



RULEMAKING ISSUE
(Notation Vote)

March 20, 1986

SECY-86-92

To: The Commissioners

From: Victor Stello, Jr. Acting Executive Director for Operations

Subject: 10 CFR PART 60--DISPOSAL OF HIGH-LEVEL RADIOACTIVE WASTES IN GEOLOGIC REPOSITORIES--PROPOSED AMENDMENTS TO ELIMINATE INCONSISTENCIES WITH THE EPA HIGH-LEVEL WASTE STANDARDS

Category: This paper involves a minor policy question.

Purpose: To request Commission approval to publish proposed amendments to 10 CFR Part 60, which would, in accordance with the Nuclear Waste Policy Act, eliminate inconsistencies with the EPA Standard for HLW Geologic Repositories.

Background: Final procedures which established a regulatory framework for licensing the disposal of high-level radioactive wastes (HLW) in geologic repositories were published by the NRC on February 25, 1981 (46 FR 13971). Final technical criteria against which license applications would be reviewed under 10 CFR Part 60 were published by the NRC on June 21, 1983 (48 FR 28194).

The Nuclear Waste Policy Act of 1982 (NWPA) directs the Environmental Protection Agency (EPA) to "promulgate generally applicable standards for protection of the general environment from offsite releases from radioactive material in repositories" (Sec. 121). The final EPA Standard--40 CFR Part 191--was published on September 19, 1985 (50 FR 38066). The NWPA also directs NRC to insure that its regulations "shall not be inconsistent with any comparable standards promulgated by (EPA)" (Sec. 121). The staff has analyzed the final EPA Standard and determined that some modifications to Part 60 are necessary to assure consistency between Part 60 and the EPA Standard. Several modifications concerning EPA's "assurance requirements" have been discussed with the EPA staff and brought to the attention of the Commission in SECY-85-272 - Report on the Environmental Protection Agency's

Contacts: D. Fehring, NMSS
427-4796
C. Prichard, RES
427-4586
R. Wolf, ELD
92-8694

8604240182 XA

Environmental Standards for High-Level Radioactive Wastes. In responding to SECY-85-272, the Commission directed the staff to submit the rulemaking package which conforms 10 CFR Part 60 with the EPA standard.¹

Discussion:

In preparing the proposed amendments, the staff has tried to address the concerns expressed by the Advisory Committee on Reactor Safeguards (ACRS) regarding the implementation of the EPA standard in a licensing context.² Two of the concerns of the ACRS deserve additional discussion here. First, the ACRS has stated that the level of risk allowed by the EPA HLW standards is much lower than that allowed by other standards for radiological and non-radiological hazards. However, ~~the staff believes that under certain reasonable scenarios and assumptions (e.g., the size of the population at risk) the EPA standards might in fact be comparable to other standards now in place for other nuclear activities. Since the risks allowed by the EPA standards can be viewed in such widely different ways, the staff has concentrated on the achievability of the standards rather than on comparisons with the risks allowed by other standards.~~

What assumption re: population in order to be reasonable?

The ACRS was also concerned that the low level of allowable risk, combined with the probabilistic nature of the standards, will make the standards difficult to implement in an actual repository licensing review. ~~NRC contractor studies (documented in NUREG/CR-3235) concluded that repository sites likely can be found for which repository performance can be demonstrated to be in compliance with EPA HLW standards using analytical techniques which exist or are under development. However, the conclusion is supported by an implicit assumption that research will identify the processes and phenomena important to repository performance, and that research efforts will validate the models and assumptions which are the bases of those techniques. Moreover, the staff has further developed its review regarding its ability to implement the EPA standards in the enclosed draft Federal Register notice, including additional discussion of the relationship between a numerical, probabilistic standard and the qualitative, "reasonable assurance" specification for the required level of confidence that the release limits will be met.~~

On January 15, 1986 the staff met with the ACRS Subcommittee on Waste Management to discuss these proposed amendments to Part 60 in accordance with the directions to the staff contained in the staff

¹Staff requirements memorandum from Samuel J. Chilk to William J. Dircks, dated November 27, 1985 (Enclosure D).

²Memorandum to Chairman Palladino from David A. Ward, dated November 14, 1985 (Enclosure H).

The application of these conditions should not impose any further conservatism on an already highly conservative standard.

It is unfortunate that the ACRS comments on the EPA standards were made available at a time when Commission options to act without seriously delaying the repository program had, for the most part, been foreclosed. I would hope that in future reviews of NRC activities under the NWA the ACRS could be involved at an earlier stage so that valuable technical advice and input could be used to timely and best advantage by the Commission.

cc: Chairman Palladino
Commissioner Roberts
Commissioner Asselstine
Commissioner Zech
OGC
OPE
EDO



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

November 20, 1985

OFFICE OF THE
COMMISSIONER

MEMORANDUM FOR: Samuel J. Chilk, Secretary
FROM: Frederick M. Bernthal *FB*
SUBJECT: REAFFIRMATION OF VOTE ON SECY-85-272

Upon extensive examination of the ACRS objections to the EPA standard (including their most recent comments presented in a letter of 11/14/85) and of the analysis of available Commission options presented by OGC, I reaffirm my approval of SECY-85-272.

The ACRS has criticized the EPA standard on the grounds that

1. it is overly stringent, mandating a level of protection that is far in excess of that provided by other existing environmental standards, and
2. implementation of the standard by NRC in licensing a repository will be difficult if not impossible.

My review of the question suggests that the momentary confusion over the EPA standard arose from imprecise wording on the part of EPA and Staff in attempting to explain the origin of the cumulative probability distribution function of repository release upon which the interpretation of 40 CFR 191 is based. Nevertheless, I continue to have reservations, both as to the application of the EPA standard, and as to the reasonableness and consistency of the standard when viewed in light of other societal risks (cf. comments of ACRS Members Dade Moeller and Hal Lewis).

Be that as it may, the Nuclear Waste Policy Act clearly assigns to the EPA the responsibility for establishing the environmental standard. Given that our staff has repeatedly asserted that the standards as published can be implemented, there appears to be little basis on which to challenge a policy decision that is, strictly speaking, that of EPA.

But I agree with the suggestion of ACRS Member, Dr. Dade Moeller that the Commission request the Committee on Interagency Radiation Research and Policy Coordination (CIRRPC) to develop guidelines for use by Federal agencies that would foster consistency in the risk estimates and risk management of low doses of radiation.

I also agree with Commissioner Zech and the Chairman that any remaining ACRS concerns should be addressed to the fullest extent possible in the rulemaking that will be necessary to conform Part 60 to the EPA standard. In particular, care should be taken to avoid any ambiguity in the application of probabilistic conditions placed on the post-closure containment requirements.

EPA ASSURANCE REQUIREMENTS AND
PROPOSED CHANGES TO PART 60

1.a. EPA Assurance Requirement:

(a) Active institutional controls over disposal sites should be maintained for as long a period of time as is practicable after disposal; however, performance assessments that assess isolation of the wastes from the accessible environment shall not consider any contributions from active institutional controls for more than 100 years after disposal.

(In Working Draft No. 8 "active institutional control" means: (1) controlling access to a disposal site by any means other than passive institutional controls, (2) performing maintenance operations or remedial actions at a site, (3) controlling or cleaning up releases from a site, or (4) monitoring parameters related to disposal system performance.)

b. Discussion:

The Commission's existing provisions (§60.52) related to license termination will determine the length of time for which institutional controls should be maintained, and there is therefore no need to alter Part 60 based on the first part of this assurance requirement.

The second part of this assurance requirement would require that "active" institutional controls be excluded from consideration (after 100 years) when the Commission assesses the isolation characteristics of a repository. The staff understands that remedial actions (or other active institutional controls) would not be relied upon under Part 60 to compensate for a poor site or inadequate engineered barriers. However, in the definition of "unanticipated events and processes," Part 60 expressly contemplates that, in assessing human intrusion scenarios, the Commission would assume that "institutions are able to assess risk and to take remedial action at a level of social organization and technological competence equivalent to, or superior to, that which was applied in initiating the processes or events concerned" (emphasis added). Therefore, it might appear at first blush that Part 60 is at odds with the draft EPA standards.

The "remedial action" is not, however, the same in the two documents. The EPA standards have in mind a planned capability to maintain a site and, if necessary, to take remedial action at a site in order to assure that isolation is achieved. The staff agrees that such a capability should not be relied upon. The extent to which corrective action may be taken after an unanticipated intrusion occurs is an entirely different matter. The Commission may wish to consider, for example, the extent to which the application of the limited societal response capability assumed by the rule (e.g., sealing boreholes consistent with current petroleum industry practice) could reduce the likelihood of releases exceeding the values specified in the EPA standards, or could eliminate certain hypothetical scenarios such as systematic and persistent intrusions into a site.

The NRC and EPA staffs are in substantive agreement that planned remedial capabilities should not be relied upon for repository safety, and agree that the wording below should be proposed for public comment. The EPA staff may provide comment on this wording to help clarify the distinction between expected societal responses versus planned capabilities for remedial actions.

c. Proposed Changes to Part 60:

Add definitions to §60.2 as follows:

"Active institutional control" means: (1) controlling access to a site by any means other than passive institutional controls, (2) performing maintenance operations or remedial actions at a site, (3) controlling or cleaning up releases from a site, or (4) monitoring parameters related to geologic repository performance.

"Passive institutional control" means: (1) permanent markers placed at a site, (2) public records and archives, (3) government ownership and regulations regarding land or resource use, and (4) other methods of preserving knowledge about the location, design, and contents of a geologic repository.

Add a new §60.114 as follows:

§60.114 Institutional Controls

Neither active nor passive institutional controls shall be deemed to assure compliance with the overall performance objective set out at § 60.112 for more than 100 years after disposal. However, the effects of institutional controls may be considered in assessing, for purposes of that section, the likelihood and consequences of processes and events affecting the geologic setting.

2.a. EPA Assurance Requirement:

(b) Disposal systems shall be monitored after disposal to detect any substantial and detrimental deviations from expected performance. This monitoring shall be done with techniques that do not jeopardize the isolation of the wastes and shall be conducted until there are no significant concerns to be addressed by further monitoring.

b. Discussion:

Part 60 currently requires completion of a performance confirmation program prior to repository closure, but does not require monitoring during the period following closure but prior to license termination. The Commission chose not to require post-closure monitoring because of doubts about the usefulness of such monitoring and because of fears that monitoring in or near a repository after closure could degrade repository performance. The type of monitoring envisioned by EPA does not involve direct monitoring of the repository itself (which might degrade repository performance). Rather, EPA proposes monitoring of such parameters as regional groundwater flow characteristics. The staff agrees that such monitoring may, in some cases, provide desirable information beyond that which would be obtained in the performance confirmation program which Part 60 now requires to be continued until permanent closure. The staff therefore proposes to require monitoring as an extension of performance confirmation, as appropriate, when such monitoring can be conducted without degrading repository performance.

c. Proposed Changes to Part 60:

Add to §60.21(c) a new ¶ (9) as follows:

(9) A general description of the program for post-permanent closure monitoring of the geologic repository.

Re-number the current ¶ (9) through (15) accordingly.

Revise §60.51(a)(1) to read:

(1) A detailed description of the program for post-permanent closure monitoring of the geologic repository in accordance with §60.144. As a minimum, this description shall:

- (i) identify those parameters that will be monitored;
- (ii) indicate how each parameter will be used to evaluate the expected performance of the repository; and
- (iii) discuss the length of time over which each parameter should be monitored to adequately confirm the expected performance of the repository.

Add to §60.52(c) a new ¶ (3) as follows:

(3) That the results available from the post-permanent closure monitoring program confirm the expectation that the repository will comply with the performance objectives set out at §60.112 and §60.113; and

Renumber the current ¶ (3) as ¶ (4).

Add a new §60.144 as follows:

§60.144 Monitoring After Permanent Closure

A program of monitoring shall be conducted after permanent closure to monitor all repository characteristics which can reasonably be expected to provide material confirmatory information regarding long-term repository performance, provided that the means for conducting such monitoring will not degrade repository performance. This program shall be continued until termination of a license.

Include in the Supplementary Information of the Federal Register notice proposing these changes the following paragraph:

Part 60 currently requires DOE to carry out a performance confirmation program which is to continue until repository closure. Part 60 does not now require monitoring after repository closure because of the likelihood that post-closure monitoring of the underground facility would degrade repository performance. The Commission recognizes, however, that monitoring such parameters as regional groundwater flow characteristics may, in some cases, provide desirable information beyond that which would be obtained in the performance confirmation program. The proposed requirement for post-permanent closure monitoring requires that such monitoring be continued until termination of a license. The Commission intends that a repository license not be terminated until such time as the Commission is convinced that there is no significant additional information to be obtained from such monitoring which would be material to a finding of reasonable assurance that long-term repository performance would be in accordance with the established performance objectives.

3.a. EPA Assurance Requirement:

(c) Disposal sites shall be designated by the most permanent markers, records, and other passive institutional controls practicable to indicate the dangers of the wastes and their location.

b. Discussion:

No revisions to Part 60 are needed. §60.21(c)(8), 60.51(a)(2), and 60.121 contain equivalent provisions.

4.a. EPA Assurance Requirement:

(d) Disposal systems shall use several different types of barriers to isolate the wastes from the environment. Both engineered and natural barriers shall be included.

b. Discussion:

The staff considers that Part 60 already requires use of both engineered and natural barriers. Nevertheless, in order to avoid any possible confusion regarding the provisions of §60.113(b), the staff proposes to add additional clarifying language to §60.113.

c. Proposed Changes to Part 60:

Add a new ¶ (d) to §60.113 as follows:

(d) Notwithstanding the provisions of (b) above, the geologic repository shall incorporate a system of multiple barriers, both engineered and natural.

In the Supplementary Information of the Federal Register notice proposing these changes include the following:

Questions might arise regarding the types of engineered or natural materials or structures which would be considered to constitute barriers. The Commission notes that §60.2 now contains the definition: "'Barrier' means any material or structure that prevents or substantially delays movement of water or radionuclides." Thus, the Commission considers that the new paragraph to be added to §60.113 will confirm the Commission's commitment to a multiple barrier approach as contemplated by Section 121(b)(1)(B) of the Nuclear Waste Policy Act.

5.a. EPA Assurance Requirement:

(e) Places where there has been mining for resources, or where there is a reasonable expectation of exploration for scarce or easily accessible resources, or where there is a significant concentration of any material that is not widely available from other sources, should be avoided in selecting disposal sites. Resources to be considered shall include minerals, petroleum or natural gas, valuable geologic formations, and ground waters that are either irreplaceable because there is no reasonable alternative source of drinking water available for substantial populations or that are vital to the preservation of unique and sensitive ecosystems. Such places shall not be used for disposal of the wastes covered by this Part unless the favorable characteristics of such places compensate for their greater likelihood of being disturbed in the future.

b. Discussion:

Part 60 contains provisions equivalent to this assurance requirement in §60.122(c)(17), (18) and (19). Part 60 does not, however, address "a significant concentration of any material that is not widely available from other sources."

It is possible that the economic value of materials could change in the future in a way which might attract future exploration or development detrimental to repository performance. The staff proposes to add an additional potentially adverse condition to Part 60 related to significant concentrations of material that is not widely available from other sources. As with the other potentially adverse conditions, the presence of such a condition would require an evaluation of the effect of the condition on repository performance as specified in §60.122(a)(2)(ii), but would not preclude selection of a site for repository construction. (It should be noted that DOE's siting guidelines contain an identical provision in 10 CFR 960.4-2-8-1.)

c. Proposed Changes to Part 60:

Add a new § (18) to §60.122(c) as follows:

(18) The presence of significant concentrations of any naturally-occurring material that is not widely available from other sources.

Renumber the current § (18) through (21) accordingly.

6.a. EPA Assurance Requirement:

(f) Disposal systems shall be selected so that removal of most of the wastes is not precluded for a reasonable period of time after disposal.

D. Discussion:

EPA's concept of "removal" is significantly different from "retrieval" in Part 60. EPA wants to preclude disposal concepts such as deep well injection for which it would be virtually impossible to remove or recover wastes regardless of the time and resources employed. For a mined geologic repository wastes could be located and recovered, albeit at great cost, even after repository closure. EPA therefore considers that a repository complies with this assurance requirement, and no revision to Part 60 is needed.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CHAIRMAN

December 2, 1985

The Honorable Lee Thomas
Administrator
U.S. Environmental Protection Agency
Washington, D.C. 20460

Dear Mr. Thomas:

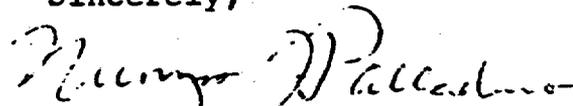
On May 10 and 11, 1982 the Nuclear Regulatory Commission (NRC) submitted formal comments on the Environmental Protection Agency's proposed environmental standards for management and disposal of high-level radioactive wastes. Among other things, we stated our view that the proposed "assurance requirements" and "procedural requirements" contained in those proposed standards involved matters of implementation and thus went beyond the limits of EPA's jurisdiction.

In letters dated July 19 and August 15, 1984 Acting Chairman Roberts and Former Administrator Ruckelshaus, respectively, agreed that the staffs of EPA and NRC should attempt to develop modifications to 10 CFR Part 60 to incorporate the principles of EPA's proposed assurance and procedural requirements. EPA could then delete these requirements or make them applicable only to facilities not licensed by the NRC, eliminating any potential problems of jurisdictional overlap.

The NRC staff recently reported to the Commission several proposed changes to Part 60 which have been worked out by the NRC and EPA staff (text enclosed). Consistent with the provisions of the Administrative Procedure Act, the Commission will propose these changes for incorporation into Part 60 now that the final EPA high-level waste standards have been published. The NRC staff anticipates submittal of a rulemaking package, incorporating both these wording changes and other conforming amendments, to the Commission within 120 days.

The Commission appreciates the cooperation shown by the EPA staff in working to reach this agreement.

Sincerely,


Nunzio J. Palladino

Enclosure:
Proposed changes to
10 CFR Part 60



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 20, 1985

MEMORANDUM FOR: Samuel J. Chilk
Secretary

FROM: Nunzio J. Palladino *NJP*

SUBJECT: REAFFIRMATION OF VOTE ON SECY-85-272,
"REPORT ON EPA'S STANDARDS FOR HLW
DISPOSAL"

The October 21 Commission meeting with staff, ACRS and others presented conflicting views as to the deficiencies and degree of restrictiveness of the EPA Standards. I understand subsequent meetings between the ACRS and staff were held for additional discussions on this matter.

I have reviewed the ACRS follow-up letter of November 14 in which additional comments were offered. These comments reiterate ACRS concerns that the standards contain deficiencies and inconsistencies but remain silent on whether the standard is too strict. The staff still maintains that its approach is workable.

On balance, I reaffirm my approval of releasing the letter to EPA as I modified in the draft October 1, 1985 Staff Requirements Memorandum.

I would stress the importance of staff to clearly articulate, in changes to 10 CFR Part 60, how we interpret the EPA's Standards and address any other ACRS concerns per Commissioner Zech's suggestion. In line with ACRS comments, the staff should accelerate efforts to develop analytical methods to be used in making a determination that a licensee is complying with the EPA Standards. These methods should receive as broad an input and review as possible.

In developing the package on 10 CFR Part 60, we must assure that staff and ACRS interact with each other early in the process and alert the Commission to problems as expeditiously as possible.

cc: Commissioner Roberts
Commissioner Asselstine
Commissioner Bernthal
Commissioner Zech
OGC
OPE
SECY



OFFICE OF THE
COMMISSIONER

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

October 25, 1985

MEMORANDUM FOR: Samuel J. Chilk, Secretary
FROM: Lando W. Zech, Jr.
SUBJECT: SECY 85-272

I have reviewed and carefully considered the ACRS' advice that the EPA standards, in the opinion of the ACRS, are "unreasonably restrictive and contain serious deficiencies" together with their conclusion that the standards "will undoubtedly introduce unnecessary obstacles into the licensing process." I have also considered the DOE and EPA statements in support of the standards and their conclusion that the standards are reasonable and achievable. The NRC staff has concluded that the EPA standards are reasonable, achievable and flexible enough that they can be implemented.

In view of the conflicting advice provided to the Commission, OGC has provided options which the Commission may exercise and concluded that since "the ACRS concerns [are] governed by the policy and technical issues we have described rather than any strictly legal considerations, we make no recommendation on how the Commission should proceed, other than that it should not act without hearing from the NRC staff and fully assessing all the factors we have described." The staff has responded to the Commission at the October 21, 1985 public meeting and addressed the ACRS concerns. The staff has advised the Commission that the staff, as well as DOE and EPA, do not agree with the ACRS that the standards are overly restrictive and contain serious deficiencies. The staff stated that they believed, as did DOE and EPA, that the standards were flexible enough and could be executed.

With all due respect to the advice of the ACRS, I reaffirm my approval of SECY-85-272 in support of the DOE, EPA and staff recommendation.

However, I suggest that the staff be directed to address the ACRS' concerns when developing the package conforming Part 60 to the EPA standards. I understand they may do this by defining the basis for their assurance that adequate flexibility exists in the standards for them to be implemented.

cc: Chairman Palladino
Commissioner Roberts
Commissioner Asselstine
Commissioner Zech
EDO
ACRS

Ralph Stein, Acting Director
Engineering and Licensing Division
Office of Civilian Radioactive Waste Management
DOE

Chairman Palladino requested Commissioners to reexamine their
vote sheets on SECY-85-272 and inform the Office of the Secretary
as to their present position on this paper.

(OCM)

(SECY Suspense: 10/30/85)

cc: Chairman Palladino
Commissioner Roberts
Commissioner Asselstine
Commissioner Bernthal
Commissioner Zech
Commission Staff Offices
EDO
PDR - Advance
DCS - 016 Phillips



OFFICE OF THE
SECRETARY

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

IN RESPONSE, PLEASE
REFER TO: M851021B

October 29, 1985

MEMORANDUM FOR THE RECORD

FROM: Samuel J. Chilk, Secretary

SUBJECT: STAFF REQUIREMENTS - DISCUSSION WITH EPA,
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS, AND
STAFF ON EPA STANDARD FOR HLW, 1:30 P.M.,
MONDAY, OCTOBER 21, 1985, COMMISSIONERS'
CONFERENCE ROOM, D.C. OFFICE (OPEN TO PUBLIC
ATTENDANCE)

The Commission met with representatives of the Environmental Protection Agency (EPA), the Advisory Committee on Reactor Safeguards (ACRS), and staff to discuss EPA's environmental standards for the management and disposal of high-level radioactive waste. Representatives of the Department of Energy also took part in this meeting.

In addition to NRC staff, the following individuals participated in this meeting:

Sheldon Meyers, Acting Director
Office of Radiation Programs
EPA

Terry Yosie, Director
EPA Science Advisory Board

Floyd Galpin, Chief
Waste Management Standards Branch
Criteria and Standards Division
Office of Radiation Programs
EPA

Daniel Egan, Health Physicist
Waste Management Standards Branch
Criteria and Standards Division
Office of Radiation Programs
EPA

Dade Moeller
ACRS

William Purcell, Associate Director
Office of Geologic Repositories
DOE

requirements memorandum from Samuel J. Chilk to William J. Dircks dated November 27, 1985. Comments offered by the subcommittee members are documented in the transcripts of the meeting, as is the subcommittee's recommendation to proceed with the rulemaking package after accommodating, to the extent practical, those comments. The enclosed draft Federal Register notice has accommodated most of the subcommittee's comments. ~~Others, notably the equivalence of "reasonable assurance" with "reasonable expectation" and of "undisturbed performance" with "in the absence of unanticipated processes and events" are being proposed for public review and comment along with the other material in the notice.~~

The notice also discusses the interpretation of the term "reasonable assurance" in Part 60 and clearly states that this is considered to be equivalent in meaning to the EPA's term "reasonable expectation," which is found in the standard.

The staff is proposing to adopt EPA's definition of "controlled area," which is different from that currently in Part 60. In doing so, the staff has included text within the Supplementary Information section of the proposed Federal Register notice explaining the staff's reasons for adopting EPA's definition and the effects of this definition on the related concepts of the "disturbed zone" and "groundwater travel time."

Other significant amendments to the rule are; (1) Changes in certain definitions to achieve consistency between the standard and the rule, (2) ~~Addition of a requirement that estimates of cumulative releases over 10,000 years from all anticipated and unanticipated processes and events shall be incorporated into an overall probability distribution of cumulative release, to the extent practicable, in demonstrating compliance with the standard,~~ (3) Requiring information on the program for post-permanent closure monitoring of the repository, (4) Replacing the current Part 60 language which requires compliance with "such generally applicable environmental standards for radioactivity as may have been established by the Environmental Protection Agency" with the specific limits promulgated by EPA, (5) Incorporation of provisions of the "assurance requirements" where appropriate, (6) Adding the individual dose limits which are found in the standard and (7) Incorporating the special sources of groundwater protection requirements which are found in the standard.

The EPA staff is in agreement with the general approach of the proposed notice. Specific comments on the wording of the proposed amendments to Part 60 may be submitted by EPA during the public comment period.

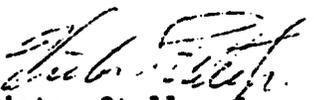
Commission resource needs to implement the provisions of 10 CFR Part 60 have been reflected in programmatic budget requests. Thus, no significant new resource expenditures will be required by issuance of the amendments.

Recommendations:

That the Commission:

1. Approve for publication as proposed amendments to 10 CFR Part 60 contained in the Federal Register notice (Enclosure A) which revise Part 60 to eliminate inconsistencies with the EPA HLW Standard.
2. Certify that this rule, if promulgated, will not have a significant economic impact on a substantial number of small entities. This certification is necessary in order to satisfy the requirements of the Regulatory Flexibility Act, 5 U.S.C 605(a)
3. Note:
 - a. Enclosure B contains a copy of the final EPA HLW standard as published in the Federal Register on September 19, 1985.
 - b. As provided by the Nuclear Waste Policy Act of 1982, no environmental assessment is being prepared in connection with this action.
 - c. The Chief Counsel for Advocacy of the Small Business Administration will be informed by the Division of Rules and Records of the certification regarding economic impact on small entities.
 - d. The Subcommittee on Energy and the Environment of the House Interior and Insular Affairs Committee, the Subcommittee on Nuclear Regulation of the Senate Committee on the Environment and Public Works, the Subcommittee on Energy, Nuclear Proliferation and Federal Services of the Senate Committee on Government Affairs, and the Subcommittee on Energy and Power of the House Interstate and Foreign Commerce Committee will be informed by a letter similar to Enclosure C.
 - e. This rule contains no new or amended recordkeeping, reporting, or application requirement, or any other type of information collection requirement, subject to the Paperwork Reduction Act (Pub. L. 96-511).
 - f. A regulatory analysis is presented in Enclosure E.

- g. The Office of Public Affairs has determined that it is necessary to issue a public announcement similar to Enclosure F in connection with these proposed amendments.
- h. The changes proposed to be made in 10 CFR Part 60 are provided in comparative text as Enclosure G.
- i. The draft Federal Register Notice (Appendix A) states that provisions of 10 CFR 50.109 on backfitting do not apply to this rulemaking because the rule is not applicable to production and utilization facilities licensed under 10 CFR Part 50.
- j. The press has reported, and the EPA staff has confirmed, that legal challenges have been filed opposing the EPA standards - no other details are currently known. The staff is monitoring this litigation.


Victor Stello, Jr.
Acting Executive Director
for Operations

Enclosures:

- A. Federal Register Notice for Proposed Amendments to Part 60
- B. EPA HLW standard
- C. Draft Congressional Letter
- D. Staff Requirements Memorandum from Samuel J. Chilk to William J. Dircks, dated November 27, 1985
- E. Regulatory Analysis
- F. Public Announcement
- G. Comparative Text
- H. Memorandum to Chairman Palladino from David A. Ward, dated November 14, 1985

Commissioners' comments or consent should be provided directly to the Office of the Secretary by c.o.b. Friday, April 4, 1986.

Commission Staff Office comments, if any, should be submitted to the Commissioners NLT Friday, March 28, 1986, with an information copy to the Office of the Secretary. If the paper is of such a nature that it requires additional time for analytical review and comment, the Commissioners and the Secretariat should be apprised of when comments may be expected.

DISTRIBUTION:

Commissioners

OGC

OPE

OI

OCA

OIA

OPA

REGIONAL OFFICES

EDO

ELD

ACRS

ASLBP

ASLAP

SECY

ENCLOSURE A

NUCLEAR REGULATORY COMMISSION

10 CFR Part 60

Disposal of High-Level Radioactive Wastes in Geologic Repositories;
Conforming Amendments

AGENCY: Nuclear Regulatory Commission.

ACTION: Proposed rule.

SUMMARY: The Nuclear Regulatory Commission (NRC) is proposing to amend its regulations for disposal of high-level radioactive wastes in geologic repositories. The amendments are necessary to conform existing NRC regulations to the environmental standards for management and disposal of high-level radioactive wastes promulgated by the Environmental Protection Agency (EPA) on September 19, 1985. The proposed rule would incorporate all the substantive requirements of the environmental standards and make several changes in the wording used by EPA in order to maintain consistency with the current wording of the NRC regulations.

DATE: Comment period expires _____. Comments received after this date will be considered if it is practical to do so, but assurance of consideration cannot be given except as to comments received on or before this date.

ADDRESSES: Written comments may be submitted to the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch. Comments may also be delivered to Room 1121, 1717 H Street NW, Washington, DC, from 8:15 a.m. to 5:00 p.m. weekdays. Copies of the documents referred to in this notice and comments

received may be examined at the NRC Public Document Room, 1717 H Street, NW, Washington, DC.

FOR FURTHER INFORMATION CONTACT: Daniel J. Fehringer, Division of Waste Management, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC, 20555, telephone (301) 427-4796.

SUPPLEMENTARY INFORMATION:

Background

Section 121 of the Nuclear Waste Policy Act of 1982 (NWSA), 42 USC 10141, directs the Environmental Protection Agency (EPA) to "promulgate generally applicable standards for protection of the general environment from offsite releases from radioactive material in repositories." EPA published its final high-level radioactive waste (HLW) standards in the Federal Register on September 19, 1985 (50 FR 38066). Section 121 of the NWSA further specifies that the regulations of the NRC "shall not be inconsistent with any comparable standards promulgated by [EPA]."

The Nuclear Regulatory Commission has previously published rules (10 CFR Part 60, 46 FR 13980, February 25, 1981, 48 FR 28204, June 21, 1983) which established procedures and technical criteria for disposal of HLW in a geologic repository by the U.S. Department of Energy (DOE). This notice describes the interpretations and analyses which the Commission considers to be appropriate for implementation of the EPA standards, and identifies modifications to the Commission's regulations which are considered appropriate to maintain consistency with the standards promulgated by EPA.

It should be noted that "working draft" versions of the EPA standards were available to the Commission when Part 60 was being developed, and the Commission structured its regulations to be compatible with those draft standards. (See, for example, 48 FR 28195-28205, June 21, 1983, where the Commission discussed its final technical criteria, and NUREG-0804, the staff's analysis of public comments on the proposed technical criteria. NUREG-0804 is available in the NRC Public Document Room.) Since many of the general features

of the "working drafts" remain present in the final standards, Part 60 is largely consistent with those standards. EPA has, however, sometimes used different terminology to describe concepts already present in Part 60. To maintain the overall structure of Part 60, and to avoid introduction of duplicative terminology which could prove confusing in a licensing review, the Commission prefers to retain its own established terms. Most of the amendments to Part 60 proposed in this notice involve direct incorporation within Part 60 of the substantive requirements of the EPA standards, reworded as necessary to conform to the terminology of Part 60. (Additional proposed amendments derive from EPA's "assurance requirements," as discussed in Section III of this notice. One further amendment, unrelated to the EPA standards, is proposed for clarification of existing wording in Part 60.) With the issuance of this rule, no substantive changes are intended in the requirements of the EPA standards or in the environmental protection they afford.

The EPA standards specify certain limits on radiation exposures and releases of radioactive material during two principal stages: first, the period of management and storage operations at a repository and, second, the long-term period after waste disposal has been completed. These standards, and the proposed rules to implement them during operations and after closure, are discussed in Section I below, while Section II provides some further observations regarding the manner in which the Commission intends to apply the EPA standards in its licensing proceedings. Section III describes additional proposed rules related to certain "assurance requirements" which are present in EPA's standards but which are not applicable to NRC-licensed facilities. In order to avoid potential jurisdictional problems which might arise if this section of the EPA standards were applied to NRC-licensed facilities, the NRC is proposing to add substantially equivalent provisions to its regulations. Finally, this notice presents a section-by-section analysis of the proposed rule (Section IV), followed by the specific text of the proposed amendments to Part 60. (The organization of Section IV follows that of Part 60 while the text of Section I is organized to present a section-by-section discussion of the EPA standards. Parts of Section IV are therefore repetitions of information presented in Section I.)

I. Limits on Exposures and Releases

The limits established by EPA for the period of repository operations appear at 40 CFR 191.03. The limits applicable to the period after disposal include "containment requirements" (limits on cumulative releases of radionuclides to the environment for 10,000 years) in §191.13, "individual protection requirements" in §191.15, and "ground water protection requirements" in §191.16. Implementation of each of these sections is discussed in the following paragraphs.

Standards for repository operations (§191.03). The standards for repository operations are virtually identical to the standards previously promulgated by EPA for the uranium fuel cycle (42 FR 2860, January 13, 1977), and will be implemented in the same manner.* DOE will be expected to demonstrate, through analyses of anticipated facility performance, that the dose limits of these standards, as well as the standards for protection against radiation set out in 10 CFR Part 20, will not be exceeded. Releases of radionuclides and resulting doses during operations are amenable to monitoring, and DOE will be required to conduct a monitoring program to confirm that the limits are complied with. Section 60.111(a) would be amended to include the EPA dose limits. Section 60.101(a)(2) already includes a provision requiring "reasonable assurance" that the release limits be achieved, and it is not necessary to repeat this language in the release limits of §60.111. It is also not necessary to employ the terms "management" and "storage," as EPA has done, since all preclosure repository operations are already subject to the provisions of §60.111.

*It should be noted that a potential ambiguity exists in this section of EPA's HLW standards and in EPA's uranium fuel cycle standards. Both standards limit the annual dose equivalent to any member of the public to "25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other critical organ" (emphasis added). The Commission has always interpreted these limits as if the word "and" were replaced by "or." Thus, the Commission would not consider it acceptable to allow an annual dose equivalent of 25 millirems to the whole body and an additional 25 millirems to any other organ. The Commission will continue to implement these limits as it has in the past, but will encourage EPA to clarify the wording quoted above.

Postclosure standards. The EPA postclosure standards are all expressed in terms of a "reasonable expectation" of meeting specified levels of performance. EPA explained that it selected this term because "'reasonable assurance' has come to be associated with a level of confidence that may not be appropriate for the very long-term analytical projections that are called for by 191.13." The Commission is sensitive to the need to account for the uncertainties involved in predicting performance over 10,000 years, and the difficulties as well as the importance of doing so. The Commission has attempted to address this concern in the existing language of §60.101(a)(2). That section requires a finding of reasonable assurance, "making allowance for the time period, hazards, and uncertainties involved, that the outcome will be in conformance" with the relevant criteria. Rather than adopt an additional concept such as "reasonable expectation," the Commission proposes to add additional explanatory text, derived from EPA's wording, to its existing discussion of reasonable assurance. This text will make clear the Commission's belief that its concept of reasonable assurance, although somewhat different from previous usage in reactor licensing, is appropriate for evaluations of repository performance where long-term issues and substantial uncertainties are inherent in projections of repository performance. The Commission considers that the level of confidence associated with its concept of reasonable assurance is the same as that sought by EPA in the use of the term "reasonable expectation."

In the case of the individual protection requirements (40 CFR 191.15), the standards limit the annual dose equivalent to any member of the public in the accessible environment. A new provision in section 60.112(b) is proposed that would include the dose limits established by EPA as well as the additional specifications, which the Commission finds to be reasonable, with regard to consideration of all pathways including consumption of drinking water from a "significant source of ground water," as defined by EPA.

The EPA standards require that the individual protection requirements be achieved only for "undisturbed performance" of a geologic repository ("disposal system" in EPA's terminology). The proposed amendment to Part 60 makes no

reference to "undisturbed performance." Instead, it provides that the standard is to be met "in the absence of unanticipated processes and events." The Commission considers the concepts of undisturbed performance and the absence of unanticipated processes and events to be identical. As used by EPA (40 CFR 191.12(p)), "undisturbed performance" refers to the predicted behavior of a disposal system if it is "not disrupted by human intrusion or the occurrence of unlikely natural events." Since human intrusion and unlikely natural processes and events are precisely the types of "unanticipated processes and events" defined in §60.2, the two concepts are the same. Thus, the Commission considers that the phrase "in the absence of unanticipated processes and events" has the same meaning as "undisturbed performance" in the EPA standards. To maintain the overall structure of Part 60, and to avoid introduction of duplicative language, the Commission prefers to retain its own established terms.

The engineered barriers of a repository will, in many cases, be instrumental in achieving compliance with both the individual protection requirements and the groundwater protection requirements discussed below. The Commission notes that the existing provisions of Part 60 require the engineered barriers of a repository to achieve their containment and release rate performance objectives "assuming anticipated processes and events." Thus, equating "undisturbed performance" with "anticipated processes and events" causes no change in the types of conditions for which the engineered barriers must be designed.

The ground water protection requirements (40 CFR 191.16) focus on the quality of any "special source of ground water," which is defined, generally, as a source of drinking water in an area that includes and surrounds the geologic repository. This area extends for five kilometers beyond the controlled area. The standard applies to water "withdrawn" from such a special source. The Commission is proposing to include the EPA standard as a new performance objective (§60.112(c)). Once again the rule applies in the absence of unanticipated processes and events instead of "undisturbed performance."

The containment requirements (40 CFR 191.13) restrict the total amount of

radioactive material released to the environment for 10,000 years following permanent closure of a repository. EPA provides a table listing release limits for the significant radionuclides present in HLW or spent fuel. The values in this table were derived, based on environmental transport and dosimetry considerations, so that the amount of each radionuclide listed in the table will, if released to the environment, produce approximately the same number of population health effects. The standard further specifies different release limits for releases with differing likelihoods of occurrence. The Commission is proposing to incorporate these requirements as a new performance objective (§60.112(a)), along with a new §60.115 containing EPA's table of release limits.

The regulation goes on to state that the disposal systems shall be designed to provide a reasonable expectation - "based on performance assessments" - that the release limits are satisfied. While the proposed amendments incorporate most of the EPA standard in its precise terms, they omit the reference to performance assessments. Part 60 already requires analyses virtually identical to those contemplated by EPA, but the Commission proposes to add additional wording to §60.21(c)(1)(ii)(C) to emphasize consistency with the EPA standards.

The Commission notes, in this connection, that EPA's reference to estimating the cumulative releases caused by all significant processes and events, to be incorporated in an overall probability distribution of cumulative release to the extent practicable, does not modify the principles underlying Part 60. As was observed when NRC's final technical criteria were published in 1983 (48 FR 28204), the Commission expects that the information considered in a licensing proceeding will include probability distribution functions for the consequences from anticipated and unanticipated processes and events. Further information concerning the Commission's plans for assessing repository performance is contained in Section II of this notice.

II. Additional Comments on Implementation of the EPA Standards

Four sections of the EPA standards contain numerical requirements for which compliance must be demonstrated -- standards for repository operations, post-closure individual and groundwater protection requirements and containment requirements restricting the total amount of radionuclides projected to be released to the environment after repository closure. The discussion of Section I of this notice articulates the Commission's interpretation of the standards that have been issued by EPA. Additional comments related to implementation of each of these sections are presented in the following paragraphs.

Standards for repository operations. As discussed previously, the standards for repository operations are virtually identical to the standards previously promulgated by EPA for the uranium fuel cycle, and will be implemented in the same manner. A license applicant will be expected to demonstrate, through analyses of anticipated facility performance, that the dose limits of these standards will not be exceeded. Doses during operations are amenable to monitoring, and the applicant will be required to conduct a monitoring program to confirm that the dose limits are complied with.

Individual and groundwater protection requirements. The individual and groundwater protection requirements are applicable for the first 1,000 years after permanent closure of a repository. Monitoring is not practical for this period of time and the applicant will therefore be required to demonstrate compliance with these requirements through analyses of projected repository performance. Two general approaches might be pursued by DOE. First, DOE might choose to calculate the expected concentrations of radionuclides in certain groundwaters potentially useable by humans in the future. Such calculations would include projections of waste package and engineered barrier performance (to provide a source term) as well as evaluations of the direction, velocity and volumetric flow rates of groundwaters near the repository. The EPA standards specify the types of groundwaters to be considered in such analyses (through the definitions of the terms "significant" and "special" sources of groundwater), and these concepts will be incorporated directly into Part 60.

Alternatively, DOE might choose to show compliance with these requirements by demonstrating that other barriers, such as the waste packages or the emplacement medium (e.g., salt), will provide substantially complete containment for the first 1,000 years after permanent closure thereby preventing contamination of the groundwaters of concern.

If DOE chooses to calculate the expected concentrations of radionuclides in groundwaters, rather than to rely on containment by engineered barriers, it will also be necessary to calculate potential doses to individuals in the future. The individual protection requirements limit the annual dose equivalent to any member of the public in the accessible environment. If a "significant source of groundwater" (as defined) is present, the Commission will assume that a hypothetical individual resides at the boundary of the controlled area and obtains his domestic water supply from a well at that location. If no such source of groundwater is present, the location of the maximally exposed individual and the pathways by which he might be exposed to radionuclides released from a repository must be examined on a site-specific basis.

The individual protection requirements also necessitate assumptions about the dietary patterns and other potential modes of ingestion of radionuclides during the next 1,000 years. The Commission will assume that current patterns remain unchanged, unless it can be convincingly demonstrated that a change is likely to occur (e.g., reduced groundwater consumption due to depletion of an aquifer).

Both the individual and groundwater protection requirements are applicable only for "undisturbed performance" of a repository system. As discussed in Section I, this term is considered to be equivalent to "anticipated processes and events," as currently defined in Part 60. The Commission will therefore require a demonstration of compliance with these requirements assuming the occurrence of anticipated processes and events, but will not require a demonstration of compliance in the event of unanticipated processes and events.

Containment requirements. The containment requirements are applicable for 10,000 years after repository closure. Therefore, compliance with these

requirements must also be evaluated by analyses of projected repository performance rather than by monitoring. The containment requirements call for significantly different analyses than those discussed above. This section of the EPA standards restricts the total amount of radioactive material released to the environment for 10,000 years following permanent closure of a repository. This section further specifies different release limits for releases with differing likelihoods of occurrence. Notwithstanding the quantitative probabilistic form of the EPA containment requirements (40 CFR 191.13), the Commission finds that there is adequate flexibility therein to allow them to be implemented using the licensing procedures of 10 CFR Parts 2 and 50. A further discussion of these matters is appropriate in order to avoid ambiguity in the application of the probabilistic conditions.

As the Commission emphasized when the technical criteria for geologic repositories were promulgated in final form (48 FR 28204), there are two distinct elements underlying a finding that a proposed facility satisfies the desired performance objective for long-term isolation of radioactive waste. There is, first, a standard of performance - some statement regarding the quantity of radioactive material that may be released to the accessible environment. This standard can be expressed in quantitative terms, and may include numerical requirements for the probabilities of exceeding certain levels of release.

The second element of a finding relates to the confidence that is needed by the factfinder in order to be able to conclude that the standard of performance has been met. The Commission has insisted, and the EPA has agreed, that this level of confidence must be expressed qualitatively. The licensing decisions that must be made in connection with a repository involve substantial uncertainties, many of which are not quantifiable (e.g., those pertaining to the correctness of the models used to describe physical systems). Such uncertainties can be accommodated within the licensing process only if a qualitative test is applied for the level of confidence that the numerical performance objective will be achieved.

The essential point to be kept in mind is that findings regarding

long-term repository performance must be made with "reasonable assurance." The Commission attempted to explain this concept in the existing wording of §60.101(a) where it noted that allowance must be made for the time period, hazards, and uncertainties involved. Additional language is being proposed at this time, in the same section of Part 60, to further emphasize that qualitative judgments will need to be made including, for example, consideration of the degree of diversity or redundancy among the multiple barriers of a specific repository.

Application of a qualitative test in no way diminishes the level of safety required by a numerical standard. The applicant will be required to submit a systematic and thorough analysis of potential releases and the Commission will issue a license only if it finds a substantial, though unquantified, level of confidence that compliance with the release limits will be achieved. As we have stated previously (48 FR 28201), in order to make a finding with "reasonable assurance," the performance assessment which has been performed in the course of the licensing review must indicate that the likelihood of exceeding the EPA standard is low and, further, the Commission must be satisfied that the performance assessment is sufficiently conservative, and its limitations are sufficiently well understood, that the actual performance of the geologic repository will be within predicted limits.

The Commission will evaluate compliance with the containment requirements based on a performance assessment. Such an assessment will: (1) identify all significant processes and events which could affect the repository, (2) evaluate the likelihood of each process or event and the effect of each on release of radionuclides to the environment, and (3) to the extent practicable, combine these estimates into an overall probability distribution displaying the likelihood that the amount of radioactive material released to the environment will exceed specified values. The Commission anticipates that the overall probability distribution will be displayed in the format shown below.

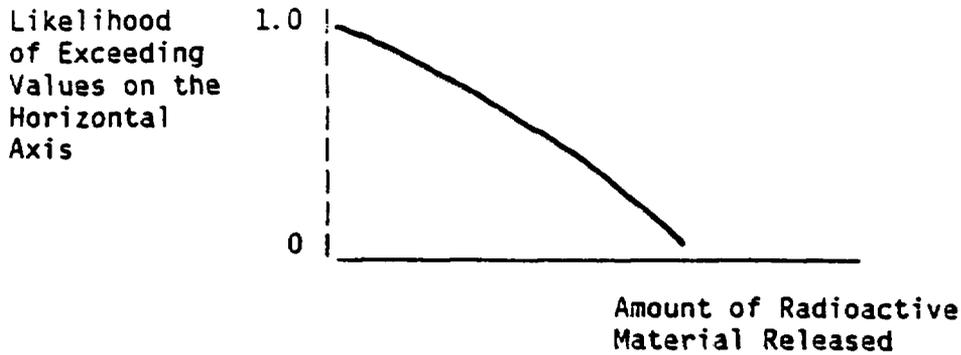


Figure 1. Illustrative "Complementary Cumulative Distribution Function."

When the results of analyses are displayed in this format, the limits of EPA's containment requirements take the form of "step functions," as shown in Figure 2.

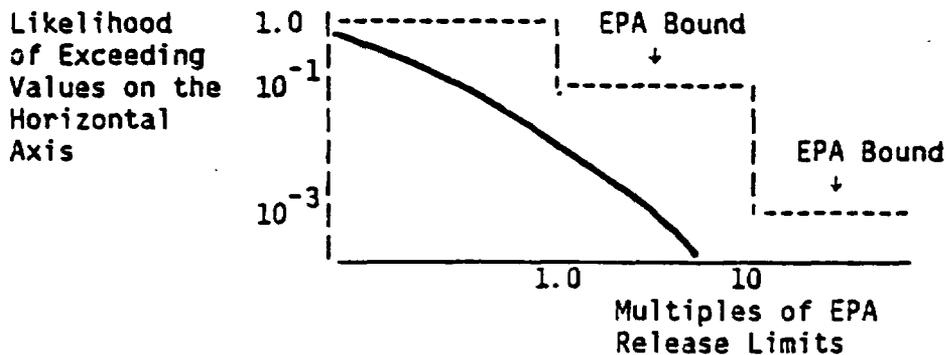


Figure 2. Graphic Representation of EPA Containment Requirements.

In Figure 2, releases which exceed the value specified in the EPA containment requirements (Table 1) must have a likelihood less than one chance in ten (over 10,000 years), and releases which exceed ten times that value must have a likelihood less than one chance in one thousand (over 10,000 years). Thus, in order to demonstrate compliance with EPA's containment requirements, the entire probability distribution must lie below the "stair-step" constraints illustrated in Figure 2.

In constructing a probability distribution of the type illustrated above, it is necessary to consider, in EPA's terms, all "significant processes and events that may affect the disposal system." This is equivalent, as we interpret the EPA standard, to all "anticipated" and "unanticipated" processes and events in the terminology of Part 60. (By the definition of "unanticipated processes and events" in Part 60, processes and events less likely than "unanticipated" are not sufficiently credible to warrant consideration.) For purposes of the proposed §60.112(a) only, which incorporates EPA's containment requirements, no distinction is to be made between "anticipated" and "unanticipated" processes and events; all such processes and events must be factored into the evaluation, including determination of such probabilities of occurrence as may be found to be appropriate. (For purposes of the proposed §60.112(b) and (c), which incorporate EPA's individual and groundwater protection requirements, only "anticipated" processes and events need be considered as discussed previously.)

The Commission will require an extensive and thorough identification of relevant processes and events, but will require analysis of the probability and/or consequence of each only to the extent necessary to determine its contribution to the overall probability distribution. If it can be shown, for example, that a particular event is so unlikely to occur that its effects on the probability distribution would not be meaningful, further analyses of the consequences of that event would not be required. Generally, categories of processes and events which can be shown to have a likelihood less than one chance in 10,000 over 10,000 years, along with categories of processes and events which otherwise can be shown not to change the remaining probability distribution of cumulative release significantly, need not receive further analysis. (The term "categories" is used to refer to general classes of processes and events, such as faulting, volcanism, or drilling. Subsets of these general categories, such as drilling which intersects a canister or fault displacement of a specific magnitude, may need to be retained in an analysis if the general category has been finely divided into a large number of specific process or event descriptions, each with reduced probabilities of occurrence.)

Treatment of uncertainties. As discussed previously, substantial uncertainties will be involved in analyses of long-term repository performance. These uncertainties may include (1) identification of basic phenomena and their potential effects on repository performance, (2) development and validation of models to describe these phenomena, (3) accuracy of available data, and (4) calculational uncertainties. Various methods may be used to accommodate such uncertainties including, for example, numerical estimates of uncertainties (expressed as probability distributions) or conservative, "bounding" models or data. Treatment of uncertainties will rely heavily on expert judgment, both for selection of an appropriate method and for application of that technique. EPA recognized the importance of uncertainties when its standards were promulgated. In Appendix B of 40 CFR Part 191 (50 FR 38088, September 19, 1985), EPA stated "substantial uncertainties are likely to be encountered in making (numerical) predictions (of repository performance). In fact, sole reliance on these numerical predictions to determine compliance may not be appropriate; the implementing agencies may choose to supplement such predictions with qualitative judgments as well." It is possible - in fact likely - that the various parties to a licensing proceeding will have significantly different views, all with technical merit, regarding the best methods to use, and these differing views may result in presentation of widely different estimates of repository performance.

Any such differences could be resolved in a number of ways. One permissible method for dealing with the uncertainties reflected in the record of the proceeding would be to rely heavily upon conservative, "bounding" analyses. Perhaps it could be shown that even if this approach were employed, the predicted performance would still satisfy the containment requirements established by EPA. On the other hand, an apparent violation of the standard (based on conservative analyses) would not necessarily preclude the Commission from finding, with reasonable assurance, that repository performance would conform to the EPA standard. After carefully evaluating the relevant uncertainties, DOE could present the same data in the form of a cumulative probability distribution that was less conservative - for example, one that

more accurately represents the best current technical understanding. Thus, alternative methods are available to DOE for treatment of uncertainties when making its demonstration of reasonable assurance of compliance with the provisions of Part 60.

It should be noted, however, that analyses based on "best estimates" of repository performance might be found to be inadequate if substantial uncertainties are present. In that case, notwithstanding the apparent conformity with the EPA standard, the Commission might ultimately conclude that it lacked the necessary reasonable assurance, considering the uncertainties involved, that the performance would meet the containment requirements.

Because uncertainties are so important in analyses of repository performance and will play such a major role in a licensing proceeding, the Commission emphasizes the importance of efforts being undertaken to foster a common technical understanding and to resolve issues, where it is practicable to do so, prior to receipt of a license application. Many of the provisions of the Nuclear Waste Policy Act are directed toward this goal. One especially important opportunity, in this regard, is DOE's preparation of site characterization plans and the review and comment process to be carried out by the Commission and other interested parties. Additionally, NRC and DOE are engaged, under an interagency procedural agreement, in ongoing technical discussions on matters that pertain to licensing requirements; these discussions are in the form of open meetings, affording other persons an opportunity to identify pertinent considerations that might also need to be addressed. The staff is also issuing staff technical positions on specific methods of analysis that would be acceptable for evaluating compliance with Part 60 technical criteria and performance objectives. As issues mature, the Commission will, where appropriate, use the rulemaking process to seek resolution of issues where a licensing proceeding might otherwise encounter difficulties due to ambiguity regarding acceptable assessment methods. Nevertheless, the data available at the time of licensing will inevitably be imperfect. It is therefore essential that every effort be made by DOE - and by any other party that develops data which it may propound at a hearing - to use careful methods to enhance, and document, the trustworthiness of the evidence which it may submit.

III. EPA Assurance Requirements

EPA's regulations (40 CFR 191.14) include certain "assurance requirements" designed, according to the rule, to provide the confidence needed for long-term compliance with the containment requirements. As noted by EPA in its preamble, the Commission took exception to the inclusion of these provisions in the regulations. The Commission viewed the assurance requirements as matters of implementation that were not properly part of the EPA's authorities assigned by Reorganization Plan No. 3 of 1970. In response to this concern, the two agencies have agreed to resolve this issue by NRC's making appropriate modifications to Part 60, reflecting the matters addressed by the assurance requirements, and by EPA's declaration that those requirements would not apply to facilities regulated by the Commission. The following discussion sets forth the Commission's views with respect to each of the EPA assurance requirements and identifies the proposed rule changes that are deemed to be appropriate under the circumstances.

EPA Assurance Requirement 40 CFR 191.14(a). Active institutional controls over disposal sites should be maintained for as long a period of time as is practicable after disposal; however, performance assessments that assess isolation of the wastes from the accessible environment shall not consider any contributions from active institutional controls for more than 100 years after disposal.

Analysis and Proposed Changes. The Commission's existing provisions (§60.52) related to license termination will determine the length of time for which institutional controls should be maintained, and there is therefore no need to alter Part 60 to reflect this part of the assurance requirement.

The second part of this assurance requirement would require that "active" institutional controls be excluded from consideration (after 100 years) when the isolation characteristics of a repository are assessed. It has always been the intent of Part 60 not to rely on remedial actions (or other active institutional controls) to compensate for a poor site or inadequate engineered barriers. However, in the definition of "unanticipated processes and events," Part 60 expressly contemplates that, in assessing human intrusion scenarios,

the Commission would assume that "institutions are able to assess risk and to take remedial action at a level of social organization and technological competence equivalent to, or superior to, that which was applied in initiating the processes or events concerned" (emphasis added). Therefore, it might appear at first examination that Part 60 is at odds with the EPA assurance requirement.

Although both the EPA regulation and Part 60 refer to "remedial action," the action being considered is not the same. The EPA assurance requirement deals with a planned capability to maintain a site and, if necessary, to take remedial action at a site in order to assure that isolation is achieved. The Commission agrees that such a capability should not be relied upon. The extent to which corrective action may be taken after an unanticipated intrusion occurs is an entirely different matter. The Commission may wish to consider, for example, the extent to which the application of the limited societal response capability assumed by the rule (e.g., sealing boreholes consistent with current petroleum industry practice) could reduce the likelihood of releases exceeding the values specified in the containment requirements or could eliminate certain hypothetical scenarios such as systematic and persistent intrusions into a site.

Subject to the comments above, the Commission concurs with the EPA's definitions of "active" and "passive" institutional controls, as well as the principle that ongoing, planned, active protective measures should not be relied upon for more than 100 years after permanent closure. We are therefore proposing to include EPA's definitions, together with a new section (§60.114) which would expressly provide that active (or passive) institutional controls shall not be deemed to assure compliance with the containment requirements over the long term. Some activities which arguably fall within EPA's definition of "active institutional controls" (e.g. remedial actions and monitoring parameters related to geologic repository performance) are relevant to assessing the likelihood and consequences of processes and events affecting the geologic setting. We are proposing, also in §60.114, to allow such activities to be considered for this purpose. We regard this as being fully consistent with the thrust of the EPA position.

EPA Assurance Requirement 40 CFR 191.14(b). Disposal systems shall be monitored after disposal to detect substantial and detrimental deviations from expected performance. This monitoring shall be done with techniques that do not jeopardize the isolation of the wastes and shall be conducted until there are no significant concerns to be addressed by further monitoring.

Analysis and Proposed Changes. Part 60 currently requires DOE to carry out a performance confirmation program which is to continue until repository closure. Part 60 does not now require monitoring after repository closure because of the likelihood that post-closure monitoring of the underground facility would degrade repository performance. The Commission recognizes, however, that monitoring such parameters as regional groundwater flow characteristics may, in some cases, provide desirable information beyond that which would be obtained in the performance confirmation program, and the Commission is proposing to require such monitoring when it can be accomplished without adversely affecting repository performance.

The proposed requirement for post-permanent closure monitoring requires that such monitoring be continued until termination of a license. The Commission intends that a repository license not be terminated until such time as the Commission is convinced that there is no significant additional information to be obtained from such monitoring which would be material to a finding of reasonable assurance that long-term repository performance would be in accordance with the established performance objectives.

A number of changes in Part 60 are proposed to reflect these views with respect to post-closure monitoring. First, a new section (§60.144) would provide for the performance confirmation program, already required by Subpart F of Part 60, to include a program of post-closure monitoring. Second, the licensing findings required at the time of license termination (§60.52(c)) would specifically be related to the results available from the post-closure monitoring program. Third, DOE would be required to provide more detailed information concerning its plans for post-closure monitoring in its original application (§60.21(c)) and when it applies to amend its license prior to permanent closure (§60.51(a)).

EPA Assurance Requirement 40 CFR 191.14(c). Disposal sites shall be designated by the most permanent markers, records, and other passive institutional controls practicable to indicate the dangers of the wastes and their location.

Analysis and Proposed Changes. The existing provisions of 10 CFR Part 60 already require that DOE take the measures set out in this assurance requirement. For further information, refer to §§60.21(c)(8) (requirement that license application describe controls to regulate land use), §60.51(a)(2) (information to be submitted, prior to permanent closure, with respect to land use controls, construction of monuments, preservation of records, etc.), and §60.121 (requirements for ownership and control of interests in land).

EPA Assurance Requirement 40 CFR 191.14(d). Disposal systems shall use different types of barriers to isolate the wastes from the accessible environment. Both engineered and natural barriers shall be included.

Analysis and Proposed Changes. This is another provision that is already inherent in Part 60. Nevertheless, in order to avoid any possible doubt in this regard, a new paragraph (§60.113(d)) would be added to state explicitly that the geologic repository shall incorporate a system of multiple barriers, both engineered and natural.

Questions might arise regarding the types of engineered or natural materials or structures which would be considered to constitute "barriers," as required by this new language. In this connection, the Commission notes that §60.2 now contains this definition: "'Barrier' means any material or structure that prevents or substantially delays movement of water or radionuclides" (emphasis added). Thus, consistent with the approach endorsed by EPA, the Commission considers that the new paragraph to be added to §60.113 will confirm its commitment to a multiple barrier approach as contemplated by Section 121(b)(1)(B) of the Nuclear Waste Policy Act.

EPA Assurance Requirement 40 CFR 191.14(e). Places where there has been mining for resources, or where there is a reasonable expectation of exploration for scarce or easily accessible resources, or where there is a significant concentration of any material that is not widely available from other sources, should be avoided in selecting disposal sites. Resources to be considered shall include minerals, petroleum or natural gas, valuable geologic formations, and ground waters that are either irreplaceable because there is no reasonable alternative source of drinking water available for substantial populations or that are vital to the preservation of unique and sensitive ecosystems. Such places shall not be used for disposal of the wastes covered by this Part [40 CFR Part 191] unless the favorable characteristics of such places compensate for their greater likelihood of being disturbed in the future.

Analysis and Proposed Changes. Part 60 contains provisions that, in large part, are equivalent to this assurance requirement. See §60.122(c)(17), (18), and (19). The existing regulation does not, however, address "a significant concentration of any material that is not widely available from other sources."

The Commission believes that there is merit in having the presence of such concentrated materials evaluated in the context of the licensing proceeding. It is, after all, quite possible that the economic value of materials could change in the future in a way which might attract future exploration or development detrimental to repository performance. By adding an additional "potentially adverse condition" to those already set out in the regulation, DOE would be required to identify the presence of the materials in question and evaluate the effect thereof on repository performance, as specified in §60.122(a)(2)(ii). It should be noted that the presence of potentially adverse conditions does not preclude the selection and use of a site for a geologic repository, provided that the conditions have been evaluated and demonstrated not to compromise performance.

EPA Assurance Requirement 40 CFR 191.14(f). Disposal systems shall be selected so that removal of most of the wastes is not precluded for a reasonable period of time after disposal.

Analysis and Proposed Changes. The Commission understands that the purpose of this assurance requirement is to discourage or preclude the use of disposal concepts such as deep well injection for which it would be virtually impossible to remove or recover wastes regardless of the time and resources employed. (This provision is thus significantly different from the Commission's retrievability requirement.) For a mined geologic repository - which is the only type of facility subject to licensing under 10 CFR Part 60 - wastes could be located and recovered (i.e. "removed," in the sense that EPA is using the term), albeit at high cost, even after repository closure. A repository would therefore meet this assurance requirement, and no further statements on the subject in Part 60 are indicated.

Petition for Rulemaking.

The Commission calls to the attention of all interested parties a pending petition for rulemaking submitted by the States of Nevada and Minnesota which deals, in large part, with the matters addressed by Section III of this notice. All relevant comments received by the Commission in response to the notice of receipt of the petition for rulemaking (published in the FEDERAL REGISTER on December 19, 1985, 50 FR 51701) will be considered along with comments received in response to this notice. It should be noted that the Commission's present proposal conforms to the approach which was discussed with EPA during the course of its rulemaking. The petition for rulemaking follows the same language very closely, but does suggest certain modifications. The Commission would be particularly interested in comments addressed to the respective merits of the language proposed herein and that proposed by the States of Nevada and Minnesota.

The Commission further notes that EPA has provided it with copies of comments regarding the assurance requirements that were received during the 40 CFR Part 191 rulemaking. These comments are available for inspection in the Commission's public document room.

IV. Section by Section Analysis of Proposed Conforming Amendments

The Commission considers that the simplest and most useful way to amend Part 60 for consistency with the EPA standards would be to incorporate directly within Part 60 all the substantive requirements of the environmental standards promulgated by EPA, modified as necessary to conform to the terminology currently used in Part 60. The following paragraphs present a section-by-section analysis of the NRC's proposed conforming amendments to Part 60.

§60.1 Purpose and scope.

This paragraph is analogous to EPA's 40 CFR 191.01 and 191.11 which state the applicability of the EPA standards. Part 60 is, however, a more specific regulation than the EPA standards in that it addresses only deep geologic repositories used for disposal of high-level radioactive wastes, while the EPA standards apply to other disposal methods and certain other types of radioactive wastes. No changes are proposed for §60.1, but the Commission notes that any regulations developed in the future for alternative disposal methods or for other types of wastes will incorporate any applicable provisions of the EPA standards.

§60.2 Definitions.

New definitions of several terms are proposed for incorporation within §60.2. These are taken directly from the EPA standards (or from 40 CFR Part 190) and are needed for purposes of implementation. These added terms are:

- 1) Active institutional control
- 2) Community water system
- 3) Passive institutional control
- 4) Significant source of groundwater
- 5) Special source of groundwater
- 6) Transmissivity
- 7) Uranium fuel cycle

In addition, the definition of "controlled area" and the related definition of "accessible environment" in the EPA standards are different from those currently in Part 60. The Commission proposes to revise its current definitions to conform to EPA's wording. In the case of "accessible environment," the change is merely editorial. The amendments to the definition of "controlled area" are also largely editorial, except for the specification of extent - i.e., that the controlled area is to encompass "no more than 100 square kilometers" and to extend "horizontally no more than five kilometers in any direction from the outer boundary of the original location of the radioactive wastes."

The Commission has reviewed this aspect of the EPA definition in the light of the policies which it articulated when the final technical criteria of 10 CFR Part 60 were adopted. One of these policies was that the controlled area "must be small enough to justify confidence that the monuments will effectively discourage subsurface disturbances." The prior rule would have authorized the establishment of a controlled area well over 300 square kilometers (about 75,000 acres) in size. While we would not deny the abstract possibility that effective controls could be instituted even over an area of that magnitude, we have much greater confidence that DOE would be able to demonstrate an ability to discourage subsurface disturbances over an area of more limited extent. It is our judgment that the 100 square kilometers that EPA has adopted, after consultation with the NRC staff, represents an appropriate limitation.

The other policy related to the definition of the "controlled area" is that it must allow the isolation capability of the rock surrounding the underground facility to be given appropriate weight in licensing reviews. This isolation capability is measured in two ways. First, it is to be taken into account in determining whether releases of radionuclides to the accessible environment are within the limits specified in the "containment requirements" (40 CFR 191.13). Second, under §60.113(a)(2), the isolation capability of the geologic setting must be such that the pre-waste-emplacment groundwater travel time along the fastest path of likely radionuclide travel from the disturbed zone to the accessible environment shall be a specified period (generally, 1000 years).

The Commission anticipates that adoption of the EPA terminology will have little effect on achievement of the containment requirements inasmuch as the controlled area is allowed a horizontal extent as large as five kilometers (presumably in the direction of radionuclide travel). Nor does the Commission anticipate that the limitation will make it impracticable to achieve a demonstration of compliance with the groundwater travel time performance objective. When the Commission adopted Part 60, it observed that the "accessible environment" might be larger (and, of course, the "controlled area" might therefore be smaller) than would be the case under the EPA standards then being considered (48 FR 28202). EPA has now moved in the direction of eliminating this difference, and the Commission's amendment, for this reason, represents no important change.

The proposed reduction in the maximum allowable extent of the controlled area (i.e., distance to the accessible environment) requires additional discussion to clarify the Commission's concepts of "disturbed zone" and "groundwater travel time." Groundwater travel time from the edge of the disturbed zone to the accessible environment is one of the criteria which the Commission identified, at the time of proposed rulemaking, as providing confidence that the wastes will be isolated for at least as long as they are most hazardous (46 FR 35280, 35281, July 8, 1981). As noted above, this objective concerns travel time from the edge of the disturbed zone rather than from the edge of the underground facility. The Commission selected the disturbed zone for the purpose of determining the groundwater travel time since the physical and chemical processes which isolate the wastes are "especially difficult to understand in the area close to the emplaced wastes because that area is physically and chemically disturbed by the heat generated by those wastes." Ibid.

One potential type of effect which could alter local groundwater flow conditions is thermal buoyancy of groundwater. Because buoyancy effects could extend over significant distances (see, e.g., M. Gordon and M. Weber, "Non-isothermal Flow Modeling of the Hanford Site," available in the NRC Public document room) and because the Commission is proposing to reduce the maximum allowable distance to the accessible environment, it is particularly

important to emphasize that the Commission did not intend such effects to serve as the basis for defining the extent of the disturbed zone. The Commission recognizes that such effects can be modeled with well developed assessment methods, and therefore were not the type of effects for which the disturbed zone concept was developed. Any contrary implication in our statement of considerations at the time the technical criteria were issued in final form (see 48 FR 28210) should be disregarded. (The staff is currently developing Generic Technical Positions discussing the disturbed zone and groundwater travel time. These technical positions will be publicly available prior to promulgation of these proposed amendments in final form, and will illustrate how the staff intends to approach these two concepts.)

Four other terms defined by EPA deserve additional discussion here.

The EPA standards contain a definition of the term "transuranic radioactive waste." The Commission does not use this term in Part 60 and thus has no need to define it there. All radioactive waste stored or disposed of at a geologic repository licensed under Part 60 - including transuranic radioactive waste - would be subject to the requirements of the EPA standards as applied by the rules proposed herein.

EPA defines the terms "storage" and "disposal" to mean retrievable storage and permanent isolation, respectively. Under Part 60, on the other hand, the term "storage" is used in the sense of Section 202 of the Energy Reorganization Act of 1974 (42 USC 5842) to refer to both long-term storage and disposal of wastes. The difference in EPA and NRC usage has no effect upon application of the EPA standards at NRC-licensed geologic repositories.

The Commission has recently defined "groundwater," for purposes of Part 60, to include all water which occurs below the land surface (50 FR 29641, July 22, 1985), while the EPA standards use the term to mean water below the land surface in a zone of saturation (emphasis added). The EPA standards use the term only in connection with the more specifically defined terms "significant source of ground water" and "special source of ground water." Thus, it is possible to identify "significant" or "special" sources of groundwater unambiguously with either definition of the term "groundwater," and the Commission therefore proposes to retain its current definition of the term.

§60.21 Content of application.

Paragraph (c)(1)(ii)(C) now requires a license application to include certain evaluations of the performance of a proposed geologic repository for the period after permanent closure. The Commission proposes to add an additional sentence to this paragraph requiring that the results of these analyses be incorporated into an overall probability distribution of cumulative releases to the extent practicable. This reflects the language of EPA's definition of "performance assessment."

The Commission also proposes to add a new paragraph to §60.21 requiring submittal of a general description of the program for post-permanent closure monitoring of the geologic repository. (See the discussion (Section III) regarding the EPA assurance requirements - specifically 40 CFR 191.14(b).)

§60.51 License amendment for permanent closure.

Paragraph (a)(1) currently requires that an application to amend a license for permanent closure must include a description of the program for post-permanent closure monitoring of the geologic repository. The Commission proposes to revise this paragraph to specify in more detail the information to be submitted, including descriptions of the parameters to be monitored and the length of time for which the monitoring is to be continued. (See also the preceding discussion regarding 40 CFR 191.14(b).)

§60.52 Termination of license.

The Commission proposes to add a new condition for license termination which would explicitly require that the results available from post-permanent closure monitoring confirm the expectation that the repository will comply with the performance objectives of Part 60. (See also the preceding discussion regarding 40 CFR 191.14(b).)

§60.101 Purpose and nature of findings.

The EPA standards use the phrase "reasonable expectation" to describe the required level of confidence that compliance will be achieved with the provisions of the standards. The Supplementary Information accompanying the EPA standards contrasts the concept of "reasonable expectation" with the

reasonable assurance standard that is used by the Commission in dealing with other licensing actions. The Commission has considered adopting EPA's "reasonable expectation" concept, but has decided that doing so would result in a needless, and potentially confusing, proliferation of terms. Instead, the Commission proposes to expand the current discussion of "reasonable assurance" in §60.101 to make clear its belief that the level of confidence associated with the term, when used in connection with the long-term issues involved in repository licensing, is the same as that sought by EPA in its use of the term "reasonable expectation."

§60.111 Performance of the geologic repository operations area through permanent closure.

Paragraph (a) currently requires compliance with "such generally applicable environmental standards for radioactivity as may have been established by the Environmental Protection Agency." The Commission proposes to replace this wording with the specific dose limits promulgated by EPA in 40 CFR 191.03(a) of its standards. The proposed wording would apply the dose limits to any member of the public outside the geologic repository operations area, consistent with EPA's phrase "any member of the public in the general environment."

The EPA provision includes wording that requires reasonable assurance of compliance with the dose limits. In Part 60, Subpart B now specifies the findings that must be made by the Commission for issuance of a license, including a finding of reasonable assurance of compliance with the performance objective of §60.111. Because Part 60 already requires that findings be made with reasonable assurance, it is unnecessary to repeat such a requirement within this proposed performance objective.

One additional amendment, unrelated to the EPA standards, is being proposed for §60.111. The current wording of this section now requires that the geologic repository operations area be designed so that radiation exposures, radiation levels, and releases of radioactive materials "will at all times be maintained within the limits specified in Part 20 . . ." (emphasis added). The words "at all times" were intended to emphasize the need to design the geologic repository operations area so that any waste retrieval found to be necessary in the future could be carried out in

conformance with the radiation protection requirements of 10 CFR Part 20. In order to clarify the meaning of the phrase "at all times," the Commission is proposing to revise this wording to read "will at all times, including the retrievability period of §60.111(b), be maintained within the limits specified in Part 20"

§60.112 Overall system performance objective for the geologic repository after permanent closure.

The current wording of this section now refers to "such generally applicable environmental standards for radioactivity as may have been established by the Environmental Protection Agency." The Commission proposes to replace this wording with the specific provisions promulgated by EPA in 40 CFR 191.13, 191.15 and 191.16 of its standards, reworded as appropriate for incorporation into Part 60.

As discussed previously, the Commission proposes to revise the language of §60.101 to make clear that its concept of the phrase "reasonable assurance" in Part 60 closely parallels the meaning intended by "reasonable expectation" in the EPA standards. Inasmuch as the findings to be made by the Commission must be made with "reasonable assurance," there is no need to use the term "reasonable expectation" in the specific standards.

EPA requires that cumulative releases of radioactivity to the environment be evaluated on the basis of "performance assessments." This concept already is built into the structure of Part 60. As discussed previously, however, the Commission is proposing an addition to §60.21 which would specifically require a license application to incorporate the results of analyses, as stated by EPA, in an overall probability distribution of cumulative releases to the extent practicable.

The individual and groundwater protection requirements of the EPA standards refer to "undisturbed performance" of a disposal system, where "undisturbed performance" is defined to mean "the predicted behavior of a disposal system, including consideration of the uncertainties in predicted behavior, if the disposal system is not disrupted by human intrusion or the

occurrence of unlikely natural events." The Commission considers undisturbed performance, as defined by EPA, to be equivalent to performance in the absence of "unanticipated processes and events," as currently defined in Part 60. The Commission is proposing to use the current Part 60 terminology rather than introduce a new term from the EPA standards.

§60.113 Performance of particular barriers after permanent closure.

Section 60.113 specifies performance objectives for individual barriers of a geologic repository, and permits the Commission to approve or specify specific numerical requirements on a case-by-case basis. The Commission considers that §60.113 clearly requires use of both engineered and natural barriers. Nevertheless, in order to avoid any possible confusion regarding the provisions of §60.113(b), the Commission proposes to add additional clarifying language to this section making it clear that a repository must incorporate a system of multiple barriers, both engineered and natural. (See the preceding discussion in Section III regarding the EPA assurance requirements - specifically 40 CFR 191.14(d).)

Paragraph (b)(1) of §60.113 now refers to "any generally applicable environmental standard for radioactivity established by the Environmental Protection Agency." The Commission proposes to replace this wording with a direct reference to the overall system performance objectives of §60.112.

§60.114 Institutional control.

The Commission proposes to add a new §60.114 to Part 60 to clarify its views regarding reliance on institutional controls. (See the preceding discussion in Section III regarding 40 CFR 191.14(a).)

§60.115 Release limits for overall system performance objectives.

The Commission proposes that the table of release limits (and accompanying notes) in Appendix A of the EPA standards be added to Part 60 in a new §60.115.

§60.122 Siting criteria.

Part 60 contains provisions related to the presence of economically valuable mineral resources at a repository site. Part 60 does not, however,

address deposits of materials which, though of limited economic value, are not reasonably available from other sources. Because the economic value of materials could change in the future, the Commission proposes to add an additional potentially adverse condition to Part 60 related to significant concentrations of material that is not reasonably available from other sources.

EPA used the term "widely available." The Commission believes that an additional consideration - the practicality of obtaining materials from alternative sources - is also germane, and the Commission is therefore proposing the phrase "reasonably available" for this potentially adverse condition. (See also the preceding discussion in Section III regarding 40 CFR 191.14(e).)

§60.144 Monitoring after permanent closure.

Part 60 currently requires DOE to carry out a performance confirmation program which is to continue until repository closure. Part 60 does not now require monitoring after repository closure because of the likelihood that post-closure monitoring of the underground facility would degrade repository performance. The Commission proposes to add a new §60.144 to Part 60 which would require post-closure monitoring of repository characteristics provided that such monitoring can be expected to provide material confirmatory information regarding long-term repository performance and provided that the means for conducting such monitoring will not degrade repository performance. (See the preceding discussion in Section III regarding 40 CFR 191.14(b).)

Environmental Impact

Pursuant to Section 121(c) of the Nuclear Waste Policy Act of 1982, this proposed rule does not require the preparation of an environmental impact statement under section 102(2)(C) of the National Environmental Policy Act of 1969 or any environmental review under subparagraph (E) or (F) of section 102(2) of this Act.

Paperwork Reduction Act Statement

The information collection requirements contained in this proposed rule are of limited applicability and affect fewer than ten respondents. Therefore,

Office of Management and Budget clearance is not required pursuant to the Paperwork Reduction Act of 1980 (44 U.S.C. 3501 et seq.).

Regulatory Flexibility Act Certification

In accordance with the Regulatory Flexibility Act of 1980 (5 U.S.C. 605(b)), the Commission certifies that this rule, if adopted, will not have a significant economic impact on a substantial number of small entities. The only entity subject to regulation under this rule is the U.S. Department of Energy, which does not fall within the scope of the definition of "small entities" set forth in the Regulatory Flexibility Act.

List of Subjects in 10 CFR Part 60

High-level waste, Nuclear power plants and reactors, Nuclear materials, Penalty, Reporting and recordkeeping requirements, Waste treatment and disposal.

Backfitting Requirements

The provisions of 10 CFR 50.109 on backfitting do not apply to this rulemaking because the rule is not applicable to production and utilization facilities licensed under 10 CFR Part 50.

For the reasons set out in the preamble and under the authority of the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974, as amended, the Nuclear Waste Policy Act of 1982, and 5 U.S.C. 553, the NRC is proposing to adopt the following amendments to 10 CFR Part 60.

PART 60 -- DISPOSAL OF HIGH-LEVEL RADIOACTIVE

WASTES IN GEOLOGIC REPOSITORIES

1. The authority citation for Part 60 continues to read as follows:
Secs. 51, 53, 62, 63, 65, 81, 161, 182, 183, 68 Stat. 929, 930, 932, 933, 935,

948, 953, 954, as amended (42 U.S.C. 2071, 2073, 2092, 2093, 2095, 2111, 2201, 2232, 2233); secs. 202, 206, 88 Stat. 1244, 1246 (42 U.S.C. 5842, 5846); secs. 10 and 14, Pub. L. 95-601, 92 Stat. 2951 (42 U.S.C. 2021a and 5851); sec. 102, Pub. L. 91-190, 83 Stat. 853 (42 U.S.C. 4332); sec. 121, Pub. L. 97-425, 96 Stat. 2228 (42 U.S.C. 10141).

For the purposes of sec. 223, 68 Stat. 958, as amended (42 U.S.C. 2273), 60.71 to 60.75 are issued under sec. 161o, 68 Stat. 950, as amended (42 U.S.C. 2201(o)).

2. Section 60.2 is amended by revising the definitions of "accessible environment" and "controlled area" and by adding seven new definitions in alphabetical order as follows:

60.2 Definitions.

* * * * *

"Accessible environment" means: (1) the atmosphere, (2) land surfaces, (3) surface waters, (4) oceans, and (5) all of the lithosphere that is beyond the controlled area.

* * * * *

"Active institutional control" means: (1) controlling access to a disposal site by any means other than passive institutional control, (2) performing maintenance operations or remedial actions at a site, (3) controlling or cleaning up releases from a site, or (4) monitoring parameters related to disposal system performance.

* * * * *

"Community water system" means a system for the provision to the public of piped water for human consumption, if such system has at least 15 service connections used by year-round residents or regularly serves at least 25 year-round residents.

* * * * *

"Controlled area" means: (1) a surface location, to be identified by passive institutional controls, that encompasses no more than 100 square kilometers and extends horizontally no more than five kilometers in any

direction from the outer boundary of the underground facility, and (2) the subsurface underlying such a surface location.

* * * * *

"Passive institutional control" means: (1) permanent markers placed at a disposal site, (2) public records and archives, (3) government ownership and regulations regarding land or resource use, and (4) other methods of preserving knowledge about the location, design, and contents of a disposal system.

* * * * *

"Significant source of groundwater" means: (1) an aquifer that: (i) is saturated with water having less than 10,000 milligrams per liter of total dissolved solids; (ii) is within 2,500 feet of the land surface; (iii) has a transmissivity greater than 200 gallons per day per foot, provided that any formation or part of a formation included within the source of groundwater has a hydraulic conductivity greater than 2 gallons per day per square foot; and (iv) is capable of continuously yielding at least 10,000 gallons per day to a pumped or flowing well for a period of at least a year; or (2) an aquifer that provides the primary source of water for a community water system as of November 18, 1985.

* * * * *

"Special source of groundwater" means those Class I groundwaters identified in accordance with the Environmental Protection Agency's Ground-Water Protection Strategy published in August 1984 that: (1) are within the controlled area encompassing a disposal system or are less than five kilometers beyond the controlled area; (2) are supplying drinking water for thousands of persons as of the date that the Department chooses a location within that area for detailed characterization as a potential site for a disposal system (e.g., in accordance with Section 112(b)(1)(B) of the NWPA); and (3) are irreplaceable in that no reasonable alternative source of drinking water is available to that population.

* * * * *

"Transmissivity" means the hydraulic conductivity integrated over the saturated thickness of an underground formation. The transmissivity of a series of formations is the sum of the individual transmissivities of each formation comprising the series.

* * * * *

"Uranium fuel cycle" means the operations of milling of uranium ore, chemical conversion of uranium, isotopic enrichment of uranium, fabrication of uranium fuel, generation of electricity by a light-water-cooled nuclear power plant using uranium fuel, and reprocessing of spent uranium fuel, to the extent that these directly support the production of electrical power for public use utilizing nuclear energy, but excludes mining operations, operations at waste disposal sites, transportation of any radioactive material in support of these operations, and the reuse of recovered non-uranium special nuclear and by-product materials from the cycle.

* * * * *

3. Section 60.21 is amended by revising paragraph (c)(1)(ii)(C), adding a new paragraph (c)(9) and redesignating the existing paragraphs (c)(9) through (c)(15) as paragraphs (c)(10) through (c)(16).

§60.21 Content of application.

* * * * *

- (c) * * *
- (1) * * *
- (ii) * * *

(C) An evaluation of the performance of the proposed geologic repository for the period after permanent closure, assuming anticipated processes and events, giving the rates and quantities of releases of radionuclides to the accessible environment as a function of time; and a similar evaluation which assumes the occurrence of unanticipated processes and events. In making such evaluations, estimated values shall be incorporated into an overall probability distribution of cumulative release to the extent practicable.

* * * * *

(9) A general description of the program for post-permanent closure monitoring of the geologic repository.

* * * * *

4. Section 60.51 is amended by revising paragraph (a)(1) to read as follows:

§60.51 License amendment for permanent closure.

(a) * * *

(1) A detailed description of the program for post-permanent closure monitoring of the geologic repository in accordance with §60.144. As a minimum, this description shall:

(i) identify those parameters that will be monitored;

(ii) indicate how each parameter will be used to evaluate the expected performance of the repository; and

(iii) discuss the length of time over which each parameter should be monitored to adequately confirm the expected performance of the repository.

* * * * *

5. Section 60.52 is amended by designating current paragraph (c)(3) as paragraph (c)(4) and by adding a new paragraph (c)(3) as follows:

§60.52 Termination of license.

* * * * *

(c) * * *

(3) That the results available from the post-permanent closure monitoring program confirm the expectation that the repository will comply with the performance objectives set out at §60.112 and §60.113; and

* * * * *

6. Section 60.101 is amended by revising paragraph (a)(2) to read as follows:

§60.101 Purpose and nature of findings.

(a) * * *

(2) While these performance objectives and criteria are generally stated in unqualified terms, it is not expected that complete assurance that they will be met can be presented. A reasonable assurance, on the basis of the record before the Commission, that the objectives and criteria will be met is the general standard that is required. For §60.112, and other portions of this subpart that impose objectives and criteria for repository performance over long times into the future, there will inevitably be greater uncertainties. Proof of the future performance of engineered barrier systems and the geologic setting over time periods of many hundreds or many thousands of years is not to be had in the ordinary sense of the word. For such long-term objectives and criteria, what is required is reasonable assurance, making allowance for the time period, hazards, and uncertainties involved, that the outcome will be in conformance with those objectives and criteria. Demonstration of compliance with such objectives and criteria will involve the use of data from accelerated tests and predictive models that are supported by such measures as field and laboratory tests, monitoring data and natural analog studies. Demonstration of compliance with the performance objectives of §60.112 will also involve predicting the likelihood and consequences of events and processes that may disturb the repository. Such predictions may involve complex computational models, analytical theories and prevalent expert judgment. Substantial uncertainties are likely to be encountered and sole reliance on numerical predictions to determine compliance may not be appropriate. In reaching a determination of reasonable assurance, the Commission may supplement numerical analyses with qualitative judgments including, for example, consideration of the degree of diversity or redundancy among the multiple barriers of a specific repository.

* * * * *

7. In section 60.111, paragraph (a) is revised to read as follows:

§60.111 Performance of the geologic repository operations area through permanent closure.

(a) Protection against radiation exposures and releases of radioactive material. The geologic repository operations area shall be designed so that until permanent closure has been completed:

(1) The annual dose equivalent to any member of the public outside the geologic repository operations area, resulting from the combination of (i) discharges of radioactive material and direct radiation from activities at the geologic repository operations area and (ii) uranium fuel cycle operations, shall not exceed 25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other critical organ.

(2) Radiation exposures and radiation levels, and releases of radioactive materials to unrestricted areas, will at all times, including the retrievability period of §60.111(b), be maintained within the limits specified in Part 20 of this chapter.

* * * * *

8. Section 60.112 is revised to read as follows:

§60.112 Overall system performance objective for the geologic repository after permanent closure.

The geologic setting shall be selected and the engineered barrier system and the shafts, boreholes and their seals shall be designed:

(a) So that, for 10,000 years following permanent closure, cumulative releases of radionuclides to the accessible environment, from all anticipated and unanticipated processes and events, shall:

(1) Have a likelihood of less than one chance in 10 of exceeding the quantities calculated in accordance with §60.115.

(2) Have a likelihood of less than one chance in 1,000 of exceeding ten times the quantities calculated in accordance with §60.115.

(b) So that for 1,000 years after permanent closure, and in the absence of unanticipated processes and events, the annual dose equivalent to any member of the public in the accessible environment does not exceed 25 millirems to the whole body or 75 millirems to any critical organ. For the purpose of applying this paragraph, all potential pathways from the geologic repository to people shall be considered, including the assumption that individuals consume 2 liters per day of drinking water from any significant source of groundwater outside of the controlled area.

(c) So that for 1,000 years after permanent closure, and in the absence of unanticipated processes and events:

(1) Except as provided in paragraph (c)(2) of this section, the radionuclide concentrations averaged over any year in water withdrawn from any portion of a special source of groundwater do not exceed:

(i) 5 picocuries per liter of radium-226 and radium-228;

(ii) 15 picocuries per liter of alpha-emitting radionuclides (including radium-226 and radium-228 but excluding radon); or

(iii) the combined concentrations of radionuclides that emit either beta or gamma radiation that would produce an annual dose equivalent to the total body or any internal organ greater than 4 millirems per year if an individual consumed 2 liters per day of drinking water from such a source of groundwater.

(2) If any of the average annual radionuclide concentrations existing in a special source of groundwater before construction of the geologic repository operations area already exceed the limits in paragraph (c)(1) of this section, the increase, caused by the geologic repository, in the existing average annual radionuclide concentrations in water withdrawn from that special source of groundwater does not exceed the limits specified in paragraph (c)(1) of this section.

9. In section 60.113, paragraph (b)(1) is revised and a new paragraph (d) is added to read as follows:

§60.113 Performance of particular barriers after permanent closure.

* * * * *

(b) * * *

(1) The overall system performance objectives of §60.112.

* * * * *

(d) Notwithstanding the provisions of paragraph (b) of this section, the geologic repository shall incorporate a system of multiple barriers, both engineered and natural.

10. A new §60.114 is added to read as follows:

§60.114 Institutional control.

Neither active nor passive institutional control shall be deemed to assure compliance with the overall system performance objectives set out at §60.112 for more than 100 years after permanent closure. However, the effects of institutional control may be considered in assessing, for purposes of that section, the likelihood and consequences of processes and events affecting the geologic setting.

11. A new §60.115 is added to read as follows:

§60.115 Release limits for overall system performance objective.

The following table shall be used to make the calculations referred to in paragraph (a) of §60.112.

TABLE 1 --RELEASE LIMITS FOR OVERALL SYSTEM PERFORMANCE OBJECTIVE
 (Cumulative Releases to the Accessible Environment
 for 10,000 Years After Disposal)

Radionuclide	Release Limit per 1000 MTHM or other unit of waste (see Notes) (curies)
Americium-241 or 243 - - - - -	100
Carbon-14 - - - - -	100
Cesium-135 or 137 - - - - -	1000
Iodine-129 - - - - -	100
Neptunium-237 - - - - -	100
Plutonium-238, 239, 240 or 242 - - - - -	100
Radium-226 - - - - -	100
Strontium-90 - - - - -	1000
Technetium-99 - - - - -	10000
Thorium-230 or 232 - - - - -	10
Tin-126 - - - - -	1000
Uranium-233, 234, 235, 236 or 238 - - - - -	100
Any other alpha-emitting radionuclide with a half-life greater than 20 years - - - - -	100
Any other radionuclide with a half-life greater than 20 years that does not emit alpha particles - - - - -	1000

Application of Table 1

NOTE 1: Units of Waste. The Release Limits in Table 1 apply to the amount of wastes in any one of the following:

(a) an amount of spent nuclear fuel containing 1,000 metric tons of heavy metal (MTHM) exposed to a burnup between 25,000 megawatt-days per metric ton of heavy metal (Mwd/MTHM) and 40,000 Mwd/MTHM;

(b) the high-level radioactive wastes generated from reprocessing each 1,000 MTHM exposed to a burnup between 25,000 Mwd/MTHM and 40,000 Mwd/MTHM;

(c) each 100,000,000 curies of gamma or beta-emitting radionuclides with half-lives greater than 20 years but less than 100 years (for use as discussed in Note 5 or with materials that are identified by the Commission as high-level radioactive waste in accordance with part (B) of the definition of high-level waste in the Nuclear Waste Policy Act (NWPA));

(d) each 1,000,000 curies of other radionuclides (i.e., gamma or beta-emitters with half-lives greater than 100 years or any alpha-emitters with half-lives greater than 20 years) (for use as discussed in Note 5 or with materials that are identified by the Commission as high-level waste in accordance with part (B) of the definition of high-level waste in the NWPA); or

(e) an amount of transuranic (TRU) wastes containing one million curies of alpha-emitting transuranic radionuclides with half-lives greater than 20 years.

NOTE 2: Release Limits for Specific Disposal Systems. To develop Release Limits for a particular disposal system, the quantities in Table 1 shall be adjusted for the amount of waste included in the disposal system compared to the various units of waste defined in Note 1. For example:

(a) If a particular disposal system contained the high-level wastes from 50,000 MTHM, the Release Limits for that system would be the quantities in Table 1 multiplied by 50 (50,000 MTHM divided by 1,000 MTHM).

(b) If a particular disposal system contained three million curies of alpha-emitting transuranic wastes, the Release Limits for that system would be

the quantities in Table 1 multiplied by three (three million curies divided by one million curies).

(c) If a particular disposal system contained both the high-level wastes from 50,000 MTHM and 5 million curies of alpha-emitting transuranic wastes, the Release Limits for that system would be the quantities in Table 1 multiplied by 55:

$$\frac{50,000 \text{ MTHM}}{1,000 \text{ MTHM}} + \frac{5,000,000 \text{ curies TRU}}{1,000,000 \text{ curies TRU}} = 55$$

NOTE 3: Adjustments for Reactor Fuels with Different Burnup. For disposal systems containing reactor fuels (or the high-level wastes from reactor fuels) exposed to an average burnup of less than 25,000 MWd/MTHM or greater than 40,000 MWd/MTHM, the units of waste defined in (a) and (b) of Note 1 shall be adjusted. The unit shall be multiplied by the ratio of 30,000 MWd/MTHM divided by the fuel's actual average burnup, except that a value of 5,000 MWd/MTHM may be used when the average fuel burnup is below 5,000 MWd/MTHM and a value of 100,000 MWd/MTHM shall be used when the average fuel burnup is above 100,000 MWd/MTHM. This adjusted unit of waste shall then be used in determining the Release Limits for the disposal system.

For example, if a particular disposal system contained only high-level wastes with an average burnup of 3,000 MWd/MTHM, the unit of waste for that disposal system would be:

$$1,000 \text{ MTHM} \times \frac{(30,000 \text{ MWd/MTHM})}{(5,000 \text{ MWd/MTHM})} = 6,000 \text{ MTHM}$$

If that disposal system contained the high-level wastes from 60,000 MTHM (with an average burnup of 3,000 MWd/MTHM), then the Release Limits for that system would be the quantities in Table 1 multiplied by ten:

$$\frac{60,000 \text{ MTHM}}{6,000 \text{ MTHM}} = 10$$

which is the same as:

$$\frac{60,000 \text{ MTHM}}{1,000 \text{ MTHM}} \times \frac{(5,000 \text{ Mwd/MTHM})}{(30,000 \text{ Mwd/MTHM})} = 10$$

NOTE 4: Treatment of Fractionated High-Level Wastes. In some cases, a high-level waste stream from reprocessing spent nuclear fuel may have been (or will be) separated into two or more high-level waste components destined for different disposal systems. In such cases, the implementing agency may allocate the Release Limit multiplier (based upon the original MTHM and the average fuel burnup of the high-level waste stream) among the various disposal systems as it chooses, provided that the total Release Limit multiplier used for that waste stream at all of its disposal systems may not exceed the Release Limit multiplier that would be used if the entire waste stream were disposed of in one disposal system.

NOTE 5: Treatment of Wastes with Poorly Known Burnups or Original MTHM. In some cases, the records associated with particular high-level waste streams may not be adequate to accurately determine the original metric tons of heavy metal in the reactor fuel that created the waste, or to determine the average burnup that the fuel was exposed to. If the uncertainties are such that the original amount of heavy metal or the average fuel burnup for particular high-level waste streams cannot be quantified, the units of waste derived from (a) and (b) of Note 1 shall no longer be used. Instead, the units of waste defined in (c) and (d) of Note 1 shall be used for such high-level waste streams. If the uncertainties in such information allow a range of values to be associated with the original amount of heavy metal or the average fuel burnup, then the calculations described in previous Notes will be conducted using the values that result in the smallest Release Limits, except that the Release Limits need not be smaller than those that would be calculated using the units of waste defined in (c) and (d) of Note 1.

NOTE 6: Use of Release Limits to Determine Compliance with §60.112(a).

Once release limits for a particular system have been determined in accordance with Notes 1 through 5, these release limits shall be used to determine compliance with the requirements of §60.112(a) as follows. In cases where a mixture of radionuclides is projected to be released to the accessible environment, the limiting values shall be determined as follows: For each radionuclide in the mixture, determine the ratio between the cumulative release quantity projected over 10,000 years and the limit for that radionuclide as determined from Table 1 and Notes 1 through 5. The sum of such ratios for all the radionuclides in the mixture may not exceed one with regard to §60.112(a)(1) and may not exceed ten with regard to §60.112(a)(2).

For example, if radionuclides A, B, and C are projected to be released in amounts Q_a , Q_b , and Q_c , and if the applicable Release Limits are RL_a , RL_b , and RL_c , then the cumulative releases over 10,000 years shall be limited so that the following relationship exists:

$$\frac{Q_a}{RL_a} + \frac{Q_b}{RL_b} + \frac{Q_c}{RL_c} \leq 1$$

12. In section 60.122, paragraph (c) is amended by redesignating the current paragraphs (c)(18) through (c)(21) as paragraphs (c)(19) through (c)(22) and by adding a new paragraph (c)(18) to read as follows:

§60.122 Siting criteria.

* * * * *

(c) * * *

(18) The presence of significant concentrations of any naturally-occurring material that is not reasonably available from other sources.

* * * * *

13. A new §60.144 is added to read as follows:

§60.144 Monitoring After Permanent Closure.

A program of monitoring shall be conducted after permanent closure to monitor all repository characteristics which can reasonably be expected to provide material confirmatory information regarding long-term repository performance, provided that the means for conducting such monitoring will not degrade repository performance. This program shall be continued until termination of a license.

Dated at Washington, D.C. this ____ day of _____ 1986.

For the Nuclear Regulatory Commission.

Samuel J. Chilk,
Secretary of the Commission

ENCLOSURE B

REGISTRATION
FEDERAL REGISTER

Thursday
September 19, 1985

Part II

**Environmental
Protection Agency**

40 CFR Part 191.

**Environmental Standards for the
Management and Disposal of Spent
Nuclear Fuel, High-Level and Transuranic
Radioactive Wastes; Final Rule**

ENCLOSURE 3

ENVIRONMENTAL PROTECTION AGENCY**40 CFR Part 191**

(AH-FRL 2870-3)

Environmental Standards for the Management and Disposal of Spent Nuclear Fuel, High-Level and Transuranic Radioactive Wastes**AGENCY:** Environmental Protection Agency.**ACTION:** Final rule.

SUMMARY: The Environmental Protection Agency (EPA) is promulgating generally applicable environmental standards for the management and disposal of spent nuclear fuel and high-level and transuranic radioactive wastes. The standards apply to management and disposal of such materials generated by activities regulated by the Nuclear Regulatory Commission (NRC) and to disposal of similar materials generated by atomic energy defense activities under the jurisdiction of the Department of Energy (DOE). These standards have been developed pursuant to the Agency's authorities and responsibilities under the Atomic Energy Act of 1954, as amended; Reorganization Plan No. 3 of 1970; and the Nuclear Waste Policy Act of 1982.

Subpart A of these standards limits the radiation exposure of members of the public from the management and storage of spent fuel or high-level or transuranic wastes prior to disposal at waste management and disposal facilities regulated by the NRC. Subpart A also limits the radiation exposures to members of the public from waste emplacement and storage operations at DOE disposal facilities that are not regulated by the NRC.

Subpart B establishes several different types of requirements for disposal of these materials. The primary standards for disposal are long-term containment requirements that limit projected releases of radioactivity to the accessible environment for 10,000 years after disposal. These release limits should insure that risks to future generations from disposal of these wastes will be no greater than the risks that would have existed if the uranium ore used to create the wastes had not been mined to begin with. A set of six qualitative assurance requirements is an equally important element of Subpart B designed to provide adequate confidence that the containment requirements will be met. The third set of requirements are limitations on exposures to individual members of the public for 1,000 years after disposal.

Finally, a set of ground water protection requirements limits radionuclide concentrations for 1,000 years after disposal in water withdrawn from most Class I ground waters to the concentrations allowed by the Agency's interim drinking water standards (unless concentrations in the Class I ground waters already exceed the limits in 40 CFR Part 141, in which case this set of requirements would limit the increases in the radionuclide concentrations to those specified in 40 CFR Part 141). Subpart B also contains informational guidance for implementation of the disposal standards to clarify the Agency's intended application of these standards, which address a time frame without precedent in environmental regulations. Although disposal of these materials in mined geologic repositories has received the most attention, the disposal standards apply to disposal by any method, except disposal directly into the oceans or ocean sediments.

This notice describes the final rule that the Agency developed after considering the public comments received on the proposed rule published on December 29, 1982, and the recommendations of a technical review conducted by the Agency's Science Advisory Board (SAB). The major comments received on the proposed standards are summarized together with the Agency's responses to them. Detailed responses to all the comments received are discussed in the Response to Comments Document prepared for this final rule.

DATE: These standards shall be promulgated for purposes of judicial review at 1:00 p.m. eastern time on October 3, 1985. These standards shall become effective on November 18, 1985.

ADDRESSES: Background Information—The technical information considered in developing this rule, including risk assessments of disposal of these wastes in mined geologic repositories, is summarized in the Background Information Document (BID) for 40 CFR Part 191, EPA 520/1-85-023. Single copies of both the BID and the Response to Comments Document, as available, may be obtained from the Program Management Office (ANR-458), Office of Radiation Programs, Environmental Protection Agency, Washington, DC 20460; telephone number (703) 557-8331.

Docket—Docket Number R-82-3 contains the rulemaking record for 40 CFR Part 191. The docket is available for inspection between 8 a.m. and 4 p.m. on weekdays in the West Tower Lobby, Gallery 1, Central Docket Section, 401 M Street, SW., Washington, DC. A

reasonable fee may be charged for copying.

FOR FURTHER INFORMATION CONTACT: Dan Egan or Ray Clark, Criteria and Standards Division (ANR-460), Office of Radiation Programs, Environmental Protection Agency, Washington, DC 20460; telephone number (703) 557-8810.

SUPPLEMENTARY INFORMATION:

Fissioning of nuclear fuel in nuclear reactors creates a small quantity of highly radioactive materials. Virtually all of these materials are retained in the "spent" fuel elements when they are removed from the reactor. If the fuel is then reprocessed to recover unfissioned uranium and plutonium, most of the radioactivity goes into acidic liquid wastes that will later be converted into various types of solid materials. These highly radioactive liquid or solid wastes from reprocessing spent nuclear fuel have traditionally been called "high-level wastes." If it is not to be reprocessed, the spent fuel itself becomes a waste. The nuclear reactors operated by the nation's electrical utilities currently generate about 2,000 metric tons of spent fuel per year. The relatively small physical quantity of these wastes is apparent when compared to the chemically hazardous wastes regulated under the Resource Conservation and Recovery Act, which are produced at a rate of about 150,000,000 metric tons per year.

Although they are produced in small quantities, proper management and disposal of high-level wastes and spent nuclear fuel are essential because of the inherent hazard of the large amounts of radioactivity they contain. Spent fuel from commercial nuclear power reactors contains about 1.6 billion curies of radionuclides with half-lives greater than 20 years. Over the next decade, this inventory is projected to grow at a rate of about 300 million curies per year from reactors currently licensed to operate. Most of this spent fuel is currently stored at reactor sites. Reprocessing reactor fuel used for national defense activities has produced about 700 million curies of radionuclides with half-lives greater than 20 years. Most of these wastes are stored in various liquid and solid forms on three Federal reservations in Idaho, Washington, and South Carolina.

In addition, a wide variety of wastes contaminated with man-made radionuclides heavier than uranium have been created by various processes, mostly from the atomic energy defense activities conducted by the DOE and its predecessor agencies (the Atomic Energy Commission and the Energy

Research and Development Administration). These wastes are usually called "transuranic" wastes. Most of them are stored at Federal reservations in Idaho, Washington, New Mexico, and South Carolina.

National Programs for Disposal of These Wastes

Since the inception of the nuclear age in the 1940's, the Federal government has assumed ultimate responsibility for the care and disposal of these wastes regardless of whether they are produced by commercial or national defense activities. In October 1978, President Ford ordered a major expansion of the Federal program to demonstrate a permanent disposal method for high-level wastes. The Agency was directed to develop generally applicable environmental standards to govern the management and disposal of these wastes as part of this initiative. Among EPA's first activities in response to this directive were a series of public workshops conducted in 1977 and 1978 to better understand the various public concerns and technical issues associated with radioactive waste disposal.

In 1981, the DOE, after completing a comprehensive programmatic environmental impact statement, decided to focus the national program on disposal in mined geologic repositories (46 FR 26677). In 1982, Congress passed the Nuclear Waste Policy Act (henceforth designated "NWSA"), which President Reagan signed into law on January 7, 1983. The NWSA contains several provisions that are relevant to this rulemaking. First, it affirmed the DOE's 1981 decision that mined repositories should receive primary emphasis in the national program, although research on some other technologies would be continued. Second, it established formal procedures regarding the evaluation and selection of sites for geologic repositories, including steps for the interaction of affected States and Indian tribes with the Federal Government regarding site selection decisions. Third, the NWSA levied a fee on utilities that generate electrical power with nuclear reactors in order to pay for Federal management and disposal of their spent fuel or high-level wastes. Fourth, the NWSA reiterated the existing responsibilities of the Federal agencies involved in the national program to develop mined geologic repositories, and it assigned some additional tasks regarding site evaluation. Finally, the Act provided a timetable for several key milestones that the Federal agencies were to meet in carrying out the program.

Section 121 of the NWSA reiterated the Agency's responsibility for developing the overall framework of requirements needed to assure protection of public health and the environment, in accordance with the Agency's authorities under the Atomic Energy Act of 1954 and Reorganization Plan Number 3 of 1970. Section 121 also called for the Agency to promulgate these standards by January 7, 1984. The Agency did not meet this deadline. On February 8, 1985, the Natural Resources Defense Council and four other environmental interest groups filed suit to bring about compliance with the NWSA mandate. This litigation was settled by the Agency and the plaintiffs agreeing to a consent order requiring promulgation not later than August 15, 1985. The generally applicable environmental standards promulgated by this notice satisfy the terms of this consent order. However, they also represent the culmination of an effort that began almost nine years ago and that has included frequent interactions with the public to help formulate standards responsive to the concerns about disposal of these dangerous materials.

Objective and Implementation of the Standards

In developing the standards for disposal of spent nuclear fuel and high-level and transuranic radioactive wastes, the Agency has carefully evaluated the capabilities of mined geologic repositories to isolate the wastes from the environment. Because such repositories are capable of performing so well, it has been possible to choose containment requirements that will provide exceptionally good protection to current and future populations for at least 10,000 years after disposal. In fact, EPA's analyses indicate that the small residual risks allowed by the disposal standards would be comparable to the risks that future populations would have been exposed to if the uranium ore used to produce the high-level wastes had not been mined to begin with.¹ The Agency

¹Specifically, the Agency estimates that compliance with the disposal standards would allow no more than 1,000 premature deaths from cancer in the first 10,000 years after disposal of the high-level wastes from 100,000 metric tons of reactor fuel, an average of no more than one premature death every ten years. As this residual risk level is referred to in the following discussion, it should be remembered that it is a speculative calculation that is primarily intended as a tool for comparing risk levels; it should not be considered a reliable projection of the "real" number of health effects resulting from compliance with the disposal standards.

believes that achieving this protection should not significantly increase the cost or difficulty of carrying out the national program for disposing of the wastes from commercial nuclear power plants. In addition, the containment requirements in the final rule are complemented by six qualitative assurance requirements designed to provide confidence that the containment requirements will be met, given the substantial uncertainties inherent in predictions of systems performance over 10,000 years. Because of this comprehensive framework, the Agency is confident that the national program to dispose of these wastes will be carried out with exceptional protection of public health and the environment.

The Nuclear Regulatory Commission (NRC) and the DOE are responsible for implementing these standards. The NRC has already promulgated procedural and technical requirements in 10 CFR Part 60 for disposal of high-level wastes in mined geologic repositories (46 FR 13971, 48 FR 28194). The NRC will obtain compliance with 40 CFR Part 191 for disposal of all high-level wastes by issuing licenses to the DOE, in accordance with 10 CFR Part 60, at various steps in the construction and operation of a repository. The NWSA directs the DOE to select a number of potential sites for geologic repositories, successively reducing this set of alternatives from five to three to one, in consultation with affected States and Indian Tribes and with participation by the public in key steps in the selection process. The DOE will accomplish this through use of site selection guidelines (10 CFR Part 960) that it has developed in accordance with section 112 of the NWSA. Both NRC's 10 CFR Part 60 and DOE's 10 CFR Part 960 incorporate the standards the Agency is promulgating today as the overall performance requirements for a geologic repository. Both of these other rules were designed in concert with EPA's ongoing development of 40 CFR Part 191. However, both the NRC and DOE must now review these regulations to determine what specific changes will be needed to properly implement the final version of 40 CFR Part 191.

Review of the Proposed Standards

On December 29, 1982, shortly before the NWSA was enacted, the Agency published 40 CFR Part 191 for public review (47 FR 58198) and asked that comments be received by May 2, 1983. Eighty-three substantive replies were received from a broad spectrum of private citizens, public interest groups, members of the scientific community,

representatives of industry, and State and Federal agencies. These responses contained information and recommendations regarding seven issues on which the Agency sought further public comment (48 FR 21666). Questions concerning these issues were directed to all of the witnesses at two public hearings held during May 1983 in Washington, D.C. and in Denver (48 FR 13444). Copies of these questions were also sent to all those who responded to the initial request for comment, and the availability of these questions was announced in the Federal Register (48 FR 21666). The comment period was then held open until June 20, 1983, to receive responses to these additional questions. Responses to major comments—including all those specifically highlighted for public review—are summarized below. Detailed responses to the full range of comments received is described in the Response to Comments Document prepared for the final rule.

Review of the Technical Basis of the Standards

In parallel with this public review and comment, the Agency conducted an independent scientific review of the technical basis for the proposed 40 CFR Part 191 through a special Subcommittee of the Agency's Science Advisory Board (SAB) (48 FR 509). This Subcommittee held nine public meetings from January 18, 1983, through September 21, 1983, and prepared a final report that was transmitted on February 17, 1984. While finding that the Agency had generally prepared comprehensive and scientifically competent technical analyses to support the proposed standards, the SAB review developed 46 findings and recommendations regarding specific improvements in the technical analyses and in the standards themselves. Since many of the SAB recommendations were to be considered in developing the final rule, the Agency sought public comment on the information and recommendations presented in the final SAB report (49 FR 19604).

Most of the SAB recommendations involve specific details of the technical assessments and judgments the Agency made in developing these standards. After evaluating the public comments received on the SAB report, the Agency agrees with almost all of the SAB's technical recommendations and has made corresponding changes in the technical basis of the final rule. A few of the Subcommittee's recommendations have implications that involve broader policy judgments. These recommendations have been treated as

part of the public comment record and are described below as the major comments on the proposed 40 CFR Part 191 are discussed. A complete itemization of the Agency's responses to each of the findings and recommendations of the SAB is contained in the Response to Comments Document, together with a synopsis of the public comments on the SAB report.

Summary of the Final Rule

The rule being promulgated today establishes generally applicable environmental standards for the management and disposal of spent nuclear fuel, high-level radioactive wastes, and transuranic radioactive wastes. The final rule differs in a number of ways from the proposed rule because of changes the Agency has made in response to public comments and in response to the recommendations of the technical review by the Agency's Science Advisory Board. This section provides an overview of the major provisions of the final rule, and changes from the proposed rule are noted. More detail on many of these provisions is provided later as part of the discussion of the comments considered in development of 40 CFR Part 191. The final rule:

(1) Applies to management and disposal of spent nuclear fuel, high-level radioactive wastes as defined by the NWSA, and transuranic wastes containing more than 100 nanocuries per gram of alpha-emitting transuranic isotopes, except for wastes that either the NRC or the Administrator determines do not need the degree of isolation required by this rule. (The proposed rule applied to spent nuclear fuel, high-level wastes exceeding a specific set of concentration limits, and to transuranic wastes containing more than 100 nanocuries per gram.)

(2) Through Subpart A, "Standards for Management and Storage," establishes limits on annual doses to members of the public of 25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other organ from exposures associated with management, storage, and preparation for disposal of any of these materials at facilities regulated by the NRC. These limits apply to the combined exposures from all NRC-licensed facilities covered by this Part or 40 CFR Part 190, the Agency's standards for the commercial uranium fuel cycle. Subpart A also limits annual doses to members of the public from management and storage operations at DOE disposal facilities that are not regulated by the NRC to 25 millirems to the whole body and 75 millirems to any other organ. (The

proposed rule applied to the combined exposures from operations regulated by 40 CFR Part 190, waste management and storage operations regulated by the NRC or Agreement States, and waste management and storage operations conducted at all DOE facilities.) Subpart A also contains a provision that allows the Administrator to issue alternative standards for waste management and storage operations at DOE disposal facilities that are not regulated by the NRC. (The proposed rule contained a provision to allow the implementing agency, either the NRC or the DOE, to grant variances for unusual operating conditions.)

(3) Establishes several sets of requirements for disposal of these wastes through Subpart B, "Standards for Disposal." The primary standards are *containment requirements* that limit projected releases of radioactivity to the accessible environment for 10,000 years after disposal. Equally important is a set of six *assurance requirements* chosen to provide adequate confidence that the containment requirements will be met. In addition, Subpart B of the final rule includes *individual protection requirements* that limit annual exposures from the disposal facility to members of the public in the accessible environment to 25 millirems to the whole body and 75 millirems to any organ for 1,000 years after disposal. The Subpart also contains *ground water protection requirements* that limit radioactivity concentrations in water withdrawn from most Class I ground waters near a disposal system (as defined in conjunction with the Agency's Ground Water Protection Strategy published in August 1984) for 1,000 years after disposal. Finally, Subpart B provides *guidance for implementation* that indicates how the Agency intends the various numerical standards to be applied. (The proposed rule contained only containment requirements, assurance requirements, and procedural requirements; this last category provided some of the basis for the "guidance for implementation" in the final rule.) Major provisions of each of these sets of requirements include the following:

(a) The containment requirements (Section 191.13) limit the total projected release of specific radionuclides over the entire 10,000-year period after disposal. Releases from all expected and accidental causes are included, except for releases from conceivable events that are judged to have an incredibly small likelihood of occurrence. Quantitative terms are used to identify the probabilities of the releases to which

the containment requirements apply; however, the final rule acknowledges that determination of compliance will have to tolerate much larger uncertainties than would be appropriate for short-term estimates and that judgments may have to be substituted for quantitative predictions in certain situations. Disposal in compliance with the containment requirements is projected to cause no more than 1,000 premature cancer deaths over the entire 10,000-year period from disposal of all existing high-level wastes and most of the wastes yet to be produced by currently operating reactors—an average of 0.1 fatality per year. This level of residual risk to future generations would be comparable to the risks that those generations would have faced from the uranium ore used to create the wastes if the ore had never been mined. Actual risks will probably be significantly less because of the conservative approach called for by the other parts of Subpart B. (The quantitative probabilities in the proposed rule were an order of magnitude smaller than those incorporated into the final rule. The release limits in the final rule are different than those in the proposed rule due to changes in EPA's technical analyses that were recommended by the SAB Subcommittee; however, the level of residual risk is the same as for the proposed rule.)

(b) The assurance requirements (Section 191.14) call for cautious steps to be taken in disposing of these wastes because of the inherent uncertainties in selecting and designing disposal systems that must be very effective for more than 10,000 years. The assurance requirements incorporate the following principles:

(i) Although active institutional controls, such as guarding and maintaining a disposal site, should be encouraged, they cannot be relied upon to isolate these wastes from the environment for more than 100 years after disposal. (The proposed rule limited reliance to "a few hundred years" after disposal.)

(ii) Disposal systems must be monitored to detect substantial changes from their expected performance until the implementing agency determines that there are no significant concerns to be addressed by further monitoring. (This requirement was not included in the proposed rule.)

(iii) The sites where disposal systems are located must be identified by permanent markers, widespread records, and other passive institutional controls to warn future generations of the dangers and location of the wastes.

(iv) Disposal systems must use several different types of barriers, including both engineered and natural ones, to isolate the wastes from the environment to help guard against unexpectedly poor performance from one type of barrier.

(v) Sites for disposal systems should be selected to avoid places where resources have previously been mined, where there is a reasonable expectation of exploration for scarce or easily accessible resources, or where there is a significant concentration of any material which is not otherwise available. (The wording in the proposed rule would have ruled out sites with a significant possibility of being considered for resource exploration in the future. The final rule revises this requirement to allow use of sites with some resource potential if they have other significant advantages compared to potential alternative sites.)

(vi) Recovery of most of the wastes must not be precluded for a reasonable period after disposal if unforeseen events require this in the future.

(c) The individual protection requirements (Section 191.15) limit annual exposures to members of the public in the accessible environment from the disposal system to 25 millirems to the whole body and 75 millirems to any organ. These requirements apply to, undisturbed performance of the disposal system for 1,000 years after disposal. All potential pathways of radiation exposure from the disposal system to people must be considered, including the assumption that individuals consume all of their drinking water (2 liters per day) from any "significant source of ground water" located outside the "controlled area" established around a disposal system. A "significant source" is identified by several parameters intended to describe an aquifer sufficient to meet the needs of a "community water system" as defined in the Agency's National Interim Primary Drinking Water Regulations (40 CFR Part 141). (No explicit individual protection requirements were included in the proposed rule.)

(d) The ground water protection requirements (Section 191.16) limit the concentrations of radioactivity (or the increases in concentrations, if preexisting concentrations already exceed these limits) in waters withdrawn from most Class I sources of ground water near a disposal system to no more than 15 picocuries per liter of alpha-emitting radionuclides (including no more than 5 picocuries per liter of radium-226 and radium-228 but excluding radon) and to no more than the combined concentrations of radionuclides that emit either beta or

gamma radiation that would produce an annual dose equivalent to the total body or any internal organ greater than 4 millirems if individuals consumed all of their drinking water from that source of ground water. These concentration limits are similar to those set in 40 CFR Part 141 for community water systems. Like the individual protection requirements, the ground water protection requirements apply to undisturbed behavior of the disposal system for a period of 1,000 years after disposal. (No explicit ground water protection requirements were included in the proposed rule.)

(e) Section 191.17 of the final rule establishes minimum procedural requirements that the Administrator must follow if additional information considered in the future indicates that it would be appropriate to modify any portion of the disposal standards through further rulemaking. (No similar provision was included in the proposed rule.)

(f) The "guidance for implementation" included as Appendix B to the final rule describes certain analytical approaches and assumptions through which the Agency intends the various long-term numerical standards of Subpart B to be applied. This guidance is particularly important because there are no precedents for the implementation of such long-term environmental standards, which will require consideration of extensive analytical projections of disposal system performance. (The proposed rule contained a corresponding, but less extensive, section entitled "procedural requirements.")

Overall Approach of the Final Rule

In general, the Agency developed the various elements of this rule by balancing several perspectives. One set of considerations was the expected capabilities of the waste management and disposal technologies to reduce both short- and long-term risks to public health and the environment. These capabilities were examined through a number of performance assessments of the waste management, storage, and disposal facilities planned for the wastes generated by commercial nuclear power plants. Since detailed plans have not yet been determined for disposition of the wastes generated by atomic energy defense activities, similar assessments were generally not performed for these materials. A second consideration, where applicable, was consistency with related environmental standards for radiation exposure. A third factor was evaluation of various

benchmarks to assess the acceptability of the residual risks that might be allowed by the rule. This was particularly important for the disposal standards, where there were few precedents to guide the Agency's judgments. Finally, the Agency placed considerable emphasis on the public concerns expressed during the various phases of this rulemaking, particularly where these concerns involved addressing the substantial uncertainties inherent in the unprecedented time periods of interest.

The final rule reflects a combination of all these perspectives—no single factor predominated. For instance, no portion of this rule is based solely on projections of the "best" protection that technology might provide. If this had been the case, the rule would have been significantly different. On the other hand, the rule cannot be interpreted as setting precedents for "acceptable risk" levels to future generations that should not be exceeded regardless of the circumstances. Instead, because of a number of unique circumstances, the Agency has been able to develop standards for the management and disposal of these wastes that are both reasonably achievable—with little, if any, effort beyond that already planned for commercial wastes—and that limit risks to levels that the Agency believes are clearly acceptably small. The following paragraphs describe how these various perspectives were used in developing the final rule.

Standards for Management and Storage (Subpart A)

Upon surveying the expected performance of the technologies planned for the management, storage, and preparation of these wastes for disposal, the Agency found that the likely exposures to members of the public would generally be very small. Therefore, compatibility with related radiation protection standards became a more important perspective for Subpart A.

For waste management and storage operations to be regulated by the NRC, the most relevant existing standards are those provisions of 40 CFR Part 190 that limit annual exposures of members of the public to 25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other organ from uranium fuel cycle facilities. Accordingly, the Agency has decided to extend this coverage to include such waste management and storage operations so that the combined exposure from all of the NRC-licensed facilities covered under Part 190 and Subpart A of Part 191 shall not exceed

these limits. This will include all operations prior to final closure at high-level waste disposal facilities, since these are to be regulated by the NRC.

For waste management and storage operations conducted at atomic energy defense facilities operated for the Department of Energy (which are not regulated by the NRC), the most relevant existing standards are the 40 CFR Part 81 limitations on air emissions of radionuclides that were recently promulgated under the Agency's Clean Air Act authorities (50 FR 5190). These standards limit annual exposures to members of the public to 25 millirems to the whole body and 75 millirems to any organ, with less stringent alternative standards available if it can be shown that no member of the public will receive a continuous exposure of more than 100 millirems per year or an infrequent exposure of more than 500 millirems per year from all sources (excluding natural background and medical exposures.) These Clean Air Act standards are applicable to those facilities not covered by 40 CFR Parts 190, 191 or 192. For DOE waste disposal facilities covered by this rule but not regulated by NRC (i.e., those for defense transuranic wastes), the Agency has included standards in Subpart A similar to those included in the Clean Air Act rule.

For other DOE waste management and storage operations, which are usually conducted on large facilities with many other potential sources of radionuclide emissions, the Agency believes that continued regulation under the broader scope of 40 CFR Part 81 is the most effective and practical approach. Otherwise, similar types of emissions from adjoining operations would have to be assessed and regulated through separate rules developed under different authorities; this would cause complex implementation practices without providing any additional protection.

Standards for Disposal (Subpart B)

Developing the standards for disposal of spent fuel and high-level and transuranic wastes involved much more unusual circumstances than those for waste management and storage. Because these materials are dangerous for so long, very long time frames are of interest. Standards must be implemented in the design phase for these disposal systems because active surveillance cannot be relied upon over such periods. At the same time, the standards must accommodate large uncertainties, including uncertainties in our current knowledge about disposal system behavior and the inherent

uncertainties regarding the distant future. Subpart B addresses these issues by combining several different types of standards. The primary objective of these standards is to isolate most of the wastes from man's environment by limiting long-term releases and the associated risks to populations. In addition, Subpart B limits risks to individuals in ways compatible with this primary objective.

Although developed primarily through consideration of mined geologic repositories, these disposal standards apply to disposal of spent fuel and high-level and transuranic radioactive wastes by any method—with one exception. The standards do not apply to ocean disposal or disposal in ocean sediments because such disposal of high-level waste is prohibited by the Marine Protection, Research, and Sanctuaries Act of 1972. If this law is ever changed to allow such disposal (DOE continues to study the feasibility of this technology, consistent with the NWSA), the Agency will develop appropriate regulations in accordance with the different authorities that would apply.

Also, these disposal standards do not apply to wastes that have already been disposed of. The various provisions of Subpart B are intended to be met through a combination of steps involving disposal system site selection, design, and operational techniques (i.e., engineered barriers). Therefore, the Agency believes it appropriate that these disposal standards only apply to disposal occurring after the standards have been promulgated—so that they can be taken into consideration in devising the proper selection of controls. Some transuranic wastes produced in support of national defense programs were disposed of before the current DOE procedures for transuranic waste management were adopted in 1970. The exclusion of wastes already disposed of applies to these transuranic wastes, for which selection of disposal system sites, designs, and operational techniques are no longer options.

Containment Requirements (Section 191.13)

To develop the containment requirements, the Agency assumed that some aspects of the future can be predicted well enough to guide the selection and development of disposal systems for these wastes. A period of 10,000 years was considered because that appears to be long enough to distinguish geologic repositories with relatively good capabilities to isolate wastes from those with relatively poor capabilities. On the other hand, this

period is short enough so that major geologic changes are unlikely and repository performance might be reasonably projected.

The Agency assessed the performance of a number of model geologic repositories similar to those systems now being considered by DOE. Potential radionuclide releases over 10,000 years were evaluated, and very general models of environmental transport and a linear, non-threshold dose-effect relationship were used to relate these releases to the incidence of premature cancer deaths they might cause. For the various repository types, these assessments indicate that disposal of the wastes from 100,000 metric tons of reactor fuel would cause a population risk ranging from no more than about ten to a little more than one hundred premature deaths over the entire 10,000-year period, assuming that the existing provisions of 10 CFR Part 60 regarding engineered barriers are met.

The Agency also evaluated the health risks that future generations would be exposed to from the amount of uranium ore needed to produce 100,000 metric tons of reactor fuel, if this ore had not been mined to begin with. Population risks ranging between 10 and 100,000 premature cancer deaths over 10,000 years were associated with this much unmined uranium ore, depending upon the analytical assumptions made.

These analyses, which have been updated from those prepared for the proposed standards, reinforce the Agency's conclusion that limiting radionuclide releases to levels associated with no more than 1,000 premature cancer deaths over 10,000 years from disposal of the wastes from 100,000 metric tons of reactor fuel satisfies two important objectives. First, it provides a level of protection that appears reasonably achievable by the various options being considered within the national program for commercial wastes. Second, the Agency believes that such a limitation would clearly keep risks to future populations at acceptably small levels, particularly because it appears to limit risks to no more than the midpoint of the range of estimated risks that future generations would have been exposed to if the uranium ore used to create the wastes had never been mined. Thus, because mined geologic repositories appear capable of providing such good protection, the Agency has decided to establish containment requirements that meet these two objectives.

The specific release limits for different radionuclides in Table 1 of the final rule were developed by estimating how many curies of each radionuclide would

cause 1,000 premature deaths over 10,000 years if released to the environment. The limits were then stated in terms of the allowable release from 1,000 metric tons of reactor fuel (so that the actual curie values in Table 1 correspond to a risk level of 10 premature deaths over 10,000 years). All of these limits have been rounded to the nearest order of magnitude because of the approximate nature of these calculations. For particular disposal systems, release limits based upon the amount of waste in the system will be developed and will be used in a formula that insures that the desired risk level will not be exceeded if releases of more than one type of radionuclide are predicted. For some of the wastes covered by this rule, 1,000 metric tons of reactor fuel is not an appropriate unit of waste. In these situations, the various Notes to Table 1 provide instructions on how to calculate the proper release limits. In particular, the final rule includes provisions for high-level wastes from reactor fuels that have received substantially different uses in national defense applications (and contain much different amounts of radioactivity) than is typical of most reactor fuel used to generate electricity. The proposed rule would have allowed releases for these different types of fuels to occur in much different proportions to their total radioactivity than the Agency intended.

The release limits apply to radionuclides that are projected to move into the "accessible environment", during the first 10,000 years after disposal. The accessible environment includes all of the atmosphere, land surface, surface waters, and oceans. However, it does not include the lithosphere (and the ground water within it) that is below the "controlled area" surrounding a disposal system. The standards are formulated this way because the properties of the geologic media around a mined repository are expected to provide much of the disposal system's capability to isolate these wastes over these long time periods. Thus, a certain area of the natural environment is envisioned to be dedicated to keeping these dangerous materials away from future generations and may not be suitable for certain other uses. In the final rule, this "controlled area" is not to exceed 100 square kilometers and is not to extend more than five kilometers in any direction from the original emplacement of the wastes in the disposal system. The implementing agencies may choose a smaller area whenever appropriate.

The containment requirements apply to accidental disruptions of a disposal system as well as to any expected

releases. Accordingly, they are stated in terms of the probability of releases occurring. This is done in two steps.

First, the release limits calculated in accordance with Notes 1 through 5 to Table 1 apply to those release levels that are projected to occur with a cumulative probability greater than 0.1 for the entire 10,000-year period over which these disposal standards apply. This includes the total releases from those processes that are expected to occur as well as relatively likely disruptions (which the Agency assumes will primarily include predictions of inadvertent human intrusion).

Second, these release limits multiplied by ten apply to all of the releases projected to occur with a cumulative probability greater than 0.001 over the 10,000-year period. The Agency expects that this will include releases that might occur from the more likely natural disruptive events, such as fault movement and breccia pipe formation (near soluble media such as salt formations). This range of probabilities was selected to include the anticipated uncertainties in predicting the likelihood of these natural phenomena. Greater releases are allowed for these circumstances because they are so unlikely to occur.

Finally, the containment requirements place no limits on releases projected to occur with a cumulative probability of less than 0.001 over 10,000 years. Probabilities this small would tend to be limited to phenomena such as the appearance of new volcanos outside of known areas of volcanic activity, and the Agency believes there is no benefit to public health or the environment from trying to regulate the consequences of such very unlikely events.

The containment requirements call for a "reasonable expectation" that their various quantitative tests be met. This phrase reflects the fact that unequivocal numerical proof of compliance is neither necessary nor likely to be obtained. A similar qualitative test, that of "reasonable assurance," has been used with NRC regulations for many years. Although the Agency's intent is similar, the NRC phrase has not been used in 40 CFR Part 191 because "reasonable assurance" has come to be associated with a level of confidence that may not be appropriate for the very long-term analytical projections that are called for by 191.13. The use of a different test of judgment is meant to acknowledge the unique considerations likely to be encountered upon implementation of these disposal standards.

Assurance Requirements (Section 191.14)

In contrast to the containment requirements, the assurance requirements were developed from that point of view that there may be major uncertainties and gaps in our knowledge of the expected behavior of disposal systems over many thousands of years. Therefore, no matter how promising the analytical projections of disposal system performance appear to be, these materials should be disposed in a cautious manner that reduces the likelihood of unanticipated types of releases. Because of the inherent uncertainties associated with these long time periods, the Agency believes that the principles embodied in the assurance requirements are important complements to the containment requirements that should insure that the level of protection desired is likely to be achieved.

Each of the assurance requirements was chosen to reduce the potential harm from some aspect of our uncertainty about the future. Designing disposal systems with limited reliance on active institutional controls reduces the risks if future generations do not maintain surveillance of disposal sites. On the other hand, planning for long-term monitoring helps reduce the chances that unexpectedly poor performance of a disposal system would go unnoticed. Using extensive markers and records and avoiding resources when selecting disposal sites both serve to reduce the chances that people may inadvertently disrupt a disposal system because of incomplete understanding of its location, design, or hazards. Designing disposal systems to include multiple types of barriers, both engineered and natural, reduces the risks if one type of barrier performs more poorly than current knowledge indicates. Finally, designing disposal systems so that it is feasible for the wastes to be located and recovered gives future generations an opportunity to rectify the situation if new discoveries indicate compelling reasons (which would not be foreseeable now) to change the way these wastes are disposed of.

The proposed standards contained two other assurance requirements intended to reduce the risks of uncertainty. One of them called for these wastes to be disposed of promptly to reduce the uncertainties associated with storing these materials for indefinitely long times with methods that require active human involvement. However—after this rule was published for public comment—the NWPA was enacted, setting up mandates and

procedures intended to insure development of the necessary disposal systems for spent fuel and high-level wastes. Furthermore, the Department has made substantial progress towards developing a repository for disposal of the transuranic wastes from atomic energy defense activities. Because of these steps, the Agency decided that the call for prompt disposal was no longer needed, and this assurance requirement has not been included in the final rule.

The other proposed assurance requirement deleted from the final rule is the provision that called for releases to be kept as small as reasonably achievable even when the numerical containment requirements have been complied with. This would have increased the confidence of achieving the desired level of protection even if there were major uncertainties in analytical projections of long-term isolation. However, the Agency does not believe that it is necessary to retain this assurance requirement in the final standards because of two aspects of the related rules subsequently promulgated by the NRC and DOE for disposal of spent fuel and high-level wastes.

First, NRC's 10 CFR Part 80 implemented the multiple barrier principle by requiring very good performance from two types of engineered components: A 300 to 1,000-year lifetime for waste packages during which there would be essentially no expected release of waste, and a subsequent long-term release rate from the waste form of no more than one part in 100,000 per year. The Agency fully endorses this approach and believes that it represents the best performance reasonably achievable for currently foreseeable engineered components. Second, the DOE has included a provision in its site selection guidelines (10 CFR Part 960) that calls for significant emphasis to be placed on selecting sites that demonstrate the lowest releases over 100,000 years compared to the other alternatives available. Particularly because of the longer time frames involved in this comparison, the Agency believes that this provides adequate encouragement to choose sites that provide the best isolation capabilities available. Therefore, the concept of keeping long-term releases as small as reasonably achievable has been embodied by other agencies' regulations for both the engineered and natural components of disposal systems.

The final rule incorporates the five remaining assurance requirements plus the requirement for long-term monitoring, but it makes them

applicable only to disposal facilities are not regulated by the NRC. In its comments on the proposed rule, the NRC objected to inclusion of the assurance requirements, asserting that they were not properly part of the Agency's authorities assigned by Reorganization Plan No. 3 of 1970. The Agency continues to believe that provisions such as the assurance requirements are an appropriate part of generally applicable standards where they are necessary to establish the regulatory context for numerical standards—as they are in these circumstances because of the major uncertainties involved. However, the two agencies have agreed to resolve this issue by having the Commission modify 10 CFR Part 60 where necessary to incorporate the intent of the assurance requirements, rather than have them included in 40 CFR Part 191 for NRC-licensed disposal facilities. Thus, 10 CFR Part 60 will establish the context needed for appropriate implementation of 40 CFR Part 191.

The NRC staff is preparing the appropriate revisions to Part 60 and has told the Agency that they will be published in the Federal Register for public review and comment within approximately 120 days of today's promulgation of 40 CFR Part 191. EPA has provided NRC with all of the comments received on the assurance requirements during the 40 CFR Part 191 rulemaking, and the Agency will participate in the NRC rulemaking to facilitate our objective of having the intent of all of the assurance requirements embodied in Federal regulation. Finally, the Agency will review the record and outcome of the Part 60 rulemaking to determine if any subsequent modifications to 40 CFR Part 191 are needed.

Individual and Ground Water Protection Requirements (Sections 191.15 and 191.16)

While the primary objective of both the proposed and final disposal standards has been to limit potential long-term releases from disposal systems (and the population risks associated with such releases), these two sections have been added to the final rule to provide protection for those individuals in the vicinity of a disposal system. There are a number of difficult issues involved in formulating standards for individual protection in this situation, as discussed later in the "Release Limits vs. Individual Dose Limits" section. However, after evaluating the various comments received on this topic, the Agency

believes that there are also important advantages in providing for individual protection in ways compatible with the containment and assurance requirements. In discussing this issue, the SAB Subcommittee stated that: "We support the use of a population risk criteria. We believe it is impractical to provide absolute protection to every individual for all postulated events or for very long periods. On the other hand, in our view it is important that, for the first several hundred years, residents of the region immediately outside the accessible environment have very great assurance that they will suffer no, or negligible, ill effects from the repository."

The individual protection requirements in the final rule limit the annual exposure from the disposal system to a member of the public in the accessible environment, for the first 1,000 years after disposal, to no more than 25 millirems to the whole body or 75 millirems to any organs. These limitations apply to the predicted behavior of the disposal system, including consideration of the uncertainties in predicted behavior, assuming that the disposal system is not disrupted by human intrusion or the occurrence of unlikely natural events. The Agency chose the limits of 25 millirem/year to the whole body and 75 millirem/year to any organ because it believes that they represent a sufficiently stringent level of protection for situations where no more than a few individuals are likely to receive this exposure. If such an individual were exposed to this level over a lifetime (which seems particularly unlikely given the localized pathways through which waste might escape from a geologic repository), the Agency estimates this would cause a 5×10^{-6} chance of incurring a premature fatal cancer.

In choosing a time period for these requirements to protect individuals nearby disposal systems, the Agency took into account concerns such as those expressed by the SAB by examining the effects of choosing different time frames. As 10,000 years was chosen for the containment requirements because it is long enough to encourage use of disposal sites with natural characteristics that enhance long-term isolation, 1,000 years was chosen for the individual protection provisions because the Agency's assessments indicate it is long enough to insure that particularly good engineered barriers would need to be used at potential sites where some ground water would be expected to flow through a mined geologic repository. Use of a time

much shorter than 1,000 years would not call for substantial engineered barriers even at disposal sites with a lot of ground water flow.

On the other hand, demonstrating compliance with individual exposure limits for times much longer than 1,000 years appears to be quite difficult because of the analytical uncertainties involved. It would require predicting radionuclide concentrations—even from releases of tiny portions of the waste—in all the possible ground water pathways flowing in all directions from the disposal system, at all depths down to 2,500 feet, as a function of time over many thousands of years. At some of the sites being considered (and possibly all of them, depending upon what is discovered during site characterization) the only certain way to comply with such requirements for periods on the order of 10,000 years appears to be to use very expensive engineered barriers that would rule out any potential releases over most of this period. While such barriers could provide longer-term protection for individuals, they would not provide substantial benefits to populations because the containment and assurance requirements already reduce population risks to very small levels.

Based on all of these considerations, the Agency has decided that a 1,000-year duration is adequate for quantitative limits on individual exposures after disposal. For longer time periods, several of the qualitative assurance requirements should help to reduce the chances that individuals will receive serious radiation exposures. In addition, 40 CFR Part 191 in no way limits the future applicability of the Agency's drinking water standards (40 CFR Part 141)—which protect community water supply systems through institutional controls—or of similar standards that future generations may choose to adopt.

In assessing the performance of a disposal system with regard to individual exposures, all pathways of radioactive material or radiation from the disposal system to people shall be considered. In particular, the assessments must assume that individuals consume all of their drinking water (2 liters per day) from any portion of a "significant source of ground water" anywhere outside of the "controlled area" surrounding the disposal system. Significant sources of ground water are defined to include underground formations that are likely to be able to provide enough water for a community water system as defined in 40 CFR Part 141. (More information regarding this

definition is provided later in the "Release Limits vs. Individual Dose Limits" discussion.) Formations that could only provide smaller amounts of potable water have not been included because the Agency wants to avoid discriminating against the use of low-productivity geologic formations that might provide very good long-term isolation as disposal sites. The Agency believes this is reasonable for these standards because of the very small number of such disposal facilities that are contemplated (no more than three or four over the next 100 years.) However, the Agency has no plans to use this classification for other ground water related standards, which usually affect a far greater number of situations.

The Agency has not required these individual protection provisions to assume ground water use within the controlled area because geologic media within the controlled area are an integral part of the disposal system's capability to provide long-term isolation. (But if the implementing agency plans to allow individuals to use ground water within the controlled area, such planned use would have to be considered within the pathways evaluated to determine compliance with § 191.15.) The potential loss of ground water resources is very small because of the small number of such disposal facilities contemplated. Nevertheless, the Agency has also added ground water protection requirements to the final rule (Section 191.16) that protect certain sources of ground water even within the controlled area. These ground water protection requirements are similar to the individual protection requirements because they apply to undisturbed performance for 1,000 years after disposal. However, the ground water protection requirements apply only to those Class I ground waters, as they are identified in accordance with the Agency's Ground-Water Protection Strategy published in August 1984, that meet the following three conditions: (1) They are within the controlled area or near (less than five kilometers beyond) the controlled area; (2) they are supplying drinking water for thousands of persons as of the date that the Department selects the site for extensive exploration as a potential location of a disposal system; and (3) they are irreplaceable in that no reasonable alternative source of drinking water is available to that population.

For such Class I ground waters, § 191.16 limits the radionuclide concentrations in water withdrawn from any portion of them to no more than concentration limits similar to those

established for the output of community water systems in 40 CFR Part 141. However, if the preexisting concentrations of radioactivity in the Class I aquifer already exceed any of these limits at a particular site, § 191.18 then limits any increases in the preexisting concentrations to these same concentration limits. The Agency believes these provisions are necessary and adequate to avoid any significant degradation of the important drinking water resources provided by these Class I ground waters.

Alternative Provisions for Disposal (Section 191.17)

In developing the disposal standards, the Agency has had to make many assumptions about the characteristics of disposal systems that have not been built, about plans for disposal that are only now being formulated, and about the probable adequacy of technical information that will not be collected for many years. Thus, although the Agency believes that the disposal standards being promulgated today are appropriate based upon current knowledge, we cannot rule out the possibility that future information may indicate needs to modify the standards.

In recognition of this possibility, § 191.17 of the final rule sets forth procedures under which the Administrator may develop modifications to Subpart B, should the need arise. Any such changes would have to proceed through the usual notice-and-comment rulemaking process, and § 191.17 stipulates that such a rulemaking would require a public comment period of at least 90 days, to include public hearings in affected areas of the country. Although such procedures are common practice in rulemakings of this type, they are not required by the statutes relevant to this rule (Administrative Procedures Act mandates can be satisfied by a comment period as short as 14 days). Thus, § 191.17 insures an opportunity for significant public interaction regarding any proposed changes to the disposal standards.

There are several areas of uncertainty the Agency is aware of that might cause suggested modifications of the standards in the future. One of these concerns implementation of the containment requirements for mined geologic repositories. This will require collection of a great deal of data during site characterization, resolution of the inevitable uncertainties in such information, and adaptation of this information into probabilistic risk assessments. Although the Agency is currently confident that this will be

successfully accomplished, such projections over thousands of years to determine compliance with an environmental regulation are unprecedented. If—after substantial experience with these analyses is acquired—disposal systems that clearly provide good isolation cannot reasonably be shown to comply with the containment requirements, the Agency would consider whether modifications to Subpart B were appropriate.

Another situation that might lead to suggested revisions would be if additional information were developed regarding the disposal of certain wastes that appeared to make it inappropriate to retain generally applicable standards addressing all of the wastes covered by this rule. For example, the DOE is considering disposal of some defense wastes by stabilizing them in their current storage tanks, rather than relocating them to a mined repository. The Agency has not assessed the ramifications of such disposal yet, and it is certainly possible that it could be carried out in compliance with all the provisions of Subpart B being promulgated today. However, it is also possible that there may be benefits associated with such disposal that would warrant changes in Subpart B for these types of waste. If so, § 191.17 would govern the consideration of any such revisions.

Other examples of developments that might offer reasons to consider alternative provisions in the future include: The use of reactor fuel cycles or utilizations substantially different than today's; new models of the environmental transport and biological effects of radionuclides that indicate major changes (i.e., approaching an order of magnitude) in the relative risks associated with different radionuclides and the level of protection sought by the disposal standards; or information that indicates that particular assurance requirements might not be needed in certain situations to insure adequate confidence of long-term environmental protection.

Guidance for Implementation (Appendix B)

This supplement to the final rule is based upon some of the analytical assumptions that the Agency made in developing the technical basis used for formulating the numerical disposal standards. These analytical assumptions incorporate information assembled as part of the technical basis used to develop the proposed rule. In particular, Appendix B discusses: (1) The consideration of all barriers of a disposal system in performance

assessments; (2) reasonable limitations on the scope of performance assessments; (3) the use of average or "mean" values in expressing the results of performance assessments; (4) the types of assumptions regarding the effectiveness of institutional controls; and (5) limiting assumptions regarding the frequency and severity of inadvertent human intrusion into geologic repositories.

The implementing agencies are responsible for selecting the specific information to be used in these and other aspects of performance assessments to determine compliance with 40 CFR Part 191. However, the Agency believes it is important that the assumptions used by the implementing agencies are compatible with those used by EPA in developing this rule. Otherwise, implementation of the disposal standards may have effects quite different than those anticipated by EPA. The final rule to be published in the Code of Federal Regulations will include this informational appendix as guidance to the implementing agencies. Although the other agencies are not bound to follow this guidance, EPA recommends that it be carefully considered in planning for the application of 40 CFR Part 191. The Agency will monitor implementation of the disposal standards as it develops over the next several years to determine whether any changes to the rule are called for to meet the Agency's objectives for these standards.

Comments on Issues Highlighted for Public Review

The Agency particularly requested public comment on six issues associated with the proposed rule (47 FR 58198). After these comments were received, additional comments and information were requested on seven issues raised by the initial comments (48 FR 21668). Two of these seven issues (the definition of high-level waste and the use of individual dose limitations in the disposal standards) had been included among the first six issues that were highlighted. Thus, a total of eleven questions received particular attention during the public review and comment process. The following paragraphs summarize the comments received on each of these issues and the Agency's responses to them, including descriptions of any resulting changes made in the final rule.

Definition of "High-Level Waste"

Traditionally, the term "high-level waste" has meant the highly radioactive liquid wastes remaining from the

recovery of uranium and plutonium in a nuclear fuel reprocessing plant, and other liquid or solid forms into which such liquid wastes are converted to facilitate managing them. This traditional use of the term has not included radioactive materials from other sources, no matter how radioactive they are. However, somewhat different definitions of high-level waste have appeared in certain laws and regulations affecting specific aspects of radioactive waste management. Most notably, some of these definitions have included unprocessed spent fuel as the prospects for a commercial fuel reprocessing industry became more uncertain.

In the proposed rule, high-level waste was defined in the traditional sense, including spent fuel if disposed of without reprocessing. But the proposed definition also included minimum radioactivity concentrations below which such materials would not be subject to the stringent isolation requirements of 40 CFR Part 191. To identify these minimum concentrations, the maximum concentrations that the NRC determined that it would generally accept in near-surface disposal facilities under 10 CFR Part 61 (47 FR 57446) were adapted. Since this represented a modification of the traditional meaning of high-level waste, the Agency particularly sought comment on this aspect of the proposed rule.

Shortly after 40 CFR Part 191 was published for public review, the NWPA was enacted. The NWPA distinguished between spent nuclear fuel and high-level waste, and it defined high-level waste to include both: "(A) The highly radioactive material resulting from the reprocessing of spent nuclear fuel, including liquid waste produced directly in reprocessing and any solid material derived from such liquid waste that contains fission products in sufficient concentrations; and (B) other highly radioactive material that the Commission, consistent with existing law, determines by rule requires permanent isolation." This definition allow for inclusion of highly radioactive material not related to reprocessing of spent nuclear fuel, and it reflects the concept that some derivatives of nuclear fuel reprocessing may not contain sufficient radioactivity to warrant exceptional isolation.

Many of the comments regarding the proposed definition suggested that EPA adopt the definition in the NWPA, although in response to the specific questions distributed in conjunction with the Agency's public hearings, many

responders thought that the Agency should define the phrase "sufficient concentrations" contained in part A of the NWPA definition. However, several commenters argued that the proposed lower limits for high-level waste concentrations had been improperly taken out of the context of 10 CFR Part 61 and could require expensive disposal of wastes with relatively small hazards.

After considering these comments and other information currently available, the Agency decided to incorporate the NWPA definition of high-level waste in the final 40 CFR Part 191 without further elaboration of the phrase "sufficient concentrations." The Agency recognizes that this introduces some uncertainty regarding the applicability of this rule. However, the Commission is now beginning a rulemaking that should assemble the technical information needed to develop a more specific definition of high-level wastes. Since the NRC definition would not necessarily apply to all the situations covered by 40 CFR Part 191 (e.g., management and storage of defense high-level wastes prior to disposal is not regulated by NRC), the Agency will follow the Commission's rulemaking to determine what appropriate elaborations of the NWPA definition should be incorporated into 40 CFR Part 191. Upon completion of the NRC rulemaking, the Agency will initiate steps to appropriately modify this rule. In addition, EPA will address disposal of any radioactive wastes that are not covered by 40 CFR Part 191 or 40 CFR Part 192 (the Agency's standards for disposal of uranium mill tailings) as it considers standards for disposal of low-level radioactive wastes (48 FR 39563).

Finally, incorporating the NWPA definition of high-level waste also includes the phrase "consistent with existing law" when describing the NRC's responsibilities to identify materials as high-level waste. Promulgation of 40 CFR Part 191 with this definition does not signify Agency acceptance or endorsement of any particular interpretation of the phrase "consistent with existing law." The Agency presumes that the Commission will specify the applicability of its existing authorities as it conducts the relevant rulemaking efforts.

The Level of Protection

In the proposed rule, the containment requirements for disposal systems limited the residual risks to no more than an estimated 1,000 premature cancer deaths over the first 10,000 years after disposal of the wastes from 100,000 metric tons of heavy metal (MTHM) used as fuel in a nuclear reactor. The

Agency pointed out that a variety of mined repository designs using different combinations of geologic media and engineered controls were expected to meet these requirements. It was also estimated that the residual risks to future generations appeared to be no greater than if the uranium ore used to create the wastes had not been mined. EPA particularly asked for comment on whether it had taken an appropriate and reasonable approach in choosing this level of protection based upon these considerations.

Most of the public comments found this approach satisfactory. However, some commenters argued that the risks from unmined uranium ore did not necessarily define an acceptably low level of residual risks. They pointed out that such risks may vary from place to place (and a high-level waste repository could "redistribute" them) and that society sometimes does take measures to clean-up naturally-occurring radioactivity, implying that such natural risks are not always "acceptable."

On the other hand, some commenters felt that the level of protection sought in the proposed rule was far too stringent when compared to risks allowed and accepted by society from other activities. For example, the SAB Subcommittee recommended that the desired level of protection be relaxed by at least a factor of ten for this reason, coupled with the Subcommittee's concern that the uncertainties in analytical projections over thousands of years could make it difficult to demonstrate compliance with the proposed containment requirements.

After evaluating the public comments and updated performance assessments of geologic repositories, the Agency has retained the proposed level of protection as the basis for the long-term containment requirements in the final rule—even though it is true that long-term assessments of repository performance will encounter substantial uncertainties, as the SAB Subcommittee pointed out. Three reasons support this decision.

First, revising the performance assessments in accordance with many of the technical recommendations of the SAB has reinforced the Agency's conclusion that the proposed level of protection can reasonably be achieved by a variety of combinations of repository sites and designs—and EPA's regulatory impact analyses indicate that this level of protection can be achieved without significant effects on the cost of disposing of these wastes.

Second, comparing this level of protection with the comparable risks

from equivalent amounts of unmined uranium ore continues to reinforce the Agency's belief that this is an acceptably small residual risk for future generations. Therefore, the Agency believes that this level of protection represents a reasonable basis for these disposal standards.

Third, rather than relax the level of protection, the Agency has chosen to address the uncertainties that concerned the SAB Subcommittee by adding § 191.13(b) and by providing a more detailed "Guidance for Implementation" section to replace the proposed "Procedural Requirements." For example, this guidance points out that the entire range of possible projections of releases need not meet the containment requirements. Rather, compliance should be based upon the projections that the implementing agencies believe are more realistic. Furthermore, these revisions acknowledge that the quantitative calculations needed may have to be supplemented by reasonable qualitative judgments in order to appropriately determine compliance with the disposal standards.

In retaining the proposed level of protection, the Agency emphasizes that it is making a decision applicable only to the circumstances involving disposal of spent nuclear fuel and high-level and transuranic wastes. This rule cannot be used to establish precedents such as "no incremental risk to future generations" for extrapolation to other disposal problems. For other situations, evaluations of technological feasibility and cost-effectiveness must be considered for the particular set of circumstances. If mined geologic repositories were not capable of providing such good protection, the Agency might have chosen considerably different standards.

Time Period for Containment Requirements

Many commenters addressed the 10,000-year period used for the proposed containment requirements. A few argued that this period was too long and that EPA should only be concerned with a few hundred to a thousand years. A number of commenters supported the focus on 10,000 years. However, many commenters felt that it was inappropriate for the standards to ignore the period after 10,000 years. Some suggested that the containment requirements should address periods ranging from 50,000 to 500,000 years.

In the proposed rule, the Agency indicated that 10,000 years was chosen, in part, because compliance with quantitative standards for a

substantially longer period would have entailed considerably more uncertain calculations. There was no intention to indicate that times beyond 10,000 years were unimportant, but the Agency felt that a disposal system capable of meeting the proposed containment requirements for 10,000 years would continue to protect people and the environment well beyond 10,000 years. The SAB Subcommittee reviewed and supported these technical arguments for limiting the containment requirements to a 10,000-year period. Those commenters who argued for longer periods did not suggest effective ways that might compensate for the substantially greater uncertainties inherent in longer projections of disposal system performance.

However, many of the commenters and the SAB Subcommittee suggested that more qualitative or comparative assessments beyond 10,000 years might be appropriate. The Agency agreed with these comments and worked with the DOE to formulate comparative assessment provisions that have been incorporated into the final version of the Department's site selection guidelines (10 CFR Part 960). These provisions call for comparisons of the projected releases from undisturbed performance of alternative repository sites over 100,000 years to be a significant consideration in site selection. Since natural barriers are expected to provide the primary protection for such long time frames, this provision should allow for appropriate consideration of longer time periods without requiring the absolute values of these very uncertain calculations to meet a specific quantitative test. With the inclusion of this comparative test in 10 CFR Part 960, the Agency believes that no modification is needed in 40 CFR Part 191.

Use of Quantitative Probabilities in the Containment Requirements

The containment requirements in the proposed rule applied to two categories of potential releases ("reasonably foreseeable" and "very unlikely") based upon their projected probabilities of occurrence over the first 10,000 years after disposal. In its comments on the proposed rule, the NRC objected to the proposed quantitative definitions of these probabilities on the basis that calculation of such probabilities could be so uncertain that it would be impractical to determine whether the standards had been complied with. Instead, the NRC suggested substitution of qualitative terms to identify the two categories of potential releases. The wording proposed by the NRC was

formulated in terms of releases that might be caused by geologic processes and events.

In the second round of comment, the Agency sought information on whether to adopt the NRC's recommended wording or to retain definitions based on quantitative probabilities. Although a number of commenters agreed with the NRC position, the preponderance of comments supported retention of the quantitative probabilities. The SAB Subcommittee strongly supported retention of the probabilistic structure, but with substantially less restrictive probabilities and with the proviso that the Agency be sure that such conditions would be ". . . practical to meet and [would] not lead to serious impediments, legal or otherwise, to the licensing of high-level waste repositories." After considering all of this information, the Agency has revised the structure of the containment requirements in several ways that will retain quantitative objectives for long-term containment while allowing the implementing agencies enough flexibility to make qualitative judgments when necessary.

First, the final rule does not use the terms "reasonably foreseeable" and "very unlikely" releases. Instead, the permissible probabilities for two different levels of cumulative releases (over 10,000 years after disposal) are now incorporated directly into the containment requirements.

Second, the numerical probabilities associated with the two release categories have been increased by an order of magnitude to reflect further assessments of the uncertainties associated with projecting the probabilities of geologic events such as fault movement.

Third, the final rule clearly indicates that comprehensive performance assessments, including estimates of the probabilities of various potential releases whenever meaningful estimates are practicable, are needed to determine compliance with the containment requirements.

Fourth, a paragraph has been added to the final containment requirements (Section 191.13) to emphasize that unequivocal proof of compliance is neither expected nor required because of the substantial uncertainties inherent in such long-term projections. Instead, the appropriate test is a reasonable expectation of compliance based upon practically obtainable information and analysis. This paragraph was patterned after a paragraph that considered similar issues in NRC's 10 CFR Part 60.

Finally, the "Guidance for Implementation" section has been

added (Appendix B). This part of the rule describes the Agency's assumptions regarding performance assessments and uncertainties and should discourage overly restrictive or inappropriate implementation of the containment requirements.

The Agency believes that these revisions to the proposed rule preserve an objective framework for application of the containment requirements that requires very stringent isolation while allowing the implementing agencies adequate flexibility to handle specific uncertainties that may be encountered.

Within this framework, the possibility of inadvertent human intrusion into or nearby a repository requires special attention. Such intrusion can significantly disrupt the containment afforded by a geologic repository (as well as being dangerous for the intruders), and repositories should be selected and designed to reduce the risks from such potential disruptions. However, assessing the ways and the reasons that people might explore underground in the future—and evaluating the effectiveness of passive controls to deter such exploration near a repository—will entail informed judgment and speculation. It will not be possible to develop a "correct" estimate of the probability of such intrusion. The Agency believes that performance assessments should consider the possibilities of such intrusion, but that limits should be placed on the severity of the assumptions used to make the assessments. Appendix B to the final rule describes a set of parameters about the likelihood and consequences of inadvertent intrusion that the Agency assumed were the most pessimistic that would be reasonable in making performance assessments. The implementing agencies may adopt these assumptions or develop similar ones of their own. However, as indicated under the discussion of institutional controls, the Agency does not believe that institutional controls can be relied upon to completely eliminate the possibility of inadvertent intrusion.

Definition of "Accessible Environment"

The containment requirements limit releases to the "accessible environment" for 10,000 years after disposal. In the proposed rule, ground water within 10 kilometers of a disposal system was excluded from the definition of accessible environment. This definition was intended to reflect the concept that the geologic media surrounding a mined repository are part of the long-term containment system, with disposal sites being selected so that the surrounding media prevent or

retard transport of radionuclides through ground water. Such surrounding media would be dedicated for this purpose, with the intention to prohibit incompatible activities (either those that might disrupt the disposal system or those that could cause significant radiation exposures) in perpetuity. Applying standards to the ground water contained within these geologic media surrounding a repository would ignore the role of this natural barrier, and it could reduce the incentive to search for sites with characteristics that would enhance long-term containment of these wastes. (At the same time, the Agency recognized that the institutional controls designed to reserve this area around a disposal system cannot be considered infallible, and other provisions of the rule are designed to reduce the consequences of potential failures.)

Many commenters objected to the definition of accessible environment incorporated in the proposed rule. Some recommended that all ground water, or all "potable" ground water, should be included. Others agreed that it was appropriate to exclude some ground water in the immediate vicinity of a repository, but argued that the proposed 10-kilometer distance was too long—particularly for ground water sources that were likely to be used in the future. A few commenters thought that the proposed definition was too restrictive by including all ground water beyond 10 kilometers; they suggested that poor quality ground water sources unlikely to be used in the future should not be part of the accessible environment at all.

After considering these comments, the Agency has decided to make several changes in the definition of the "accessible environment." First, the concept of a "controlled area" has been adopted from NRC's 10 CFR Part 60. This establishes an area around a disposal system that is to be identified by markers, records, and other passive institutional controls intended to prohibit incompatible activities from the area. Consistent with the proposed 40 CFR Part 191, the current NRC definition of "controlled area" limits its distance from the edge of a repository to no more than 10 kilometers. The final 40 CFR Part 191 defines "accessible environment" to include: (1) The atmosphere, land surfaces, surface waters, and the oceans, wherever they are located; and (2) portions of the lithosphere—and the ground water within it—that are beyond the controlled area.

Second, the Agency has made the definition of the "controlled area" more restrictive than that currently

incorporated in 10 CFR Part 60. This revised definition limits the controlled area to a distance no greater than five kilometers from the original emplacement of wastes in a disposal system, rather than 10 kilometers. Furthermore, the revised definition limits the area encompassed by the controlled area to no more than 100 square kilometers, which is approximately the area that would be encompassed by a controlled area at a distance of three kilometers from all sides of a typical repository configuration. (A distance of five kilometers from all sides of a typical repository would correspond to an area of about 200 square kilometers, whereas a distance of ten kilometers from all sides corresponds to an area of almost 500 square kilometers.) This revised definition substantially reduces the area of the lithosphere that would have been removed from the "accessible environment" defined in the proposed rule, and it somewhat reduces the distance used in the proposed rule. The five-kilometer distance was chosen to retain reasonable compatibility with the NRC's requirement for a preemplacement ground water travel time of 1,000 years to the accessible environment (one of the 10 CFR Part 60 requirements developed in concert with the proposed rule), while still providing for greater isolation than called for by the proposed rule. This definition of the accessible environment will allow a controlled area to be established asymmetrically around a repository based upon the particular characteristics of a site.

Release Limits vs. Individual Dose Limits

The Agency believes that the containment requirements in § 191.13 will insure that the overall population risks to future generations from disposal of these wastes will be acceptably small. However, the situation with regard to potential individual doses is more complicated. Even with good engineering controls, some waste may eventually (i.e., several hundreds or thousands of years after disposal) be released into any ground water that might be in the immediate vicinity of a geologic repository. Since ground water generally provides relatively little dilution, anyone using such contaminated ground water in the future may receive a substantial radiation exposure (e.g., several rems per year or more). This possibility is inherent in collecting a very large amount of radioactivity in a small area.

The proposed rule did not contain any numerical restrictions on such potential individual doses after disposal. Rather, the proposal relied on several of the qualitative assurance requirements to greatly reduce the likelihood of such exposures. In particular, the assurance requirement calling for extensive permanent markers and records was intended to perpetuate information to future generations about the dangers of intruding into the vicinity of a repository. The assurance requirement to avoid sites with significant resources was intended to reduce the incentive to explore around a repository even if the information passed on was ignored or misunderstood. And the assurance requirements to use multiple barriers, both engineered and natural, and to keep releases as small as reasonably achievable were intended to encourage reduction of releases to ground water beyond that needed to meet the containment requirements—further reducing the potential for harmful individual exposures.

This approach to potential individual exposures was highlighted for comment when 40 CFR Part 191 was proposed. After receiving many recommendations to incorporate a limitation on individual doses after disposal, the Agency sought comment on further details of such a limitation in the second round of comments. For example, EPA asked whether such a limitation should apply to ground water use, whether it should apply only for ground water at some distance from a geologic repository or for any ground water source, and whether reliance on existing individual dose limitations (such as 40 CFR Part 141 or 10 CFR Part 20) for protection regarding ground water would be adequate.

The responses resulting from these questions offered a wide range of suggestions. A number of commenters opposed inclusion of an individual dose limitation for disposal on the grounds that calculations to judge compliance with such a standard would be highly speculative and not an appropriate basis upon which to judge the adequacy of a disposal system. In contrast, some other commenters argued that an individual dose standard in the 5 to 25 millirems per year range should apply to use of ground water in the accessible environment for an indefinitely long period into the future. Another group of commenters supported inclusion of some limitation on individual exposure, but only to the extent that it would not compromise the primary intent of long-term isolation and containment of the wastes.

These comments did not offer information that changed the Agency's perception of some of the problems associated with individual dose limitations for disposal. First, relying *only* upon an individual dose standard for disposal could encourage disposal methods that would enhance dilution of any wastes released. Thus, disposal sites near bodies of surface water or large sources of ground water might be preferred—which the Agency believes is an inappropriate policy that would usually increase overall population exposures.

This concern could be met by *adding* an individual dose limitation to the proposed containment requirements, rather than replacing them. However, the Agency's performance assessments of geologic repositories indicate that doses from using ground water close to a repository can become substantial (e.g., several rems per year) after a few hundred or thousand years, because the geological and geochemical characteristics of appropriate sites tend to concentrate eventual releases of wastes in any ground water that is close to the site. A study published by the National Academy of Sciences in April 1983 confirms this potential for large individual doses if flowing ground water can contact the wastes after the waste canisters are presumed to start leaking. Although it might be possible to find certain geologic settings that avoid this problem, such restrictive siting prerequisites could substantially delay development of disposal systems without providing significantly more protection to populations. Furthermore, even if reasonable limitations on individual exposure might be met at certain sites for very long times, demonstrating compliance with such limitations could be very difficult because of the additional complexities involved in estimating individual exposures rather than amounts of radioactivity released. The SAB Subcommittee report generally agreed with the technical aspects of these conclusions.

On the other hand, analyses of repository systems with good engineering controls show that they should be able to prevent significant doses from ground water use for at least a thousand years after disposal. Such protection would be compatible with both the proposed containment and assurance requirements. Accordingly, the SAB Subcommittee recommended that the Agency include a requirement limiting individual doses for the first 500 years after disposal, and one of the States that commented on the proposed

rule suggested an individual dose limit for 1,000 years after disposal.

After considering all of this information, the Agency has decided to include two new sections in the final rule. The first (Section 191.15) limits exposures to members of the public after disposal, while the second (Section 191.16) limits concentrations in water withdrawn from certain important sources of ground water after disposal.

The individual protection requirements in § 191.15 limit exposures from a disposal system to individuals in the accessible environment to 25 millirems per year to the whole body and 75 millirems per year to any organ. These limits apply only to undisturbed performance of the disposal system (i.e., without any consideration of human intrusion or disruption by unlikely natural events), and they apply for the first 1,000 years after disposal. All potential pathways of radiation or radioactive material from the disposal system to people (associated with undisturbed performance) shall be considered, including the assumption that an individual drinks two liters per day of water from any "significant source of ground water" outside of the "controlled area" surrounding a disposal system. If the implementing agency plans to allow individuals to use ground water within the controlled area, such planned use would also have to be considered within the pathways evaluated to determine compliance with § 191.15.

"Significant sources of ground water" are defined to include any aquifer currently providing the primary source of water for a community water system or any aquifer that satisfies all of the following five conditions: (1) It is saturated with water containing less than 10,000 milligrams per liter of total dissolved solids; (2) it is within 2,500 feet of the land surface; (3) it has a transmissivity of a least 200 gallons per day per foot, provided that (4) each of the underground formations or parts of underground formations included within the aquifer must have an individual hydraulic conductivity greater than 2 gallons per day per square foot; and (5) it must be capable of providing a sustained yield of 10,000 gallons per day of water to a pumped or flowing well.

Although such quantitative distinctions are inevitably somewhat arbitrary, the Agency believes that they provide reasonable demarcations to identify underground formations that could meet the needs of community water systems in the future. The selected transmissivity of 200 gallons per day per foot and the sustained yield

of 10,000 gallons per day roughly correspond to the size of a ground water source required to support the needs of about 20 households; this is similar to the size of the community water system considered in 40 CFR Part 141. The water quality criterion of 10,000 milligrams per liter of total dissolved solids has been used in several previous Agency regulations and is based upon congressional guidance in the legislative history of the Safe Drinking Water Act. The maximum depth criterion of 2,500 feet was chosen because almost all of the wells used to provide water to significant numbers of people do not extend below this depth. The minimum hydraulic conductivity criterion of 2 gallons per day per square foot was chosen to insure that only reasonably permeable formations are considered, rather than including unproductive formations that might be in the vicinity of a "significant source of ground water."

The ground water protection requirements in § 191.16(a) limit the concentrations in water withdrawn from any "special source of ground water" in the vicinity of a disposal system to concentrations similar to those established for the output of community water systems by 40 CFR Part 141: (1) 5 picocuries per liter of radium-226 and radium-228; (2) 15 picocuries per liter of alpha-emitting radionuclides (including radium-226 and radium-228 but excluding radon); or (3) the combined concentrations of radionuclides that emit either beta or gamma radiation that would produce an annual dose equivalent to the total body or any internal organ greater than 4 millirems per year if an individual continuously consumed 2 liters per day of drinking water from that source of water. However, if the preexisting radionuclide concentrations in the special source of ground water already exceed any of these limits, then § 191.16(b) limits any increases in the preexisting concentrations to the concentration limits set in § 191.16(a). Like the individual protection requirements, the ground water protection requirements apply only for undisturbed performance of the disposal system and apply for the first 1,000 years after disposal. Unlike the individual protection requirements, the ground water requirements would apply to a "special source" if it was within the controlled area.

"Special sources" are defined to include only those Class I ground waters—to be identified in accordance with the Agency's Ground-Water Protection Strategy published in August 1984—that meet the following three

conditions: (1) They are within the controlled area or near (less than five kilometers beyond) the controlled area; (2) they are supplying drinking water for thousands of persons as of the date that the Department selects the site for extensive exploration as a potential location of a disposal system; and (3) they are irreplaceable in that no reasonable alternative source of drinking water is available to that population.

Need for the Assurance Requirements

The preceding issues dealt with the quantitative requirements of the disposal standards. While numerical standards are important to bring about appropriate selection and design of disposal systems, the Agency has long recognized that the numerical standards chosen for Subpart B, by themselves, do not provide either an adequate context for environmental protection or a sufficient basis to foster public confidence in the national program. There are too many uncertainties in projecting the behavior of natural and engineered components for many thousands of years—and too many opportunities for mistakes or poor judgments in such calculations—for the numerical requirements on overall system performance in Subpart B to be the sole basis to determine the acceptability of disposal systems for these very hazardous wastes. These uncertainties and potential errors in quantitative analysis could ultimately prevent the degree of protection sought by the Agency from being achieved. (Theoretically, it might be possible to develop adequate confidence in achieving this level of protection by choosing much more stringent numerical standards, but this could lead to substantial difficulties in implementation.) Therefore, the proposed standards also included qualitative assurance requirements chosen to ensure that cautious steps are taken to reduce the problems caused by these uncertainties. The proposed rule emphasized that the assurance requirements were an essential complement to the quantitative containment requirements that were selected.

In its comments on the proposed rule, the NRC argued that the assurance requirements were not properly part of the Agency's generally applicable standards. The Commission agreed that the overall numerical performance standards were not sufficient, but suggested that its regulations and procedures were the appropriate vehicle to provide the necessary confidence that the inherent uncertainties would not

compromise environmental protection. The Agency believes that it does have the authority to give regulatory expression to the context within which it has chosen to establish one set of numerical standards rather than another. However, because it might not be appropriate to exercise this authority, the Agency sought public comment on the need for the assurance requirements in the second round of comments.

The preponderance of comments received on this question strongly supported retention of the assurance requirements in 40 CFR Part 191. In particular, virtually all of the various State governments that commented on the rule described the assurance requirements as an essential part of the regulations governing disposal of these wastes. Subsequently, two of these States, Nevada and Minnesota, petitioned the Commission to incorporate the assurance requirements proposed as part of 40 CFR Part 191 into its own rules (50 FR 18267).

Based upon these comments, the Agency and the NRC have reached an agreement that should accomplish the desired regulatory goals while avoiding the jurisdictional issue. EPA has included the assurance requirements in the final rule, modified as appropriate in response to other comments. However, these requirements will not be applicable to disposal facilities to be licensed by the Commission. Instead, as discussed previously, the NRC staff plans to propose modifications to 10 CFR Part 60, developed in consultation with EPA, for public review and comment within approximately 120 days to insure that the objectives of all of the assurance requirements in 40 CFR Part 191 will be accomplished through compliance with 10 CFR Part 60. The Agency has provided the Commission with all of the comments received by EPA regarding the assurance requirements, so that the NRC can use them in its rulemaking. In addition, the Agency will participate in the NRC rulemaking to facilitate incorporation of the principles of all of the assurance requirements in Federal regulation. Finally, the Agency will review the record and outcome of the Part 60 rulemaking to determine if any subsequent modifications to 40 CFR Part 191 are needed.

Approach Toward Institutional Controls

The Agency particularly sought comment on its proposed approach to reliance on institutional controls. The proposed rule limited reliance on "active institutional controls" (such as controlling access to a disposal site,

performing maintenance operations, or cleaning up releases) to a reasonable period of time after disposal, described as on the order of a "few hundred years." On the other hand, "passive institutional controls" (such as permanent markers, records, archives, and other methods of preserving knowledge) were considered to be at least partially effective for a longer period of time.

Few commenters argued with the distinction between active and passive institutional controls, or with the amount of reliance the proposed rule envisioned for passive controls. However, many commenters felt that "a few hundred years" was too long a period to count on active controls. Accordingly, the final rule limits reliance on active institutional controls to no more than 100 years after disposal. This was the time period the Agency considered in criteria for radioactive waste disposal that were proposed for public comment in 1978 (43 FR 53262), a period that was generally supported by the commenters on that proposal. After this time, no contribution from any of the active institutional controls can be projected to prevent or limit potential releases of waste from a disposal system.

The concept of passive institutional controls has now been incorporated into the definition of "controlled area" that is used to establish one of the boundaries for applicability of the containment requirements and the individual protection requirements in the final rule. Because the assumptions made about the effectiveness of passive institutional controls can strongly affect implementation of the containment requirements, the Agency's intent has been elaborated in the "guidance for implementation" section. The Federal Government is committed to retaining control over disposal sites for these wastes as long as possible. Accordingly (and in compliance with one of the assurance requirements), an extensive system of explanatory markers and records will be instituted to warn future generations about the location and dangers of these wastes. These passive controls have not been assumed to prevent all possibilities of inadvertent human intrusion, because there will always be a realistic chance that some individuals will overlook or misunderstand the markers and records. (For example, exploratory drilling operations occasionally intrude into areas that clearly would have been avoided if existing information had been obtained and properly evaluated.) However, the Agency assumed that

society in general will retain knowledge about these wastes and that future societies should be able to deter systematic or persistent exploitation of a disposal site.

The Agency also assumed that passive institutional controls should reduce the chance of inadvertent intrusion compared to the likelihood if no markers and records were in place. Specific judgments about the chances and consequences of intrusion should be made by the implementing agencies when more information about particular disposal sites and passive control systems is available. The parameters described in the "guidance for implementation" represent the most severe assumptions that the Agency believed were reasonable to use in its analyses to evaluate the feasibility of compliance with this rule (analyses that are summarized in the BID). The implementing agencies are free to use other assumption if they develop information considered adequate to support those judgments.

The role envisioned for institutional controls in this rulemaking has been adapted from the general approach the Agency has followed in its activities involving disposal of radioactive wastes since the initial public workshops conducted in 1977 and 1978. The Agency's overall objective has been to protect public health and the environment from disposal of radioactive wastes without relying upon institutional controls for extended periods of time—because such controls do not appear to be reliable enough over the very long periods that these wastes remain dangerous. Instead, the Agency has pursued standards that call for isolation of the wastes through the physical characteristics of disposal system siting and design, rather than through continuing maintenance and surveillance. This principle was enunciated in the general criteria published for public comment in 1978 (43 FR 53262), and it has been incorporated into the Agency's standards for disposal of uranium mill tailings (48 FR 590, 48 FR 45928).

This approach has been tailored to fit two circumstances associated with mined geologic repositories. First, 40 CFR Part 191 places containment requirements on a broad range of potential unplanned releases as well as the expected behavior of the disposal system. Therefore, determining compliance with the standards involves performance assessments that consider the probabilities and consequences of a variety of disruptive events, including potential human intrusion. Not allowing

passive institutional controls to be taken into account to some degree when estimating the consequences of inadvertent human intrusion could lead to less protective geologic media being selected for repository sites. The Agency's analyses indicate that repositories in salt formations have particularly good capabilities to isolate the wastes from flowing ground water and, hence, the accessible environment. However, salt formations are also relatively easy to mine and are often associated with other types of resources. If performance assessments had to assume that future societies will have no way to ever recognize and limit the consequences of inadvertent intrusion (from solution mining of salt, for example), the scenarios that would have to be studied would be more likely to eliminate salt media from consideration than other rock types. Yet, this could rule out repositories that may provide the best isolation, compared to other alternatives, if less pessimistic assumptions about survival of knowledge were made.

The second circumstance that the Agency considered in evaluating the approach towards institutional controls taken in this rule is the fact that the mined geologic repositories planned for disposal of the materials covered by 40 CFR Part 191 are different from the disposal systems envisioned for any other types of waste. The types of inadvertent human activities that could lead to significant radiation exposures or releases of material from geologic repositories appear to call for much more intensive and organized effort than those which could cause problems at, for example, an unattended surface disposal site. It appears reasonable to assume that information regarding the disposal system is more likely to reach (and presumably deter) people undertaking such organized efforts than it is to inform individuals involved in mundane activities.

These considerations led the Agency to conclude that a limited role for passive institutional controls would be appropriate when projecting the long-term performance of mined geologic repositories to judge compliance with these standards. However, such assumptions would not necessarily be applicable to other Agency actions where different issues are involved.

Avoiding Sites With Natural Resources

The proposed rule contained an assurance requirement that would have prohibited use of sites where there is a reasonable expectation that future exploration for scarce or easily

accessible resources might occur. The comments received on this issue generally agreed that sites with resources should be avoided. However, some commenters suggested that the requirement should be more restrictive, to include "potentially accessible" resources. Other commenters argued that the Agency should be less restrictive regarding sites with possible resource potential—discouraging but not prohibiting their use—because other attributes of the site might overcome the relative disadvantages presented by resource potential.

After considering these comments, the Agency agreed with the latter viewpoint. This judgment was reinforced by the belief that disposal sites should be chosen after comparative evaluation of a variety of alternatives, and the proposed assurance requirement could have inhibited this process. Therefore, this assurance requirement has been revised in the final rule to identify resource potential as a disincentive but not as an outright prohibition for site selection. Instead, the revised assurance requirement states that places with resource potential shall not be used "unless the favorable characteristics of such places compensate for their greater likelihood of being disturbed in the future."

This wording implies a qualitative comparison, because the Agency is not aware of quantitative formulas comprehensive enough to provide adequate comparisons to govern site selection. However, the Agency does not intend that sites with resource potential can be used merely upon identification of a few features that might be more favorable than at a site without significant resources. Rather, sites with resources should only be used if it is reasonably certain that they would provide better overall protection than the practical alternatives that are available.

The following example illustrates the effect of the change in this assurance requirement. When discussing the proposed assurance requirement, the Agency implied that disposal in salt domes might not be acceptable because such formations seemed more likely than others to attract exploration in the future. The modification of this assurance requirement in the final rule means that salt domes should not be peremptorily removed from consideration, but should be compared against all of the characteristics of alternative sites in terms of the overall environmental protection expected.

Long-Term Monitoring

The proposed rule addressed active institutional controls over a disposal site only in a negative sense—to prohibit reliance upon them for more than a few hundred years after disposal. The Agency's intent was to be sure that long-term protection of the environment did not depend upon positive actions by future generations. Almost all commenters agreed with this intent, although many suggested a shorter period of reliance was appropriate (see the preceding discussion under "Approach Towards Institutional Controls").

However, several commenters (including most of the States) also urged addition of a requirement for long-term monitoring of a repository after disposal. This view did not deny the need to select and design disposal systems without depending upon active controls in the future. However, it broadened this perspective by arguing that a disposal system so designed should still be monitored for a long time after disposal to guard against unexpected failures.

The Agency had not considered this viewpoint in developing the proposed rule. Accordingly, further information on this idea was sought during the "second round" of public comment, and the Agency surveyed the capabilities and expectations of long-term monitoring approaches. Evaluating this information led the Agency to several conclusions:

(1) Perhaps most importantly, the techniques used for monitoring after disposal must not jeopardize the long-term isolation capabilities of the disposal system. Furthermore, plans to conduct monitoring after disposal should never become an excuse to relax the care with which systems to isolate these wastes must be selected, designed, constructed, and operated.

(2) Monitoring for radionuclide releases to the accessible environment is not likely to be productive. Even a poorly performing geologic repository is very unlikely to allow measurable releases to the accessible environment for several hundreds of years or more, particularly in view of the engineered controls needed to comply with 10 CFR Part 60. A monitoring system based only on detecting radionuclide releases—a system which would almost certainly not be detecting anything for several times the history of the United States—is not likely to be maintained for long enough to be of much use.

(3) Within the above constraints, however, there are likely to be monitoring approaches which may, in a relatively short time, significantly improve confidence that a repository is

performing as intended. Two examples are of particular interest. One involves the concept of monitoring ground water sources at a variety of distances for benign tracers intentionally released to the ground water in the repository; this approach can evaluate the delay involved in ground water movement from the repository to the environment and can serve to validate expectations of the performance expected from the system's natural barriers. Another concept involves monitoring the small uplift of the land surface over the repository in order to validate predictions of the system's thermal behavior. Both of these approaches can be carried out without enhancing pathways for the wastes to escape from the repository.

Based on these conclusions and the public comments on this question, the Agency has included a provision for long-term monitoring after disposal in the assurance requirements of the final rule: "Disposal systems shall be monitored after disposal to detect substantial and detrimental deviations from expected performance. This monitoring shall be done with techniques that do not jeopardize the isolation of the wastes and shall be conducted until there are no significant concerns to be addressed by further monitoring." This new provision is consistent with the overall intent of the assurance requirements: To take prudent and cautious steps necessary to minimize the risks posed by the inherent uncertainties in expectations of the future. Beyond this broad mandate, however, the Agency has not specified the details of a monitoring program. That is properly left to the implementing agencies. Furthermore, the precise objectives of an appropriate monitoring program probably should not be spelled out until much more information is gathered about the characteristics and expected behavior of specific sites and designs.

Ability To Recover Wastes After Disposal

The proposed rule included an assurance requirement that recovery of these wastes be feasible for "a reasonable period of time" after disposal. The Agency specifically sought comment on whether this was a desirable provision, since it would rule out certain disposal concepts, such as deep-well injection of liquid wastes. The comments received were split about evenly between those who thought the provision should be retained and those who thought it was detrimental to the overall rule. Many of those who opposed

the requirement argued that it would encourage designing a geologic repository to make retrieving waste relatively easy—which might compromise the isolation capabilities of the repository or which might encourage recovery of the waste to make use of some intrinsic value it might retain (the potential energy content of spent nuclear fuel, for example).

The intent of this provision was not to make recovery of waste easy or cheap, but merely possible in case some future discovery or insight made it clear that the wastes needed to be relocated. EPA reiterates the statement in the preamble to the proposal that *any* current concept for a mined geologic repository meets this requirement *without* any additional procedures or design features. For example, there is no intent to require that a repository shaft be kept open to allow future recovery. To meet this assurance requirement, it only need be technologically feasible (assuming current technology levels) to be able to mine the sealed repository and recover the waste—albeit at substantial cost and occupational risk. The Commission's requirements for multiple engineered barriers within a repository (10 CFR Part 60) adequately address any concerns about the feasibility of recovering wastes from a repository.

Therefore, this provision should not have any effect upon plans for mined geologic repositories. Rather, it is intended to call into question any other disposal concept that might not be so reversible—because the Agency believes that future generations should have options to correct any mistakes that this generation might unintentionally make. Almost all of the commenters agreed with the validity of this objective. Accordingly, the Agency has decided to retain this assurance requirement in the final rule as proposed.

Health Impacts of 40 CFR Part 191

Waste Management and Storage. Waste management and storage activities conducted in accordance with Subpart A would limit the maximum risk to a member of the public in the general environment to a 5×10^{-6} chance of incurring a premature fatal cancer over a lifetime. Of course, a risk this large would exist only for an individual continuously exposed to the full amount of the dose limits over his or her lifetime. Because the Agency believes that such continuous exposure is very unlikely, the actual risks to individuals are expected to be much lower. It is theoretically possible under the final rule that an individual could be exposed to 25 millirems per year (to the whole

body) from *both* an NRC-licensed facility and a DOE facility not licensed by NRC, for a total of 50 millirem/year. However, the Agency believes that this is particularly improbable and does not foresee a significant public health impact from this possibility.

Waste Disposal. A disposal system complying with Subpart B would confine almost all of the radioactive wastes to the immediate vicinity of the repository for a very long time. Because the wastes would be so well isolated from the environment, the Agency is confident that any risks to future populations would be very small. Similarly, risks to most future individuals would also be very small (and effectively zero in almost all cases)—except for the possibility that an individual in the distant future might use ground water from the vicinity of a repository. In this case, there is a chance that such an individual might receive a substantial exposure. The following paragraphs describe the possible health impacts of the residual risks from a disposal system that would be in compliance with 40 CFR Part 191.

Population Risks: With regard to exposure of populations, the Agency has estimated the potential long-term health risks to future generations from various types of mined geologic repositories using very general models of environmental transport and a linear, nonthreshold dose-effect relationship between radiation exposures and premature deaths from cancer. Food chains, ways of life, and the size and geographical distributions of populations will undoubtedly change over a 10,000-year period. Unlike geological processes, factors such as these cannot be usefully predicted over such long periods of time. Thus, in making these health effects projections, the Agency found it necessary to depend upon very general models of environmental pathways and to assume current population distributions and death rates. The SAB Subcommittee evaluated these models carefully, and, although a number of specific changes were recommended for particular parameters, the Subcommittee endorsed the general approach. As a consequence of using these generalized models, EPA's projections are intended to be used primarily as a tool for comparing the performance of one waste disposal system to another and for comparison of the risks of waste disposal with those of undisturbed ore bodies. The results of these analyses should not be considered a reliable projection of the "real" or absolute number of health effects

resulting from compliance with the disposal standards.

These health risk models were used to assess the long-term health risks from several different model repositories containing the wastes from 100,000 MTHM—which could include all existing wastes and the future wastes from all currently operating reactors. The Agency estimates that this quantity of waste, when disposed of in accordance with the proposed standards, would cause no more than 1,000 premature deaths from cancer in the first 10,000 years after disposal: an average of no more than one premature death every 10 years. Most of the model repositories considered had projected population risks at least a factor of ten below this, or about 100 deaths over 10,000 years. The projections for the actual repositories that are constructed are expected to be closer to this lower figure. Any such increase in the number of cancer deaths would be very small compared to today's incidence of cancer, which kills about 350,000 people per year in the United States. Similarly, any such increase would be much less than the approximately 6,000 premature cancer deaths per year that the same linear, non-threshold dose-effect relationship predicts for the nation due to natural background radiation.

Individual Risks: With regard to exposures of individuals, the Agency examined the potential doses to persons who might use ground water from the immediate vicinity of a repository at various times in the future. For these analyses, only the expected undisturbed performance of a repository was considered (e.g. there was no evaluation of exposures that might occur if a repository was disrupted by movement of a fault). In most of the cases studied, no exposures occurred for more than one thousand years after disposal. After that, these analyses predict that significant exposures (on the order of a few rems per year in the vicinity of the repository over the next several thousands of years) may appear for some of the geologic media considered. These projections are similar to those contained in the April 1983 report published by the National Academy of Sciences. The BID contains more detailed descriptions of the Agency's individual dose calculations.

Intergenerational Risk: As described earlier, the Agency has chosen to rely on provisions that limit risks to populations as the primary standards for the long-term performance of disposal systems. Although the projections of the residual population risk are clearly very small, the discontinuity between when the

wastes are generated and when the projected health effects manifest themselves made it difficult to determine what level of residual risk should be allowed by these disposal standards. The difficulty arose because most of the benefits derived in the process of waste production fall upon the current generation, while most of the risks fall upon future generations. Thus, a potential problem of intergenerational equity with respect to the distribution of risks and benefits became apparent. This problem is sometimes referred to as the intergenerational risk issue, and it is not unique to the disposal of high-level radioactive wastes. If the Agency tried to insure that these standards fully satisfied a criterion of intergenerational equity with respect to the distribution of risks and benefits, it might appear that no risk should be passed on to future generations. This is a condition which the Agency believes cannot be met by disposal technologies foreseeable within this century. However, there is one particular factor which has reinforced EPA's decision about the reasonableness of the risks permitted under the disposal standards. This is the following evaluation of the risks associated with undisturbed uranium ore bodies. Additionally, for the purpose of comparing the risks permitted under the standards to other radiation risks which people are currently exposed to, a brief discussion of the risks from other natural sources of radiation is also included.

Uranium Ore: Most uranium ore in the United States occurs in permeable geologic strata containing flowing ground water. Radionuclides in the ore, particularly uranium and radium, continuously enter this ground water. EPA estimated the potential risks from these undisturbed ore bodies using the same generalized environmental models that were used for releases from a waste repository. The effects associated with the amount of ore needed to produce the high-level wastes that would fill the model geologic repository can vary considerably. Part of this variation corresponds to actual differences from one ore body to another; part can be attributed to uncertainties in the assessment. After revising the population risk models in accordance with the recommendations of the SAB Subcommittee, these estimates of the risks from unmined ore bodies ranged from about 10 to more than 100,000 excess cancer deaths over 10,000 years. Thus, leaving the ore unmined appears to present a risk to future generations comparable to the risks from disposal of wastes covered by these standards.

Variations in Natural Background: Radionuclides occur naturally in the earth in very large amounts, and are produced in the atmosphere by cosmic radiation. Everyone is exposed to natural background radiation from these natural radionuclides and from direct exposure to cosmic radiation. Individual exposures average about 100 millirems per year, with a range of about 60 to 200 millirem/year. These natural background radiation levels have remained relatively constant for a very long time. According to the same linear, nonthreshold dose effect relationship used in EPA's other analyses, an increase of one millirem per year (about one percent) in natural background in the United States would result in about 60 additional deaths per year, or 600,000 over a 10,000-year period.

Natural Radionuclide Concentrations in Ground Water: One source of this exposure to natural background radiation comes from naturally occurring radionuclides found in ground water. Radium is the most important of the naturally occurring radioactive materials likely to occur in public water supply systems, but uranium is also found in ground waters due to its natural occurrence. Surveys of radionuclides in ground water systems indicate: a United States range of 0.1 to 50 picocuries (pCi) per liter for radium-226 (with isolated sources exceeding 100 pCi/liter); up to 74 pCi/liter for all alpha-emitting radionuclides other than uranium (although most of the alpha-emitting concentrations are below 3 pCi/liter); and up to 650 pCi/liter for total uranium concentrations. Elevated radium-226 concentrations are found along the Atlantic coastal region and the Midwest; low levels are usually found in the treated water supplies in the western States. Elevated uranium and alpha-emitting radionuclide concentrations are generally limited to the Rocky Mountain region and Maine and Pennsylvania in the east.

The Agency's primary drinking water regulations (40 CFR Part 141) limit the contamination levels for radium-226 and radium-228 to 5 pCi/liter and the levels for total alpha-emitting contamination (excluding radon and uranium) to 15 pCi/liter. Elevated concentrations of radium in drinking water are generally a problem associated with smaller community water systems, with an estimated 500 systems exceeding 5 pCi/liter. The Agency's risk assessments indicate that continuous consumption of water containing the maximum amount of radium allowed may cause between 0.7 and 3 cancers per year per million exposed persons.

Environmental Impacts

A Draft Environmental Impact Statement (EIS) was prepared for the proposed rule, in accordance with the Agency's procedures for the voluntary preparation of EIS's (30 FR 37419). However, section 121(c) of the NWPA subsequently exempted this action from preparation of an EIS under section 102(2)(C) of the National Environmental Policy Act of 1969 (NEPA) and from any environmental review under subparagraph (E) or (F) of section 102(2) of the NEPA. Accordingly, a Final EIS has not been prepared for promulgation of this rule. The potential health impacts of this action are summarized above, and much of the information that would have been contained in a Final EIS is documented in the Background Information Document that accompanies this final version of 40 CFR Part 191.

Regulatory Impacts

This rule was submitted to the Office of Management and Budget (OMB) for review as required by Executive Order 12291. The final rule has not been classified as a "major rule" in accordance with the guidelines provided by the Executive Order. Any comments received from OMB and EPA's responses to those comments are available for public inspection in the docket cited above under the heading "ADDRESSER."

The Agency has had to take an unusual approach in considering the regulatory impacts of this proposed action—as required by Executive Order 12291. In most cases, a regulation concerns an ongoing activity and may be considered a burden whose costs should be judged against the regulatory benefits. Here, it was not possible to quantify the costs and benefits of this action compared to the consequences of no regulation because there is no specific "baseline" program to consider. The appropriate regulations must be established before the regulated activity can even begin. Thus, the typical perspectives on costs and benefits are altered. Instead, the Agency evaluated how the costs of commercial waste management and disposal might change in response to different levels of protection from the containment requirements. Similar evaluations were not performed for the wastes from atomic energy defense activities because sufficient information was not available.

To evaluate the effects of different levels of protection, EPA considered the performance of different repository designs in several different geologic

media. The costs of the various engineering controls that might be needed to meet different levels of protection were estimated. In addition, allowances were made for the increased research and development costs that might be needed to demonstrate compliance with the standards if projected performance for a particular disposal system indicated releases less than an order of magnitude below the long-term radionuclide release limits in § 191.13.

Since the regulatory impact analyses that supported the proposed rule were performed, the NRC has promulgated minimum requirements for the engineered barriers of a disposal system (in 10 CFR Part 60), more data concerning disposal sites being considered by the Department have become available, and the Agency has reviewed its performance assessments to reduce overestimates of long-term risks in accordance with the SAB review. After evaluating all of this new information, the Agency believes that there need not be any significant additional costs to the national program for disposal of commercial wastes caused by retaining the proposed level of protection in the final rule, compared to the costs of choosing levels considerably less stringent. In other words, all of the disposal sites being evaluated by the Department, assuming compliance with the existing requirements of 10 CFR Part 60, are expected to be able to meet these disposal standards without additional precautions beyond those already planned.

List of Subjects in 40 CFR Part 191

Environmental protection, Nuclear energy, Radiation protection, Uranium, Waste treatment and disposal.

Regulatory Flexibility Certification

In accordance with the Regulatory Flexibility Act of 1980, 5 U.S.C. 605(b), the Administrator hereby certifies that this rule will not have any significant impact on small businesses or other entities, and that a Regulatory Flexibility Analysis is not required. This rule will affect only a small number of facilities, most of which are or will be operated by the United States Government.

Dated: August 15, 1985.

Lee M. Thomas,
Administrator.

A new Part 191 is hereby added to Title 40, Code of Federal Regulations, as follows:

SUBCHAPTER F—RADIATION PROTECTION PROGRAMS

PART 191—ENVIRONMENTAL RADIATION PROTECTION STANDARDS FOR MANAGEMENT AND DISPOSAL OF SPENT NUCLEAR FUEL, HIGH-LEVEL AND TRANSURANIC RADIOACTIVE WASTES

Subpart A—Environmental Standards for Management and Storage

Sec.

- 191.01 Applicability.
- 191.02 Definitions.
- 191.03 Standards.
- 191.04 Alternative standards.
- 191.05 Effective date.

Subpart B—Environmental Standards for Disposal

- 191.11 Applicability.
- 191.12 Definitions.
- 191.13 Containment requirements.
- 191.14 Assurance requirements.
- 191.15 Individual protection requirements.
- 191.16 Ground water protection requirements.
- 191.17 Alternative provisions for disposal.
- 191.18 Effective date.

Appendix A Table for Subpart B

Appendix B Guidance for Implementation of Subpart B

Authority: The Atomic Energy Act of 1954, as amended; Reorganization Plan No. 3 of 1970; and the Nuclear Waste Policy Act of 1982.

Subpart A—Environmental Standards for Management and Storage

§ 191.01 Applicability.

This Subpart applies to:

(a) Radiation doses received by members of the public as a result of the management (except for transportation) and storage of spent nuclear fuel or high-level or transuranic radioactive wastes at any facility regulated by the Nuclear Regulatory Commission or by Agreement States, to the extent that such management and storage operations are not subject to the provisions of Part 190 of title 40; and

(b) Radiation doses received by members of the public as a result of the management and storage of spent nuclear fuel or high-level or transuranic wastes at any disposal facility that is operated by the Department of Energy and that is not regulated by the Commission or by Agreement States.

§ 191.02 Definitions.

Unless otherwise indicated in this Subpart, all terms shall have the same meaning as in Subpart A of Part 190.

(a) "Agency" means the Environmental Protection Agency.

(b) "Administrator" means the Administrator of the Environmental Protection Agency.

(c) "Commission" means the Nuclear Regulatory Commission.

(d) "Department" means the Department of Energy.

(e) "NWSA" means the Nuclear Waste Policy Act of 1982 (Pub. L. 97-425).

(f) "Agreement State" means any State with which the Commission or the Atomic Energy Commission has entered into an effective agreement under subsection 274b of the Atomic Energy Act of 1954, as amended (68 Stat. 919).

(g) "Spent nuclear fuel" means fuel that has been withdrawn from a nuclear reactor following irradiation, the constituent elements of which have not been separated by reprocessing.

(h) "High-level radioactive waste," as used in this Part, means high-level radioactive waste as defined in the Nuclear Waste Policy Act of 1982 (Pub. L. 97-425).

(i) "Transuranic radioactive waste," as used in this Part, means waste containing more than 100 nanocuries of alpha-emitting transuranic isotopes, with half-lives greater than twenty years, per gram of waste, except for: (1) High-level radioactive wastes; (2) wastes that the Department has determined, with the concurrence of the Administrator, do not need the degree of isolation required by this Part; or (3) wastes that the Commission has approved for disposal on a case-by-case basis in accordance with 10 CFR Part 61.

(j) "Radioactive waste," as used in this Part, means the high-level and transuranic radioactive waste covered by this Part.

(k) "Storage" means retention of spent nuclear fuel or radioactive wastes with the intent and capability to readily retrieve such fuel or waste for subsequent use, processing, or disposal.

(l) "Disposal" means permanent isolation of spent nuclear fuel or radioactive waste from the accessible environment with no intent of recovery, whether or not such isolation permits the recovery of such fuel or waste. For example, disposal of waste in a mined geologic repository occurs when all of the shafts to the repository are backfilled and sealed.

(m) "Management" means any activity, operation, or process (except for transportation) conducted to prepare spent nuclear fuel or radioactive waste for storage or disposal, or the activities associated with placing such fuel or waste in a disposal system.

(n) "Site" means an area contained within the boundary of a location under the effective control of persons possessing or using spent nuclear fuel or radioactive waste that are involved in

any activity, operation, or process covered by this Subpart.

(o) "General environment" means the total terrestrial, atmospheric, and aquatic environments outside sites within which any activity, operation, or process associated with the management and storage of spent nuclear fuel or radioactive waste is conducted.

(p) "Member of the public" means any individual except during the time when that individual is a worker engaged in any activity, operation, or process that is covered by the Atomic Energy Act of 1954, as amended.

(q) "Critical organ" means the most exposed human organ or tissue exclusive of the integumentary system (skin) and the cornea.

§ 191.03 Standards.

(a) Management and storage of spent nuclear fuel or high-level or transuranic radioactive wastes at all facilities regulated by the Commission or by Agreement States shall be conducted in such a manner as to provide reasonable assurance that the combined annual dose equivalent to any member of the public in the general environment resulting from: (1) Discharges of radioactive material and direct radiation from such management and storage and (2) all operations covered by Part 190; shall not exceed 25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other critical organ.

(b) Management and storage of spent nuclear fuel or high-level or transuranic radioactive wastes at all facilities for the disposal of such fuel or waste that are operated by the Department and that are not regulated by the Commission or Agreement States shall be conducted in such a manner as to provide reasonable assurance that the combined annual dose equivalent to any member of the public in the general environment resulting from discharges of radioactive material and direct radiation from such management and storage shall not exceed 25 millirems to the whole body and 75 millirems to any critical organ.

§ 191.04 Alternative standards.

(a) The Administrator may issue alternative standards from those standards established in 191.03(b) for waste management and storage activities at facilities that are not regulated by the Commission or Agreement States if, upon review of an application for such alternative standards:

(1) The Administrator determines that such alternative standards will prevent

any member of the public from receiving a continuous exposure of more than 100 millirems per year dose equivalent and an infrequent exposure of more than 500 millirems dose equivalent in a year from all sources, excluding natural background and medical procedures; and

(2) The Administrator promptly makes a matter of public record the degree to which continued operation of the facility is expected to result in levels in excess of the standards specified in 191.03(b).

(b) An application for alternative standards shall be submitted as soon as possible after the Department determines that continued operation of a facility will exceed the levels specified in 191.03(b) and shall include all information necessary for the Administrator to make the determinations called for in 191.04(a).

(c) Requests for alternative standards shall be submitted to the Administrator, U.S. Environmental Protection Agency, 401 M Street, SW., Washington, DC 20460.

§ 191.05 Effective date.

The standards in this Subpart shall be effective on November 18, 1985.

Subpart B—Environmental Standards for Disposal

§ 191.11 Applicability.

(a) This Subpart applies to:

(1) Radioactive materials released into the accessible environment as a result of the disposal of spent nuclear fuel or high-level or transuranic radioactive wastes;

(2) Radiation doses received by members of the public as a result of such disposal; and

(3) Radioactive contamination of certain sources of ground water in the vicinity of disposal systems for such fuel or wastes.

(b) However, this Subpart does not apply to disposal directly into the oceans or ocean sediments. This Subpart also does not apply to wastes disposed of before the effective date of this rule.

§ 191.12 Definitions.

Unless otherwise indicated in this Subpart, all terms shall have the same meaning as in Subpart A of this Part.

(a) "Disposal system" means any combination of engineered and natural barriers that isolate spent nuclear fuel or radioactive waste after disposal.

(b) "Waste," as used in this Subpart, means any spent nuclear fuel or radioactive waste isolated in a disposal system.

(c) "Waste form" means the materials comprising the radioactive components of waste and any encapsulating or stabilizing matrix.

(d) "Barrier" means any material or structure that prevents or substantially delays movement of water or radionuclides toward the accessible environment. For example, a barrier may be a geologic structure, a canister, a waste form with physical and chemical characteristics that significantly decrease the mobility of radionuclides, or a material placed over and around waste, provided that the material or structure substantially delays movement of water or radionuclides.

(e) "Passive institutional control" means: (1) Permanent markers placed at a disposal site, (2) public records and archives, (3) government ownership and regulations regarding land or resource use, and (4) other methods of preserving knowledge about the location, design, and contents of a disposal system.

(f) "Active institutional control" means: (1) Controlling access to a disposal site by any means other than passive institutional controls; (2) performing maintenance operations or remedial actions at a site, (3) controlling or cleaning up releases from a site, or (4) monitoring parameters related to disposal system performance.

(g) "Controlled area" means: (1) A surface location, to be identified by passive institutional controls, that encompasses no more than 100 square kilometers and extends horizontally no more than five kilometers in any direction from the outer boundary of the original location of the radioactive wastes in a disposal system; and (2) the subsurface underlying such a surface location.

(h) "Ground water" means water below the land surface in a zone of saturation.

(i) "Aquifer" means an underground geological formation, group of formations, or part of a formation that is capable of yielding a significant amount of water to a well or spring.

(j) "Lithosphere" means the solid part of the Earth below the surface, including any ground water contained within it.

(k) "Accessible environment" means: (1) The atmosphere; (2) land surfaces; (3) surface waters; (4) oceans; and (5) all of the lithosphere that is beyond the controlled area.

(l) "Transmissivity" means the hydraulic conductivity integrated over the saturated thickness of an underground formation. The transmissivity of a series of formations is the sum of the individual

transmissivities of each formation comprising the series.

(m) "Community water system" means a system for the provision to the public of piped water for human consumption, if such system has at least 15 service connections used by year-round residents or regularly serves at least 25 year-round residents.

(n) "Significant source of ground water," as used in this Part, means: (1) An aquifer that: (i) is saturated with water having less than 10,000 milligrams per liter of total dissolved solids; (ii) is within 2,500 feet of the land surface; (iii) has a transmissivity greater than 200 gallons per day per foot, provided that any formation or part of a formation included within the source of ground water has a hydraulic conductivity greater than 2 gallons per day per square foot; and (iv) is capable of continuously yielding at least 10,000 gallons per day to a pumped or flowing well for a period of at least a year; or (2) an aquifer that provides the primary source of water for a community water system as of the effective date of this Subpart.

(o) "Special source of ground water," as used in this Part, means those Class I ground waters identified in accordance with the Agency's Ground-Water Protection Strategy published in August 1984 that: (1) Are within the controlled area encompassing a disposal system or are less than five kilometers beyond the controlled area; (2) are supplying drinking water for thousands of persons as of the date that the Department chooses a location within that area for detailed characterization as a potential site for a disposal system (e.g., in accordance with Section 112(b)(1)(B) of the NWPA); and (3) are irreplaceable in that no reasonable alternative source of drinking water is available to that population.

(p) "Undisturbed performance" means the predicted behavior of a disposal system, including consideration of the uncertainties in predicted behavior, if the disposal system is not disrupted by human intrusion or the occurrence of unlikely natural events.

(q) "Performance assessment" means an analysis that: (1) Identifies the processes and events that might affect the disposal system; (2) examines the effects of these processes and events on the performance of the disposal system; and (3) estimates the cumulative releases of radionuclides, considering the associated uncertainties, caused by all significant processes and events. These estimates shall be incorporated into an overall probability distribution of cumulative release to the extent practicable.

(r) "Heavy metal" means all uranium, plutonium, or thorium placed into a nuclear reactor.

(s) "Implementing agency," as used in this Subpart, means the Commission for spent nuclear fuel or high-level or transuranic wastes to be disposed of in facilities licensed by the Commission in accordance with the Energy Reorganization Act of 1974 and the Nuclear Waste Policy Act of 1982, and it means the Department for all other radioactive wastes covered by this Part.

§ 191.13 Containment requirements.

(a) Disposal systems for spent nuclear fuel or high-level or transuranic radioactive wastes shall be designed to provide a reasonable expectation, based upon performance assessments, that the cumulative releases of radionuclides to the accessible environment for 10,000 years after disposal from all significant processes and events that may affect the disposal system shall:

(1) Have a likelihood of less than one chance in 10 of exceeding the quantities calculated according to Table 1 (Appendix A); and

(2) Have a likelihood of less than one chance in 1,000 of exceeding ten times the quantities calculated according to Table 1 (Appendix A).

(b) Performance assessments need not provide complete assurance that the requirements of 191.13(a) will be met. Because of the long time period involved and the nature of the events and processes of interest, there will inevitably be substantial uncertainties in projecting disposal system performance. Proof of the future performance of a disposal system is not to be had in the ordinary sense of the word in situations that deal with much shorter time frames. Instead, what is required is a reasonable expectation, on the basis of the record before the implementing agency, that compliance with 191.13 (a) will be achieved.

§ 191.14 Assurance requirements.

To provide the confidence needed for long-term compliance with the requirements of 191.13, disposal of spent nuclear fuel or high-level or transuranic wastes shall be conducted in accordance with the following provisions, except that these provisions do not apply to facilities regulated by the Commission (see 10 CFR Part 60 for comparable provisions applicable to facilities regulated by the Commission):

(a) Active institutional controls over disposal sites should be maintained for as long a period of time as is practicable after disposal; however, performance assessments that assess isolation of the wastes from the accessible environment

shall not consider any contributions from active institutional controls for more than 100 years after disposal.

(b) Disposal systems shall be monitored after disposal to detect substantial and detrimental deviations from expected performance. This monitoring shall be done with techniques that do not jeopardize the isolation of the wastes and shall be conducted until there are no significant concerns to be addressed by further monitoring.

(c) Disposal sites shall be designated by the most permanent markers, records, and other passive institutional controls practicable to indicate the dangers of the wastes and their location.

(d) Disposal systems shall use different types of barriers to isolate the wastes from the accessible environment. Both engineered and natural barriers shall be included.

(e) Places where there has been mining for resources, or where there is a reasonable expectation of exploration for scarce or easily accessible resources, or where there is a significant concentration of any material that is not widely available from other sources, should be avoided in selecting disposal sites. Resources to be considered shall include minerals, petroleum or natural gas, valuable geologic formations, and ground waters that are either irreplaceable because there is no reasonable alternative source of drinking water available for substantial populations or that are vital to the preservation of unique and sensitive ecosystems. Such places shall not be used for disposal of the wastes covered by this Part unless the favorable characteristics of such places compensate for their greater likelihood of being disturbed in the future.

(f) Disposal systems shall be selected so that removal of most of the wastes is not precluded for a reasonable period of time after disposal.

§ 191.15 Individual protection requirements.

Disposal systems for spent nuclear fuel or high-level or transuranic radioactive wastes shall be designed to provide a reasonable expectation that: for 1,000 years after disposal, undisturbed performance of the disposal system shall not cause the annual dose equivalent from the disposal system to any member of the public in the accessible environment to exceed 25 millirems to the whole body or 75 millirems to any critical organ. All potential pathways (associated with undisturbed performance) from the disposal system to people shall be

considered, including the assumption that individuals consume 2 liters per day of drinking water from any significant source of ground water outside of the controlled area.

§ 191.16 Ground water protection requirements.

(a) Disposal systems for spent nuclear fuel or high-level or transuranic radioactive wastes shall be designed to provide a reasonable expectation that, for 1,000 years after disposal, undisturbed performance of the disposal system shall not cause the radionuclide concentrations averaged over any year in water withdrawn from any portion of a special source of ground water to exceed:

(1) 5 picocuries per liter of radium-226 and radium-228;

(2) 15 picocuries per liter of alpha-emitting radionuclides (including radium-226 and radium-228 but excluding radon); or

(3) The combined concentrations of radionuclides that emit either beta or gamma radiation that would produce an annual dose equivalent to the total body or any internal organ greater than 4 millirems per year if an individual consumed 2 liters per day of drinking water from such a source of ground water.

(b) If any of the average annual radionuclide concentrations existing in a special source of ground water before construction of the disposal system already exceed the limits in 191.16(a), the disposal system shall be designed to provide a reasonable expectation that, for 1,000 years after disposal, undisturbed performance of the disposal system shall not increase the existing average annual radionuclide concentrations in water withdrawn from that special source of ground water by more than the limits established in 191.16(a).

§ 191.17 Alternative provisions for disposal.

The Administrator may, by rule, substitute for any of the provisions of Subpart B alternative provisions chosen after:

(a) The alternative provisions have been proposed for public comment in the Federal Register together with information describing the costs, risks, and benefits of disposal in accordance with the alternative provisions and the reasons why compliance with the existing provisions of Subpart B appears inappropriate;

(b) A public comment period of at least 90 days has been completed, during which an opportunity for public hearings in affected areas of the country has been provided; and

(c) The public comments received have been fully considered in developing the final version of such alternative provisions.

§ 191.18 Effective date.

The standards in this Subpart shall be effective on September 19, 1985.

Appendix A—Table for Subpart B

TABLE 1.—RELEASE LIMITS FOR CONTAINMENT REQUIREMENTS

(Cumulative releases to the accessible environment for 10,000 years after disposal)

Radionuclide	Release limit per 1,000 MTHM or other unit of waste (see notes) (curies)
Americium-241 or -243	100
Carbon-14	100
Cesium-136 or -137	1,000
Iodine-129	100
Neptunium-237	100
Plutonium-238, -239, -240, or -242	100
Radium-226	100
Strontium-90	1,000
Technetium-99	10,000
Thorium-230 or -232	10
Tin-126	1,000
Uranium-233, -234, -235, -236, or -238	100
Any other alpha-emitting radionuclide with a half-life greater than 20 years	100
Any other radionuclide with a half-life greater than 20 years that does not emit alpha particles	1,000

Application of Table 1

Note 1: Units of Waste. The Release Limits in Table 1 apply to the amount of wastes in any one of the following:

(a) An amount of spent nuclear fuel containing 1,000 metric tons of heavy metal (MTHM) exposed to a burnup between 25,000 megawatt-days per metric ton of heavy metal (MWD/MTHM) and 40,000 MWD/MTHM;

(b) The high-level radioactive wastes generated from reprocessing each 1,000 MTHM exposed to a burnup between 25,000 MWD/MTHM and 40,000 MWD/MTHM;

(c) Each 100,000,000 curies of gamma or beta-emitting radionuclides with half-lives greater than 20 years but less than 100 years (for use as discussed in Note 5 or with materials that are identified by the Commission as high-level radioactive waste in accordance with part B of the definition of high-level waste in the NWPAA);

(d) Each 1,000,000 curies of other radionuclides (i.e., gamma or beta-emitters with half-lives greater than 100 years or any alpha-emitters with half-lives greater than 20 years) (for use as discussed in Note 5 or with materials that are identified by the

Commission as high-level radioactive waste in accordance with part B of the definition of high-level waste in the NWPAA); or

(e) An amount of transuranic (TRU) wastes containing one million curies of alpha-emitting transuranic radionuclides with half-lives greater than 20 years.

Note 2: Release Limits for Specific Disposal Systems. To develop Release Limits for a particular disposal system, the quantities in Table 1 shall be adjusted for the amount of waste included in the disposal system compared to the various units of waste defined in Note 1. For example:

(a) If a particular disposal system contained the high-level wastes from 50,000 MTHM, the Release Limits for that system would be the quantities in Table 1 multiplied by 50 (50,000 MTHM divided by 1,000 MTHM).

(b) If a particular disposal system contained three million curies of alpha-emitting transuranic wastes, the Release Limits for that system would be the quantities in Table 1 multiplied by three (three million curies divided by one million curies).

(c) If a particular disposal system contained both the high-level wastes from 50,000 MTHM and 5 million curies of alpha-emitting transuranic wastes, the Release Limits for that system would be the quantities in Table 1 multiplied by 53:

$$\frac{50,000 \text{ MTHM}}{1,000 \text{ MTHM}} + \frac{5,000,000 \text{ curies TRU}}{1,000,000 \text{ curies TRU}} = 53$$

Note 3: Adjustments for Reactor Fuels with Different Burnup. For disposal systems containing reactor fuels (or the high-level wastes from reactor fuels) exposed to an average burnup of less than 25,000 MWD/MTHM or greater than 40,000 MWD/MTHM, the units of waste defined in (a) and (b) of Note 1 shall be adjusted. The unit shall be multiplied by the ratio of 30,000 MWD/MTHM divided by the fuel's actual average burnup, except that a value of 5,000 MWD/MTHM may be used when the average fuel burnup is below 5,000 MWD/MTHM and a value of 100,000 MWD/MTHM shall be used when the average fuel burnup is above 100,000 MWD/MTHM. This adjusted unit of waste shall then be used in determining the Release Limits for the disposal system.

For example, if a particular disposal system contained only high-level wastes with an average burnup of 3,000 MWD/MTHM, the unit of waste for that disposal system would be:

$$1,000 \text{ MTHM} \times \frac{(30,000)}{(3,000)} = 10,000 \text{ MTHM}$$

If that disposal system contained the high-level wastes from 50,000 MTHM (with an average burnup of 3,000 MWD/MTHM), then

the Release Limits for that system would be the quantities in Table 1 multiplied by ten:

$$\frac{60,000 \text{ MTHM}}{6,000 \text{ MTHM}} = 10$$

which is the same as:

$$\frac{60,000 \text{ MTHM}}{1,000 \text{ MTHM}} \times \frac{(5,000 \text{ MWD/MTHM})}{(30,000 \text{ MWD/MTHM})} = 10$$

Note 4: Treatment of Fractionated High-Level Wastes. In some cases, a high-level waste stream from reprocessing spent nuclear fuel may have been (or will be) separated into two or more high-level waste components destined for different disposal systems. In such cases, the implementing agency may allocate the Release Limit multiplier (based upon the original MTHM and the average fuel burnup of the high-level waste stream) among the various disposal systems as it chooses, provided that the total Release Limit multiplier used for that waste stream at all of its disposal systems may not exceed the Release Limit multiplier that would be used if the entire waste stream were disposed of in one disposal system.

Note 5: Treatment of Wastes with Poorly Known Burnups or Original MTHM. In some cases, the records associated with particular high-level waste streams may not be adequate to accurately determine the original metric tons of heavy metal in the reactor fuel that created the waste, or to determine the average burnup that the fuel was exposed to. If the uncertainties are such that the original amount of heavy metal or the average fuel burnup for particular high-level waste streams cannot be quantified, the units of waste derived from (a) and (b) of Note 1 shall no longer be used. Instead, the units of waste defined in (c) and (d) of Note 1 shall be used for such high-level waste streams. If the uncertainties in such information allow a range of values to be associated with the original amount of heavy metal or the average fuel burnup, then the calculations described in previous Notes will be conducted using the values that result in the smallest Release Limits, except that the Release Limits need not be smaller than those that would be calculated using the units of waste defined in (c) and (d) of Note 1.

Note 6: Uses of Release Limits to Determine Compliance with 191.13 Once release limits for a particular disposal system have been determined in accordance with Notes 1 through 5, these release limits shall be used to determine compliance with the requirements of 191.13 as follows. In cases where a mixture of radionuclides is projected to be released to the accessible environment, the limiting values shall be determined as follows: For each radionuclide in the mixture, determine the ratio between the cumulative release quantity projected over 10,000 years and the limit for that radionuclide as determined from Table 1 and Notes 1 through 5. The sum of such ratios for all the radionuclides in the mixture may not exceed one with regard to 191.13(a)(1) and may not exceed ten with regard to 191.13(a)(2).

For example, if radionuclides A, B, and C are projected to be released in amounts Q_A , Q_B , and Q_C , and if the applicable Release Limits are RL_A , RL_B , and RL_C , then the cumulative releases over 10,000 years shall be limited so that the following relationship exists:

$$\frac{Q_A}{RL_A} + \frac{Q_B}{RL_B} + \frac{Q_C}{RL_C} < 1$$

Appendix B—Guidance for Implementation of Subpart B

[Note: The supplemental information in this appendix is not an integral part of 40 CFR Part 191. Therefore, the implementing agencies are not bound to follow this guidance. However, it is included because it describes the Agency's assumptions regarding the implementation of Subpart B. This appendix will appear in the Code of Federal Regulations.]

The Agency believes that the implementing agencies must determine compliance with §§ 191.13, 191.15, and 191.16 of Subpart B by evaluating long-term predictions of disposal system performance. Determining compliance with § 191.13 will also involve predicting the likelihood of events and processes that may disturb the disposal system. In making these various predictions, it will be appropriate for the implementing agencies to make use of rather complex computational models, analytical theories, and prevalent expert judgment relevant to the numerical predictions. Substantial uncertainties are likely to be encountered in making these predictions. In fact, sole reliance on these numerical predictions to determine compliance may not be appropriate; the implementing agencies may choose to supplement such predictions with qualitative judgments as well. Because the procedures for determining compliance with Subpart B have not been formulated and tested yet, this appendix to the rule indicates the Agency's assumptions regarding certain issues that may arise when implementing §§ 191.13, 191.15, and 191.16. Most of this guidance applies to any type of disposal system for the wastes covered by this rule. However, several sections apply only to disposal in mined geologic repositories and would be inappropriate for other types of disposal systems.

Consideration of Total Disposal System. When predicting disposal system performance, the Agency assumes that reasonable projections of the protection expected from all of the engineered and natural barriers of a disposal system will be considered. Portions of the disposal system should not be disregarded, even if projected performance is uncertain, except for portions of the system that make negligible contributions to the overall isolation provided by the disposal system.

Scope of Performance Assessments. Section 191.13 requires the implementing agencies to evaluate compliance through performance assessments as defined in § 191.12(q). The Agency assumes that such performance assessments need not consider

categories of events or processes that are estimated to have less than one chance in 10,000 of occurring over 10,000 years. Furthermore, the performance assessments need not evaluate in detail the releases from all events and processes estimated to have a greater likelihood of occurrence. Some of these events and processes may be omitted from the performance assessments if there is a reasonable expectation that the remaining probability distribution of cumulative releases would not be significantly changed by such omissions.

Compliance with Section 191.13. The Agency assumes that, whenever practicable, the implementing agency will assemble all of the results of the performance assessments to determine compliance with § 191.13 into a "complementary cumulative distribution function" that indicates the probability of exceeding various levels of cumulative release. When the uncertainties in parameters are considered in a performance assessment, the effects of the uncertainties considered can be incorporated into a single such distribution function for each disposal system considered. The Agency assumes that a disposal system can be considered to be in compliance with § 191.13 if this single distribution function meets the requirements of § 191.13(a).

Compliance with Sections 191.15 and 191.16. When the uncertainties in undisturbed performance of a disposal system are considered, the implementing agencies need not require that a very large percentage of the range of estimated radiation exposures or radionuclide concentrations fall below limits established in §§ 191.15 and 191.16, respectively. The Agency assumes that compliance can be determined based upon "best estimate" predictions (e.g., the mean or the median of the appropriate distribution, whichever is higher).

Institutional Controls. To comply with § 191.14(a), the implementing agency will assume that none of the active institutional controls prevent or reduce radionuclide releases for more than 100 years after disposal. However, the Federal Government is committed to retaining ownership of all disposal sites for spent nuclear fuel and high-level and transuranic radioactive wastes and will establish appropriate markers and records, consistent with § 191.14(c). The Agency assumes that, as long as such passive institutional controls endure and are understood, they: (1) can be effective in deterring systematic or persistent exploitation of these disposal sites; and (2) can reduce the likelihood of inadvertent, intermittent human intrusion to a degree to be determined by the implementing agency. However, the Agency believes that passive institutional controls can never be assumed to eliminate the chance of inadvertent and intermittent human intrusion into these disposal sites.

Consideration of Inadvertent Human Intrusion into Geologic Repositories. The most speculative potential disruptions of a mined geologic repository are those associated with inadvertent human intrusion. Some types of intrusion would have virtually no effect on a repository's containment of

waste. On the other hand, it is possible to conceive of intrusions (involving widespread societal loss of knowledge regarding radioactive wastes) that could result in major disruptions that no reasonable repository selection or design precautions could alleviate. The Agency believes that the most productive consideration of inadvertent intrusion concerns those realistic possibilities that may be usefully mitigated by repository design, site selection, or use of passive controls (although passive institutional controls should not be assumed to completely rule out the possibility of intrusion). Therefore, inadvertent and intermittent intrusion by exploratory drilling for resources (other than any provided by the disposal system itself) can be the most severe intrusion scenario assumed by the implementing agencies. Furthermore, the implementing agencies can assume that

passive institutional controls or the intruders' own exploratory procedures are adequate for the intruders to soon detect, or be warned of, the incompatibility of the area with their activities.

Frequency and Severity of Inadvertent Human Intrusion into Geologic Repositories. The implementing agencies should consider the effects of each particular disposal system's site, design, and passive institutional controls in judging the likelihood and consequences of such inadvertent exploratory drilling. However, the Agency assumes that the likelihood of such inadvertent and intermittent drilling need not be taken to be greater than 30 boreholes per square kilometer of repository area per 10,000 years for geologic repositories in proximity to sedimentary rock formations, or more than 8 boreholes per square kilometer per 10,000 years for repositories in other geologic

formations. Furthermore, the Agency assumes that the consequences of such inadvertent drilling need not be assumed to be more severe than: (1) Direct release to the land surface of all the ground water in the repository horizon that would promptly flow through the newly created borehole to the surface due to natural lithostatic pressure—or (if pumping would be required to raise water to the surface) release of 200 cubic meters of ground water pumped to the surface if that much water is readily available to be pumped; and (2) creation of a ground water flow path with a permeability typical of a borehole filled by the soil or gravel that would normally settle into an open hole over time—not the permeability of a carefully sealed borehole.

[FR Doc. 85-20331 Filed 9-18-85; 6:45 am]
BILLING CODE 4540-60-M

ENCLOSURE C

DRAFT CONGRESSIONAL LETTER

Dear Mr. Chairman:

Enclosed for your information is a copy of a notice of proposed rulemaking to be published in the Federal Register.

The Nuclear Waste Policy Act of 1982 directs the Commission to revise its regulations for licensing the disposal of high-level radioactive waste (HLW) in geologic repositories, 10 CFR Part 60, to eliminate inconsistencies with the Environmental Protection Agency's standard for HLW disposal. The standard, 40 CFR Part 191, was published on September 19, 1985 (50 FR 38066). The Commission has identified several areas in Part 60 which will require revision to eliminate inconsistencies with the standard. The proposed rulemaking would make the necessary revisions.

Robert B. Minogue, Director
Office of Nuclear Regulatory Research

Enclosure: As stated

ENCLOSURE D



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

Cys: Dircks
Roe
Rehm
Stello
GCunningham
Denton
Kerr, SP
Fehring, NMSS
(Prichard, RES
Philips

November 27, 1985

MEMORANDUM FOR: William J. Dircks
Executive Director for Operations

FROM: Samuel J. Chilk, Secretary

SUBJECT: STAFF REQUIREMENTS - NOTATION VOTE ON
SECY-85-272 - REPORT ON THE ENVIRONMENTAL
PROTECTION AGENCY'S ENVIRONMENTAL
STANDARDS FOR HIGH-LEVEL RADIOACTIVE WASTE
DISPOSAL

On September 19, 1985, the Commission (with all Commissioners agreeing) approved the proposed letter to EPA, as attached. Immediately following Commission approval, the ACRS requested that this matter be discussed with the Committee. On October 21, 1985, the Commission met with the staff, ACRS and others to discuss conflicting views.

Upon due consideration of the concerns expressed by the ACRS and the responses by the staff, the Commission reaffirmed releasing the letter to EPA.

The letter has been forwarded to the Chairman for his signature.

In addition, EDO is directed to submit to the Commission the rulemaking package which conforms 10 CFR Part 60 with the EPA Standard. The Commission also stresses the importance for the staff to clearly articulate, in the changes to Part 60, how we interpret the EPA's Standards and that the ACRS' concerns be addressed by clearly defining the basis for the assurance that adequate flexibility exists in the standards for their implementation. In particular, care should be taken to avoid any ambiguity in the application of probabilistic conditions placed on the post-closure containment requirements. (RES)

(EDO Suspense: 2/15/86)

Rec'd Off. EDO
Date... 11-29-85
Time... 1:55 P.M.

ENCLOSURE D

The Commission also agrees that the staff and the ACRS should interact with each other early in the process of developing the package on 10 CFR Part 60 as well as in future reviews of NRC activities under the NWPA so that valuable technical advice and input can be used in a timely manner by the Commission.

Chairman Palladino requested, in line with ACRS comments, that EDO accelerate its efforts to develop analytical methods to be used in making a determination that a licensee is complying with the EPA Standards. These methods should receive as broad an input and review as possible. (NMSS)

Attachment:
As stated

cc: Chairman Palladino
Commissioner Roberts
Commissioner Asselstine
Commissioner Bernthal
Commissioner Zech
OGC
OPE
ACRS



CHAIRMAN

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

The Honorable Lee Thomas
Administrator
U.S. Environmental Protection Agency
Washington, D.C. 20460

Dear Mr. Thomas:

On May 10 and 11, 1982 the Nuclear Regulatory Commission (NRC) submitted formal comments on the Environmental Protection Agency's proposed environmental standards for management and disposal of high-level radioactive wastes. Among other things, we stated our view that the proposed "assurance requirements" and "procedural requirements" contained in those proposed standards involved matters of implementation and thus went beyond the limits of EPA's jurisdiction.

In letters dated July 19 and August 15, 1984 Acting Chairman Roberts and Former Administrator Ruckelshaus, respectively, agreed that the staffs of EPA and NRC should attempt to develop modifications to 10 CFR Part 60 to incorporate the principles of EPA's proposed assurance and procedural requirements. EPA could then delete these requirements or make them applicable only to facilities not licensed by the NRC, eliminating any potential problems of jurisdictional overlap.

The NRC staff recently reported to the Commission several proposed changes to Part 60 which have been worked out by the NRC and EPA staff (text enclosed). Consistent with the provisions of the Administrative Procedure Act, the Commission will propose these changes for incorporation into Part 60 now that the final EPA high-level waste standards have been published. The NRC staff anticipates submittal of a rulemaking package, incorporating both these wording changes and other conforming amendments, to the Commission within 120 days.

The Commission appreciates the cooperation shown by the EPA staff in working to reach this agreement.

Sincerely,

Nunzio J. Palladino

Enclosure:
Proposed changes to
10 CFR Part 60

EPA ASSURANCE REQUIREMENTS AND
PROPOSED CHANGES TO PART 60

1.a. EPA Assurance Requirement:

(a) Active institutional controls over disposal sites should be maintained for as long a period of time as is practicable after disposal; however, performance assessments that assess isolation of the wastes from the accessible environment shall not consider any contributions from active institutional controls for more than 100 years after disposal.

(In Working Draft No. 8 "active institutional control" means: (1) controlling access to a disposal site by any means other than passive institutional controls, (2) performing maintenance operations or remedial actions at a site, (3) controlling or cleaning up releases from a site, or (4) monitoring parameters related to disposal system performance.)

b. Discussion:

The Commission's existing provisions (§60.52) related to license termination will determine the length of time for which institutional controls should be maintained, and there is therefore no need to alter Part 60 based on the first part of this assurance requirement.

The second part of this assurance requirement would require that "active" institutional controls be excluded from consideration (after 100 years) when the Commission assesses the isolation characteristics of a repository. The NRC staff understands that remedial actions (or other active institutional controls) would not be relied upon under Part 60 to compensate for a poor site or inadequate engineered barriers. However, in the definition of "unanticipated events and processes," Part 60 expressly contemplates that, in assessing human intrusion scenarios, the Commission would assume that "institutions are able to assess risk and to take remedial action at a level of social organization and technological competence equivalent to, or superior to, that which was applied in initiating the processes or events concerned" (emphasis added). Therefore, it might appear at first blush that Part 60 is at odds with the draft EPA standards.

2.a. EPA Assurance Requirement:

(b) Disposal systems shall be monitored after disposal to detect any substantial and detrimental deviations from expected performance. This monitoring shall be done with techniques that do not jeopardize the isolation of the wastes and shall be conducted until there are no significant concerns to be addressed by further monitoring.

b. Discussion:

Part 60 currently requires completion of a performance confirmation program prior to repository closure, but does not require monitoring during the period following closure but prior to license termination. The Commission chose not to require post-closure monitoring because of doubts about the usefulness of such monitoring and because of fears that monitoring in or near a repository after closure could degrade repository performance. The type of monitoring envisioned by EPA does not involve direct monitoring of the repository itself (which might degrade repository performance). Rather, EPA proposes monitoring of such parameters as regional groundwater flow characteristics. The NRC agrees that such monitoring may, in some cases, provide desirable information beyond that which would be obtained in the performance confirmation program which Part 60 now requires to be continued until permanent closure. The NRC therefore proposes to require monitoring as an extension of performance confirmation, as appropriate, when such monitoring can be conducted without degrading repository performance.

c. Proposed Changes to Part 60:

Add to §60.21(c) a new ¶ (9) as follows:

(9) A general description of the program for post-permanent closure monitoring of the geologic repository.

Renumber the current ¶ (9) through (15) accordingly.

Revise §60.51(a)(1) to read:

(1) A detailed description of the program for post-permanent closure monitoring of the geologic repository in accordance with §60.144. As a minimum, this description shall:

- (i) identify those parameters that will be monitored;
- (ii) indicate how each parameter will be used to evaluate the expected performance of the repository; and
- (iii) discuss the length of time over which each parameter should be monitored to adequately confirm the expected performance of the repository.

3.a. EPA Assurance Requirement:

(c) Disposal sites shall be designated by the most permanent markers, records, and other passive institutional controls practicable to indicate the dangers of the wastes and their location.

b. Discussion:

No revisions to Part 60 are needed. §60.21(c)(8), 60.51(a)(2), and 60.121 contain equivalent provisions.

5.a. EPA Assurance Requirement:

(e) Places where there has been mining for resources, or where there is a reasonable expectation of exploration for scarce or easily accessible resources, or where there is a significant concentration of any material that is not widely available from other sources, should be avoided in selecting disposal sites. Resources to be considered shall include minerals, petroleum or natural gas, valuable geologic formations, and ground waters that are either irreplaceable because there is no reasonable alternative source of drinking water available for substantial populations or that are vital to the preservation of unique and sensitive ecosystems. Such places shall not be used for disposal of the wastes covered by this Part unless the favorable characteristics of such places compensate for their greater likelihood of being disturbed in the future.

b. Discussion:

Part 60 contains provisions equivalent to this assurance requirement in §60.122(c)(17), (18) and (19). Part 60 does not, however, address "a significant concentration of any material that is not widely available from other sources."

It is possible that the economic value of materials could change in the future in a way which might attract future exploration or development detrimental to repository performance. The NRC proposes to add an additional potentially adverse condition to Part 60 related to significant concentrations of material that is not widely available from other sources. As with the other potentially adverse conditions, the presence of such a condition would require an evaluation of the effect of the condition on repository performance as specified in §60.122(a)(2)(ii), but would not preclude selection of a site for repository construction. (It should be noted that DOE's siting guidelines contain an identical provision in 10 CFR 960.4-2-8-1.)

c. Proposed Changes to Part 60:

Add a new § (18) to §60.122(c) as follows:

(18) The presence of significant concentrations of any naturally-occurring material that is not widely available from other sources.

Renumber the current § (18) through (21) accordingly.

ENCLOSURE E

REGULATORY ANALYSIS

STATEMENT OF THE PROBLEM

The Nuclear Waste Policy Act of 1982, Public Law 97-425 (NWPA) contains provisions requiring EPA to promulgate standards for the protection of public health and safety to apply to geologic repositories for HLW. Section 121 of the statute requires NRC standards or criteria for licensing geologic repositories (10 CFR 60) to "not be inconsistent" with the EPA standards. NRC promulgated 10 CFR 60 on February 25, 1981 (46 FR 13971), and final technical criteria against which license applications would be reviewed under 10 CFR 60 were promulgated on June 21, 1983 (48 FR 28194). The NWPA specifically provided for NRC to promulgate Part 60 before the EPA standards were issued. However, the law directs NRC to revise its requirements and criteria to eliminate inconsistency in the event that the EPA standards are promulgated after the promulgation of 10 CFR 60. The final EPA standard 10 -- CFR 191 -- was promulgated on September 19, 1985 (50 FR 38066).

OBJECTIVES

The objective of the proposed regulatory action is to eliminate inconsistencies between NRC regulations covering HLW geologic repositories and the EPA standard for releases from HLW geologic repositories. This action will facilitate the process of licensing HLW geologic repositories as the licensing process can take place within a consistent overall framework of standards and regulations

ALTERNATIVES

The Nuclear Waste Policy Act of 1982 specifically directs NRC to eliminate any inconsistencies between 10 CFR Part 60 and the EPA Standard so the alternatives to the proposed action are limited by statute.

- (1) Leave the provisions of 10 CFR Part 60 intact.

" ENCLOSURE

CONSEQUENCE

Alternative Action (1)

The staff's analysis of the EPA standard and the existing 10 CFR Part 60 has concluded that there are inconsistencies in several areas. The major areas of inconsistency are in; (a) definitions - particularly definitions of controlled area, and the accessible environment, (b) protection of special sources of groundwater, (c) the concept of "reasonable assurance," (d) institutional control, (e) the presence of significant concentrations of material not widely available from other sources as a siting criterion, (f) post-closure monitoring, and (g) performance of particular barriers following permanent closure.

The staff believes that to allow these inconsistencies to persist by leaving the provisions of 10 CFR Part 60 intact would create the potential for uncertainties in the licensing process for HLW geologic repositories. This would impede the U.S. program for disposal of HLW in geologic repositories.

Proposed Action

10 CFR Part 60 will be amended to revise provisions which are not consistent with the EPA HLW standard. The result will be a consistent framework for licensing HLW geologic repositories. This will make the licensing process more efficient, resulting in savings to DOE, NRC, utilities and their ratepayers, and the general public.

ENCLOSURE F

**NRC PROPOSES TO AMEND REGULATION
ON LICENSING OF HIGH-LEVEL RADIOACTIVE WASTE REPOSITORY**

The Nuclear Regulatory Commission is proposing to amend its requirements which will govern the construction and operation by the Department of Energy of geologic repositories for high-level radioactive wastes.

As proposed, and as required by the Nuclear Waste Policy Act of 1982, the amendments would incorporate the Environmental Protection Agency's recently-published "generally applicable standards for protection of the general environment from offsite releases from radioactive materials in repositories".

Four sections of the EPA standards proposed for inclusion in the NRC's requirements contain numerical guidance applicable to:

- 1) repository operations;
- 2) protection of individuals and groundwater for the first 1,000 years after repository closure; and
- 3) containment requirements restricting the total amount of radioactivity released from a repository for 10,000 years following closure.

In addition, the Commission is proposing to incorporate directly into its regulation the substantive requirements of other parts of the EPA's environmental standards--modified, as necessary, to conform with the terminology currently used by the NRC. These changes would deal, among other things, with:

- 1) definitions of terms;
- 2) contents of license application;
- 3) amendment of a license to permit repository closure;
- 4) termination of a license;
- 5) purpose and nature of findings necessary to assure compliance with EPA and NRC requirements;
- 6) overall performance objectives for a geologic repository after permanent closure;
- 7) performance of particular barriers to prevent the release of radioactive material after permanent closure of a repository;
- 8) institutional controls for radiological protection;
- 9) siting criteria governing the presence of economically, or potentially economically, valuable minerals;
- 10) monitoring of a repository after permanent closure.

SECRET

Written comments on the proposed amendments to Part 60 of the NRC's regulations should be received by (date). They should be addressed to the Secretary of the Commission, Nuclear Regulatory Commission, Washington, D. C. 20585; Attention: Docketing and Service Branch.

#

ENCLOSURE G

[COMPARATIVE TEXT]

PART 60 -- DISPOSAL OF HIGH-LEVEL RADIOACTIVE

WASTES IN GEOLOGIC REPOSITORIES

1. The authority citation for Part 60 continues to read as follows:
 Secs. 51, 53, 62, 63, 65, 81, 161, 182, 183, 68 Stat. 929, 930, 932, 933, 935,
 948, 953, 954, as amended (42 U.S.C. 2071, 2073, 2092, 2093, 2095, 2111, 2201,
 2232, 2233); secs. 202, 206, 88 Stat. 1244, 1246 (42 U.S.C. 5842, 5846); secs.
 10 and 14, Pub. L. 95-601, 92 Stat. 2951 (42 U.S.C. 2021a and 5851); sec. 102,
 Pub. L. 91-190, 83 Stat. 853 (42 U.S.C. 4332); sec. 121, Pub. L. 97-425, 96
 Stat. 2228 (42 U.S.C. 10141).

For the purposes of sec. 223, 68 Stat. 958, as amended (42 U.S.C. 2273),
 §§60.71 to 60.75 are issued under sec. 161o, 68 Stat. 950, as amended (42
 U.S.C. 2201(o)).

2. Section 60.2 is amended by revising the definitions of "accessible
 environment" and "controlled area" and by adding seven new definitions in
 alphabetical order as follows:

§60.2 Definitions.

* * * * *

"Accessible environment" means: (1) [~~the~~]the atmosphere, (2) [~~the~~] land
 surfaces, (3) surface waters, (4) oceans, and (5) [~~the-portion~~] all of the
 lithosphere that is [~~outside~~] beyond the controlled area.

* * * * *

"Active institutional control" means: (1) controlling access to a disposal site by any means other than passive institutional control, (2) performing maintenance operations or remedial actions at a site, (3) controlling or cleaning up releases from a site, or (4) monitoring parameters related to disposal system performance.

* * * * *

"Community water system" means a system for the provision to the public of piped water for human consumption, if such system has at least 15 service connections used by year-round residents or regularly serves at least 25 year-round residents.

* * * * *

"Controlled area" [means-a-surface-location,-to-be-marked-by-suitable monuments,-extending-horizontally-no-more-than-10-kilometers-in-any-direction from-the-outer-boundary-of-the-underground-facility,-and-the-underlying subsurface,-which-area-has-been-committed-to-use-as-a-geologic-repository-and from-which-incompatible-activities-would-be-restricted-following-permanent closure.] means: (1) a surface location, to be identified by passive institutional controls, that encompasses no more than 100 square kilometers and extends horizontally no more than five kilometers in any direction from the outer boundary of the underground facility, and (2) the subsurface underlying such a surface location.

* * * * *

"Passive institutional control" means: (1) permanent markers placed at a disposal site, (2) public records and archives, (3) government ownership and regulations regarding land or resource use, and (4) other methods of preserving knowledge about the location, design, and contents of a disposal system.

* * * * *

"Significant source of groundwater" means: (1) an aquifer that: (i) is saturated with water having less than 10,000 milligrams per liter of total dissolved solids; (ii) is within 2,500 feet of the land surface; (iii) has a transmissivity greater than 200 gallons per day per foot, provided that any formation or part of a formation included within the source of groundwater has a hydraulic conductivity greater than 2 gallons per day per square foot; and (iv) is capable of continuously yielding at least 10,000 gallons per day to a pumped or flowing well for a period of at least a year; or (2) an aquifer that provides the primary source of water for a community water system as of November 18, 1985.

* * * * *

"Special source of groundwater" means those Class I groundwaters identified in accordance with the Environmental Protection Agency's Ground-Water Protection Strategy published in August 1984 that: (1) are within the controlled area encompassing a disposal system or are less than five kilometers beyond the controlled area; (2) are supplying drinking water for thousands of persons as of the date that the Department chooses a location within that area for detailed characterization as a potential site for a disposal system (e.g., in accordance with Section 112(b)(1)(B) of the NHPA); and (3) are irreplaceable in that no reasonable alternative source of drinking water is available to that population.

* * * * *

"Transmissivity" means the hydraulic conductivity integrated over the saturated thickness of an underground formation. The transmissivity of a series of formations is the sum of the individual transmissivities of each formation comprising the series.

* * * * *

"Uranium fuel cycle" means the operations of milling of uranium ore, chemical conversion of uranium, isotopic enrichment of uranium, fabrication of uranium fuel, generation of electricity by a light-water-cooled nuclear power plant using uranium fuel, and reprocessing of spent uranium fuel, to the extent that these directly support the production of electrical power for public use utilizing nuclear energy, but excludes mining operations, operations at waste disposal sites, transportation of any radioactive material in support of these operations, and the reuse of recovered non-uranium special nuclear and by-product materials from the cycle.

* * * * *

3. Section 60.21 is amended by revising paragraph (c)(1)(ii)(C), adding a new paragraph (c)(9) and redesignating the existing paragraphs (c)(9) through (c)(15) as paragraphs (c)(10) through (c)(16).

§60.21 Content of application.

* * * * *

- (c) * * *
- (1) * * *
- (ii) * * *

(C) An evaluation of the performance of the proposed geologic repository for the period after permanent closure, assuming anticipated processes and events, giving the rates and quantities of releases of radionuclides to the accessible environment as a function of time; and a similar evaluation which assumes the occurrence of unanticipated processes and events. In making such evaluations, estimated values shall be incorporated into an overall probability distribution of cumulative release to the extent practicable.

* * * * *

(9) A general description of the program for post-permanent closure monitoring of the geologic repository.

* * * * *

4. Section 60.51 is amended by revising paragraph (a)(1) to read as follows:

§60.51 License amendment for permanent closure.

(a) * * *

(1) A detailed description of the program for post-permanent closure monitoring of the geologic repository[] in accordance with §60.144. As a minimum, this description shall:

(i) identify those parameters that will be monitored;

(ii) indicate how each parameter will be used to evaluate the expected performance of the repository; and

(iii) discuss the length of time over which each parameter should be monitored to adequately confirm the expected performance of the repository.

* * * * *

5. Section 60.52 is amended by designating current paragraph (c)(3) as paragraph (c)(4) and by adding a new paragraph (c)(3) as follows:

§60.52 Termination of license.

* * * * *

(c) * * *

(3) That the results available from the post-permanent closure monitoring program confirm the expectation that the repository will comply with the performance objectives set out at §60.112 and §60.113; and

* * * * *

6. Section 60.101 is amended by revising paragraph (a)(2) to read as follows:

§60.101 Purpose and nature of findings.

(a) * * *

(2) While these performance objectives and criteria are generally stated in unqualified terms, it is not expected that complete assurance that they will be met can be presented. A reasonable assurance, on the basis of the record before the Commission, that the objectives and criteria will be met is the general standard that is required. For §60.112, and other portions of this subpart that impose objectives and criteria for repository performance over long times into the future, there will inevitably be greater uncertainties. Proof of the future performance of engineered barrier systems and the geologic setting over time periods of many hundreds or many thousands of years is not to be had in the ordinary sense of the word. For such long-term objectives and criteria, what is required is reasonable assurance, making allowance for the time period, hazards, and uncertainties involved, that the outcome will be in conformance with those objectives and criteria. Demonstration of compliance with such objectives and criteria will involve the use of data from accelerated tests and predictive models that are supported by such measures as field and laboratory tests, monitoring data and natural analog studies. Demonstration of compliance with the performance objectives of §60.112 will also involve predicting the likelihood and consequences of events and processes that may disturb the repository. Such predictions may involve complex computational models, analytical theories and prevalent expert judgment. Substantial uncertainties are likely to be encountered and sole reliance on numerical predictions to determine compliance may not be appropriate. In reaching a determination of reasonable assurance, the Commission may supplement numerical analyses with qualitative judgments including, for example, consideration of the degree of diversity or redundancy among the multiple barriers of a specific repository.

* * * * *

7. In section 60.111, paragraph (a) is revised to read as follows:

§60.111 Performance of the geologic repository operations area through permanent closure.

(a) Protection against radiation exposures and releases of radioactive material. The geologic repository operations area shall be designed so that until permanent closure has been completed [~~radiation exposures and radiation levels, and releases of radioactive materials to unrestricted areas, will at all times be maintained within the limits specified in Part 20 of this chapter and such generally applicable environmental standards for radioactivity as may have been established by the Environmental Protection Agency.~~] :

(1) The annual dose equivalent to any member of the public outside the geologic repository operations area, resulting from the combination of (i) discharges of radioactive material and direct radiation from activities at the geologic repository operations area and (ii) uranium fuel cycle operations, shall not exceed 25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other critical organ.

(2) Radiation exposures and radiation levels, and releases of radioactive materials to unrestricted areas, will at all times, including the retrievability period of §60.111(b), be maintained within the limits specified in Part 20 of this chapter.

*

*

*

*

*

8. Section 60.112 is revised to read as follows:

§60.112 Overall system performance objective for the geologic repository after permanent closure.

The geologic setting shall be selected and the engineered barrier system and the shafts, boreholes and their seals shall be designed [~~to assure that releases of radioactive materials to the accessible environment following permanent closure conform to such generally applicable environmental standards as may have been established by the Environmental Protection Agency with respect to both anticipated processes and events and unanticipated processes and events.~~] :

(a) So that, for 10,000 years following permanent closure, cumulative releases of radionuclides to the accessible environment, from all anticipated and unanticipated processes and events, shall:

(1) Have a likelihood of less than one chance in 10 of exceeding the quantities calculated in accordance with §60.115.

(2) Have a likelihood of less than one chance in 1,000 of exceeding ten times the quantities calculated in accordance with §60.115.

(b) So that for 1,000 years after permanent closure, and in the absence of unanticipated processes and events, the annual dose equivalent to any member of the public in the accessible environment does not exceed 25 millirems to the whole body or 75 millirems to any critical organ. For the purpose of applying this paragraph, all potential pathways from the geologic repository to people shall be considered, including the assumption that individuals consume 2 liters per day of drinking water from any significant source of groundwater outside of the controlled area.

(c) So that for 1,000 years after permanent closure, and in the absence of unanticipated processes and events:

(1) Except as provided in paragraph (c)(2) of this section, the radionuclide concentrations averaged over any year in water withdrawn from any portion of a special source of groundwater do not exceed:

(i) 5 picocuries per liter of radium-226 and radium-228;

(ii) 15 picocuries per liter of alpha-emitting radionuclides (including radium-226 and radium-228 but excluding radon); or

(iii) the combined concentrations of radionuclides that emit either beta or gamma radiation that would produce an annual dose equivalent to the total body or any internal organ greater than 4 millirems per year if an individual consumed 2 liters per day of drinking water from such a source of groundwater.

(2) If any of the average annual radionuclide concentrations existing in a special source of groundwater before construction of the geologic repository operations area already exceed the limits in paragraph (c)(1) of this section, the increase, caused by the geologic repository, in the existing average annual radionuclide concentrations in water withdrawn from that special source of groundwater does not exceed the limits specified in paragraph (c)(1) of this section.

9. In section 60.113, paragraph (b)(1) is revised and a new paragraph (d) is added to read as follows:

§60.113 Performance of particular barriers after permanent closure.

* * * * *

(b) * * *

(1) [~~Any generally applicable environmental standard for radioactivity established by the Environmental Protection Agency.~~] The overall system performance objectives of §60.112.

* * * * *

(d) Notwithstanding the provisions of paragraph (b) of this section, the geologic repository shall incorporate a system of multiple barriers, both engineered and natural.

10. A new §60.114 is added to read as follows:

§60.114 Institutional control.

Neither active nor passive institutional control shall be deemed to assure compliance with the overall system performance objectives set out at §60.112 for more than 100 years after permanent closure. However, the effects of institutional control may be considered in assessing, for purposes of that section, the likelihood and consequences of processes and events affecting the geologic setting.

11. A new §60.115 is added to read as follows:

§60.115 Release limits for overall system performance objective.

The following table shall be used to make the calculations referred to in paragraph (a) of §60.112.

TABLE 1 --RELEASE LIMITS FOR OVERALL SYSTEM PERFORMANCE OBJECTIVE
(Cumulative Releases to the Accessible Environment
for 10,000 Years After Disposal)

<u>Radionuclide</u>	<u>Release Limit per</u> <u>1000 MTHM or other unit</u> <u>of waste (see Notes)</u> <u>(curies)</u>
Americium-241 or 243 - - - - -	100
Carbon-14 - - - - -	100
Cesium-135 or 137 - - - - -	1000
Iodine-129 - - - - -	100
Neptunium-237 - - - - -	100
Plutonium-238, 239, 240 or 242 - - - - -	100
Radium-226 - - - - -	100
Strontium-90 - - - - -	1000
Technetium-99 - - - - -	10000
Thorium-230 or 232 - - - - -	10
Tin-126 - - - - -	1000
Uranium-233, 234, 235, 236 or 238 - - - - -	100
<u>Any other alpha-emitting radionuclide</u>	
<u>with a half-life greater than 20 years - - - - -</u>	<u>100</u>
<u>Any other radionuclide with a half-life greater</u>	
<u>than 20 years that does not emit alpha particles - - - -</u>	<u>1000</u>

Application of Table 1

NOTE 1: Units of Waste. The Release Limits in Table 1 apply to the amount of wastes in any one of the following:

(a) an amount of spent nuclear fuel containing 1,000 metric tons of heavy metal (MTHM) exposed to a burnup between 25,000 megawatt-days per metric ton of heavy metal (Mwd/MTHM) and 40,000 Mwd/MTHM;

(b) the high-level radioactive wastes generated from reprocessing each 1,000 MTHM exposed to a burnup between 25,000 Mwd/MTHM and 40,000 Mwd/MTHM;

(c) each 100,000,000 curies of gamma or beta-emitting radionuclides with half-lives greater than 20 years but less than 100 years (for use as discussed in Note 5 or with materials that are identified by the Commission as high-level radioactive waste in accordance with part (B) of the definition of high-level waste in the Nuclear Waste Policy Act (NWPA));

(d) each 1,000,000 curies of other radionuclides (i.e., gamma or beta-emitters with half-lives greater than 100 years or any alpha-emitters with half-lives greater than 20 years) (for use as discussed in Note 5 or with materials that are identified by the Commission as high-level waste in accordance with part (B) of the definition of high-level waste in the NWPA); or

(e) an amount of transuranic (TRU) wastes containing one million curies of alpha-emitting transuranic radionuclides with half-lives greater than 20 years.

NOTE 2: Release Limits for Specific Disposal Systems. To develop Release Limits for a particular disposal system, the quantities in Table 1 shall be adjusted for the amount of waste included in the disposal system compared to the various units of waste defined in Note 1. For example:

(a) If a particular disposal system contained the high-level wastes from 50,000 MTHM, the Release Limits for that system would be the quantities in Table 1 multiplied by 50 (50,000 MTHM divided by 1,000 MTHM).

(b) If a particular disposal system contained three million curies of alpha-emitting transuranic wastes, the Release Limits for that system would be the quantities in Table 1 multiplied by three (three million curies divided by one million curies).

(c) If a particular disposal system contained both the high-level wastes from 50,000 MTHM and 5 million curies of alpha-emitting transuranic wastes, the Release Limits for that system would be the quantities in Table 1 multiplied by 55:

$$\begin{array}{r} 50,000 \text{ MTHM} \\ \hline 1,000 \text{ MTHM} \end{array} + \begin{array}{r} 5,000,000 \text{ curies TRU} \\ \hline 1,000,000 \text{ curies TRU} \end{array} = 55$$

NOTE 3: Adjustments for Reactor Fuels with Different Burnup. For disposal systems containing reactor fuels (or the high-level wastes from reactor fuels) exposed to an average burnup of less than 25,000 MWd/MTHM or greater than 40,000 MWd/MTHM, the units of waste defined in (a) and (b) of Note 1 shall be adjusted. The unit shall be multiplied by the ratio of 30,000 MWd/MTHM divided by the fuel's actual average burnup, except that a value of 5,000 MWd/MTHM may be used when the average fuel burnup is below 5,000 MWd/MTHM and a value of 100,000 MWd/MTHM shall be used when the average fuel burnup is above 100,000 MWd/MTHM. This adjusted unit of waste shall then be used in determining the Release Limits for the disposal system.

For example, if a particular disposal system contained only high-level wastes with an average burnup of 3,000 MWd/MTHM, the unit of waste for that

disposal system would be:

$$1,000 \text{ MTHM} \times \frac{(30,000 \text{ MWd/MTHM})}{(5,000 \text{ MWd/MTHM})} = 6,000 \text{ MTHM}$$

If that disposal system contained the high-level wastes from 60,000 MTHM (with an average burnup of 3,000 MWd/MTHM), then the Release Limits for that system would be the quantities in Table 1 multiplied by ten:

$$\frac{60,000 \text{ MTHM}}{6,000 \text{ MTHM}} = 10$$

which is the same as:

$$\frac{60,000 \text{ MTHM}}{1,000 \text{ MTHM}} \times \frac{(5,000 \text{ MWd/MTHM})}{(30,000 \text{ MWd/MTHM})} = 10$$

NOTE 4: Treatment of Fractionated High-Level Wastes. In some cases, a high-level waste stream from reprocessing spent nuclear fuel may have been (or will be) separated into two or more high-level waste components destined for different disposal systems. In such cases, the implementing agency may allocate the Release Limit multiplier (based upon the original MTHM and the average fuel burnup of the high-level waste stream) among the various disposal systems as it chooses; provided that the total Release Limit multiplier used for that waste stream at all of its disposal systems may not exceed the Release Limit multiplier that would be used if the entire waste stream were disposed of in one disposal system.

NOTE 5: Treatment of Wastes with Poorly Known Burnups or Original MTHM. In some cases, the records associated with particular high-level waste streams may not be adequate to accurately determine the original metric tons of heavy metal in the reactor fuel that created the waste, or to determine the average

burnup that the fuel was exposed to. If the uncertainties are such that the original amount of heavy metal or the average fuel burnup for particular high-level waste streams cannot be quantified, the units of waste derived from (a) and (b) of Note 1 shall no longer be used. Instead, the units of waste defined in (c) and (d) of Note 1 shall be used for such high-level waste streams. If the uncertainties in such information allow a range of values to be associated with the original amount of heavy metal or the average fuel burnup, then the calculations described in previous Notes will be conducted using the values that result in the smallest Release Limits, except that the Release Limits need not be smaller than those that would be calculated using the units of waste defined in (c) and (d) of Note 1.

NOTE 6: Use of Release Limits to Determine Compliance with §60.112(a). Once release limits for a particular system have been determined in accordance with Notes 1 through 5, these release limits shall be used to determine compliance with the requirements of §60.112(a) as follows. In cases where a mixture of radionuclides is projected to be released to the accessible environment, the limiting values shall be determined as follows: For each radionuclide in the mixture, determine the ratio between the cumulative release quantity projected over 10,000 years and the limit for that radionuclide as determined from Table 1 and Notes 1 through 5. The sum of such ratios for all the radionuclides in the mixture may not exceed one with regard to §60.112(a)(1) and may not exceed ten with regard to §60.112(a)(2).

For example, if radionuclides A, B, and C are projected to be released in amounts Q_a , Q_b , and Q_c , and if the applicable Release Limits are RL_a , RL_b , and RL_c , then the cumulative releases over 10,000 years shall be limited so that the following relationship exists:

$$\frac{Q_a}{RL_a} + \frac{Q_b}{RL_b} + \frac{Q_c}{RL_c} \leq 1$$

12. In section 60.122, paragraph (c) is amended by redesignating the current paragraphs (c)(18) through (c)(21) as paragraphs (c)(19) through (c)(22) and by adding a new paragraph (c)(18) to read as follows:

§60.122 Siting criteria.

* * * * *

(c) * * *

(18) The presence of significant concentrations of any naturally-occurring material that is not reasonably available from other sources.

* * * * *

13. A new §60.144 is added to read as follows:

§60.144 Monitoring After Permanent Closure.

A program of monitoring shall be conducted after permanent closure to monitor all repository characteristics which can reasonably be expected to provide material confirmatory information regarding long-term repository performance, provided that the means for conducting such monitoring will not degrade repository performance. This program shall be continued until termination of a license.

Dated at Washington, D.C. this ____ day of _____ 1986.

For the Nuclear Regulatory Commission.

Samuel J. Chilk,
Secretary of the Commission

ENCLOSURE H



UNITED STATES
 NUCLEAR REGULATORY COMMISSION
 ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
 WASHINGTON, D. C. 20555

November 14, 1985

85 11 15 11 23

With Record File 406-3.3 WM Project _____
 Docket No. _____
 PDR _____
 LPER _____

Honorable Nunzio J. Palladino
 Chairman
 U. S. Nuclear Regulatory Commission
 Washington, D. C. 20555

Distribution: REP M-B Boyle MRK
JDB HJM
 (Return to Mail, 013-55) Fehring

Dear Dr. Palladino:

SUBJECT: ADDITIONAL ACRS COMMENTS ON EPA STANDARDS FOR A HIGH-LEVEL RADIOACTIVE WASTE REPOSITORY

During its 307th meeting, November 7-9, 1985, the Advisory Committee on Reactor Safeguards met with members of the NRC Staff and the Environmental Protection Agency (EPA) for additional discussions on the nature and implementation of the EPA Standards for a High-Level Radioactive Waste (HLW) Repository. This was also the subject of a meeting of the NRC Commissioners with the ACRS on October 10, 1985; of a meeting of the NRC Commissioners with representatives of the NRC Staff, the Department of Energy (DOE), EPA, and the ACRS on October 21, 1985; and of a combined meeting of our subcommittees on Waste Management and Metal Components on October 24-25, 1985. In addition, we reported to you on this subject in our letters of July 17, 1985 and October 16, 1985.

As a result of these meetings and associated discussions, we offer the following additional comments.

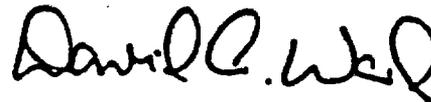
1. It is generally recognized that there is essentially no prospect that compliance with the EPA Standards can ever be demonstrated by actual observations. Determination of compliance will have to be based on the results of calculations using some agreed-upon set of release scenarios, environmental transport models, and their underlying assumptions. As stated in our letter of October 16, 1985, we believe that this has the potential for introducing obstacles in the licensing process, and it was for this reason that we recommended in our letter of July 17, 1985, that the Commission assure itself that the Staff's endorsement of this approach was correct.
2. We continue to believe that the EPA Standards contain deficiencies and inconsistencies, e.g., that the dose limits for single organs are not risk-based, and that different dose limits are being applied to NRC-licensed HLW facilities than to similar DOE facilities. Although we understand that time constraints did not permit the EPA Staff to correct these deficiencies, they nonetheless exist. In addition, there are errors in the recommended methods for the analysis and interpretation of data collected in the evaluation of the performance of a repository.

ENCLOSURE II

The NRC Staff is proposing an approach that may prove successful. However, we have no confidence that it will succeed. Our basic concern continues to be whether a formal determination can be made that a licensee is complying with the EPA Standards. To help resolve this problem, we encourage the NRC Staff to accelerate their efforts to develop analytical methods based on both deterministic and probabilistic approaches, and we recommend that a consensus be sought on these methods as they are developed. We also encourage the NRC Staff to use rule-making as a mechanism for implementing these methods, and we support the approaches being developed by the NRC Staff to utilize outside experts to help identify relevant issues and information needs.

Additional comments by ACRS Members Harold W. Lewis and Dade W. Moeller are presented below.

Sincerely,



David A. Ward
Chairman

Additional Comments by ACRS Member Harold W. Lewis

It is worth repeating and extending the statement in the ACRS letters of July 17, 1985 and October 16, 1985, that the EPA Standards are too stringent. All these problems of compliance determination derive from the fact that the EPA risk limits are far below any reasonable likelihood of detection. It is that that drives the dependence on models and calculations.

I know of no rational basis (though recognize the political constraints) for a standard involving one-tenth of a fatality per year for ten thousand years, beginning in a few hundred years. If one uses cost/benefit analysis with any reasonable estimate of the benefit of the repository; if one uses reasonable discounting of future costs against current benefits, a procedure understood by all surviving businesses and nations; if one compares with the risk or even the radioactive effluents from coal burning, the only viable alternative to nuclear power; if one compares with cosmic rays or other natural radiation; however one makes the comparison, these are unreasonably stringent standards.

I recognize that they are the product of EPA, and the result of a necessary political process, but think that the NRC should develop regulatory procedures in such a way as to make the best of a bad set of standards by moving the assessment of the risk in the direction of realism. To add the usual regulatory conservatism to the implementation of standards which are already too stringent would not be in the national interest.

I know of no risk issue (perhaps excepting UFOs) in which the discrepancy between perceived risk and actual risk is so high. That seems to be what has put us in this position, but it is still the responsibility of scientific advisors to remain rational and to deal with real risk. That is extraordinarily small here.

Additional Remarks by ACRS Member Dade W. Moeller

I recognize that many of the issues associated with the EPA Standards are controversial and subject to a range of interpretations. A primary example is the estimation of the average annual societal risk to an individual as a consequence of the operation of an HLW repository constructed and operated in accord with the EPA Standards. Depending on the number of people assumed to be exposed, one can "demonstrate" that the Standards are either comparable to the risks associated with some other existing radiation standards, or that the risks are several orders of magnitude lower. Since, at the present time, there appear to be no acceptable guides for use by Federal agencies in making risk estimates for radionuclide sources that have the potential for exposing large numbers of people at extremely low dose rates over long periods of time, I would encourage the NRC to request that the Committee on Interagency Radiation Research and Policy Coordination (CIRRPC) undertake to develop such guides. I understand that the CIRRPC would be receptive to such a request.



OFFICE OF THE
SECRETARY

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

May 15, 1986

MEMORANDUM FOR: Victor Stello, Jr.
Executive Director for Operations

FROM: *(1.13)* Samuel J. Chilk, Secretary
2-

SUBJECT: STAFF REQUIREMENTS - SECY-86-92 -- "10 CFR
PART 60 -- DISPOSAL OF HIGH-LEVEL RADIO-
ACTIVE WASTES IN GEOLOGIC REPOSITORIES --
PROPOSED AMENDMENTS TO ELIMINATE
INCONSISTENCIES WITH THE EPA HIGH-LEVEL
WASTE STANDARDS"

The Commission, (with all Commissioners agreeing) has approved publication in the Federal Register of the proposed amendments to 10 CFR Part 60, which eliminates inconsistencies with the EPA HLW standard.

You should forward the Federal Register Notice for signature and publication.

(EDO)

(SECY SUSPENSE: 6/13/86)

cc: Chairman Palladino
Commissioner Roberts
Commissioner Asselstine
Commissioner Bernthal
Commissioner Zech
Commission Staff Offices

NOTATION VOTE

RESPONSE SHEET

TO: SAMUEL J. CHILK, SECRETARY OF THE COMMISSION

FROM: COMMISSIONER ASSELSTINE

SUBJECT: SECY-86-92 - 10 CFR PART 60--DISPOSAL OF HIGH-LEVEL
RADIOACTIVE WASTES IN GEOLOGIC REPOSITORIES--PROPOSED
AMENDMENTS TO ELIMINATE INCONSISTENCIES WITH THE EPA
HIGH-LEVEL WASTE STANDARDS

APPROVED ✓ DISAPPROVED _____ ABSTAIN _____
NOT PARTICIPATING _____ REQUEST DISCUSSION _____

COMMENTS:

Samuel J. Chilk
SIGNATURE
5-12-86
DATE

Entered on "AS" YES NO

SECRETARIAT NOTE: PLEASE ALSO RESPOND TO AND/OR COMMENT ON OGC/OPE MEMORANDUM IF ONE HAS BEEN ISSUED ON THIS PAPER.

NOTATION VOTE

RESPONSE SHEET

TO: SAMUEL J. CHILK, SECRETARY OF THE COMMISSION

FROM: COMMISSIONER ROBERTS

SUBJECT: SECY-86-92 - 10 CFR PART 60--DISPOSAL OF HIGH-LEVEL
RADIOACTIVE WASTES IN GEOLOGIC REPOSITORIES--PROPOSED
AMENDMENTS TO ELIMINATE INCONSISTENCIES WITH THE EPA
HIGH-LEVEL WASTE STANDARDS

APPROVED _____ DISAPPROVED _____ ABSTAIN _____
NOT PARTICIPATING _____ REQUEST DISCUSSION _____

COMMENTS:



SIGNATURE

5 9 86

DATE

Entered on "AS"

YES NO

SECRETARIAT NOTE: PLEASE ALSO RESPOND TO AND/OR COMMENT ON OGC/OPE
MEMORANDUM IF ONE HAS BEEN ISSUED ON THIS PAPER.

NOTATION VOTE

RESPONSE SHEET

TO: SAMUEL J. CHILK, SECRETARY OF THE COMMISSION

FROM: CHAIRMAN PALLADINO

SUBJECT: SECY-86-92 - 10 CFR PART 60--DISPOSAL OF HIGH-LEVEL
RADIOACTIVE WASTES IN GEOLOGIC REPOSITORIES--PROPOSED
AMENDMENTS TO ELIMINATE INCONSISTENCIES WITH THE EPA
HIGH-LEVEL WASTE STANDARDS

APPROVED xx DISAPPROVED _____ ABSTAIN _____
NOT PARTICIPATING _____ REQUEST DISCUSSION _____

COMMENTS:

N. Palladino
SIGNATURE
4-30-86
DATE

Entered on "AS"

YES NO

SECRETARIAT NOTE: PLEASE ALSO RESPOND TO AND/OR COMMENT ON OGC/OPE
MEMORANDUM IF ONE HAS BEEN ISSUED ON THIS PAPER.

NOTATION VOTE

RESPONSE SHEET

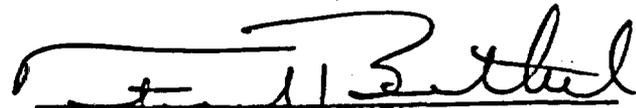
TO: SAMUEL J. CHILK, SECRETARY OF THE COMMISSION

FROM: COMMISSIONER BERNTHAL

SUBJECT: SECY-86-92 - 10 CFR PART 60--DISPOSAL OF HIGH-LEVEL
RADIOACTIVE WASTES IN GEOLOGIC REPOSITORIES--PROPOSED
AMENDMENTS TO ELIMINATE INCONSISTENCIES WITH THE EPA
HIGH-LEVEL WASTE STANDARDS

APPROVED DISAPPROVED ABSTAIN
NOT PARTICIPATING REQUEST DISCUSSION

COMMENTS:


SIGNATURE

5/14/86
DATE

Entered on "AS"

YES

NO

SECRETARIAT NOTE: PLEASE ALSO RESPOND TO AND/OR COMMENT ON OGC/OPE
MEMORANDUM IF ONE HAS BEEN ISSUED ON THIS PAPER.

NOTATION VOTE

RESPONSE SHEET

TO: SAMUEL J. CHILK, SECRETARY OF THE COMMISSION

FROM: COMMISSIONER ZECH

SUBJECT: SECY-86-92 - 10 CFR PART 60--DISPOSAL OF HIGH-LEVEL
RADIOACTIVE WASTES IN GEOLOGIC REPOSITORIES--PROPOSED
AMENDMENTS TO ELIMINATE INCONSISTENCIES WITH THE EPA
HIGH-LEVEL WASTE STANDARDS

APPROVED DISAPPROVED _____ ABSTAIN _____
NOT PARTICIPATING _____ REQUEST DISCUSSION _____

COMMENTS:

Larson W. Zech Jr.

SIGNATURE

5-14-86

DATE

Entered on "AS"

YES NO

SECRETARIAT NOTE: PLEASE ALSO RESPOND TO AND/OR COMMENT ON OGC/OPE
MEMORANDUM IF ONE HAS BEEN ISSUED ON THIS PAPER.

NUCLEAR REGULATORY COMMISSION

10 CFR Part 60

Disposal of High-Level Radioactive Wastes in Geologic Repositories;
Conforming Amendments

AGENCY: Nuclear Regulatory Commission.

ACTION: Proposed rule.

SUMMARY: The Nuclear Regulatory Commission (NRC) is proposing to amend its regulations for disposal of high-level radioactive wastes in geologic repositories. The amendments are necessary to conform existing NRC regulations to the environmental standards for management and disposal of high-level radioactive wastes promulgated by the Environmental Protection Agency (EPA) on September 19, 1985. The proposed rule would incorporate all the substantive requirements of the environmental standards and make several changes in the wording used by EPA in order to maintain consistency with the current wording of the NRC regulations.

DATE: Comment period expires Aug. 18, 1986 Comments received after this date will be considered if it is practical to do so, but assurance of consideration cannot be given except as to comments received on or before this date.

ADDRESSES: Written comments may be submitted to the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch. Comments may also be delivered to Room 1121, 1717 H Street NW, Washington, DC, from 8:15 a.m. to 5:00 p.m. weekdays. Copies of the documents referred to in this notice and comments

received may be examined at the NRC Public Document Room, 1717 H Street, NW, Washington, DC.

FOR FURTHER INFORMATION CONTACT: Daniel J. Fehringer, Division of Waste Management, Office of Nuclear Material Safety and Safeguards, U.S. Nuclear Regulatory Commission, Washington, DC, 20555, telephone (301) 427-4796.

SUPPLEMENTARY INFORMATION:

Background

Section 121 of the Nuclear Waste Policy Act of 1982 (NWPAA), 42 USC 10141, directs the Environmental Protection Agency (EPA) to "promulgate generally applicable standards for protection of the general environment from offsite releases from radioactive material in repositories." EPA published its final high-level radioactive waste (HLW) standards in the Federal Register on September 19, 1985 (50 FR 38066). Section 121 of the NWPAA further specifies that the regulations of the NRC "shall not be inconsistent with any comparable standards promulgated by [EPA]."

The Nuclear Regulatory Commission has previously published rules (10 CFR Part 60, 46 FR 13980, February 25, 1981, 48 FR 28204, June 21, 1983) which established procedures and technical criteria for disposal of HLW in a geologic repository by the U.S. Department of Energy (DOE). This notice describes the interpretations and analyses which the Commission considers to be appropriate for implementation of the EPA standards, and identifies modifications to the Commission's regulations which are considered appropriate to maintain consistency with the standards promulgated by EPA.

It should be noted that "working draft" versions of the EPA standards were available to the Commission when Part 60 was being developed, and the Commission structured its regulations to be compatible with those draft standards. (See, for example, 48 FR 28195-28205, June 21, 1983, where the Commission discussed its final technical criteria, and NUREG-0804, the staff's analysis of public comments on the proposed technical criteria. NUREG-0804 is available in the NRC Public Document Room.) Since many of the general features

of the "working drafts" remain present in the final standards, Part 60 is largely consistent with those standards. EPA has, however, sometimes used different terminology to describe concepts already present in Part 60. To maintain the overall structure of Part 60, and to avoid introduction of duplicative terminology which could prove confusing in a licensing review, the Commission prefers to retain its own established terms. Most of the amendments to Part 60 proposed in this notice involve direct incorporation within Part 60 of the substantive requirements of the EPA standards, reworded as necessary to conform to the terminology of Part 60. (Additional proposed amendments derive from EPA's "assurance requirements," as discussed in Section III of this notice. One further amendment, unrelated to the EPA standards, is proposed for clarification of existing wording in Part 60.) With the issuance of this rule, no substantive changes are intended in the requirements of the EPA standards or in the environmental protection they afford.

The EPA standards specify certain limits on radiation exposures and releases of radioactive material during two principal stages: first, the period of management and storage operations at a repository and, second, the long-term period after waste disposal has been completed. These standards, and the proposed rules to implement them during operations and after closure, are discussed in Section I below, while Section II provides some further observations regarding the manner in which the Commission intends to apply the EPA standards in its licensing proceedings. Section III describes additional proposed rules related to certain "assurance requirements" which are present in EPA's standards but which are not applicable to NRC-licensed facilities. In order to avoid potential jurisdictional problems which might arise if this section of the EPA standards were applied to NRC-licensed facilities, the NRC is proposing to add substantially equivalent provisions to its regulations. Finally, this notice presents a section-by-section analysis of the proposed rule (Section IV), followed by the specific text of the proposed amendments to Part 60. (The organization of Section IV follows that of Part 60 while the text of Section I is organized to present a section-by-section discussion of the EPA standards. Parts of Section IV are therefore repetitions of information presented in Section I.)

I. Limits on Exposures and Releases

The limits established by EPA for the period of repository operations appear at 40 CFR 191.03. The limits applicable to the period after disposal include "containment requirements" (limits on cumulative releases of radionuclides to the environment for 10,000 years) in §191.13, "individual protection requirements" in §191.15, and "ground water protection requirements" in §191.16. Implementation of each of these sections is discussed in the following paragraphs.

Standards for repository operations (§191.03). The standards for repository operations are virtually identical to the standards previously promulgated by EPA for the uranium fuel cycle (42 FR 2860, January 13, 1977), and will be implemented in the same manner.¹ DOE will be expected to demonstrate, through analyses of anticipated facility performance, that the dose limits of these standards, as well as the standards for protection against radiation set out in 10 CFR Part 20, will not be exceeded. Releases of radionuclides and resulting doses during operations are amenable to monitoring, and DOE will be required to conduct a monitoring program to confirm that the limits are complied with. Section 60.111(a) would be amended to include the EPA dose limits. Section 60.101(a)(2) already includes a provision requiring "reasonable assurance" that the release limits be achieved, and it is not necessary to repeat this language in the release limits of §60.111. It is also not necessary to employ the terms "management" and "storage," as EPA has done, since all preclosure repository operations are already subject to the provisions of §60.111.

¹It should be noted that a potential ambiguity exists in this section of EPA's HLW standards and in EPA's uranium fuel cycle standards. Both standards limit the annual dose equivalent to any member of the public to "25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other critical organ" (emphasis added). The Commission has always interpreted these limits as if the word "and" were replaced by "or." Thus, the Commission would not consider it acceptable to allow an annual dose equivalent of 25 millirems to the whole body and an additional 25 millirems to any other organ. The Commission will continue to implement these limits as it has in the past, but will encourage EPA to clarify the wording quoted above.

Postclosure standards. The EPA postclosure standards are all expressed in terms of a "reasonable expectation" of meeting specified levels of performance. EPA explained that it selected this term because "'reasonable assurance' has come to be associated with a level of confidence that may not be appropriate for the very long-term analytical projections that are called for by 191.13." The Commission is sensitive to the need to account for the uncertainties involved in predicting performance over 10,000 years, and the difficulties as well as the importance of doing so. The Commission has attempted to address this concern in the existing language of §60.101(a)(2). That section requires a finding of reasonable assurance, "making allowance for the time period, hazards, and uncertainties involved, that the outcome will be in conformance" with the relevant criteria. Rather than adopt an additional concept such as "reasonable expectation," the Commission proposes to add additional explanatory text, derived from EPA's wording, to its existing discussion of reasonable assurance. This text will make clear the Commission's belief that its concept of reasonable assurance, although somewhat different from previous usage in reactor licensing, is appropriate for evaluations of repository performance where long-term issues and substantial uncertainties are inherent in projections of repository performance. The Commission considers that the level of confidence associated with its concept of reasonable assurance is the same as that sought by EPA in the use of the term "reasonable expectation."

In the case of the individual protection requirements (40 CFR 191.15), the standards limit the annual dose equivalent to any member of the public in the accessible environment. A new provision in section 60.112(b) is proposed that would include the dose limits established by EPA as well as the additional specifications, which the Commission finds to be reasonable, with regard to consideration of all pathways including consumption of drinking water from a "significant source of ground water," as defined by EPA.

The EPA standards require that the individual protection requirements be achieved only for "undisturbed performance" of a geologic repository ("disposal system" in EPA's terminology). The proposed amendment to Part 60 makes no

reference to "undisturbed performance." Instead, it provides that the standard is to be met "in the absence of unanticipated processes and events." The Commission considers the concepts of undisturbed performance and the absence of unanticipated processes and events to be identical. As used by EPA (40 CFR 191.12(p)), "undisturbed performance" refers to the predicted behavior of a disposal system if it is "not disrupted by human intrusion or the occurrence of unlikely natural events." Since human intrusion and unlikely natural processes and events are precisely the types of "unanticipated processes and events" defined in §60.2, the two concepts are the same. Thus, the Commission considers that the phrase "in the absence of unanticipated processes and events" has the same meaning as "undisturbed performance" in the EPA standards. To maintain the overall structure of Part 60, and to avoid introduction of duplicative language, the Commission prefers to retain its own established terms.

The engineered barriers of a repository will, in many cases, be instrumental in achieving compliance with both the individual protection requirements and the groundwater protection requirements discussed below. The Commission notes that the existing provisions of Part 60 require the engineered barriers of a repository to achieve their containment and release rate performance objectives "assuming anticipated processes and events." Thus, equating "undisturbed performance" with "anticipated processes and events" causes no change in the types of conditions for which the engineered barriers must be designed.

The ground water protection requirements (40 CFR 191.16) focus on the quality of any "special source of ground water," which is defined, generally, as a source of drinking water in an area that includes and surrounds the geologic repository. This area extends for five kilometers beyond the controlled area. The standard applies to water "withdrawn" from such a special source. The Commission is proposing to include the EPA standard as a new performance objective (§60.112(c)). Once again the rule applies in the absence of unanticipated processes and events instead of "undisturbed performance."

The containment requirements (40 CFR 191.13) restrict the total amount of radioactive material released to the environment for 10,000 years following permanent closure of a repository. EPA provides a table listing release limits for the significant radionuclides present in HLW or spent fuel. The

values in this table were derived, based on environmental transport and dosimetry considerations, so that the amount of each radionuclide listed in the table will, if released to the environment, produce approximately the same number of population health effects. The standard further specifies different release limits for releases with differing likelihoods of occurrence. The Commission is proposing to incorporate these requirements as a new performance objective (§60.112(a)), along with a new §60.115 containing EPA's table of release limits.

The regulation goes on to state that the disposal systems shall be designed to provide a reasonable expectation - "based on performance assessments" - that the release limits are satisfied. While the proposed amendments incorporate most of the EPA standard in its precise terms, they omit the reference to performance assessments. Part 60 already requires analyses virtually identical to those contemplated by EPA, but the Commission proposes to add additional wording to §60.21(c)(1)(ii)(C) to emphasize consistency with the EPA standards.

The Commission notes, in this connection, that EPA's reference to estimating the cumulative releases caused by all significant processes and events, to be incorporated in an overall probability distribution of cumulative release to the extent practicable, does not modify the principles underlying Part 60. As was observed when NRC's final technical criteria were published in 1983 (48 FR 28204), the Commission expects that the information considered in a licensing proceeding will include probability distribution functions for the consequences from anticipated and unanticipated processes and events. Further information concerning the Commission's plans for assessing repository performance is contained in Section II of this notice.

II. Additional Comments on Implementation of the EPA Standards

Four sections of the EPA standards contain numerical requirements for which compliance must be demonstrated -- standards for repository operations, post-closure individual and groundwater protection requirements and containment requirements restricting the total amount of radionuclides projected to be released to the environment after repository closure. The discussion of

Section I of this notice articulates the Commission's interpretation of the standards that have been issued by EPA. Additional comments related to implementation of each of these sections are presented in the following paragraphs.

Standards for repository operations. As discussed previously, the standards for repository operations are virtually identical to the standards previously promulgated by EPA for the uranium fuel cycle, and will be implemented in the same manner. A license applicant will be expected to demonstrate, through analyses of anticipated facility performance, that the dose limits of these standards will not be exceeded. Doses during operations are amenable to monitoring, and the applicant will be required to conduct a monitoring program to confirm that the dose limits are complied with.

Individual and groundwater protection requirements. The individual and groundwater protection requirements are applicable for the first 1,000 years after permanent closure of a repository. Monitoring is not practical for this period of time and the applicant will therefore be required to demonstrate compliance with these requirements through analyses of projected repository performance. Two general approaches might be pursued by DOE. First, DOE might choose to calculate the expected concentrations of radionuclides in certain groundwaters potentially useable by humans in the future. Such calculations would include projections of waste package and engineered barrier performance (to provide a source term) as well as evaluations of the direction, velocity and volumetric flow rates of groundwaters near the repository. The EPA standards specify the types of groundwaters to be considered in such analyses (through the definitions of the terms "significant" and "special" sources of groundwater), and these concepts will be incorporated directly into Part 60.

Alternatively, DOE might choose to show compliance with these requirements by demonstrating that other barriers, such as the waste packages or the emplacement medium (e.g., salt), will provide substantially complete containment for the first 1,000 years after permanent closure thereby preventing contamination of the groundwaters of concern.

If DOE chooses to calculate the expected concentrations of radionuclides in groundwaters, rather than to rely on containment by engineered barriers, it will also be necessary to calculate potential doses to individuals in the future. The individual protection requirements limit the annual dose equivalent to any member of the public in the accessible environment. If a "significant source of groundwater" (as defined) is present, the Commission will assume that a hypothetical individual resides at the boundary of the controlled area and obtains his domestic water supply from a well at that location. If no such source of groundwater is present, the location of the maximally exposed individual and the pathways by which he might be exposed to radionuclides released from a repository must be examined on a site-specific basis.

The individual protection requirements also necessitate assumptions about the dietary patterns and other potential modes of ingestion of radionuclides during the next 1,000 years. The Commission will assume that current patterns remain unchanged, unless it can be convincingly demonstrated that a change is likely to occur (e.g., reduced groundwater consumption due to depletion of an aquifer).

Both the individual and groundwater protection requirements are applicable only for "undisturbed performance" of a repository system. As discussed in Section I, this term is considered to be equivalent to "anticipated processes and events," as currently defined in Part 60. The Commission will therefore require a demonstration of compliance with these requirements assuming the occurrence of anticipated processes and events, but will not require a demonstration of compliance in the event of unanticipated processes and events.

Containment requirements. The containment requirements are applicable for 10,000 years after repository closure. Therefore, compliance with these

requirements must also be evaluated by analyses of projected repository performance rather than by monitoring. The containment requirements call for significantly different analyses than those discussed above. This section of the EPA standards restricts the total amount of radioactive material released to the environment for 10,000 years following permanent closure of a repository. This section further specifies different release limits for releases with differing likelihoods of occurrence. Notwithstanding the quantitative probabilistic form of the EPA containment requirements (40 CFR 191.13), the Commission finds that there is adequate flexibility therein to allow them to be implemented using the licensing procedures of 10 CFR Parts 2 and 60. A further discussion of these matters is appropriate in order to avoid ambiguity in the application of the probabilistic conditions.

As the Commission emphasized when the technical criteria for geologic repositories were promulgated in final form (48 FR 28204), there are two distinct elements underlying a finding that a proposed facility satisfies the desired performance objective for long-term isolation of radioactive waste. There is, first, a standard of performance - some statement regarding the quantity of radioactive material that may be released to the accessible environment. This standard can be expressed in quantitative terms, and may include numerical requirements for the probabilities of exceeding certain levels of release.

The second element of a finding relates to the confidence that is needed by the factfinder in order to be able to conclude that the standard of performance has been met. The Commission has insisted, and the EPA has agreed, that this level of confidence must be expressed qualitatively. The licensing decisions that must be made in connection with a repository involve substantial uncertainties, many of which are not quantifiable (e.g., those pertaining to the correctness of the models used to describe physical systems). Such uncertainties can be accommodated within the licensing process only if a qualitative test is applied for the level of confidence that the numerical performance objective will be achieved.

The essential point to be kept in mind is that findings regarding long-term repository performance must be made with "reasonable assurance." The Commission attempted to explain this concept in the existing wording of §60.101(a) where it noted that allowance must be made for the time period, hazards, and uncertainties involved. Additional language is being proposed at this time, in the same section of Part 60, to further emphasize that qualitative judgments will need to be made including, for example, consideration of the degree of diversity or redundancy among the multiple barriers of a specific repository.

Application of a qualitative test in no way diminishes the level of safety required by a numerical standard. The applicant will be required to submit a systematic and thorough analysis of potential releases and the Commission will issue a license only if it finds a substantial, though unquantified, level of confidence that compliance with the release limits will be achieved. As we have stated previously (48 FR 28201), in order to make a finding with "reasonable assurance," the performance assessment which has been performed in the course of the licensing review must indicate that the likelihood of exceeding the EPA standard is low and, further, the Commission must be satisfied that the performance assessment is sufficiently conservative, and its limitations are sufficiently well understood, that the actual performance of the geologic repository will be within predicted limits.

The Commission will evaluate compliance with the containment requirements based on a performance assessment. Such an assessment will: (1) identify all significant processes and events which could affect the repository, (2) evaluate the likelihood of each process or event and the effect of each on release of radionuclides to the environment, and (3) to the extent practicable, combine these estimates into an overall probability distribution displaying the likelihood that the amount of radioactive material released to the environment will exceed specified values. The Commission anticipates that the overall probability distribution will be displayed in the format shown below.

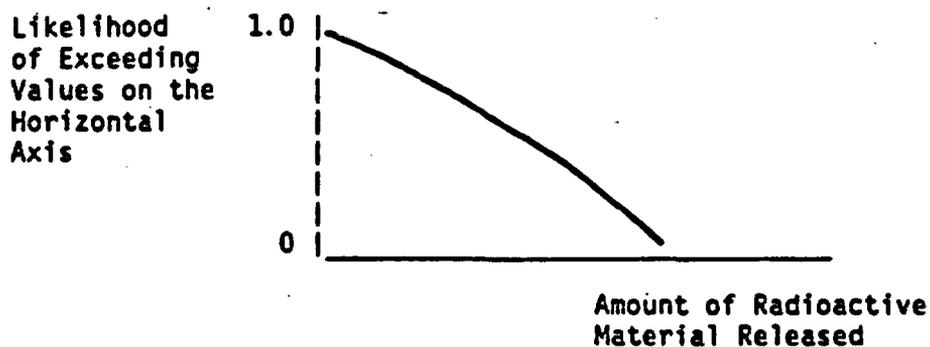


Figure 1. Illustrative "Complementary Cumulative Distribution Function."

When the results of analyses are displayed in this format, the limits of EPA's containment requirements take the form of "step functions," as shown in Figure 2.

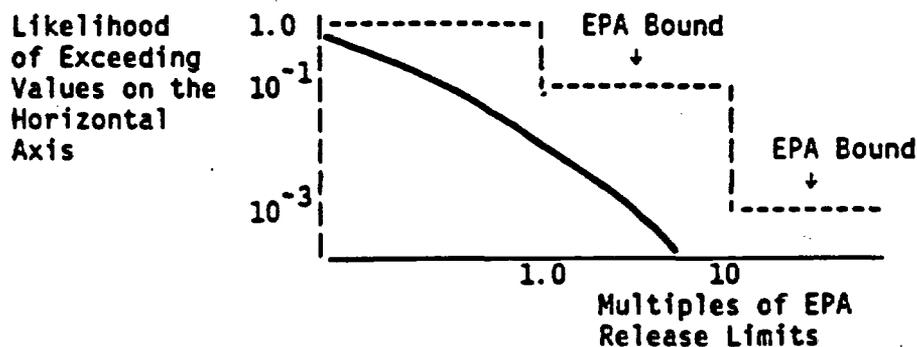


Figure 2. Graphic Representation of EPA Containment Requirements.

In Figure 2, releases which exceed the value specified in the EPA containment requirements (Table 1) must have a likelihood less than one chance in ten (over 10,000 years), and releases which exceed ten times that value must have a likelihood less than one chance in one thousand (over 10,000 years). Thus, in order to demonstrate compliance with EPA's containment requirements, the entire probability distribution must lie below the "stair-step" constraints illustrated in Figure 2.

In constructing a probability distribution of the type illustrated above, it is necessary to consider, in EPA's terms, all "significant processes and events that may affect the disposal system." This is equivalent, as we interpret the EPA standard, to all "anticipated" and "unanticipated" processes and events in the terminology of Part 60. (By the definition of "unanticipated processes and events" in Part 60, processes and events less likely than "unanticipated" are not sufficiently credible to warrant consideration.) For purposes of the proposed §60.112(a) only, which incorporates EPA's containment requirements, no distinction is to be made between "anticipated" and "unanticipated" processes and events; all such processes and events must be factored into the evaluation, including determination of such probabilities of occurrence as may be found to be appropriate. (For purposes of the proposed §60.112(b) and (c), which incorporate EPA's individual and groundwater protection requirements, only "anticipated" processes and events need be considered as discussed previously.)

The Commission will require an extensive and thorough identification of relevant processes and events, but will require analysis of the probability and/or consequence of each only to the extent necessary to determine its contribution to the overall probability distribution. If it can be shown, for example, that a particular event is so unlikely to occur that its effects on the probability distribution would not be meaningful, further analyses of the consequences of that event would not be required. Generally, categories of processes and events which can be shown to have a likelihood less than one chance in 10,000 over 10,000 years, along with categories of processes and events which otherwise can be shown not to change the remaining probability distribution of cumulative release significantly, need not receive further analysis. (The term "categories" is used to refer to general classes of processes and events, such as faulting, volcanism, or drilling. Subsets of these general categories, such as drilling which intersects a canister or fault displacement of a specific magnitude, may need to be retained in an analysis if the general category has been finely divided into a large number of specific process or event descriptions, each with reduced probabilities of occurrence.)

Treatment of uncertainties. As discussed previously, substantial uncertainties will be involved in analyses of long-term repository performance. These uncertainties may include (1) identification of basic phenomena and their potential effects on repository performance, (2) development and validation of models to describe these phenomena, (3) accuracy of available data, and (4) calculational uncertainties. Various methods may be used to accommodate such uncertainties including, for example, numerical estimates of uncertainties (expressed as probability distributions) or conservative, "bounding" models or data. Treatment of uncertainties will rely heavily on expert judgment, both for selection of an appropriate method and for application of that technique. EPA recognized the importance of uncertainties when its standards were promulgated. In Appendix B of 40 CFR Part 191 (50 FR 38088, September 19, 1985), EPA stated "substantial uncertainties are likely to be encountered in making (numerical) predictions (of repository performance). In fact, sole reliance on these numerical predictions to determine compliance may not be appropriate; the implementing agencies may choose to supplement such predictions with qualitative judgments as well." It is possible - in fact likely - that the various parties to a licensing proceeding will have significantly different views, all with technical merit, regarding the best methods to use, and these differing views may result in presentation of widely different estimates of repository performance.

Any such differences could be resolved in a number of ways. One permissible method for dealing with the uncertainties reflected in the record of the proceeding would be to rely heavily upon conservative, "bounding" analyses. Perhaps it could be shown that even if this approach were employed, the predicted performance would still satisfy the containment requirements established by EPA. On the other hand, an apparent violation of the standard (based on conservative analyses) would not necessarily preclude the Commission from finding, with reasonable assurance, that repository performance would conform to the EPA standard. After carefully evaluating the relevant uncertainties, DOE could present the same data in the form of a cumulative probability distribution that was less conservative - for example, one that

more accurately represents the best current technical understanding. Thus, alternative methods are available to DOE for treatment of uncertainties when making its demonstration of reasonable assurance of compliance with the provisions of Part 60.

It should be noted, however, that analyses based on "best estimates" of repository performance might be found to be inadequate if substantial uncertainties are present. In that case, notwithstanding the apparent conformity with the EPA standard, the Commission might ultimately conclude that it lacked the necessary reasonable assurance, considering the uncertainties involved, that the performance would meet the containment requirements.

Because uncertainties are so important in analyses of repository performance and will play such a major role in a licensing proceeding, the Commission emphasizes the importance of efforts being undertaken to foster a common technical understanding and to resolve issues, where it is practicable to do so, prior to receipt of a license application. Many of the provisions of the Nuclear Waste Policy Act are directed toward this goal. One especially important opportunity, in this regard, is DOE's preparation of site characterization plans and the review and comment process to be carried out by the Commission and other interested parties. Additionally, NRC and DOE are engaged, under an interagency procedural agreement, in ongoing technical discussions on matters that pertain to licensing requirements; these discussions are in the form of open meetings, affording other persons an opportunity to identify pertinent considerations that might also need to be addressed. The staff is also issuing staff technical positions on specific methods of analysis that would be acceptable for evaluating compliance with Part 60 technical criteria and performance objectives. As issues mature, the Commission will, where appropriate, use the rulemaking process to seek resolution of issues where a licensing proceeding might otherwise encounter difficulties due to ambiguity regarding acceptable assessment methods. Nevertheless, the data available at the time of licensing will inevitably be imperfect. It is therefore essential that every effort be made by DOE - and by any other party that develops data which it may propound at a hearing - to use careful methods to enhance, and document, the trustworthiness of the evidence which it may submit.

III. EPA Assurance Requirements

EPA's regulations (40 CFR 191.14) include certain "assurance requirements" designed, according to the rule, to provide the confidence needed for long-term compliance with the containment requirements. As noted by EPA in its preamble, the Commission took exception to the inclusion of these provisions in the regulations. The Commission viewed the assurance requirements as matters of implementation that were not properly part of the EPA's authorities assigned by Reorganization Plan No. 3 of 1970. In response to this concern, the two agencies have agreed to resolve this issue by NRC's making appropriate modifications to Part 60, reflecting the matters addressed by the assurance requirements, and by EPA's declaration that those requirements would not apply to facilities regulated by the Commission. The following discussion sets forth the Commission's views with respect to each of the EPA assurance requirements and identifies the proposed rule changes that are deemed to be appropriate under the circumstances.

EPA Assurance Requirement 40 CFR 191.14(a). Active institutional controls over disposal sites should be maintained for as long a period of time as is practicable after disposal; however, performance assessments that assess isolation of the wastes from the accessible environment shall not consider any contributions from active institutional controls for more than 100 years after disposal.

Analysis and Proposed Changes. The Commission's existing provisions (§60.52) related to license termination will determine the length of time for which institutional controls should be maintained, and there is therefore no need to alter Part 60 to reflect this part of the assurance requirement.

The second part of this assurance requirement would require that "active" institutional controls be excluded from consideration (after 100 years) when the isolation characteristics of a repository are assessed. It has always been the intent of Part 60 not to rely on remedial actions (or other active institutional controls) to compensate for a poor site or inadequate engineered barriers. However, in the definition of "unanticipated processes and events," Part 60 expressly contemplates that, in assessing human intrusion scenarios,

the Commission would assume that "institutions are able to assess risk and to take remedial action at a level of social organization and technological competence equivalent to, or superior to, that which was applied in initiating the processes or events concerned" (emphasis added). Therefore, it might appear at first examination that Part 60 is at odds with the EPA assurance requirement.

Although both the EPA regulation and Part 60 refer to "remedial action," the action being considered is not the same. The EPA assurance requirement deals with a planned capability to maintain a site and, if necessary, to take remedial action at a site in order to assure that isolation is achieved. The Commission agrees that such a capability should not be relied upon. The extent to which corrective action may be taken after an unanticipated intrusion occurs is an entirely different matter. The Commission may wish to consider, for example, the extent to which the application of the limited societal response capability assumed by the rule (e.g., sealing boreholes consistent with current petroleum industry practice) could reduce the likelihood of releases exceeding the values specified in the containment requirements or could eliminate certain hypothetical scenarios such as systematic and persistent intrusions into a site.

Subject to the comments above, the Commission concurs with the EPA's definitions of "active" and "passive" institutional controls, as well as the principle that ongoing, planned, active protective measures should not be relied upon for more than 100 years after permanent closure. We are therefore proposing to include EPA's definitions, together with a new section (§60.114) which would expressly provide that active (or passive) institutional controls shall not be deemed to assure compliance with the containment requirements over the long term. Some activities which arguably fall within EPA's definition of "active institutional controls" (e.g. remedial actions and monitoring parameters related to geologic repository performance) are relevant to assessing the likelihood and consequences of processes and events affecting the geologic setting. We are proposing, also in §60.114, to allow such activities to be considered for this purpose. We regard this as being fully consistent with the thrust of the EPA position.

EPA Assurance Requirement 40 CFR 191.14(b). Disposal systems shall be monitored after disposal to detect substantial and detrimental deviations from expected performance. This monitoring shall be done with techniques that do not jeopardize the isolation of the wastes and shall be conducted until there are no significant concerns to be addressed by further monitoring.

Analysis and Proposed Changes. Part 60 currently requires DOE to carry out a performance confirmation program which is to continue until repository closure. Part 60 does not now require monitoring after repository closure because of the likelihood that post-closure monitoring of the underground facility would degrade repository performance. The Commission recognizes, however, that monitoring such parameters as regional groundwater flow characteristics may, in some cases, provide desirable information beyond that which would be obtained in the performance confirmation program, and the Commission is proposing to require such monitoring when it can be accomplished without adversely affecting repository performance.

The proposed requirement for post-permanent closure monitoring requires that such monitoring be continued until termination of a license. The Commission intends that a repository license not be terminated until such time as the Commission is convinced that there is no significant additional information to be obtained from such monitoring which would be material to a finding of reasonable assurance that long-term repository performance would be in accordance with the established performance objectives.

A number of changes in Part 60 are proposed to reflect these views with respect to post-closure monitoring. First, a new section (§60.144) would provide for the performance confirmation program, already required by Subpart F of Part 60, to include a program of post-closure monitoring. Second, the licensing findings required at the time of license termination (§60.52(c)) would specifically be related to the results available from the post-closure monitoring program. Third, DOE would be required to provide more detailed information concerning its plans for post-closure monitoring in its original application (§60.21(c)) and when it applies to amend its license prior to permanent closure (§60.51(a)).

EPA Assurance Requirement 40 CFR 191.14(c). Disposal sites shall be designated by the most permanent markers, records, and other passive institutional controls practicable to indicate the dangers of the wastes and their location.

Analysis and Proposed Changes. The existing provisions of 10 CFR Part 60 already require that DOE take the measures set out in this assurance requirement. For further information, refer to §§60.21(c)(8) (requirement that license application describe controls to regulate land use), §60.51(a)(2) (information to be submitted, prior to permanent closure, with respect to land use controls, construction of monuments, preservation of records, etc.), and §60.121 (requirements for ownership and control of interests in land).

EPA Assurance Requirement 40 CFR 191.14(d). Disposal systems shall use different types of barriers to isolate the wastes from the accessible environment. Both engineered and natural barriers shall be included.

Analysis and Proposed Changes. This is another provision that is already inherent in Part 60. Nevertheless, in order to avoid any possible doubt in this regard, a new paragraph (§60.113(d)) would be added to state explicitly that the geologic repository shall incorporate a system of multiple barriers, both engineered and natural.

Questions might arise regarding the types of engineered or natural materials or structures which would be considered to constitute "barriers," as required by this new language. In this connection, the Commission notes that §60.2 now contains this definition: "'Barrier' means any material or structure that prevents or substantially delays movement of water or radionuclides" (emphasis added). Thus, consistent with the approach endorsed by EPA, the Commission considers that the new paragraph to be added to §60.113 will confirm its commitment to a multiple barrier approach as contemplated by Section 121(b)(1)(B) of the Nuclear Waste Policy Act.

EPA Assurance Requirement 40 CFR 191.14(e). Places where there has been mining for resources, or where there is a reasonable expectation of exploration for scarce or easily accessible resources, or where there is a significant concentration of any material that is not widely available from other sources, should be avoided in selecting disposal sites. Resources to be considered shall include minerals, petroleum or natural gas, valuable geologic formations, and ground waters that are either irreplaceable because there is no reasonable alternative source of drinking water available for substantial populations or that are vital to the preservation of unique and sensitive ecosystems. Such places shall not be used for disposal of the wastes covered by this Part [40 CFR Part 191] unless the favorable characteristics of such places compensate for their greater likelihood of being disturbed in the future.

Analysis and Proposed Changes. Part 60 contains provisions that, in large part, are equivalent to this assurance requirement. See §60.122(c)(17),(18), and (19). The existing regulation does not, however, address "a significant concentration of any material that is not widely available from other sources."

The Commission believes that there is merit in having the presence of such concentrated materials evaluated in the context of the licensing proceeding. It is, after all, quite possible that the economic value of materials could change in the future in a way which might attract future exploration or development detrimental to repository performance. By adding an additional "potentially adverse condition" to those already set out in the regulation, DOE would be required to identify the presence of the materials in question and evaluate the effect thereof on repository performance, as specified in §60.122(a)(2)(ii). It should be noted that the presence of potentially adverse conditions does not preclude the selection and use of a site for a geologic repository, provided that the conditions have been evaluated and demonstrated not to compromise performance.

EPA Assurance Requirement 40 CFR 191.14(f). Disposal systems shall be selected so that removal of most of the wastes is not precluded for a reasonable period of time after disposal.

Analysis and Proposed Changes. The Commission understands that the purpose of this assurance requirement is to discourage or preclude the use of disposal concepts such as deep well injection for which it would be virtually impossible to remove or recover wastes regardless of the time and resources employed. (This provision is thus significantly different from the Commission's retrievability requirement.) For a mined geologic repository - which is the only type of facility subject to licensing under 10 CFR Part 60 - wastes could be located and recovered (i.e. "removed," in the sense that EPA is using the term), albeit at high cost, even after repository closure. A repository would therefore meet this assurance requirement, and no further statements on the subject in Part 60 are indicated.

Petition for Rulemaking.

The Commission calls to the attention of all interested parties a pending petition for rulemaking submitted by the States of Nevada and Minnesota which deals, in large part, with the matters addressed by Section III of this notice. All relevant comments received by the Commission in response to the notice of receipt of the petition for rulemaking (published in the FEDERAL REGISTER on December 19, 1985, 50 FR 51701) will be considered along with comments received in response to this notice. It should be noted that the Commission's present proposal conforms to the approach which was discussed with EPA during the course of its rulemaking. The petition for rulemaking follows the same language very closely, but does suggest certain modifications. The Commission would be particularly interested in comments addressed to the respective merits of the language proposed herein and that proposed by the States of Nevada and Minnesota.

The Commission further notes that EPA has provided it with copies of comments regarding the assurance requirements that were received during the 40 CFR Part 191 rulemaking. These comments are available for inspection in the Commission's public document room.

IV. Section by Section Analysis of Proposed Conforming Amendments

The Commission considers that the simplest and most useful way to amend Part 60 for consistency with the EPA standards would be to incorporate directly within Part 60 all the substantive requirements of the environmental standards promulgated by EPA, modified as necessary to conform to the terminology currently used in Part 60. The following paragraphs present a section-by-section analysis of the NRC's proposed conforming amendments to Part 60.

§60.1 Purpose and scope.

This paragraph is analogous to EPA's 40 CFR 191.01 and 191.11 which state the applicability of the EPA standards. Part 60 is, however, a more specific regulation than the EPA standards in that it addresses only deep geologic repositories used for disposal of high-level radioactive wastes, while the EPA standards apply to other disposal methods and certain other types of radioactive wastes. No changes are proposed for §60.1, but the Commission notes that any regulations developed in the future for alternative disposal methods or for other types of wastes will incorporate any applicable provisions of the EPA standards.

§60.2 Definitions.

New definitions of several terms are proposed for incorporation within §60.2. These are taken directly from the EPA standards (or from 40 CFR Part 190) and are needed for purposes of implementation. These added terms are:

- 1) Active institutional control
- 2) Community water system
- 3) Passive institutional control
- 4) Significant source of groundwater
- 5) Special source of groundwater
- 6) Transmissivity
- 7) Uranium fuel cycle

In addition, the definition of "controlled area" and the related definition of "accessible environment" in the EPA standards are different from those currently in Part 60. The Commission proposes to revise its current definitions to conform to EPA's wording. In the case of "accessible environment," the change is merely editorial. The amendments to the definition of "controlled area" are also largely editorial, except for the specification of extent - i.e., that the controlled area is to encompass "no more than 100 square kilometers" and to extend "horizontally no more than five kilometers in any direction from the outer boundary of the original location of the radioactive wastes."

The Commission has reviewed this aspect of the EPA definition in the light of the policies which it articulated when the final technical criteria of 10 CFR Part 60 were adopted. One of these policies was that the controlled area "must be small enough to justify confidence that the monuments will effectively discourage subsurface disturbances." The prior rule would have authorized the establishment of a controlled area well over 300 square kilometers (about 75,000 acres) in size. While we would not deny the abstract possibility that effective controls could be instituted even over an area of that magnitude, we have much greater confidence that DOE would be able to demonstrate an ability to discourage subsurface disturbances over an area of more limited extent. It is our judgment that the 100 square kilometers that EPA has adopted, after consultation with the NRC staff, represents an appropriate limitation.

The other policy related to the definition of the "controlled area" is that it must allow the isolation capability of the rock surrounding the underground facility to be given appropriate weight in licensing reviews. This isolation capability is measured in two ways. First, it is to be taken into account in determining whether releases of radionuclides to the accessible environment are within the limits specified in the "containment requirements" (40 CFR 191.13). Second, under §60.113(a)(2), the isolation capability of the geologic setting must be such that the pre-waste-emplacement groundwater travel time along the fastest path of likely radionuclide travel from the disturbed zone to the accessible environment shall be a specified period (generally, 1000 years).

The Commission anticipates that adoption of the EPA terminology will have little effect on achievement of the containment requirements inasmuch as the controlled area is allowed a horizontal extent as large as five kilometers (presumably in the direction of radionuclide travel). Nor does the Commission anticipate that the limitation will make it impracticable to achieve a demonstration of compliance with the groundwater travel time performance objective. When the Commission adopted Part 60, it observed that the "accessible environment" might be larger (and, of course, the "controlled area" might therefore be smaller) than would be the case under the EPA standards then being considered (48 FR 28202). EPA has now moved in the direction of eliminating this difference, and the Commission's amendment, for this reason, represents no important change.

The proposed reduction in the maximum allowable extent of the controlled area (i.e., distance to the accessible environment) requires additional discussion to clarify the Commission's concepts of "disturbed zone" and "groundwater travel time." Groundwater travel time from the edge of the disturbed zone to the accessible environment is one of the criteria which the Commission identified, at the time of proposed rulemaking, as providing confidence that the wastes will be isolated for at least as long as they are most hazardous (46 FR 35280, 35281, July 8, 1981). As noted above, this objective concerns travel time from the edge of the disturbed zone rather than from the edge of the underground facility. The Commission selected the disturbed zone for the purpose of determining the groundwater travel time since the physical and chemical processes which isolate the wastes are "especially difficult to understand in the area close to the emplaced wastes because that area is physically and chemically disturbed by the heat generated by those wastes." Ibid.

One potential type of effect which could alter local groundwater flow conditions is thermal buoyancy of groundwater. Because buoyancy effects could extend over significant distances (see, e.g., M. Gordon and M. Weber, "Non-isothermal Flow Modeling of the Hanford Site," available in the NRC Public document room) and because the Commission is proposing to reduce the maximum allowable distance to the accessible environment, it is particularly

important to emphasize that the Commission did not intend such effects to serve as the basis for defining the extent of the disturbed zone. The Commission recognizes that such effects can be modeled with well developed assessment methods, and therefore were not the type of effects for which the disturbed zone concept was developed. Any contrary implication in our statement of considerations at the time the technical criteria were issued in final form (see 48 FR 28210) should be disregarded. (The staff is currently developing Generic Technical Positions discussing the disturbed zone and groundwater travel time. These technical positions will be publicly available prior to promulgation of these proposed amendments in final form, and will illustrate how the staff intends to approach these two concepts.)

Four other terms defined by EPA deserve additional discussion here.

The EPA standards contain a definition of the term "transuranic radioactive waste." The Commission does not use this term in Part 60 and thus has no need to define it there. All radioactive waste stored or disposed of at a geologic repository licensed under Part 60 - including transuranic radioactive waste - would be subject to the requirements of the EPA standards as applied by the rules proposed herein.

EPA defines the terms "storage" and "disposal" to mean retrievable storage and permanent isolation, respectively. Under Part 60, on the other hand, the term "storage" is used in the sense of Section 202 of the Energy Reorganization Act of 1974 (42 USC 5842) to refer to both long-term storage and disposal of wastes. The difference in EPA and NRC usage has no effect upon application of the EPA standards at NRC-licensed geologic repositories.

The Commission has recently defined "groundwater," for purposes of Part 60, to include all water which occurs below the land surface (50 FR 29641, July 22, 1985), while the EPA standards use the term to mean water below the land surface in a zone of saturation (emphasis added). The EPA standards use the term only in connection with the more specifically defined terms "significant source of ground water" and "special source of ground water." Thus, it is possible to identify "significant" or "special" sources of groundwater unambiguously with either definition of the term "groundwater," and the Commission therefore proposes to retain its current definition of the term.

§60.21 Content of application.

Paragraph (c)(1)(ii)(C) now requires a license application to include certain evaluations of the performance of a proposed geologic repository for the period after permanent closure. The Commission proposes to add an additional sentence to this paragraph requiring that the results of these analyses be incorporated into an overall probability distribution of cumulative releases to the extent practicable. This reflects the language of EPA's definition of "performance assessment."

The Commission also proposes to add a new paragraph to §60.21 requiring submittal of a general description of the program for post-permanent closure monitoring of the geologic repository. (See the discussion (Section III) regarding the EPA assurance requirements - specifically 40 CFR 191.14(b).)

§60.51 License amendment for permanent closure.

Paragraph (a)(1) currently requires that an application to amend a license for permanent closure must include a description of the program for post-permanent closure monitoring of the geologic repository. The Commission proposes to revise this paragraph to specify in more detail the information to be submitted, including descriptions of the parameters to be monitored and the length of time for which the monitoring is to be continued. (See also the preceding discussion regarding 40 CFR 191.14(b).)

§60.52 Termination of license.

The Commission proposes to add a new condition for license termination which would explicitly require that the results available from post-permanent closure monitoring confirm the expectation that the repository will comply with the performance objectives of Part 60. (See also the preceding discussion regarding 40 CFR 191.14(b).)

§60.101 Purpose and nature of findings.

The EPA standards use the phrase "reasonable expectation" to describe the required level of confidence that compliance will be achieved with the provisions of the standards. The Supplementary Information accompanying the EPA standards contrasts the concept of "reasonable expectation" with the

reasonable assurance standard that is used by the Commission in dealing with other licensing actions. The Commission has considered adopting EPA's "reasonable expectation" concept, but has decided that doing so would result in a needless, and potentially confusing, proliferation of terms. Instead, the Commission proposes to expand the current discussion of "reasonable assurance" in §60.101 to make clear its belief that the level of confidence associated with the term, when used in connection with the long-term issues involved in repository licensing, is the same as that sought by EPA in its use of the term "reasonable expectation."

§60.111 Performance of the geologic repository operations area through permanent closure.

Paragraph (a) currently requires compliance with "such generally applicable environmental standards for radioactivity as may have been established by the Environmental Protection Agency." The Commission proposes to replace this wording with the specific dose limits promulgated by EPA in 40 CFR 191.03(a) of its standards. The proposed wording would apply the dose limits to any member of the public outside the geologic repository operations area, consistent with EPA's phrase "any member of the public in the general environment."

The EPA provision includes wording that requires reasonable assurance of compliance with the dose limits. In Part 60, Subpart B now specifies the findings that must be made by the Commission for issuance of a license, including a finding of reasonable assurance of compliance with the performance objective of §60.111. Because Part 60 already requires that findings be made with reasonable assurance, it is unnecessary to repeat such a requirement within this proposed performance objective.

One additional amendment, unrelated to the EPA standards, is being proposed for §60.111. The current wording of this section now requires that the geologic repository operations area be designed so that radiation exposures, radiation levels, and releases of radioactive materials "will at all times be maintained within the limits specified in Part 20 . . ." (emphasis added). The words "at all times" were intended to emphasize the need to design the geologic repository operations area so that any waste retrieval found to be necessary in the future could be carried out in

conformance with the radiation protection requirements of 10 CFR Part 20. In order to clarify the meaning of the phrase "at all times," the Commission is proposing to revise this wording to read "will at all times, including the retrievability period of §60.111(b), be maintained within the limits specified in Part 20"

§60.112 Overall system performance objective for the geologic repository after permanent closure.

The current wording of this section now refers to "such generally applicable environmental standards for radioactivity as may have been established by the Environmental Protection Agency." The Commission proposes to replace this wording with the specific provisions promulgated by EPA in 40 CFR 191.13, 191.15 and 191.16 of its standards, reworded as appropriate for incorporation into Part 60.

As discussed previously, the Commission proposes to revise the language of §60.101 to make clear that its concept of the phrase "reasonable assurance" in Part 60 closely parallels the meaning intended by "reasonable expectation" in the EPA standards. Inasmuch as the findings to be made by the Commission must be made with "reasonable assurance," there is no need to use the term "reasonable expectation" in the specific standards.

EPA requires that cumulative releases of radioactivity to the environment be evaluated on the basis of "performance assessments." This concept already is built into the structure of Part 60. As discussed previously, however, the Commission is proposing an addition to §60.21 which would specifically require a license application to incorporate the results of analyses, as stated by EPA, in an overall probability distribution of cumulative releases to the extent practicable.

The individual and groundwater protection requirements of the EPA standards refer to "undisturbed performance" of a disposal system, where "undisturbed performance" is defined to mean "the predicted behavior of a disposal system, including consideration of the uncertainties in predicted behavior, if the disposal system is not disrupted by human intrusion or the

occurrence of unlikely natural events." The Commission considers undisturbed performance, as defined by EPA, to be equivalent to performance in the absence of "unanticipated processes and events," as currently defined in Part 60. The Commission is proposing to use the current Part 60 terminology rather than introduce a new term from the EPA standards.

§60.113 Performance of particular barriers after permanent closure.

Section 60.113 specifies performance objectives for individual barriers of a geologic repository, and permits the Commission to approve or specify specific numerical requirements on a case-by-case basis. The Commission considers that §60.113 clearly requires use of both engineered and natural barriers. Nevertheless, in order to avoid any possible confusion regarding the provisions of §60.113(b), the Commission proposes to add additional clarifying language to this section making it clear that a repository must incorporate a system of multiple barriers, both engineered and natural. (See the preceding discussion in Section III regarding the EPA assurance requirements - specifically 40 CFR 191.14(d).)

Paragraph (b)(1) of §60.113 now refers to "any generally applicable environmental standard for radioactivity established by the Environmental Protection Agency." The Commission proposes to replace this wording with a direct reference to the overall system performance objectives of §60.112.

§60.114 Institutional control.

The Commission proposes to add a new §60.114 to Part 60 to clarify its views regarding reliance on institutional controls. (See the preceding discussion in Section III regarding 40 CFR 191.14(a).)

§60.115 Release limits for overall system performance objectives.

The Commission proposes that the table of release limits (and accompanying notes) in Appendix A of the EPA standards be added to Part 60 in a new §60.115.

§60.122 Siting criteria.

Part 60 contains provisions related to the presence of economically valuable mineral resources at a repository site. Part 60 does not, however,

address deposits of materials which, though of limited economic value, are not reasonably available from other sources. Because the economic value of materials could change in the future, the Commission proposes to add an additional potentially adverse condition to Part 60 related to significant concentrations of material that is not reasonably available from other sources.

EPA used the term "widely available." The Commission believes that an additional consideration - the practicality of obtaining materials from alternative sources - is also germane, and the Commission is therefore proposing the phrase "reasonably available" for this potentially adverse condition. (See also the preceding discussion in Section III regarding 40 CFR 191.14(e).)

§60.144 Monitoring after permanent closure.

Part 60 currently requires DOE to carry out a performance confirmation program which is to continue until repository closure. Part 60 does not now require monitoring after repository closure because of the likelihood that post-closure monitoring of the underground facility would degrade repository performance. The Commission proposes to add a new §60.144 to Part 60 which would require post-closure monitoring of repository characteristics provided that such monitoring can be expected to provide material confirmatory information regarding long-term repository performance and provided that the means for conducting such monitoring will not degrade repository performance. (See the preceding discussion in Section III regarding 40 CFR 191.14(b).)

Environmental Impact

Pursuant to Section 121(c) of the Nuclear Waste Policy Act of 1982, this proposed rule does not require the preparation of an environmental impact statement under section 102(2)(C) of the National Environmental Policy Act of 1969 or any environmental review under subparagraph (E) or (F) of section 102(2) of this Act.

Paperwork Reduction Act Statement

The information collection requirements contained in this proposed rule are of limited applicability and affect fewer than ten respondents. Therefore,

Office of Management and Budget clearance is not required pursuant to the Paperwork Reduction Act of 1980 (44 U.S.C. 3501 et seq.).

Regulatory Flexibility Act Certification

In accordance with the Regulatory Flexibility Act of 1980 (5 U.S.C. 605(b)), the Commission certifies that this rule, if adopted, will not have a significant economic impact on a substantial number of small entities. The only entity subject to regulation under this rule is the U.S. Department of Energy, which does not fall within the scope of the definition of "small entities" set forth in the Regulatory Flexibility Act.

List of Subjects in 10 CFR Part 60

High-level waste, Nuclear power plants and reactors, Nuclear materials, Penalty, Reporting and recordkeeping requirements, Waste treatment and disposal.

Backfitting Requirements

The provisions of 10 CFR 50.109 on backfitting do not apply to this rulemaking because the rule is not applicable to production and utilization facilities licensed under 10 CFR Part 50.

For the reasons set out in the preamble and under the authority of the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974, as amended, the Nuclear Waste Policy Act of 1982, and 5 U.S.C. 553, the NRC is proposing to adopt the following amendments to 10 CFR Part 60.

PART 60 -- DISPOSAL OF HIGH-LEVEL RADIOACTIVE

WASTES IN GEOLOGIC REPOSITORIES

1. The authority citation for Part 60 continues to read as follows:
Secs. 51, 53, 62, 63, 65, 81, 161, 182, 183, 68 Stat. 929, 930, 932, 933, 935,

948, 953, 954, as amended (42 U.S.C. 2071, 2073, 2092, 2093, 2095, 2111, 2201, 2232, 2233); secs. 202, 206, 88 Stat. 1244, 1246 (42 U.S.C. 5842, 5846); secs. 10 and 14, Pub. L. 95-601, 92 Stat. 2951 (42 U.S.C. 2021a and 5851); sec. 102, Pub. L. 91-190, 83 Stat. 853 (42 U.S.C. 4332); sec. 121, Pub. L. 97-425, 96 Stat. 2228 (42 U.S.C. 10141).

For the purposes of sec. 223, 68 Stat. 958, as amended (42 U.S.C. 2273), §§60.71 to 60.75 are issued under sec. 161o, 68 Stat. 950, as amended (42 U.S.C. 2201(o)).

2. Section 60.2 is amended by revising the definitions of "accessible environment" and "controlled area" and by adding seven new definitions in alphabetical order as follows:

§60.2 Definitions.

* * * * *

"Accessible environment" means: (1) the atmosphere, (2) land surfaces, (3) surface waters, (4) oceans, and (5) all of the lithosphere that is beyond the controlled area.

* * * * *

"Active institutional control" means: (1) controlling access to a disposal site by any means other than passive institutional control, (2) performing maintenance operations or remedial actions at a site, (3) controlling or cleaning up releases from a site, or (4) monitoring parameters related to disposal system performance.

* * * * *

"Community water system" means a system for the provision to the public of piped water for human consumption, if such system has at least 15 service connections used by year-round residents or regularly serves at least 25 year-round residents.

* * * * *

"Controlled area" means: (1) a surface location, to be identified by passive institutional controls, that encompasses no more than 100 square kilometers and extends horizontally no more than five kilometers in any

direction from the outer boundary of the underground facility, and (2) the subsurface underlying such a surface location.

* * * * *

"Passive institutional control" means: (1) permanent markers placed at a disposal site, (2) public records and archives, (3) government ownership and regulations regarding land or resource use, and (4) other methods of preserving knowledge about the location, design, and contents of a disposal system.

* * * * *

"Significant source of groundwater" means: (1) an aquifer that: (i) is saturated with water having less than 10,000 milligrams per liter of total dissolved solids; (ii) is within 2,500 feet of the land surface; (iii) has a transmissivity greater than 200 gallons per day per foot, provided that any formation or part of a formation included within the source of groundwater has a hydraulic conductivity greater than 2 gallons per day per square foot; and (iv) is capable of continuously yielding at least 10,000 gallons per day to a pumped or flowing well for a period of at least a year; or (2) an aquifer that provides the primary source of water for a community water system as of November 18, 1985.

* * * * *

"Special source of groundwater" means those Class I groundwaters identified in accordance with the Environmental Protection Agency's Ground-Water Protection Strategy published in August 1984 that: (1) are within the controlled area encompassing a disposal system or are less than five kilometers beyond the controlled area; (2) are supplying drinking water for thousands of persons as of the date that the Department chooses a location within that area for detailed characterization as a potential site for a disposal system (e.g., in accordance with Section 112(b)(1)(B) of the NWPA); and (3) are irreplaceable in that no reasonable alternative source of drinking water is available to that population.

* * * * *

"Transmissivity" means the hydraulic conductivity integrated over the saturated thickness of an underground formation. The transmissivity of a series of formations is the sum of the individual transmissivities of each formation comprising the series.

* * * * *

"Uranium fuel cycle" means the operations of milling of uranium ore, chemical conversion of uranium, isotopic enrichment of uranium, fabrication of uranium fuel, generation of electricity by a light-water-cooled nuclear power plant using uranium fuel, and reprocessing of spent uranium fuel, to the extent that these directly support the production of electrical power for public use utilizing nuclear energy, but excludes mining operations, operations at waste disposal sites, transportation of any radioactive material in support of these operations, and the reuse of recovered non-uranium special nuclear and by-product materials from the cycle.

* * * * *

3. Section 60.21 is amended by revising paragraph (c)(1)(ii)(C), adding a new paragraph (c)(9) and redesignating the existing paragraphs (c)(9) through (c)(15) as paragraphs (c)(10) through (c)(16).

§60.21 Content of application.

* * * * *

- (c) * * *
- (1) * * *
- (ii) * * *

(C) An evaluation of the performance of the proposed geologic repository for the period after permanent closure, assuming anticipated processes and events, giving the rates and quantities of releases of radionuclides to the accessible environment as a function of time; and a similar evaluation which assumes the occurrence of unanticipated processes and events. In making such evaluations, estimated values shall be incorporated into an overall probability distribution of cumulative release to the extent practicable.

* * * * *

(9) A general description of the program for post-permanent closure monitoring of the geologic repository.

* * * * *

4. Section 60.51 is amended by revising paragraph (a)(1) to read as follows:

§60.51 License amendment for permanent closure.

(a) * * *

(1) A detailed description of the program for post-permanent closure monitoring of the geologic repository in accordance with §60.144. As a minimum, this description shall:

(i) identify those parameters that will be monitored;

(ii) indicate how each parameter will be used to evaluate the expected performance of the repository; and

(iii) discuss the length of time over which each parameter should be monitored to adequately confirm the expected performance of the repository.

* * * * *

5. Section 60.52 is amended by designating current paragraph (c)(3) as paragraph (c)(4) and by adding a new paragraph (c)(3) as follows:

§60.52 Termination of license.

* * * * *

(c) * * *

(3) That the results available from the post-permanent closure monitoring program confirm the expectation that the repository will comply with the performance objectives set out at §60.112 and §60.113; and

* * * * *

6. Section 60.101 is amended by revising paragraph (a)(2) to read as follows:

§60.101 Purpose and nature of findings.

(a) * * - *

(2) While these performance objectives and criteria are generally stated in unqualified terms, it is not expected that complete assurance that they will be met can be presented. A reasonable assurance, on the basis of the record before the Commission, that the objectives and criteria will be met is the general standard that is required. For §60.112, and other portions of this subpart that impose objectives and criteria for repository performance over long times into the future, there will inevitably be greater uncertainties. Proof of the future performance of engineered barrier systems and the geologic setting over time periods of many hundreds or many thousands of years is not to be had in the ordinary sense of the word. For such long-term objectives and criteria, what is required is reasonable assurance, making allowance for the time period, hazards, and uncertainties involved, that the outcome will be in conformance with those objectives and criteria. Demonstration of compliance with such objectives and criteria will involve the use of data from accelerated tests and predictive models that are supported by such measures as field and laboratory tests, monitoring data and natural analog studies. Demonstration of compliance with the performance objectives of §60.112 will also involve predicting the likelihood and consequences of events and processes that may disturb the repository. Such predictions may involve complex computational models, analytical theories and prevalent expert judgment. Substantial uncertainties are likely to be encountered and sole reliance on numerical predictions to determine compliance may not be appropriate. In reaching a determination of reasonable assurance, the Commission may supplement numerical analyses with qualitative judgments including, for example, consideration of the degree of diversity or redundancy among the multiple barriers of a specific repository.

* * * * *

7. In section 60.111, paragraph (a) is revised to read as follows:

§60.111 Performance of the geologic repository operations area through permanent closure.

(a) Protection against radiation exposures and releases of radioactive material. The geologic repository operations area shall be designed so that until permanent closure has been completed:

(1) The annual dose equivalent to any member of the public outside the geologic repository operations area, resulting from the combination of (i) discharges of radioactive material and direct radiation from activities at the geologic repository operations area and (ii) uranium fuel cycle operations, shall not exceed 25 millirems to the whole body, 75 millirems to the thyroid, and 25 millirems to any other critical organ.

(2) Radiation exposures and radiation levels, and releases of radioactive materials to unrestricted areas, will at all times, including the retrievability period of §60.111(b), be maintained within the limits specified in Part 20 of this chapter.

* * * * *

8. Section 60.112 is revised to read as follows:

§60.112 Overall system performance objective for the geologic repository after permanent closure.

The geologic setting shall be selected and the engineered barrier system and the shafts, boreholes and their seals shall be designed:

(a) So that, for 10,000 years following permanent closure, cumulative releases of radionuclides to the accessible environment, from all anticipated and unanticipated processes and events, shall:

(1) Have a likelihood of less than one chance in 10 of exceeding the quantities calculated in accordance with §60.115.

(2) Have a likelihood of less than one chance in 1,000 of exceeding ten times the quantities calculated in accordance with §60.115.

(b) So that for 1,000 years after permanent closure, and in the absence of unanticipated processes and events, the annual dose equivalent to any member of the public in the accessible environment does not exceed 25 millirems to the whole body or 75 millirems to any critical organ. For the purpose of applying this paragraph, all potential pathways from the geologic repository to people shall be considered, including the assumption that individuals consume 2 liters per day of drinking water from any significant source of groundwater outside of the controlled area.

(c) So that for 1,000 years after permanent closure, and in the absence of unanticipated processes and events:

(1) Except as provided in paragraph (c)(2) of this section, the radionuclide concentrations averaged over any year in water withdrawn from any portion of a special source of groundwater do not exceed:

(i) 5 picocuries per liter of radium-226 and radium-228;

(ii) 15 picocuries per liter of alpha-emitting radionuclides (including radium-226 and radium-228 but excluding radon); or

(iii) the combined concentrations of radionuclides that emit either beta or gamma radiation that would produce an annual dose equivalent to the total body or any internal organ greater than 4 millirems per year if an individual consumed 2 liters per day of drinking water from such a source of groundwater.

(2) If any of the average annual radionuclide concentrations existing in a special source of groundwater before construction of the geologic repository operations area already exceed the limits in paragraph (c)(1) of this section, the increase, caused by the geologic repository, in the existing average annual radionuclide concentrations in water withdrawn from that special source of groundwater does not exceed the limits specified in paragraph (c)(1) of this section.

9. In section 60.113, paragraph (b)(1) is revised and a new paragraph (d) is added to read as follows:

§60.113 Performance of particular barriers after permanent closure.

	*		*		*		*		*
(b)	*	*	*						
(1)	The overall system performance objectives of §60.112.								
	*		*		*		*		*

(d) Notwithstanding the provisions of paragraph (b) of this section, the geologic repository shall incorporate a system of multiple barriers, both engineered and natural.

10. A new §60.114 is added to read as follows:

§60.114 Institutional control.

Neither active nor passive institutional control shall be deemed to assure compliance with the overall system performance objectives set out at §60.112 for more than 100 years after permanent closure. However, the effects of institutional control may be considered in assessing, for purposes of that section, the likelihood and consequences of processes and events affecting the geologic setting.

11. A new §60.115 is added to read as follows:

§60.115 Release limits for overall system performance objective.

The following table shall be used to make the calculations referred to in paragraph (a) of §60.112.

TABLE 1 --RELEASE LIMITS FOR OVERALL SYSTEM PERFORMANCE OBJECTIVE
 (Cumulative Releases to the Accessible Environment
 for 10,000 Years After Disposal)

Radionuclide	Release Limit per 1000 MTHM or other unit of waste (see Notes) (curies)
Americium-241 or 243 - - - - -	100
Carbon-14 - - - - -	100
Cesium-135 or 137 - - - - -	1000
Iodine-129 - - - - -	100
Neptunium-237 - - - - -	100
Plutonium-238, 239, 240 or 242 - - - - -	100
Radium-226 - - - - -	100
Strontium-90 - - - - -	1000
Technetium-99 - - - - -	10000
Thorium-230 or 232 - - - - -	10
Tin-126 - - - - -	1000
Uranium-233, 234, 235, 236 or 238 - - - - -	100
Any other alpha-emitting radionuclide with a half-life greater than 20 years - - - - -	100
Any other radionuclide with a half-life greater than 20 years that does not emit alpha particles - - - - -	1000

Application of Table 1

NOTE 1: Units of Waste. The Release Limits in Table 1 apply to the amount of wastes in any one of the following:

(a) an amount of spent nuclear fuel containing 1,000 metric tons of heavy metal (MTHM) exposed to a burnup between 25,000 megawatt-days per metric ton of heavy metal (Mwd/MTHM) and 40,000 Mwd/MTHM;

(b) the high-level radioactive wastes generated from reprocessing each 1,000 MTHM exposed to a burnup between 25,000 Mwd/MTHM and 40,000 Mwd/MTHM;

(c) each 100,000,000 curies of gamma or beta-emitting radionuclides with half-lives greater than 20 years but less than 100 years (for use as discussed in Note 5 or with materials that are identified by the Commission as high-level radioactive waste in accordance with part (B) of the definition of high-level waste in the Nuclear Waste Policy Act (NWPA));

(d) each 1,000,000 curies of other radionuclides (i.e., gamma or beta-emitters with half-lives greater than 100 years or any alpha-emitters with half-lives greater than 20 years) (for use as discussed in Note 5 or with materials that are identified by the Commission as high-level waste in accordance with part (B) of the definition of high-level waste in the NWPA); or

(e) an amount of transuranic (TRU) wastes containing one million curies of alpha-emitting transuranic radionuclides with half-lives greater than 20 years.

NOTE 2: Release Limits for Specific Disposal Systems. To develop Release Limits for a particular disposal system, the quantities in Table 1 shall be adjusted for the amount of waste included in the disposal system compared to the various units of waste defined in Note 1. For example:

(a) If a particular disposal system contained the high-level wastes from 50,000 MTHM, the Release Limits for that system would be the quantities in Table 1 multiplied by 50 (50,000 MTHM divided by 1,000 MTHM).

(b) If a particular disposal system contained three million curies of alpha-emitting transuranic wastes, the Release Limits for that system would be

the quantities in Table 1 multiplied by three (three million curies divided by one million curies).

(c) If a particular disposal system contained both the high-level wastes from 50,000 MTHM and 5 million curies of alpha-emitting transuranic wastes, the Release Limits for that system would be the quantities in Table 1 multiplied by 55:

$$\frac{50,000 \text{ MTHM}}{1,000 \text{ MTHM}} + \frac{5,000,000 \text{ curies TRU}}{1,000,000 \text{ curies TRU}} = 55$$

NOTE 3: Adjustments for Reactor Fuels with Different Burnup. For disposal systems containing reactor fuels (or the high-level wastes from reactor fuels) exposed to an average burnup of less than 25,000 Mwd/MTHM or greater than 40,000 Mwd/MTHM, the units of waste defined in (a) and (b) of Note 1 shall be adjusted. The unit shall be multiplied by the ratio of 30,000 Mwd/MTHM divided by the fuel's actual average burnup, except that a value of 5,000 Mwd/MTHM may be used when the average fuel burnup is below 5,000 Mwd/MTHM and a value of 100,000 Mwd/MTHM shall be used when the average fuel burnup is above 100,000 Mwd/MTHM. This adjusted unit of waste shall then be used in determining the Release Limits for the disposal system.

For example, if a particular disposal system contained only high-level wastes with an average burnup of 3,000 Mwd/MTHM, the unit of waste for that disposal system would be:

$$1,000 \text{ MTHM} \times \frac{(30,000 \text{ Mwd/MTHM})}{(5,000 \text{ Mwd/MTHM})} = 6,000 \text{ MTHM}$$

If that disposal system contained the high-level wastes from 60,000 MTHM (with an average burnup of 3,000 Mwd/MTHM), then the Release Limits for that system would be the quantities in Table 1 multiplied by ten:

$$\frac{60,000 \text{ MTHM}}{6,000 \text{ MTHM}} = 10$$

which is the same as:

$$\frac{60,000 \text{ MTHM}}{\text{-----}} \times \frac{(5,000 \text{ Mwd/MTHM})}{\text{-----}} = 10$$

$$1,000 \text{ MTHM} \quad (30,000 \text{ Mwd/MTHM})$$

NOTE 4: Treatment of Fractionated High-Level Wastes. In some cases, a high-level waste stream from reprocessing spent nuclear fuel may have been (or will be) separated into two or more high-level waste components destined for different disposal systems. In such cases, the implementing agency may allocate the Release Limit multiplier (based upon the original MTHM and the average fuel burnup of the high-level waste stream) among the various disposal systems as it chooses, provided that the total Release Limit multiplier used for that waste stream at all of its disposal systems may not exceed the Release Limit multiplier that would be used if the entire waste stream were disposed of in one disposal system.

NOTE 5: Treatment of Wastes with Poorly Known Burnups or Original MTHM. In some cases, the records associated with particular high-level waste streams may not be adequate to accurately determine the original metric tons of heavy metal in the reactor fuel that created the waste, or to determine the average burnup that the fuel was exposed to. If the uncertainties are such that the original amount of heavy metal or the average fuel burnup for particular high-level waste streams cannot be quantified, the units of waste derived from (a) and (b) of Note 1 shall no longer be used. Instead, the units of waste defined in (c) and (d) of Note 1 shall be used for such high-level waste streams. If the uncertainties in such information allow a range of values to be associated with the original amount of heavy metal or the average fuel burnup, then the calculations described in previous Notes will be conducted using the values that result in the smallest Release Limits, except that the Release Limits need not be smaller than those that would be calculated using the units of waste defined in (c) and (d) of Note 1.

NOTE 6: Use of Release Limits to Determine Compliance with §60.112(a).

Once release limits—for a particular system have been determined in accordance with Notes 1 through 5, these release limits shall be used to determine compliance with the requirements of §60.112(a) as follows. In cases where a mixture of radionuclides is projected to be released to the accessible environment, the limiting values shall be determined as follows: For each radionuclide in the mixture, determine the ratio between the cumulative release quantity projected over 10,000 years and the limit for that radionuclide as determined from Table 1 and Notes 1 through 5. The sum of such ratios for all the radionuclides in the mixture may not exceed one with regard to §60.112(a)(1) and may not exceed ten with regard to §60.112(a)(2).

For example, if radionuclides A, B, and C are projected to be released in amounts Q_a , Q_b , and Q_c , and if the applicable Release Limits are RL_a , RL_b , and RL_c , then the cumulative releases over 10,000 years shall be limited so that the following relationship exists:

$$\frac{Q_a}{RL_a} + \frac{Q_b}{RL_b} + \frac{Q_c}{RL_c} \leq 1$$

12. In section 60.122, paragraph (c) is amended by redesignating the current paragraphs-(c)(18) through (c)(21) as paragraphs (c)(19) through (c)(22) and by adding a new paragraph (c)(18) to read as follows:

§60.122 Siting criteria.

* * * * *
(c) * * *

(18) The presence of significant concentrations of any naturally-occurring material that is not reasonably available from other sources.

* * * * *

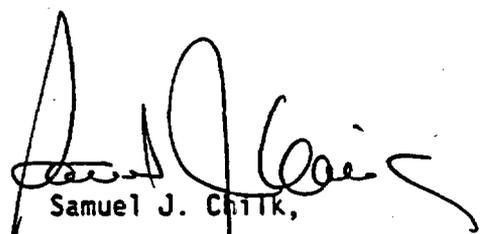
13. A new §60.144 is added to read as follows:

§60.144 Monitoring After Permanent Closure.

A program of monitoring shall be conducted after permanent closure to monitor all repository characteristics which can reasonably be expected to provide material confirmatory information regarding long-term repository performance, provided that the means for conducting such monitoring will not degrade repository performance. This program shall be continued until termination of a license.

Dated at Washington, D.C. this 13th day of June 1986.

For the Nuclear Regulatory Commission.


Samuel J. Chittk,
Secretary of the Commission

Enclosure 2



Department of Energy
Washington, DC 20585

JAN 23 1989

Honorable Lando W. Zech, Jr.
Chairman
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Chairman:

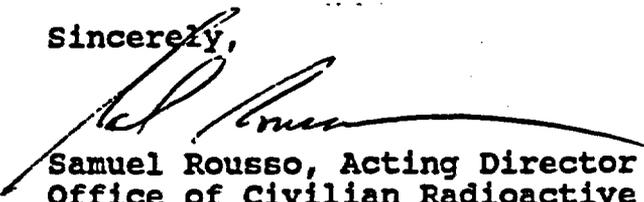
The Nuclear Waste Policy Amendments Act of 1987 (NWPAA, Public Law 100-203) mandates that the Department of Energy (DOE) prepare a report to the Congress on the use of dry cask storage at nuclear reactor sites to meet the utility industry's spent nuclear fuel storage needs through the start of operation of a permanent geologic repository. As part of the overall study, the DOE solicited comments from the Nuclear Regulatory Commission (NRC), State and local governments, utility companies, interested parties, and the public on the Initial Version of the Dry Cask Storage Study which was published in September 1988.

In response to the comments received, we have prepared the final version of the report, a copy of which is enclosed. The Department was in agreement with the NRC comments on the initial version of the report. Our detailed responses to all of the comments received are contained in Part II of this final report.

Thirty copies of the final report are being mailed to Mr. John Roberts of your staff. Because the NWPAA requires that the views of the NRC be included in the Dry Cask Storage Study, we would appreciate NRC's final written comments by February 17, 1989, so that they can also be submitted to Congress with the final report.

Thank you for your interest and participation in this important activity.

Sincerely,



Samuel Rousso, Acting Director
Office of Civilian Radioactive
Waste Management

Enclosure:
Final Version Dry Cask Storage Study

cc:
Hugh L. Thompson, Jr., USNRC
Richard E. Cunningham, USNRC
John P. Roberts, USNRC

Enclosure 2

Enclosure 4



Department of Energy

Washington, DC 20585

JAN 4 1989

Honorable Lando W. Zech, Jr.
Chairman
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Chairman Zech:

In response to your letter of November 18, 1988, which provided comments on the "Initial Version Dry Cask Storage Study" (DOE/RW-0196), I would like to thank you for your overall assessment of the document. We attempted to make it a straightforward, responsible, technical document and were gratified by your response. We will, of course, be responding to all of your comments in the comment response section of the final report. However, in consideration of the significance of the comment in the second paragraph of your letter, concerning compatibility of the various steps in the spent fuel management process, I want to give you an early indication of our current thinking.

The Department agrees that all of the steps in the spent fuel management process should be coordinated to enhance the safety and efficiency of the operations and plans to increase its efforts to ensure that this coordination takes place. This has already been recognized by both the Department and the nuclear utilities, and elements of this general coordination issue have already been identified as topics to be addressed through the process for resolving issues concerning the standard contract for disposal of spent fuel. This contract establishes, among other things, the contractual terms and conditions for the waste acceptance process.

The issue resolution process associated with the contract is a mechanism for identifying and ultimately overcoming obstacles to the effective and efficient implementation of the contract. The issue resolution process was described in the June 1988 issue of the "Annual Capacity Report" (DOE/RW-0191) and is commonly referred to as "the ACR issue resolution process." The Department intends to discuss with the utilities at the next meeting in the ACR issue resolution process the general coordination issue that you have raised, to identify opportunities for and the timing of steps to address any coordination elements that are not already being addressed. Any elements of the general coordination issue that are not appropriate for resolution through the ACR issue resolution process will be taken up separately by the Department working with the utilities, through the auspices of the Edison Electric Institute's Utility Nuclear Waste and Transportation Program (successor to the separate Utility Nuclear Waste Management Group and Transportation Group).

Enclosure 4

As described in the Department's "Initial Version Dry Cask Storage Study," several different technologies for expanding at-reactor storage are in various stages of development. No single technology is likely to meet the requirements of all the utilities. Furthermore, the utilities believe that they need to retain the flexibility to choose the option that best suits their requirements, while choosing systems that incorporate compatibility elements that are jointly developed based on system requirements.

As more information is developed about each of the technologies, it will be appropriate and natural to consider certain features or interfaces within each of the technology categories for compatibility with the Federal Waste Management System. These features or interfaces could include items such as dimensions, weights, payloads, materials, heat and radiation limits, and handling features.

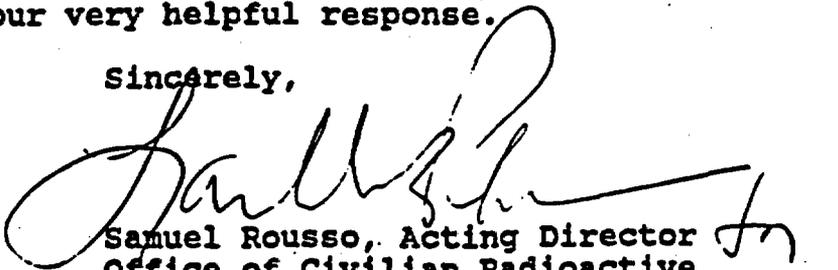
The compatible elements of each of the major types of technology can then serve as focal points for combined Federal and utility efforts to ensure that the various technologies interface satisfactorily with the Federal Waste Management System. Such a process will allow time for major programmatic issues (such as the need for a Monitored Retrievable Storage facility) to be resolved, more information concerning the various at-reactor storage technologies to be developed, and the waste disposal package design and handling requirements to become better defined.

In the meantime, the Department's near-term shipping cask designs will be oriented toward development of the basic designs needed to ship the bulk of the fuel (i.e., maintaining compatibility with the 80 percent that are intact spent fuel assemblies stored in water filled pools). The Department's longer term shipping cask design efforts will consider modifications to these basic designs to maximize the efficiency of handling as much of the remaining 20 percent of the spent fuel as possible, primarily the portion whose storage incorporates the compatibility features discussed above.

Finally, the Department will separately consider how to handle any spent fuel that is stored in ways that do not comply with the compatible techniques established in cooperation with the utility industry, recognizing that such fuel may be subject to delayed acceptance under the terms of the standard disposal contract.

Again, let me thank you for your very helpful response.

Sincerely,



Samuel Rousso, Acting Director
Office of Civilian Radioactive
Waste Management

*HLW. New York
Storage*



Department of Energy
Washington, DC 20585

JAN 4 1989

Honorable Lando W. Zech, Jr.
Chairman
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Chairman Zech:

In response to your letter of November 18, 1988, which provided comments on the "Initial Version Dry Cask Storage Study" (DOE/RW-0196), I would like to thank you for your overall assessment of the document. We attempted to make it a straight-forward, responsible, technical document and were gratified by your response. We will, of course, be responding to all of your comments in the comment response section of the final report. However, in consideration of the significance of the comment in the second paragraph of your letter, concerning compatibility of the various steps in the spent fuel management process, I want to give you an early indication of our current thinking.

The Department agrees that all of the steps in the spent fuel management process should be coordinated to enhance the safety and efficiency of the operations and plans to increase its efforts to ensure that this coordination takes place. This has already been recognized by both the Department and the nuclear utilities, and elements of this general coordination issue have already been identified as topics to be addressed through the process for resolving issues concerning the standard contract for disposal of spent fuel. This contract establishes, among other things, the contractual terms and conditions for the waste acceptance process.

The issue resolution process associated with the contract is a mechanism for identifying and ultimately overcoming obstacles to the effective and efficient implementation of the contract. The issue resolution process was described in the June 1988 issue of the "Annual Capacity Report" (DOE/RW-0191) and is commonly referred to as "the ACR issue resolution process." The Department intends to discuss with the utilities at the next meeting in the ACR issue resolution process the general coordination issue that you have raised, to identify opportunities for and the timing of steps to address any coordination elements that are not already being addressed. Any elements of the general coordination issue that are not appropriate for resolution through the ACR issue resolution process will be taken up separately by the Department working with the utilities, through the auspices of the Edison Electric Institute's Utility Nuclear Waste and Transportation Program (successor to the separate Utility Nuclear Waste Management Group and Transportation Group).

1/10...To EDO for Appropriate Action...Cpys to: RF, Cmr...89-0020

As described in the Department's "Initial Version Dry Cask Storage Study," several different technologies for expanding at-reactor storage are in various stages of development. No single technology is likely to meet the requirements of all the utilities. Furthermore, the utilities believe that they need to retain the flexibility to choose the option that best suits their requirements, while choosing systems that incorporate compatibility elements that are jointly developed based on system requirements.

As more information is developed about each of the technologies, it will be appropriate and natural to consider certain features or interfaces within each of the technology categories for compatibility with the Federal Waste Management System. These features or interfaces could include items such as dimensions, weights, payloads, materials, heat and radiation limits, and handling features.

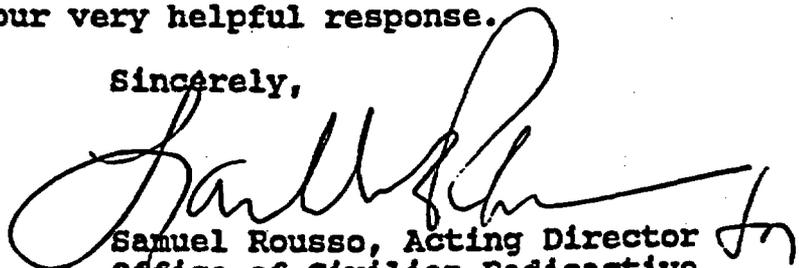
The compatible elements of each of the major types of technology can then serve as focal points for combined Federal and utility efforts to ensure that the various technologies interface satisfactorily with the Federal Waste Management System. Such a process will allow time for major programmatic issues (such as the need for a Monitored Retrievable Storage facility) to be resolved, more information concerning the various at-reactor storage technologies to be developed, and the waste disposal package design and handling requirements to become better defined.

In the meantime, the Department's near-term shipping cask designs will be oriented toward development of the basic designs needed to ship the bulk of the fuel (i.e., maintaining compatibility with the 80 percent that are intact spent fuel assemblies stored in water filled pools). The Department's longer term shipping cask design efforts will consider modifications to these basic designs to maximize the efficiency of handling as much of the remaining 20 percent of the spent fuel as possible, primarily the portion whose storage incorporates the compatibility features discussed above.

Finally, the Department will separately consider how to handle any spent fuel that is stored in ways that do not comply with the compatible techniques established in cooperation with the utility industry, recognizing that such fuel may be subject to delayed acceptance under the terms of the standard disposal contract.

Again, let me thank you for your very helpful response.

Sincerely,



Samuel Rousso, Acting Director
Office of Civilian Radioactive
Waste Management



Department of Energy
Washington, DC 20585

JAN 4 1989

Honorable Lando W. Zech, Jr.
Chairman
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Chairman Zech:

In response to your letter of November 18, 1988, which provided comments on the "Initial Version Dry Cask Storage Study" (DOE/RW-0196), I would like to thank you for your overall assessment of the document. We attempted to make it a straightforward, responsible, technical document and were gratified by your response. We will, of course, be responding to all of your comments in the comment response section of the final report. However, in consideration of the significance of the comment in the second paragraph of your letter, concerning compatibility of the various steps in the spent fuel management process, I want to give you an early indication of our current thinking.

The Department agrees that all of the steps in the spent fuel management process should be coordinated to enhance the safety and efficiency of the operations and plans to increase its efforts to ensure that this coordination takes place. This has already been recognized by both the Department and the nuclear utilities, and elements of this general coordination issue have already been identified as topics to be addressed through the process for resolving issues concerning the standard contract for disposal of spent fuel. This contract establishes, among other things, the contractual terms and conditions for the waste acceptance process.

The issue resolution process associated with the contract is a mechanism for identifying and ultimately overcoming obstacles to the effective and efficient implementation of the contract. The issue resolution process was described in the June 1988 issue of the "Annual Capacity Report" (DOE/RW-0191) and is commonly referred to as "the ACR issue resolution process." The Department intends to discuss with the utilities at the next meeting in the ACR issue resolution process the general coordination issue that you have raised, to identify opportunities for and the timing of steps to address any coordination elements that are not already being addressed. Any elements of the general coordination issue that are not appropriate for resolution through the ACR issue resolution process will be taken up separately by the Department working with the utilities, through the auspices of the Edison Electric Institute's Utility Nuclear Waste and Transportation Program (successor to the separate Utility Nuclear Waste Management Group and Transportation Group).

1/10...To EDO for Appropriate Action...Cpys to: RF, Cms...89-0020

As described in the Department's "Initial Version Dry Cask Storage Study," several different technologies for expanding at-reactor storage are in various stages of development. No single technology is likely to meet the requirements of all the utilities. Furthermore, the utilities believe that they need to retain the flexibility to choose the option that best suits their requirements, while choosing systems that incorporate compatibility elements that are jointly developed based on system requirements.

As more information is developed about each of the technologies, it will be appropriate and natural to consider certain features or interfaces within each of the technology categories for compatibility with the Federal Waste Management System. These features or interfaces could include items such as dimensions, weights, payloads, materials, heat and radiation limits, and handling features.

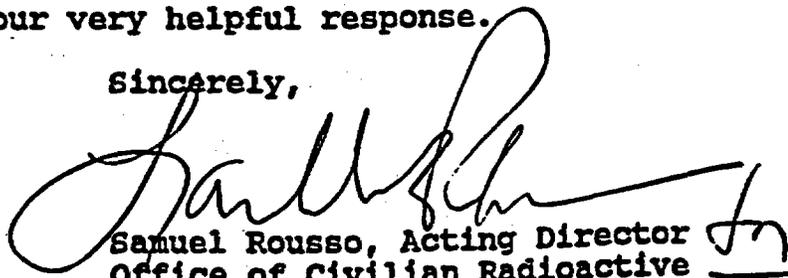
The compatible elements of each of the major types of technology can then serve as focal points for combined Federal and utility efforts to ensure that the various technologies interface satisfactorily with the Federal Waste Management System. Such a process will allow time for major programmatic issues (such as the need for a Monitored Retrievable Storage facility) to be resolved, more information concerning the various at-reactor storage technologies to be developed, and the waste disposal package design and handling requirements to become better defined.

In the meantime, the Department's near-term shipping cask designs will be oriented toward development of the basic designs needed to ship the bulk of the fuel (i.e., maintaining compatibility with the 80 percent that are intact spent fuel assemblies stored in water filled pools). The Department's longer term shipping cask design efforts will consider modifications to these basic designs to maximize the efficiency of handling as much of the remaining 20 percent of the spent fuel as possible, primarily the portion whose storage incorporates the compatibility features discussed above.

Finally, the Department will separately consider how to handle any spent fuel that is stored in ways that do not comply with the compatible techniques established in cooperation with the utility industry, recognizing that such fuel may be subject to delayed acceptance under the terms of the standard disposal contract.

Again, let me thank you for your very helpful response.

Sincerely,



Samuel Rousso, Acting Director
Office of Civilian Radioactive
Waste Management



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 18, 1988

*HLW/Dry Cask
Storage*

United States Department of Energy
ATTN: Mr. Sam Rousso, Acting Director
Office of Civilian Radioactive Waste Management
RW-322 Forrestal Building
1000 Independence Avenue, S.W.
Washington, D.C. 20585

Gentlemen:

I am responding to your September 1, 1988 request for the comments of the Nuclear Regulatory Commission (NRC) on the Department of Energy's (DOE) Initial Version Dry Cask Storage Study (DOE/RW-0196). The Commission's detailed comments are enclosed. In general, we find it a well-balanced presentation of spent fuel storage requirements, of the in-pool consolidated fuel storage and dry storage technologies available to address those requirements in at-reactor storage, and of the impacts and costs of such storage.

The Commission is concerned, however, that inadequate attention is being given to ensure the compatibility of the various steps in the storage, transport, and disposal of spent fuel and thereby to enhance the safety and efficiency of fuel handling. With a proliferation of storage options, it appears likely that fuel to be removed from reactor sites in some instances may have to be returned to reactor pools to be unloaded and then loaded into transportation casks for shipment off site. In addition, subsequent operations at the repository, or a monitored retrievable storage (MRS) site, may be needed to repackage the fuel for ultimate disposal. The Commission believes that radiation exposures and other handling risks should be minimized in the entire process from removing the fuel from the reactor pool the first time to its ultimate disposal. System analysis and action at this early stage could result in minimizing these handling risks, and we suggest that DOE proceed on this course of analysis and action to achieve cask design compatibility to the greatest extent possible.

I hope that our comments on this draft report have been helpful. If you have any questions, please contact Hugh L. Thompson, Jr. (telephone number 492-3352). The Commission looks forward to commenting on your final report.

Sincerely,
Original signed by
Lando W. Zech, Jr.

Lando W. Zech, Jr.

Enclosure:
Detailed Comments

cc: Charles Head, DOE

Originating Office: EDO/NMSS

SECY-88-305

OFFICE	SECY	OCM	OCM				
SURNAME	C. Butts	pm	Zech				
DATE	11/17/88	11/16/88	11/15/88				

NRC COMMENTS ON DOE DRY CASK STORAGE STUDY (DOE/RW-0196)

Page

- viii See the partial paragraph at the top of the page. While as a practical matter topical reports have been referenced in dry storage licensing, there is no requirement to do so. A license applicant could simply submit a new design detailed in the Safety Analysis Report submitted as part of the site-specific license application.
- viii and 13 See the first full paragraph and the sixth full paragraph, respectively. A utility does not apply for a general license. Rather, the utility would register with the NRC to use a certified cask, thus committing to the specified conditions of the general license.
- viii Insert the following under lined wording at the end of first sentence, fourth full paragraph:
- "...for dry storage in that dry cask storage involves a new license under 10 CFR Part 72, while consolidation where it increases the number of assemblies permitted to be stored in the pool involves an amendment to the 10 CFR Part 50 license. To the extent that utilities have consolidated spent fuel rods as a demonstration, they have performed these limited consolidations under 10 CFR 50.59. First of all,..."
- x See the first full paragraph. In the third sentence the words, "for at least 30 years beyond the expiration of the operating license," refer to reactor pool storage also.
- 9 Delete the last sentence from the partial paragraph at the top of the page.
- Revise the second sentence in the first full paragraph by incorporating the underlined words as follows:
- "...all spent fuel storage pools where an evaluation shows that the pool can support the additional weight..."
- 11 See the fourth paragraph. Please note that 10 CFR Part 72 has been recently amended (53 FR 31651, August 19, 1988). It is now entitled "Licensing Requirements for the Independent Storage of Spent Nuclear Fuel and High-Level Radioactive Waste."

- 11 (cont'd) Delete the last paragraph (including the continuation at the top of page 12) and replace it with the following paragraph:
- "Consolidation where it increases the number of assemblies permitted to be stored in the spent fuel pool involves an amendment to the 10 CFR Part 50 license. To the extent that utilities have consolidated limited numbers of spent fuel rods as a demonstration, they have performed these consolidation programs under 10 CFR 50.59."
- 12 See the second full paragraph. Again there is some lack of clarity in explaining the use of a reviewed topical report as a reference in a site-specific application. It may be advantageous to a license applicant to do so, since it is an action that can reduce uncertainty (the proposed design having been reviewed by NRC staff) and effort, but it is not required by regulation.
- 13 Revise the first sentence in the third full paragraph to read:
- "Consistent with this objective, the NRC staff is currently developing a proposed rule for consideration by the Commission which would amend 10 CFR Part 72 to provide a process for..."
- 13 Revise the first sentence of the fourth full paragraph to read:
- "The proposal under development envisions an amendment to 10 CFR Part 72 to specify the following process...
- 29 See the first full paragraph. The first sentence concerning cask loading in the storage pool and boron is incorrect. Nuclear criticality analyses to date have not taken credit for the presence of boron in the water.
- 30 Section 4.1.1.2, paragraph 3. Gesellschaft fur Nuklear Service mbH is improperly identified. This should be General Nuclear Systems, Inc., the United States partnership.
- 34 Section 4.1.2.3, paragraph 1 (same comment as above for p 30).

- 35 Section 4.1.2.3, paragraph 1. This section should note that the technical review panel set up by the NRC to investigate the acceptability of nodular cast iron for transportation casks has been completed. It was concluded that nodular cast iron should not be used as the primary structural material in spent fuel shipping casks based on the material properties and available information at this time.
- 42 Section 4.1.4.2, paragraph 2, line 3. The words, "safety analysis," should be topical report.
- 47 Revise the first sentence in the second paragraph in incorporating the underlined words as follows:

"...with rod consolidation are primarily economic in nature. They include the difficulties..."
- 49 Delete the first two sentences in the second full paragraph and replace them with the following sentences:

"As explained in Chapter 2, consolidation where it increases the number of assemblies permitted to be stored in the spent fuel pool involves an amendment to 10 CFR Part 50 license. To the extent that utilities have consolidated rods as a demonstration, they have performed these limited consolidations under 10 CFR 50.59."

Delete the words "local or" in the third full paragraph, third sentence.
- 50 Revise the wording in the second full paragraph, fourth sentence with the underlined words as follows:

"...and will have to start by 1997 when a full-core..."

Reference: "Spent Fuel Storage Requirements (1987)" DOE/RL-87-11, page 3.13, Table 3.4 shows full core reserve at Millstone 2 not lost until 1997.
- 72 See the first paragraph. The reactor operating license may be amended at the end of plant operating life. Thus, spent fuel may be stored in the reactor pool under a "possession only" license pursuant to 10 CFR Part 50. The reactor license cannot be terminated until the reactor is decommissioned. To fully decommission the reactor, all spent fuel must be removed from the site.

81-85

In the Chapter 6, DOE discusses the use of the Nuclear Waste Fund to support additional at-reactor storage. The study finds that Nuclear Waste Policy Act of 1982 does not authorize DOE to use the Fund for providing such storage. DOE also finds, as a matter of policy, that the Nuclear Waste Fund should not be used for at-reactor storage unless such storage would provide overall benefits to the waste-management system. As these findings do not involve health and safety issues, NRC takes no position thereon.



OM 7 - I C NW

UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON NUCLEAR WASTE
WASHINGTON, D.C. 20555

July 1, 1988

The Honorable Lando W. Zech, Jr.
Chairman
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Chairman Zech:

SUBJECT: PROPOSED RULE ON STORAGE OF SPENT NUCLEAR FUEL IN CASKS AT NUCLEAR POWER REACTOR SITES

During the first meeting of the Advisory Committee on Nuclear Waste, June 27-29, 1988, we met with the NRC Staff to discuss the proposed rule on "Storage of Spent Nuclear Fuel in NRC Approved Storage Casks at Nuclear Power Reactor Sites" (referenced).

Overall, we endorse the development of this rule. Formulation of regulations designed to address this subject on a generic basis will be constructive. We offer the following specific comments:

1. The portion of the rule that restricts the storage of spent fuel at a given site to only fuel that was produced at that site should be re-examined. Since a utility with multiple nuclear power plant sites may desire to centralize its storage of spent fuel at one location, it appears useful to include in the rule guidance for obtaining approval of such an approach.
2. Since the above approach would require that the fuel be transported and ultimately all such fuel will need to be shipped to a site for final disposal, it would appear useful to design the casks with the safety of, and doses associated with, subsequent operations in mind.
3. Finally, since several NRC offices will be responsible for implementing this rule, we urge that careful attention be addressed to the division of responsibilities within the NRC.

Sincerely,

Dade W. Moeller
Chairman

Reference:

U. S. Nuclear Regulatory Commission, Proposed Rule dated June 6, 1988 (7590-01), "Storage of Spent Nuclear Fuel in NRC Approved Storage Casks at Nuclear Power Reactor Sites"