACHMENT DC LOADS 007 02/04/2004 02/04/2004 02/04/2009 Æ ATT-8.1 ATTACHMENT D/G STOP 005 03/06/2002 02/03/2003 02/03/2008 Œ ATT-8.2 ATTACHMENT GEN DEGAS 008 06/20/2002 08/17/1999 08/17/2004 Œ ATT-8.3 ATTACHMENT NONVITAL 004 03/06/2002 02/03/2003 02/03/2008 F ATT-8.4 Œ ATTACHMENT SVUV 005 03/06/2002 02/03/2003 02/03/2008 ATT-8.5 ATTACHMENT LOSS OF OFFSITE POWER 001 08/26/2003 05/02/2007 盱 05/02/2002 ATT-9.0 009 01/07/2004 F ATTACHMENT LETDOWN 03/06/2002 03/06/2007 ATT-9.1 F ATTACHMENT EXCESS L/D 005 03/06/2002 10/31/2001 10/31/2006 Œ ATT-10.0 03/06/2002 03/27/2008 ATTACHMENT FAULTED S/G 006 03/27/2003 F ATT-11.0 ATTACHMENT IA CONCERNS 003 06/26/2003 03/27/2003 03/27/2008 ATT-11.1 ATTACHMENT IA SUPPLY 03/06/2002 03/27/2008 003 03/27/2003 F ATT-11.2 ATTACHMENT DIESEL AIR COMPRESSOR 004 11/18/2002 03/10/2003 03/10/2008 F ATT-12.0 ATTACHMENT N2 PORVS 005 02/12/2003 02/12/2003 02/12/2008 Æ ATT-13.0 F ATTACHMENT NC 003 02/12/2003 02/12/2003 02/12/2008 ATT-14.0 ATTACHMENT NORMAL RHR COOLING 003 03/06/2002 04/06/2004 04/06/2009 臣

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INPUT PARAMETERS: TYPE: PRATT STATUS VALUE(S): EF. QU 5 YEARS ONLY: PRATT **EOP ATTACHMENTS** PROCEDURE **EFFECT** LAST NEXT ST NUMBER PROCEDURE TITLE REV DATE REVIEW REVIEW ATT-14.1 盱 ATTACHMENT RHR COOL 006 04/30/2003 01/08/2002 01/08/2007 ATT-14.2 F ATTACHMENT RHR ISOL 003 02/12/2003 02/12/2003 02/12/2008 ATT-14.3 F ATTACHMENT RHR NPSH 003 03/06/2002 01/28/2004 01/28/2009 ATT-14.5 ATTACHMENT RHR SYSTEM 003 03/20/2003 02/03/2003 02/03/2008 F ATT-14.6 ATTACHMENT RHR PRESS REDUCTION 002 03/06/2002 01/28/2004 01/28/2009 Æ F ATT-14.7 ATTACHMENT ADJUST RHR FLOW 000 02/17/2004 02/17/2004 02/17/2009 F ATT-15.0 ATTACHMENT RCP START 009 03/06/2002 03/17/2000 03/17/2005 F ATT-15.1 ATTACHMENT RCP DIAGNOSTICS 003 04/24/1997 02/03/2003 02/03/2008 ATT-15.2 Œ ATTACHMENT SEAL COOLING 005 03/06/2002 02/03/2003 02/03/2008 ATT-16.0 F ATTACHMENT RUPTURED S/G 012 04/29/2004 01/11/2000 01/11/2005 ATT-16.1 ATTACHMENT SGTL 002 03/06/2002 09/08/2000 09/08/2005 Œ. ATT-16.2 ATTACHMENT RCS BORON FOR SGTL 003 11/26/2003 09/08/2000 09/08/2005 Œ ATT-17.0 **ATTACHMENT SD-1** 016 10/10/2003 02/29/2000 02/28/2005 F ATT-17,1 006 03/06/2002 01/30/2006 F ATTACHMENT SD-2 01/30/2001 ATT-18.0 F ATTACHMENT SFP - RWST 005 03/06/2002 02/03/2003 02/03/2008 ATT-20.0 ATTACHMENT VENT TIME 003 07/26/1994 02/03/2003 02/03/2008 Ħ ATT-21.0 ATTACHMENT RCS ISOLATION 002 03/06/2002 02/03/2003 02/03/2008 F ATT-22.0 ATTACHMENT RESTORING FEED FLOW 003 05/02/2002 01/22/2002 01/22/2007 F ATT-23.0 ATTACHMENT TRANSFER 4160V LOADS 000 02/26/1999 01/28/2004 01/28/2009 F ATT-24.0 ATTACHMENT TRANSFER BATTERY TO TSC 000 09/08/2000 09/08/2000 09/08/2005 臣 ATT-26.0 F ATTACHMENT RETURN TO NORMAL OPERATIONS 000 10/31/2001 10/31/2001 10/31/2006 **PRATT** TOTAL: 51

GRAND TOTAL: 51

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		STATUS VALUE(S): EF, QU	5 YEARS OF		مان مان مان مان مان مان مان المان مان مان مان مان مان مان مان مان مان	· # 4.5 +679 5
PROCEDURE NUMBER	PROCEDURE TITLE		EFFECT V DATE	LAST, REVIEW	NEXT REVIEW	S
E-0	REACTOR TRIP OR SAFETY INJECTION	030		03/24/2003	03/24/2008	Е
E-1	LOSS OF REACTOR OR SECONDARY COOLANT	029	02/17/2004	03/24/2003	03/24/2008	E
E-2	FAULTED STEAM GENERATOR ISOLATION	01	05/30/2003	03/24/2003	03/24/2008	E
E-3	STEAM GENERATOR TUBE RUPTURE	03	04/29/2004	03/24/2003	03/24/2008	E

GRAND TOTAL: 4

EOP:	TITLE:	REV: 12
ATT-16.0	ATTACHMENT RUPTURED S/G	
	·	PAGE 1 of 4

Responsible Manager Resulting Date 4-29-2004

NOTE: Intermediate Building environment should be assessed for radiological and other personnel safety concerns.

- PART A. Dispatch AO with locked valve key to complete local isolation of ruptured S/G as follows:
 - Check closed the ruptured S/G MSIV

o S/G A, V-3517 OR

o S/G B, V-3516

<u>IF</u> ruptured S/G MSIV is <u>NOT</u> closed, <u>THEN</u> perform the following to close ruptured S/G MSIV:

- o S/G A
 - Close IA isol to S/G A MSIV, V-5408A
 - Open emerg vent valves V-5471 AND V-5473
 - Notify Control Room S/G A MSIV is closed

<u>OR</u>

- o S/G B
 - Close IA isol to S/G B MSIV, V-5409B
 - Open emerg vent valves V-5472 AND V-5474
 - Notify Control Room S/G B MISV is closed
- 2) Ensure <u>BOTH</u> S/G MSIV bypass valves closed (INT BLDG steam header area):
 - o S/G A, V-3615
 - o S/G B, V-3614
- 3) <u>IF</u> the ruptured S/G MSIV can <u>NOT</u> be closed, <u>THEN</u> perform the | following:
 - a) Close Air Ejector/Gland steam root valve, V-3540 (Main steam header TURB BLDG).
 - b) Close flange heating isolation valves, MOV-3601A and MOV-3602A.
 - c) Notify Control Room that main flowpaths are isolated.

EOP:	TITLE:	REV: 12
ATT-16.0	ATTACHMENT RUPTURED S/G	
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CAUTION

CONTROL ROOM SHOULD BE NOTIFIED BEFORE ISOLATING TDAFW PUMP STEAM FLOW.

NOTE: The following steps may be done in any order for PART A.

- 4) Locally close TDAFW Pump steam root valve
 - o S/G A, V-3505 -OR-
 - o S/G B, V-3504
- 5) Locally close the following steam valves from the ruptured S/G:
 - o Steam to sampling system valve (INT BLDG steam header area):
 S/G A, close V-3413A
 -OR-

S/G B, close V-3412A

- o Support heating steam valve (INT BLDG steam header area):

 S/G A, close V-3669

 -OR
 S/G B, close V-3668
- O Upstream trap isolation valve (TURB BLDG near MFW reg vlvs):
 S/G A, close V-3521
 -OR-

S/G B, close V-3520

CAUTION

CONTROL ROOM SHOULD BE NOTIFIED BEFORE ISOLATING TDAFW PUMP FEED FLOW.

6) Locally close TDAFW pump manual feedwater isolation valve to ruptured S/G (INT BLDG steam header area):

S/G A, V-4005 -OR-S/G B, V-4006

EOP:	TITLE:	REV: 12
ATT-16.0	ATTACHMENT RUPTURED S/G	100. 12
	8 54	PAGE 3 of 4

- Bypass condensate polishing demineralizers as follows: 7)
 - Place AVT bypass valve controller in MANUAL (east a. end of AVT panel).
 - b. Open bypass valve.
 - Isolate ALL inservice demineralizers as follows:
 - Place the Mixed Bed Service Selector switch to override <u>AND</u> select the bed to be removed from service.
 - 2) Place the 4 position selector switch for the selected bed to OFF.
 Repeat steps 1 and 2 for each inservice bed.
 - 3)
- Locally place TURB RM WALL EXH FAN switches to CLOSE. 8)
- 9) Locally place TURB RM ROOF VENT switches to CLOSE.

EOP:	TITLE:	REV: 12
ATT-16.0	ATTACHMENT RUPTURED S/G	MDV. IZ
		PAGE 4 of 4

CAUTION

PART B OF THIS ATTACHMENT SHOULD ONLY BE PERFORMED IF RUPTURED S/G MSIV CANNOT BE CLOSED.

PART B. Dispatch AO to locally perform the following when ruptured S/G MSIV cannot be closed, if areas are accessible:

1) Complete isolation by closing the following valves:

NOTE: Substeps may be done in any order.

- a) MFW regulating valve and bypass valve manual isolation valves for both S/Gs:
 - o S/G A, V-3985 and V-3989
 - o S/G B, V-3984 and V-3988
- b) Reheat steam chain valves:
 - o 1A MSR, V-3551
 - o 1B MSR, V-3550
 - o 2A MSR, V-3553
 - o 2B MSR, V-3552
- c) Steam dump header isolation and bypass valves (Main steam header TURB BLDG on platform overhead)
 - o V-3532 and V-3659
 - o V-3533 and V-3658
- d) Reheat steamline warmup valves (warmup vlvs located east end of 1A and 2A MSRs TURB BLDG middle floor):
 - o V-3645
 - o V-3646
 - o V-3647
 - o V-3648
- e) Reheat steamline common vent, V-8500 (at condenser north of 1A MSR).
- f) Steam to trap header isolation valves
 - o V-8513 (Main steam header TURB BLDG)
 - o V-8529 (south side EH skid)
- g) Steam trap isolation and bypass valves
 - o V-3596 (south side of EH skid)
 - o V-3598 (south side of EH skid)

EOP:	STEAM GENERATOR TUBE RUPTURE	REV: 37
F-2	SIEAM GENERATOR TOBE ROPTORE	PAGE 1 of 42

ROCHESTER GAS AND ELECTRIC CORPORATION

GINNA STATION

CONTROLLED COPY NUMBER _

23

RESPONSIBLE MANAGER

H-29-200H
EFFECTIVE DATE

CATEGORY 1.0

REVIEWED BY:

EOP:	TITLE:	REV: 37
E-3	STEAM GENERATOR TUBE RUPTURE	PAGE 2 of 42

A. PURPOSE - This procedure provides actions to terminate leakage of reactor coolant into the secondary system following a steam generator tube rupture (SGTR), this procedure should also be used for a SGTR in one S/G and a fault in the other S/G.

B. ENTRY CONDITIONS/SYMPTOMS

- 1. ENTRY CONDITIONS This procedure is entered from:
 - a. E-0, REACTOR TRIP OR SAFETY INJECTION, when condenser air ejector radiation or blowdown radiation is abnormal, or
 - b. E-0, REACTOR TRIP OR SAFETY INJECTION, E-1, LOSS OF REACTOR OR SECONDARY COOLANT, E-2, FAULTED STEAM GENERATOR ISOLATION, and FR-H.3, RESPONSE TO STEAM GENERATOR HIGH LEVEL, when secondary radiation is abnormal, or
 - c. E-0, REACTOR TRIP OR SAFETY INJECTION, E-1, LOSS OF REACTOR OR SECONDARY COOLANT, E-2, FAULTED STEAM GENERATOR ISOLATION, ES-1.2, POST LOCA COOLDOWN AND DEPRESSURIZATION, ECA-3.3, SGTR WITHOUT PRESSURIZER PRESSURE CONTROL, when an intact S/G narrow range level increases in an uncontrolled manner.
 - d. ECA-3.3, SGTR WITHOUT PRESSURIZER PRESSURE CONTROL, when pressurizer pressure control is restored.
 - e. Any FOLDOUT page that has E-3 transition criteria whenever either S/G level increases in an uncontrolled manner or either S/G has abnormal radiation.

EOP:	TITLE:	REV: 37
E-3	STEAM GENERATOR TUBE RUPTURE	5.55
	· [8]	PAGE 3 of 42

STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: o FOLDOUT page should be open AND monitored periodically.

- o Critical Safety Function Status Trees should be monitored (Refer to Appendix 1 for Red Path Summary).
- o Personnel should be available for sampling during this procedure.
- o Conditions should be evaluated for Site Contingency Reporting (EPIP-1.0, GINNA STATION EVENT EVALUATION AND CLASSIFICATION).
- o Adverse CNMT values should be used whenever CNMT pressure is greater than 4 psig or CNMT radiation is greater than 10^{+05} R/hr.
- * 1 Monitor RCP Trip Criteria:
 - a. RCP status ANY RCP RUNNING
- a. Go to Step 2.
- b. SI pumps AT LEAST TWO RUNNING
- b. Go to Step 2.
- c. RCS pressure minus maximum S/G pressure LESS THAN 175 psig [400 psig adverse CNMT]
- c. Go to Step 2.

d. Stop both RCPs

EOP: TITLE: **REV: 37** STEAM GENERATOR TUBE RUPTURE E-3 PAGE 4 of 42 ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED STEP IF OFFSITE POWER IS LOST AFTER SI RESET. THEN MANUAL ACTION MAY BE REQUIRED TO RESTART SAFEGUARDS EQUIPMENT. (REFER TO ATT-8.5, ATTACHMENT LOSS OF OFFSITE POWER) 2 Identify Ruptured S/G(s): Perform the following: o Unexpected increase in either a. Reset SI S/G narrow range level b. Continue with Steps 10 -ORthrough 16. WHEN ruptured S/G(s) identified. THEN do Steps o High radiation indication on 3 through 9. main steamline radiation monitor • R-31 for S/G A • R-32 for S/G B -ORo AO reports local indication of high steamline radiation · -ORo RP reports high radiation from S/G activity sample

EOP:	TITLE: STEAM GENERATOR TUBE RUPTURE	REV:	37	
E-3		PAGE	5 of	42

ACTION/EXPECTED RESPONSE

STEP

RESPONSE NOT OBTAINED

- IF THE TDAFW PUMP IS THE ONLY AVAILABLE SOURCE OF FEED FLOW. STEAM SUPPLY TO THE TDAFW PUMP MUST BE MAINTAINED FROM ONE S/G.
- o AT LEAST ONE S/G SHALL BE MAINTAINED AVAILABLE FOR RCS COOLDOWN.
- 3 Isolate Flow From Ruptured S/G(s):
 - a. Adjust ruptured S/G ARV controller to 1050 psig in AUTO
 - b. Check ruptured S/G ARV CLOSED
 - isolate.
 - c. Close ruptured S/G TDAFW pump steam supply valve and place in PULL STOP
 - S/G A. MOV-3505A
 - S/G B. MOV-3504A
 - d. Verify ruptured S/G blowdown valve - CLOSED
 - S/G A. AOV-5738
 - S/G B. AOV-5737

- b. WHEN ruptured S/G pressure less than 1050 psig. THEN verify S/G ARV closed. IF NOT closed. THEN place controller in MANUAL and close S/G ARV.
 - IF S/G ARV can NOT be closed. THEN dispatch AO to locally
- c. Dispatch AO with locked valve key to locally isolate steam from ruptured S/G to TDAFW pump.
 - S/G A, V-3505
 - S/G B. V-3504
- d. Place S/G blowdown and sample valve isolation switch to CLOSE.
 - IF blowdown can NOT be isolated manually, THEN dispatch AO to locally isolate blowdown.
 - S/G A. V-5701
 - S/G B. V-5702

			
•	EOP: TITLE:		
		• .	REV: 37
•	E-3	STEAM GENERATOR TUBE RUPTURE	··
•			PAGE 6 of 42
1 1			
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	STEP ACTION/EXPECT	ED RESPONSE NOT OBTAINED	
	4 Complete Rupture	d S/G	
	Isolation:	- 5, 5	
•	: .		
• • •	a. Close ruptured S	/G MSIV - a. Perform the following	ng:
	RUPTURED S/G MSI		
		1) Close intact S/G	MSIV.
		0) 21	
		2) Place intact S/G	
	1.	controller at 100 AUTO.	oo psig in
	,	AUIO.	
	, :	3) Adjust condenser	steam dump
		controller to 10	
		AUTO.	F6
		4) Place condenser,	
		mode selector sw	itch to
•		MANUAL.	
		5) Alexandra - Land	
		5) Adjust reheat sto controller cam to	eam supply
\bigcirc		reheat steam sup	
		renear becam sup	ory varves.
		6) Ensure turbine s	top valves -
	•	CLOSED.	
			·
	•	7) Dispatch AO to co	
•		ruptured S/G iso	lation (Refer
		to ATT-16.0. ATTA RUPTURED S/G, par	ACHMENT
		RUFTURED 5/G, par	its A allu b).
•		8) Go to step 5.	
·			•
,	b. Dispatch AO to c	omplete ruptured	,
	S/G isolation (R		
,	ATT-16.0. ATTACH	MENT RUPTURED	
,	S/G part A)		
		•	
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E-3	STEAM GENERATOR TUBE RUPTURE	PAGE 7 of 42

ACTION/EXPECTED RESPONSE

STEP

RESPONSE NOT OBTAINED

IF ANY RUPTURED S/G IS FAULTED. FEED FLOW TO THAT S/G SHOULD REMAIN ISOLATED DURING SUBSEQUENT RECOVERY ACTIONS UNLESS NEEDED FOR RCS COOLDOWN.

- 5 Check Ruptured S/G Level:
 - a. Narrow range level GREATER THAN 5% [25% adverse CNMT]
- a. IF ruptured S/G NOT faulted. THEN perform the following: -
 - 1) Maintain feed flow to ruptured S/G until level greater than 5% [25% adverse CNMT].
 - 2) Continue with Step 6. WHEN ruptured S/G level greater than 5% [25% adverse CNMT]. THEN do Steps 5b through e.
- b. Close MDAFW pump discharge valve b. Dispatch AO to locally close to ruptured S/G
 - S/G A, MOV-4007
 - S/G B. MOV-4008
- c. Pull stop MDAFW pump for ruptured S/G
- d. Close TDAFW pump flow control valve to ruptured S/G
 - S/G A. AOV-4297
 - S/G B. AOV-4298
- e. Verify MDAFW pump crosstie valves - CLOSED
 - MOV-4000A
 - MOV-4000B

valve.

- d. Dispatch AO with locked valve key to locally close TDAFW pump manual feedwater isolation valve to ruptured S/G.
 - S/G A, V-4005
 - S/G B, V-4006
- e. Manually close valves.

STEP ACT

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- 6 Verify Ruptured S/G Isolated:
 - a. Check ruptured MSIV CLOSED
 - b. Check TDAFW pump steam supply from ruptured S/G ISOLATED
 - c. Ruptured S/G pressure GREATER
 . THAN 300 PSIG
- 7 Establish Condenser Steam Dump Pressure Control:
 - a. Verify condenser available:
 - o Intact S/G MSIV OPEN
 - o Annunciator G-15, STEAM DUMP ARMED - LIT
 - b. Adjust condenser steam dump controller HC-484 to maintain intact S/G pressure and verify in AUTO
 - c. Place steam dump mode selector switch to MANUAL

- a. Direct AO to immediately isolate air ejector/gland steam supply and flange heating steam.
 (Refer to ATT-16.0, ATTACHMENT RUPTURED S/G, part A).
- b. Continue efforts to isolate steam supply from ruptured S/G:
 - S/G A, MOV-3505A OR V-3505
 - S/G B, MOV-3504A OR V-3504
- c. Go to ECA-3.1. SGTR WITH LOSS OF REACTOR COOLANT SUBCOOLED RECOVERY DESIRED, Step 1.
- a. Adjust S/G ARV controllers to maintain intact S/G pressure in AUTO and go to Step 8.

EOP: TITLE: REV: 37
E-3 STEAM GENERATOR TUBE RUPTURE PAGE 9 of 42

STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

IF OFFSITE POWER IS LOST AFTER SI RESET. THEN MANUAL ACTION MAY BE REQUIRED TO RESTART.SAFEGUARDS EQUIPMENT. (REFER TO ATT-8.5. ATTACHMENT LOSS OF OFFSITE POWER)

8 Reset SI

EOP: TITLE: **REV: 37** E-3 STEAM GENERATOR TUBE RUPTURE

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STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

RUPTURED S/G LEVEL SHALL BE MAINTAINED GREATER THAN 5% [25% ADVERSE CNMT] DURING THE RCS COOLDOWN, UNLESS THE RUPTURED S/G IS ALSO FAULTED.

Following initiation of controlled cooldown or depressurization, RCP trip criteria is no longer applicable.

9 Initiate RCS Cooldown:

a. Determine required core exit temperature from below table

RUPTURED	REQUIRED CORE EXIT
SG PRESSURE	TEMPERATURE (°F)
1100 PSIG	525 [517 adverse CNMT]
1000 PSIG	512 [504 adverse CNMT]
900 PSIG	499 [490 adverse CNMT]
800 PSIG	484 [475 adverse CNMT]
700 PSIG	468 [457 adverse CNMT]
600 PSIG	449 [437 adverse CNMT]
500 PSIG	428 [413 adverse CNMT]
400 PSIG	402 [384 adverse CNMT]
300 PSIG	369 [344 adverse CNMT]

- b. IF ruptured S/G MSIV closed. THEN initiate dumping steam to condenser from intact S/G at maximum rate
- b. Manually or locally initiate steam dump from intact S/G at maximum rate using S/G ARV.

<u>IF</u> no intact S/G available. <u>THEN</u> perform the following:

o Use faulted S/G.

-OR-

- o <u>IF</u> a ruptured S/G must be · used. THEN go to ECA-3.1.
 SGTR WITH LOSS OF REACTOR
 COOLANT - SUBCOOLED RECOVERY
 DESIRED. Step 1.
- c. Continue with Step 10. WHEN core exit T/Cs less than required. THEN do Step 9d.
- c. Core exit T/Cs LESS THAN REQUIRED TEMPERATURE
- d. Stop RCS cooldown and stabilize core exit T/Cs less than required temperature

STEP |

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: TDAFW pump flow control AOVs may drift open on loss of IA.

10 Monitor Intact S/G Level:

- a. Narrow range level GREATER
 THAN 5% [25% adverse CNMT]
- b. Control feed flow to maintain narrow range level between 17% intact S/G continues to increase [25% adverse CNMT] and 50% in an uncontrolled manner. The state of the intact size of t
- a. Maintain total feed flow greater than 200 gpm until narrow range level greater than 5% [25% adverse CNMT] in at least one S/G.
 - b. <u>IF</u> narrow range level in the intact S/G continues to increase in an uncontrolled manner. <u>THEN</u> go to ECA-3.1. SGTR WITH LOSS OF REACTOR COOLANT SUBCOOLED RECOVERY DESIRED. Step 1.

EOP: E-3	TITLE:	STEAM	STEAM GENERATOR TUBE RUPTURE	•	REV:	37			
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STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

CAUTION

IF ANY PRZR PORV OPENS BECAUSE OF HIGH PRZR PRESSURE. IT SHOULD BE CLOSED AFTER PRESSURE DECREASES TO LESS THAN 2335 PSIG (REFER TO STEP 11B).

- 11 Monitor PRZR PORVs And Block Valves:
 - a. Power to PORV block valves -AVAILABLE

b. PORVs - CLOSED

- a. Restore power to block valves unless block valve was closed to isolate an open PORV:
 - MOV-515. MCC D position 6C
 - MOV-516, MCC C position 6C
- b. <u>IF</u> PRZR pressure less than 2335 psig. <u>THEN</u> manually close PORVs.

IF any PORV can NOT be closed, THEN manually close its block valve. IF block valve can NOT be closed, THEN go to ECA-3.1, SGTR WITH LOSS OF REACTOR COOLANT - SUBCOOLED RECOVERY DESIRED. Step 1.

- c. Block valves AT LEAST ONE OPEN
- c. Open one block valve unless it was closed to isolate an open PORV.

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E-3 STEAM GENERATOR TUBE RUPTURE PAGE 13 of 42

STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

12 Reset CI:

- a. Depress CI reset pushbutton
- b. Verify annunciator A-26, CNMT ISOLATION EXTINGUISHED
- b. Perform the following:
 - 1) Reset SI.
 - 2) Depress CI reset pushbutton.

E-3 STEAM GENERATOR TUBE RUPTURE	: 37 E 14	of	42
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13 Monitor All AC Busses -BUSSES ENERGIZED BY OFFSITE POWER

- o Normal feed breakers to all 480 volt busses CLOSED
- o 480 volt bus voltage GREATER THAN 420 VOLTS
- o Emergency D/G output breakers OPEN

Perform the following:

- a. <u>IF</u> any AC emergency bus normal feed breaker open. <u>THEN</u> ensure associated D/G breaker closed.
- b. Perform the following as necessary:
 - 1) Close non-safeguards bus tie breakers:
 - Bus 13 to Bus 14 tie
 - Bus 15 to Bus 16 tie
 - 2) Place the following pumps in PULL STOP:
 - EH pumps
 - Turning gear oil pump
 - HP seal oil backup pump
 - 3) Restore power to MCCs.
 - A from Bus 13
 - B from Bus 15
 - E from Bus 15
 - F from Bus 15
 - 4) Start HP seal oil backup pump.
 - 5) Start CNMT RECIRC fans as necessary.
 - 6) Ensure D/G load within limits.
 - 7) <u>WHEN</u> bus 15 restored. <u>THEN</u> reset control room lighting.
- c. Try to restore offsite power to all AC busses (Refer to ER-ELEC.1, RESTORATION OF OFFSITE POWER).

E-3 STEAM GENERATO	OR TUBE RUPTURE
E-3 STEAM GENERATO	PAGE 15 of 42
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
14 Verify Adequate SW Flow:	
a. Check at least two SW pumps - RUNNING	 a. Manually start SW pumps as power supply permits (257 kw each).
	<u>IF</u> less than two SW pumps running, <u>THEN</u> :
	1) Ensure SW isolation.
·	2) <u>IF NO</u> SW pumps running. <u>THEN</u> perform the following:
•	a) Pull stop any D/G that is <u>NOT</u> supplied by alternate cooling. <u>AND</u> immediately depress VOLTAGE SHUTDOWN pushbutton.
	b) Refer to ATT-2.4. ATTACHMENT NO SW PUMPS.
	3) <u>IF</u> only one SW pump running. <u>THEN</u> refer to AP-SW.2. LOSS OF SERVICE WATER.

b. Dispatch AO to establish normal shutdown alignment (Refer to ATT-17.0, ATTACHMENT SD-1)

TITLE: EOP: REV: ,37 E-3 STEAM GENERATOR TUBE RUPTURE PAGE 16 of 42 ACTION/EXPECTED RESPONSE STEP RESPONSE NOT OBTAINED 15 Establish IA to CNMT: a. Verify non-safeguards busses a. Perform the following: energized 1) Close non-safeguards bus tie o Bus 13 normal feed - CLOSED breakers: -OR-• Bus 13 to Bus 14 tie • Bus 15 to Bus 16 tie o Bus 15 normal feed - CLOSED 2) Verify adequate emergency D/G capacity to run air compressors (75 kw each). <u>IF NOT</u>, <u>THEN</u> perform the following: o Start diesel air compressor (refer to ATT-11.2. ATTACHMENT DIESEL AIR COMPRESSOR) -ORo Evaluate if CNMT RECIRC fans should be stopped (Refer to ATT-4.0. ATTACHMENT CNMT RECIRC FANS) b. Check SW pumps - AT LEAST TWO b. Perform the following: PUMPS RUNNING 1) Restore IA using service air compressor OR diesel air compressor (refer to ATT-11.2. ATTACHMENT DIESEL AIR COMPRESSOR) 2) Go to step 15d. c. Verify turbine building SW c. Manually align valves. isolation valves - OPEN

MOV-4613 and MOV-4670MOV-4614 and MOV-4664

This Step continued on the next page.

EOP: E-3	STEAM GENERATOR	TUBE RUPTURE	REV: 37 PAGE 17 of 42
L L	CTION/EXPECTED RESPONSE		.
	ify adequate air compressors JNNING -	d. Manually start air c as power supply perm each). <u>IF</u> air compr <u>NOT</u> be started. <u>THEN</u> to locally reset com necessary.	nits (75 kw ressors can I dispatch AO
o P	ck IA supply: Pressure - GREATER THAN 50 PSIG	e. Perform the followin 1) Continue attempts IA (Refer to AP-I	to restore
	Pressure - STABLE OR INCREASING	INSTRUMENT AIR). 2) Continue with Ste IA restored, <u>THEN</u> 15f and g.	
	et both trains of XY relays IA to CNMT AOV-5392		
g. Veri	ify IA to CNMT AOV-5392 - OPEN		
	CAUTION	<u>N</u>	
UNCONTROLL	URE SHOULD BE MONITORED. IF RCS LED MANNER TO LESS THAN 250 PSIC MUST BE MANUALLY RESTARTED TO S	G [465 PSIG ADVERSE CNMT].	
	· • • • • • • • • • • • • • • • • • • •		
16 Check Stoppe	If RHR Pumps Should Be,		
	ck RCS pressure - GREATER V 250 psig [465 psig adverse V]	a. Go to Step 17.	·
b. Stop AUTO	RHR pumps and place both in	·	

EOP: TITLE:

E-3 STEAM GENERATOR TUBE RUPTURE

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STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- 17 Establish Charging Flow:
 - a. Charging pumps ANY RUNNING
- a. Perform the following:
 - 1) IF CCW flow is lost to any RCP thermal barrier OR any RCP #1 seal outlet temperature offscale high.

 THEN dispatch AO to locally close seal injection needle valves to affected RCP.
 - V-300A for RCP A
 - V-300B for RCP B
 - 2) Ensure HCV-142 demand at 0%.
- b. IF LCV-112B can NOT be opened. THEN dispatch AO to locally open V-358, manual charging pump suction from RWST (charging pump room).

<u>IF</u> LCV-112C can <u>NOT</u> be closed. <u>THEN</u> perform the following:

- 1) Direct AO to locally open V-358, manual charging pump suction from RWST (charging pump room).
- Verify charging pump A <u>NOT</u> running and place in PULL STOP.
- 3) WHEN V-358 open, THEN direct AO to close V-268 to isolate charging pumps B and C from VCT (charging pump room).

- b. Align charging pump suction to RWST:
 - o LCV-112B OPEN
 - o LCV-112C CLOSED

- c. Start charging pumps as necessary and establish 75 gpm total charging flow
 - Charging line flow
 - Seal injection

TITLE: EOP: **REV: 37** E-3STEAM GENERATOR TUBE RUPTURE PAGE 19 of 42 STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED 18 Check If RCS Cooldown Should Be Stopped: a. Core_exit T/Cs - LESS THAN a. Do <u>NOT</u> proceed until core exit REQUIRED TEMPERATURE T/Cs less than required temperature. b. Stop RCS cooldown c. Stabilize core exit T/Cs - LESS THAN REQUIRED TEMPERATURE 19 Check Ruptured S/G Pressure -IF pressure continues to decrease STABLE OR INCREASING to less than 250 psi above the pressure of the intact S/G, THEN go to ECA-3.1. SGTR WITH LOSS OF REACTOR COOLANT - SUBCOOLED RECOVERY DESIRED, Step 1. 20 Check RCS Subcooling Based On Go to ECA-3.1, SGTR WITH LOSS OF Core Exit T/Cs - GREATER THAN REACTOR COOLANT - SUBCOOLED 20°F USING FIG-1.0, FIGURE RECOVERY DESIRED, Step 1 . MIN SUBCOOLING

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: SI ACCUMs may inject during RCS depressurization.

- 21 Depressurize RCS To Minimize Break Flow And Refill PRZR:
 - a. Check the following:

- a. Go to Step 22.
- o Ruptured S/G level LESS THAN 90% [80% adverse CNMT]
- o Any RCP RUNNING
- o IA to CNMT AVAILABLE
- b. Spray PRZR with maximum available spray until ANY of the following conditions satisfied:
 - . o PRZR level GREATER THAN 75% [65% adverse CNMT]

-OR-

o RCS pressure - LESS THAN SATURATION USING FIG-1.0. FIGURE MIN SUBCOOLING

-OR-

- o **BOTH** of the following:
 - 1) RCS pressure LESS THAN RUPTURED S/G PRESSURE
 - 2) PRZR level GREATER THAN 5% [30% adverse CNMT]
- c. Close normal PRZR spray valves:
- c. Stop associated RCP(s).
- 1) Adjust normal spray valve controller to 0% DEMAND
- 2) Verify PRZR spray valves CLOSED
 - PCV-431A
 - PCV-431B
- d. Verify auxiliary spray valve (AOV-296) CLOSED
- d. Decrease charging speed to minimum and ensure charging valve to loop B cold leg open (AOV-294).

e. Go to Step 24

EOP:	TITLE:	REV: 37
E-3	STEAM GENERATOR TUBE RUPTURE	PAGE 21 of 42

STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED

CAUTION

- O THE PRT MAY RUPTURE IF A PRZR PORV IS USED TO DEPRESSURIZE THE RCS. THIS MAY RESULT IN ABNORMAL CNMT CONDITIONS.
- o CYCLING OF THE PRZR PORV SHOULD BE MINIMIZED.
- o THE UPPER HEAD REGION MAY VOID DURING RCS DEPRESSURIZATION IF RCPS ARE NOT RUNNING. THIS MAY RESULT IN A RAPIDLY INCREASING PRZR LEVEL.

NOTE: o If auxiliary spray is in use, spray flow may be increased by closing normal charging valve AOV-294 and normal PRZR spray valves.

- o When using a PRZR PORV select one with an operable block valve.
- 22 Depressurize RCS Using PRZR PORV To Minimize Break Flow And Refill PRZR:
 - a. Verify IA to CNMT AVAILABLE
 - b. PRZR PORVs AT LEAST ONE AVAILABLE
- a. Refer to ATT-12.0, ATTACHMENT N2 PORVS to operate PORVs.
- b. <u>IF</u> auxiliary spray available. <u>THEN</u> return to Step 21b.

IF auxiliary spray can NOT be established. THEN go to ECA-3.3. SGTR WITHOUT PRESSURIZER PRESSURE CONTROL. Step 1.

This Step continued on the next page.

EOF: TITLE: ____ REV: 37
E-3 STEAM GENERATOR TUBE RUPTURE PAGE 22 of 42

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

(Step 22 continued from previous page)

- c. Open one PRZR PORV until ANY of the following conditions satisfied:
 - o PRZR level GREATER THAN 75% [65% adverse CNMT]

-OR-

o RCS pressure - LESS THAN SATURATION USING FIG-1.0, FIGURE MIN SUBCOOLING

-OR-

- o BOTH of the following:
 - 1) RCS pressure LESS THAN RUPTURED S/G PRESSURE
 - 2) PRZR level GREATER THAN
 5% [30% adverse CNMT]
- d. Close PRZR PORVs

THEN return to step 21b.

1) IF auxiliary spray can NOT

c. IF auxiliary spray available,

1) <u>IF</u> auxiliary spray can <u>NOT</u> be established. <u>THEN</u> go to ECA-3.3, SGTR WITHOUT PRESSURIZER PRESSURE CONTROL. Step 1.

d. <u>IF</u> either PRZR PORV can <u>NOT</u> be closed. <u>THEN</u> close associated block valve.

EOP: TITLE:	REV: 37	
E-3 STEAM GENERATO	PAGE 23 of	42
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
23 Check RCS Pressure - INCREASING	Close block valve for the PRZR PORV that was opened. <u>IF</u> pressure continues to decrease.	
_	THEN perform the following:	
	 a. Monitor the following conditions for indication of leakage from PRZR PORV: 	
	o PORV outlet temp (TI-438) <u>NOT</u> decreasing.	
	 PRT pressure. level or temperature continue to increase. 	
	b. Go to ECA-3.1, SGTR WITH LOSS OF REACTOR COOLANT - SUBCOOLED RECOVERY DESIRED, Step 1.	
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	• .	

EOP:	TITLE:			REV: 37
E-3	STEAM GENERA	TOR TUBE	RUPTURE	PAGE 24 of 42
				·
STEP	CTION/EXPECTED RESPONSE	R	ESPONSE NOT OBTAINED]
		* * * * *		
	<u>CAI</u>	<u>UTION</u>		
	E TERMINATED WHEN TERMINATIONS OF THE RUPTURED S/G.	ON CRITER	IA ARE SATISFIED TO P	REVENT
		* * * * *		
Termin	`		·	_
exit USIN	subcooling based on core T/Cs - GREATER THAN 0°F IG FIG-1.0. FIGURE MIN COOLING		Do <u>NOT</u> stop SI pumps ECA-3.1. SGTR WITH L REACTOR COOLANT - SU RECOVERY DESIRED. St	OSS OF BCOOLED
0 7	ondary heat sink: Cotal feed flow to S/G(s) - GREATER THAN 200 GPM AVAILAN	BLE	IF neither condition THEN do NOT stop SI ECA-3.1. SGTR WITH L REACTOR COOLANT - SU RECOVERY DESIRED. St	pumps. Go to OSS OF BCOOLED
	-OR-		•	•
	Narrow range level in at least one intact S/G - GREATER THAN 5% [25% adverse CNMT]	e		
INCF	pressure - STABLE OR REASING	c.	Do <u>NOT</u> stop SI pumps ECA-3.1. SGTR WITH L REACTOR COOLANT - SU RECOVERY DESIRED, St	OSS OF BCOOLED
	R level - GREATER THAN 5% adverse CNMT]	•	Do <u>NOT</u> stop SI pumps Step 6.	. Return to

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EOP:	TITLE:	REV:	37		
E-3	STEAM GENERATOR TUBE RUPTURE	PAGE	25	of	42

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- 25 Stop SI Pumps And Place In AUTO
- 26 Establish Required Charging Line Flow:
 - a. Charging pumps ANY RUNNING
- a. Perform the following:
 - 1) IF CCW flow is lost to any RCP thermal barrier OR any RCP #1 seal outlet temperature offscale high.

 THEN dispatch AO to locally isolate seal injection to affected RCP:
 - RCP A, V-300A
 - RCP B, V-300B
 - 2) Ensure HCV-142 open.
 - 3) Start one charging pump.
- b. Establish 20 gpm charging line flow
- 27 Monitor SI Reinitiation Criteria:
 - a. RCS subcooling based on core exit T/Cs - GREATER THAN 0°F USING FIG-1.0. FIGURE MIN SUBCOOLING
 - b. PRZR level GREATER THAN 5% [30% adverse CNMT]
- a. Manually start SI pumps as necessary and go to ECA-3.1. SGTR WITH LOSS OF REACTOR COOLANT SUBCOOLED RECOVERY DESIRED. Step 1.
- b. Control charging flow to maintain PRZR level.

IF PRZR level can NOT be maintained. THEN manually start SI pumps as necessary and go to ECA-3.1. SGTR WITH LOSS OF REACTOR COOLANT - SUBCOOLED RECOVERY DESIRED. Step 1.

EOP: TITLE:

E-3 STEAM GENERATOR TUBE RUPTURE

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STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED

28 Check If SI ACCUMs Should Be Isolated:

- a. Check the following:
 - o RCS subcooling based on core exit T/Cs - GREATER THAN 0°F USING FIG-1.0. FIGURE MIN SUBCOOLING
 - o PRZR level GREATER THAN 5% [30% adverse CNMT]
- b. Dispatch AO with locked valve key to locally close breakersfor SI ACCUM discharge valves
 - MOV-841. MCC C position 12F
 - MOV-865, MCC D position 12C
- c. Close SI ACCUM discharge valves
 - MOV-841
 - MOV-865

a. Manually operate SI pumps as necessary and go to ECA-3.1.
SGTR WITH LOSS OF REACTOR
COOLANT - SUBCOOLED RECOVERY
DESIRED. Step 1.

- c. Vent any unisolated ACCUMs:
 - 1) Open vent valves for unisolated SI ACCUMs.
 - ACCUM A, AOV-834A
 - ACCUM B. AOV-834B
 - 2) Open HCV-945.

<u>IF</u> an accumulator can <u>NOT</u> be isolated or vented. <u>THEN</u> consult TSC to determine contingency actions.

d. Locally reopen breakers for MOV-841 and MOV-865

EOP:	l e e e e e e e e e e e e e e e e e e e	REV:	37		
E-3	STEAM GENERATOR TUBE RUPTURE	PAGE	27	of	42

STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- 29 Verify Adequate SW Flow To CCW Hx:
 - a. Verify at least three SW pumps RUNNING
- a. Manually start pumps as power supply permits (257 kw each). <u>IF</u> less than two SW pumps can be operated. <u>THEN</u> perform the following:
 - 1) <u>IF NO SW pumps running. THEN perform the following:</u>
 - a) Pull stop any D/G that is <u>NOT</u> supplied by alternate cooling. <u>AND</u> immediately depress VOLTAGE SHUTDOWN pushbutton.
 - b) Refer to ATT-2.4.
 ATTACHMENT NO SW PUMPS.
 - 2) <u>IF</u> only one SW pump running. <u>THEN</u> refer to AP-SW.2. LOSS OF SERVICE WATER.
 - 3) Go to Step 36.
- b. Manually align valves.
- b. Verify AUX BLDG SW isolation valves AT LEAST ONE SET OPEN
 - MOV-4615 and MOV-4734
 - MOV-4616 and MOV-4735

This Step continued on the next page.

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E-3	STEAM GENERATOR TUBE RUPTURE	PAGE 28 of 42

STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

(Step 29 continued from previous page)

- c. Verify CNMT RECIRC fan annunciator C-2, HIGH TEMPERATURE ALARM - EXTINGUISHED
- c. Perform the following:
 - 1) Determine required SW flow to CCW HXs per table:

SW DISCHARGE ALIGNMENT	CCW HXs IN SERVICE	REQUIRED SW FLOW
Normal	2	Total of 5000 - 6000 gpm equally divided to both HXs
Normal	1	5000 - 6000 gpm to in-service HX
Alternate	2	30-33" d/p across each HX
Alternate	1	95-100" d/p across in-service HX

- 2) Direct AO to adjust SW flow to required value.
 - o IF on normal SW discharge:
 - V-4619, CCW Hx A
 - V-4620, CCW Hx B

-OR-

- o <u>IF</u> on alternate SW discharge:
 - V-4619C, CCW Hx A
 - V-4620B, CCW Hx B

EOP:	TITLE:	REV: 37
E-3	STEAM GENERATOR	PAGE 29 of 42
STEP	CTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
	If Normal CVCS ion Can Be Established	
	fy IA restored: A to CNMT (AOV-5392) - OPEN	a. Continue with Step 36. <u>WHEN</u> IA restored, <u>THEN</u> do Steps 30 through 35.
	A pressure - GREATER THAN O PSIG	
	fy instrument bus D - GIZED	b. Energize MCC B. <u>IF MCC B NOT</u> available. <u>THEN</u> perform the following:
		1) Verify MCC A energized.
	e e e e e e e e e e e e e e e e e e e	Place instrument bus D on maintenance supply.
c. CCW	pumps - ANY RUNNING	c. Perform the following:
	· · · · · · · · · · · · · · · · · · ·	 IF any RCP #1 seal outlet temperature offscale high. THEN isolate CCW to thermal barrier of affected RCP(s).
		 RCP A, MOV-749A and MOV-759A RCP B, MOV-749B and MOV-759B
		2) Manually start one CCW pump.
d. Char	ging pump - ANY RUNNING	d. Continue with Step 36. <u>WHEN</u> any charging pump running, <u>THEN</u> do Steps 31 through 35.
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EOP: TITLE: **REV: 37** STEAM GENERATOR TUBE RUPTURE E-3 PAGE 30 of 42 STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED

31 Check If Seal Return Flow Should Be Established:

- a. Verify RCP #1 seal outlet temperature - LESS THAN 235°F
- b. Verify RCP seal outlet valves -**OPEN**
 - AOV-270A
 - AOV-270B
- c. Reset both trains of XY relays for RCP seal return isolation valve MOV-313
- d. Open RCP seal return isolation d. Perform the following: valve MOV-313
- e. Verify RCP #1 seal leakoff flow e. Perform the following: - LESS THAN 6.0 GPM

- a. Go to Step 32.
- b. Manually open valves as necessary.

- - 1) Place MOV-313 switch to OPEN.
 - 2) Dispatch AO to locally open MOV-313.
- - 1) Trip the affected RCP
 - 2) Allow 4 minutes for pump coast down. THEN close the affected RCP seal discharge valve
 - RCP A. AOV-270A
 - RCP B, AOV-270B

IF both RCP seal discharge valves are shut. THEN go to Step 32.

- f. Verify RCP #1 seal leakoff flow f. Refer to AP-RCP.1, RCP SEAL - GREATER THAN 0.8 GPM
 - MALFUNCTION.

EOP: TITLE:

E-3 STEAM GENERATOR TUBE RUPTURE

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STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

32 Verify PRZR Level - GREATER THAN 13% [40% adverse CNMT]

Continue with Step 34. <u>WHEN</u> PRZR level increases to greater than 13% [40% adverse CNMT], <u>THEN</u> do Step 33.

33 Establish Normal Letdown:

- a. Establish charging line flow to REGEN Hx GREATER THAN 20 GPM
- b. Place the following switches to CLOSE:
 - Letdown orifice valves (AOV-200A, AOV-200B, and AOV-202)
 - Letdown isolation valve AOV-371
 - Loop B cold leg to REGEN Hx AOV-427
 - c. Place letdown controllers in MANUAL at 40% open
 - TCV-130
 - PCV-135
 - d. Reset both trains of XY relays for AOV-371 and AOV-427
 - e. Open AOV-371 and AOV-427
 - f. Open letdown orifice valves as necessary
 - g. Place PCV-135 in AUTO at 250 psig
 - h. Place TCV-130 in AUTO at the normal setpoint
 - i. Adjust charging pump speed and HCV-142 as necessary

<u>IF</u> RCP seal return has been established. <u>THEN</u> establish excess letdown as follows:

- Place excess letdown divert valve, AOV-312, to NORMAL.
- o Ensure CCW from excess letdown open. (AOV-745).
- o Open excess letdown isolation valve AOV-310.
- o Slowly open HCV-123 to maintain excess letdown temperature less than 195°F and pressure less than 100 psig.
- o Adjust charging pump speed as necessary.

<u>IF</u> RCP seal return <u>NOT</u> established. <u>THEN</u> consult TSC to determine if excess letdown should be placed in service.

E-3 STEAM GENERATOR	
	PAGE 32 of 4
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
34 Check VCT Makeup System: a. Adjust boric acid flow control	
valve in AUTO to 9.5 gpm b. Adjust RMW flow control valve in AUTO to 40 gpm	
c. Verify the following:1) RMW mode selector switch in AUTO	c. Adjust controls as necessary.
2) RMW control armed - RED LIGHT LIT	
d. Check VCT level: o Level - GREATER THAN 20%	d. Manually increase VCT makeup flow as follows:
O.P.	1) Ensure BA transfer pumps and

RMW pumps running. $\overline{\text{IF NOT}}$. $\overline{\text{THEN}}$ reset MCC C and MCC D UV

2) Place RMW flow control valve HCV-111 in MANUAL and increase RMW flow.

3) Increase boric acid flow as

necessary.

lockouts as necessary.

-OR-

o Level - STABLE OR INCREASING

EOP:	TITLE:	REV: 37
E-3	STEAM GENERATOR TUBE RUPTURE	PAGE 33 of 4:

35 Check Charging Pump Suction Aligned To VCT:

STEP

a. VCT level - GREATER THAN 20%

ACTION/EXPECTED RESPONSE

- a. <u>IF VCT level can NOT</u> be maintained greater than 5%. <u>THEN</u> perform the following:
 - 1) Ensure charging pump suction aligned to RWST
 - o LCV-112B open

RESPONSE NOT OBTAINED

- o LCV-112C closed
- 2) Continue with Step 36. WHEN VCT level greater than 40%, THEN do Step 35b.
- b. Verify charging pumps aligned to VCT
 - o LCV-112C OPEN
 - o LCV-112B CLOSED

b. Manually align valves as necessary.

EOP:	TITLE:	REV: 37
E-3	STEAM GENERATOR TUBE RUPTURE	D7 C7 24 5 40
	· W.	PAGE 34 of 42

	1 1		1	r		3
STEP		ACTION/EXPECTED RESPONSE	•	RESPONSE N	NOT OBTAINED	┝

RCS AND RUPTURED S/G PRESSURES MUST BE MAINTAINED LESS THAN 1050 PSIG.

*36 Control RCS Pressure And Charging Flow To Minimize RCS-To-Secondary Leakage:

> a. Perform appropriate action(s) from table:

PRZR	RUPTURED S/G NARROW RANGE LEVEL				
LEVEL	INCREASING	DECREASING	OFFSCALE HIGH		
LESS THAN 13% [40% ADVERSE CNMT]	o Increase charging flow o Depressurize RCS using Step 36b	Increase charging flow	o Increase charging flow o Maintain RCS and ruptured S/G pressure equal		
BETWEEN 13% [40% ADVERSE CNMT] AND 50%	Depressurize RCS using Step 36b	Energize PRZR heaters	Maintain RCS and ruptured S/G pressure equal		
BETWEEN 50% AND 75% [65% ADVERSE CNMT]	o Depressurize RCS using Step 36b o Decrease charging flow	Energize PRZR heaters	Maintain RCS and ruptured S/G pressure equal		
GREATER THAN 75% [65% ADVERSE CNMT]	o Decrease charging flow	Energize PRZR heaters	Maintain RCS and ruptured S/G pressure equal		

- PRZR spray, if available, to obtain desired results for Step 36a
- b. Control pressure using normal b. IF letdown is in service. THEN use auxiliary spray (AOV-296). IF NOT. THEN use one PRZR PORV.

EOP: TITLE:		REV: 37
E-3 STEAM GENERATOR	TUBE RUPTURE	PAGE 35 of 42
STEP ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED	
37 Monitor If CNMT Spray Should Be Stopped:		
a. CNMT_spray pumps - ANY RUNNING	a. Go to Step 38.	
b. Verify CNMT pressure - LESS THAN 4 PSIG	b. Continue with Step 3 CNMT pressure less 3 THEN do Steps 37c tl	than 4 psig.
c. Reset CNMT spray		
d. Check NaOH flow (FI-930) - NO FLOW	d. Place NaOH tank out switches to CLOSE.	let valve
	AOV-836AAOV-836B	
e. Stop CNMT spray pumps and place in AUTO		
f. Close CNMT spray pump discharge valves		

MOV-860AMOV-860BMOV-860CMOV-860D

EOP: TITLE: **REV: 37** STEAM GENERATOR TUBE RUPTURE E-3 PAGE 36 of 42 ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED STEP 38 Check If Emergency D/Gs Should Be Stopped: a. Verify AC emergency busses a. Try to restore offsite power energized by offsite power: (Refer to ER-ELEC.1. RESTORATION OF OFFSITE POWER). o Emergency D/G output breakers - OPEN

o AC emergency bus voltage -

o AC emergency bus normal feed breakers - CLOSED

GREATER THAN 420 VOLTS

- b. Stop any unloaded emergency D/G and place in standby (Refer to ATT-8.1, ATTACHMENT D/G STOP)
- 39 Minimize Secondary System Contamination:
 - a. Isolate reject from hotwell to CST:
 - o Place hotwell level controller (LC-107) in MANUAL at 50%
 - o Verify hotwell level STABLE
 - b. Check status of local actions to complete ruptured S/G isolation (Refer to ATT-16.0, ATTACHMENT RUPTURED S/G)

a. IF hotwell level increasing. THEN direct RP to sample hotwells for activity.

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STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

40 Energize PRZR Heaters As Necessary To Saturate PRZR Water At Ruptured S/G Pressure

- 41 Check RCP Cooling:
 - a. Check CCW to RCPs:
 - o Annunciator A-7, RCP 1A CCW RETURN HIGH TEMP OR LOW FLOW - EXTINGUISHED
 - o Annunciator A-15, RCP 1B CCW RETURN HIGH TEMP OR LOW FLOW - EXTINGUISHED
 - b. Check RCP seal injection:
 - o Labyrinth seal D/Ps GREATER THAN 15 INCHES OF WATER

-OR-

o RCP seal injection flow to each RCP - GREATER THAN 6 GPM

Establish normal cooling to RCPs (Refer to ATT-15.2, ATTACHMENT SEAL COOLING).

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STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED

CAUTION

IF RCP SEAL COOLING HAD PREVIOUSLY BEEN LOST, THEN THE AFFECTED RCP SHOULD NOT BE STARTED PRIOR TO A STATUS EVALUATION.

42 Check RCP Status

- a. Both RCPs STOPPED
- b. Ensure conditions for starting an RCP.
 - o Bus 11A or 11B energized.
 - o Refer to ATT-15.0. ATTACHMENT RCP START.
- a. Stop all but one RCP and go to step 43.
- b. <u>IF</u> conditions can <u>NOT</u> be met. <u>THEN</u> perform the following:
 - 1) Verify natural circulation (Refer to ATT-13.0, ATTACHMENT NC).

<u>IF</u> natural circulation can <u>NOT</u> be verified. <u>THEN</u> increase dumping steam.

2) Go to step 43.

This Step continued on the next page.

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STEP ACTION/EXPECTED RESPONSE RESPONSE NOT OBTAINED

(Step 42 continued from previous page)

c. Check RVLIS level (no RCPs) - ≥ 95%

d. Start one RCP

- c. <u>IF</u> RVLIS level (no RCPs) less than 95%. <u>THEN</u> perform the following:
 - o Increase PRZR level to greater than 65% [82% adverse CNMT].
 - . o Dump steam to establish RCS subcooling based on core exit T/Cs to greater than 20°F using FIG-1.0. FIGURE MIN SUBCOOLING.
 - o Energize PRZR heaters as necessary to saturate PRZR water

<u>IF</u> conditions <u>NOT</u> met. <u>THEN</u> continue with step 43. <u>WHEN</u> conditions met. <u>THEN</u> do step 42d.

d. <u>IF</u> an RCP can <u>NOT</u> be started. <u>THEN</u> verify natural circulation (Refer to ATT-13.0, ATTACHMENT NC).

 $\underline{\text{IF}}$ natural circulation can $\underline{\text{NOT}}$ be verified. $\underline{\text{THEN}}$ increase dumping steam.

STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

NOTE: Adverse CNMT conditions or loss of forced air cooling may result in failure of NIS detectors.

- 43 Check If Source Range Detectors Should Be Energized:
 - a. Source range channels -DEENERGIZED
 - b. Check intermediate range flux b. Perform the following: EITHER CHANNEL LESS THAN 10-10 AMPS
- a. Go to Step 43e.
 - - 1) IF neither intermediate range channel is decreasing. THEN initiate boration.
 - 2) Continue with Step 44. WHEN flux is LESS THAN 10-10 amps on any operable channel, THEN do Steps 43c through e.
 - c. Continue with Step 44. WHEN either condition met. THEN do Steps 43d and e.
 - c. Check the following:
 - o Both intermediate range channels - LESS THAN 10-10 AMPS

-OR-

- o Greater than 20 minutes since reactor trip
- d. Verify source range detectors d. Manually energize source range ENERGIZED
 - detectors by depressing P-6 permissive defeat pushbuttons (2 of 2).

IF source ranges can NOT be restored. THEN refer to ER-NIS.1. SR MALFUNCTION and go to Step 44.

e. Transfer Rk-45 recorder to one source range and one intermediate range channel

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STEP ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- 44 Establish Normal Shutdown Alignment:
 - a. Check condenser AVAILABLE
- a. Dispatch AO to perform ATT-17.1. ATTACHMENT SD-2.

- b. Perform the following:
 - o Open generator disconnects
 - 1G13A71
 - 9X13A73
 - o Place voltage regulator to OFF
 - o Open turbine drain valves
 - o Rotate reheater steam supply controller cam to close valves
 - o Place reheater dump valve switches to HAND
 - o Stop all but one condensate pump
- c. Verify adequate Rx head cooling:
 - 1) Verify at least one control rod shroud fan RUNNING
 - 2) Verify one Rx compartment cooling fan RUNNING
- 1) Manually start one fan as power supply permits (45 kw)
- 2) Perform the following:
 - o Dispatch AO to reset UV relays at MCC C and MCC D.
 - o Manually start one fan as power supply permits (23 kw)
- d. Verify ATT-17.0. ATTACHMENT SD-1 COMPLETE

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STEP

ACTION/EXPECTED RESPONSE

RESPONSE NOT OBTAINED

- 45 Consult TSC To Determine Appropriate Post-SGTR Cooldown Procedure:
 - o Go to ES-3.1. POST-SGTR COOLDOWN USING BACKFILL, Step 1

-OR-

o Go to ES-3.2, POST-SGTR COOLDOWN USING BLOWDOWN, Step 1

-OR-

o Go to ES-3.3, POST-SGTR COOLDOWN USING STEAM DUMP. Step 1

-END-

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E-3 APPENDIX LIST

TITLE

- 1) RED PATH SUMMARY
- 2) FIGURE MIN SUBCOOLING (FIG-1.0)
- 3) ATTACHMENT CNMT RECIRC FANS (ATT-4.0)
- 4) ATTACHMENT D/G STOP (ATT-8.1)
- 5) ATTACHMENT N2 PORVS (ATT-12.0)
- 6) ATTACHMENT NC (ATT-13.0)
- 7) ATTACHMENT SEAL COOLING (ATT-15.2)
- 8) ATTACHMENT RCP START (ATT-15.0)
- 9) ATTACHMENT RUPTURED S/G (ATT-16.0)
- 10) ATTACHMENT SD-1 (ATT-17.0)
- 11) ATTACHMENT SD-2 (ATT-17.1)
- 12) ATTACHMENT NO SW PUMPS (ATT-2.4)
- 13) ATTACHMENT LOSS OF OFFSITE POWER (ATT-8.5)
- 14) ATTACHMENT DIESEL AIR COMPRESSOR (ATT-11.2)
- 15) FOLDOUT

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RED PATH SUMMARY

a. SUBCRITICALITY - Nuclear power greater than 5%

b. CORE COOLING - Core exit T/Cs greater than 1200°F
-ORCore exit T/Cs greater than 700°F AND
RVLIS level (no RCPs) less than 52% [55%
adverse CNMT]

- c. HEAT SINK Narrow range level in all S/Gs less than 5% [25% adverse CNMT] AND total feedwater flow less than 200 gpm
- d. INTEGRITY Cold leg temperatures decrease greater than 100°F in last 60 minutes <u>AND</u> RCS cold leg temperature less than 285°F
- e. CONTAINMENT CNMT pressure greater than 60 psig

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FOLDOUT PAGE

1. LOSS OF SW CRITERIA

IF no SW pumps are available, THEN perform the following:

- a. Pull stop any D/G that is <u>NOT</u> supplied by alternate cooling, <u>AND</u> immediately depress associated VOLTAGE SHUTDOWN pushbutton.
- b. Refer to ATT-2.4, ATTACHMENT NO SW PUMPS.

2. SI REINITIATION CRITERIA

Following SI termination, <u>IF</u> either condition listed below occurs, <u>THEN</u> start SI pumps manually as necessary and go to ECA-3.1, SGTR WITH LOSS OF REACTOR COOLANT - SUBCOOLED RECOVERY DESIRED, Step 1:

o RCS subcooling based on core exit T/Cs - LESS THAN 0°F USING REQUIREMENTS OF FIG-1.0, FIGURE MIN SUBCOOLING.

OR

o PRZR level - CHARGING CAN NOT CONTROL LEVEL GREATER THAN 5% [30% adverse CNMT].

3. SECONDARY INTEGRITY CRITERIA

<u>IF</u> any S/G pressure is decreasing in an uncontrolled manner or is completely depressurized <u>AND</u> has not been isolated, <u>THEN</u> go to E-2, FAULTED S/G ISOLATION, Step 1, <u>UNLESS</u> faulted S/G needed for RCS cooldown.

4. COLD LEG RECIRCULATION SWITCHOVER CRITERION

<u>IF</u> RWST level decreases to less than 28%, <u>THEN</u> go to ES-1.3, TRANSFER TO COLD LEG RECIRCULATION, Step 1.

5. AFW SUPPLY SWITCHOVER_CRITERION

<u>IF</u> CST level decreases to less than 5 feet, <u>THEN</u> switch to alternate AFW water supply (Refer to ER-AFW.1, ALTERNATE WATER SUPPLY TO AFW PUMPS).

6. MULTIPLE S/G TUBE RUPTURE CRITERIA

<u>IF</u> any intact S/G level increases in in an uncontrolled manner <u>OR</u> <u>IF</u> any intact S/G has abnormal radiation, <u>THEN</u>. go to ECA-3.1, SGTR WITH LOSS OF REACTOR COOLANT - SUBCOOLED RECOVERY DESIRED, Step 1.