May 4, 2004

Mr. Randall K. Edington
Vice President-Nuclear and CNO
Nebraska Public Power District
P. O. Box 98
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - REQUEST FOR ADDITIONAL INFORMATION RE: LOSS-OF-COOLANT ACCIDENT (LOCA) DOSE CALCULATION METHODOLOGY AND RESOLUTION OF REMAINING LICENSE CONDITION 2.C.(6) ISSUES (TAC NO. MC1572)

Dear Mr. Edington:

By letter dated December 9, 2003, Nebraska Public Power District (NPPD or the licensee) requested the Nuclear Regulatory Commission (NRC) staff to approve its proposed license amendment request for LOCA dose calculation methodology and resolution of remaining License Condition 2.C.(6) issues. The subject amendment request also included responses to a request for additional information (RAI) issued on October 2, 2003.

On April 28, 2004, the NRC staff held a telephone call with the licensee to pose several questions regarding the RAI response. The NRC staff has reviewed the information provided during the April 28, 2004, telephone call and determined that the enclosed RAI is required in order to complete the review and approval of the changes. As agreed upon in a telephone call between Michelle Honcharik and Edward McCutchen on May 3, 2004, the licensee will respond to the RAI within 30 days.

Sincerely,

/RA/

Michelle C. Honcharik, Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosure: RAI

cc w/encl: See next page

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Sincerely,
/RA/

Michelle C. Honcharik, Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosure: RAI

cc w/encl: See next page
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REQUEST FOR ADDITIONAL INFORMATION

ISSUES RELATED TO LOSS-OF-COOLANT ACCIDENT (LOCA)

DOSE CALCULATION METHODOLOGY AND RESOLUTION OF REMAINING

LICENSE CONDITION 2.C.(6) ISSUES

COOPER NUCLEAR STATION (CNS)

All page references are to Attachment 1 of the December 9, 2003, submittal.

1. Page 7 - Please describe how the operator will be able to access both Turbine Stop Valves that are located approximately 8.5 ft. off the floor.

2. Page 8 - Please describe how the walk down was performed. For example, how many individuals/crews participated in the walk down? What actions were actually demonstrated to be performed within the "estimated 30 minutes" to accomplish the task...e.g., did the time include ingress/egress to the valve locations, access to the tools that are 25-30 feet away from the valves, etc? The illustration provided in the December 9, 2003, submittal shows an operator at the valve location dressed in "anti-C's", it appears. Did the 30 minute time estimate include time to dress-out?

3. Page 8 - "With the MSIV [main steam isolation valve] leakage assumed in the LOCA analysis and conservative assumptions, this evolution would be performed well before any radiological release to the Turbine Building could occur from Turbine Valve shaft clearance area leakage. The implementing emergency procedure directs completion within 30 hours." Please explain how the 30 hours time frame to take the required manual actions was derived. What is the minimum amount of time that an operator will have to perform the required actions before encountering a hostile environment?

4. Page 9 - Please explain what modifications were made after refueling outage 21 to improve the capability of the emergency lighting system. "Directly after a LOCA induced shutdown, the area surrounding the Turbine Stop Valves would be hot from residual heat in the piping and valves..." Though the "majority of equipment in the area is insulated," how hot is the area expected to become? Is there need for special personnel protection/equipment?

5. Page 15 (Table 1) - Please explain/clarify: Are all the manual actions in the table required to "configure the MSIV Leakage Pathway," as the table title (and Page 3) indicate? If so, please explain how the manual actions (other than the Turbine Stop Valve adjustments) are accomplished. For example, where are the locations of the other valves in relation to the Turbine Stop Valves? Does the same person who is manually adjusting the Turbine Stop Valves carry out the remaining valve closures? What steps are required to perform the remaining valve manipulations? How much time is required to complete the remaining valve manipulations? How has it been determined that the other valve manipulations can be successfully performed in the time allowable?

Enclosure

Cooper Nuclear Station

cc:

Mr. William J. Fehrman
President and Chief Executive Officer
Nebraska Public Power District
1414 15th Street
Columbus, NE 68601

Mr. Clay C. Warren
Vice President - Nuclear
Nebraska Public Power District
P. O. Box 98
Brownville, NE 68321

Mr. John R. McPhail, General Counsel
Nebraska Public Power District
P. O. Box 499
Columbus, NE 68602-0499

Mr. Paul V. Fleming, Licensing and
Regulatory Affairs Manager
Nebraska Public Power District
P.O. Box 98
Brownville, NE 68321

Mr. Michael J. Linder, Director
Nebraska Department of Environmental
Quality
P. O. Box 98922
Lincoln, NE 68509-8922

Chairman
Nemaha County Board of Commissioners
Nemaha County Courthouse
1824 N Street
Auburn, NE 68305

Ms. Cheryl K. Rogers, Program Manager
Nebraska Health & Human Services
System
Division of Public Health Assurance
Consumer Services Section
301 Centennial Mall, South
P. O. Box 95007
Lincoln, NE 68509-5007

Mr. Ronald A. Kucera, Director
of Intergovernmental Cooperation
Department of Natural Resources
P.O. Box 176
Jefferson City, MO 65102

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P. O. Box 218
Brownville, NE 68321

Regional Administrator, Region IV
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

Jerry Uhlmann, Director
State Emergency Management Agency
P. O. Box 116
Jefferson City, MO 65101

Chief, Radiation and Asbestos
Control Section
Kansas Department of Health
and Environment
Bureau of Air and Radiation
1000 SW Jackson
Suite 310
Topeka, KS 66612-1366

Mr. Daniel K. McGhee
Bureau of Radiological Health
Iowa Department of Public Health
401 SW 7th Street
Suite D
Des Moines, IA 50309

Mr. Scott Clardy, Director
Section for Environmental Public Health
P.O. Box 570
Jefferson City, MO 65102-0570