

#### GT-MHR ADVISORY GROU  $\frac{1}{\sqrt{2}}$

**Constellation**<br>Nuclear

WHO, WHAT & WHY



Pre-application NRC Meeting 12/03/01





**\* PSEG** 



# UTILITY ADVISORY MEMBERS--WHO

\* EntergyNuclear

- *Dan Keuter, Utility Advisory Board Chairman Entergy*

- Nuclear Management Company
- \* Dominion
- \* Omaha Public Power District
- \* PSE&G
- \* Constellation
- \* Other





#### **UAG STRUCTURE**



# ACADEMIC ADVISORY GROUP (AAG)

# Members

- \* University of California at Berkley
	- Per Peterson, Chair
- \* MIT
- \* University of Michigan
- \* Oregon State
- \* Texas A&M
- Wisconsin
- \* University of Florida
- \* Ohio State
- \* Purdue
- \* Penn State

# 

# UTILITY. ADVISORY MEMBERS--WHAT

UAB Purpose

- \* Provide Industry Guidance on:
	- Development and Deployment of GT-MHR
	- Coordination with Industry (NEI, NRC, EPRI & DOE) Activities to Maximize Efficiency & Prudence of Efforts
- \* Facilitate Incorporation' of Lesson Learned From Earlier Gas Reactor Licensing and Operational **Experience**

U.S. Commercialization of GT-MHR

# "SUPER-SAFE" DESIGN --WHY

\* Meltdown Proof

-High Temperature Fuel Capacity

-Passive & Conductive Cooling

\* Low Security Risk

-Below-Grade Construction Arrangement -Silo

-No Power Required

# \* No Active Safety Systems

- -No Emergency Core Cooling
- -No Emergency Diesels

\* Proliferation Resistant

- -Ceramic Fuel Makes Retrieval Difficult
- -Low Fissile Fuel Volume Fraction
- -High Fuel Burn-up / Pu Degradation

# GEN IV Benefits at CCGT Prices

# **Gas Turbine - Modular Helium Reactor Safety Approach**

みつし きょうかつがく しちかい ポプチ

44、明、好、颜料原料

地区的原料和图案

#### **Example 2 Presented to NRC Staff** 3 December 2001

**Laurence L Parme Manager, Reactor Safety & Licensing General Atomics** 

OMICS

# Modular *Gas-Reactor Safety Approach* Differs From *Earlier Reactor Designs*

- GT-MHR safety emphasizes
	- Keeping radionuclides at source during all accidents
	- Minimizing reliance on active/complex engineered systems
- \* Passive safety design based on reoptimized application of established HTGR technology
	- High temperature compatible fuel and core
	- Single phase, chemically & neutronically inert coolant
	- Specially tailored core power and geometry

Conservative, robust design with defense-indepth remain keystones of safety



# **Key to GT-MHR Safety Multiple Ceramic Fuel Coatings**



**Coated Particles Remain Intact Even at Very High Temperatures** 



*GENERAL ATOMICS* 

 $L-266(1)$  $7 - 28 - 94$  $W-9$ 

# Safety Focused on Assured Fuel Particle Integrity

*GENERAL ATOMICS* 



- Fission (heat generation) shut down without rod motion
- Heat removal assured by reactor design
	- Low power density
	- Low thermal rating per module
	- Annular core and high L/D ratio
- \* Chemical attack limited by design & materials
	- No high pressure water source in gas-turbine plant
	- Nuclear graphite, geometry, and limited air control-
	- potential oxidation

## **Heat Generation Stops During Loss of Cooling Without Rod Motion**



## **Core Temperatures Maintained at Safe Levels With and Without Reactor Trip**



IERAL ATOMICS

#### **Passive Heat Removal Changes Reactor Design Philosophy**



SIZED AND CONFIGURED TO WITHSTAND EVEN A SEVERE ACCIDENT



 $1-222(1)$ 1-12-96

 $\mathbf{I}$ 

### **ANNULAR CORE LIMITS FUEL TEMPERATURE DURING ACCIDENTS**



## **Temperature Gradient Provides Driving Force for Residual Heat Removal**

 $\blacksquare$  . The contract of the set o





#### Heat Removed Passively  $\overline{1}$ Without Circulation or Coolant



 $\frac{1}{2}$  $\label{eq:2.1} \begin{split} \mathcal{L}_{\text{max}} &= \frac{1}{2} \sum_{\mathbf{q} \in \mathcal{Q}} \frac{1}{\mathcal{A}(\mathbf{q})} \mathbf{q} \mathbf{q} \, , \\ \mathcal{L}_{\text{max}} &= \frac{1}{2} \mathcal{A}(\mathbf{q}) \mathbf{q} \, , \\ \mathcal{L}_{\text{max}} &= \frac{1}{2} \mathcal{A}(\mathbf{q}) \mathbf{q} \, , \\ \mathcal{L}_{\text{max}} &= \frac{1}{2} \mathcal{A}(\mathbf{q}) \mathbf{q} \, , \\ \mathcal{L}_{\text{max}} &= \$ 

#### FUEL TEMPERA TURES REMAIN BELOW DESIGN LIMITS DURING LOSS OF COOLING EVENTS



 $L-340(3)$ 11-16-94

# Risk From Water Ingress **Significantly Reduced in GT-MHR**

- \* No steam generators therefore no high pressure steam source
- \* Precooler and intercooler water pressure below primary coolant operating pressure
- \* Low water pressure in heat exchangers greatly reduces potential for water ingress during normal operation
- \* Liquid water transport to core under depressurized conditions as unlikely as in steam cycle system
- $\mathcal{L}_\mathcal{L} = \mathcal{L}_\mathcal{L} = \mathcal{L}_\mathcal{L}$ ju *CENERAL ATOMICS* $\label{eq:2.1} \frac{1}{2}\sum_{i=1}^n\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\pi i}\sum_{j=1}^n\frac{1}{2\$

## **Leakage is Out of Primary Coolant System Following Tube Break**



*VERAL ATOMICS* 

## Inherent and Passive Features **Control Air Attack**

- 
- \* Non-reacting coolant (helium)
- Embedded ceramic coated particles
- Air ingress limited (requires failure of Class 1 vessels) - <sup>322</sup> - 1938 - 1948 - 1958 - 1958 - 1958 - 1958 - 1958 - 1958 - 1958 - 1958 - 1958 - 1958 - 1958 - 1958
- \* Below grade, closed reactor silo (isolation)
- Air flow rate limited by core flow area (L/D  $>$  700)
- \* Slow oxidation rate (nuclear grade graphite)

 $\label{eq:2.1} \frac{1}{2} \sum_{i=1}^n \frac{$ 4 GENERAL *ATOMICS*

## Low Potential for Graphite Fires

- \* Test results successfully compared favorably to computer code (AIP) predictions
- $\cdot$  Extremely low probability of burning graphite
	- requires temperatures above those during operation or accidents, and
	- requires large quantities of air
- \* MHTGR analyses show introduction of air results in limited, decay heat driven oxidation



# **Graphite Oxidation Limited by Available Air**



### Mass Transfer, Core Temperature, & **Graphite Purity Limit Oxidation Rate**

![](_page_23_Figure_1.jpeg)

## Below-Grade Siting Augments Enhanced Safety

![](_page_24_Figure_1.jpeg)

# GT-MHR Optimization of Established Gas *Reactor* Features Provides

- Enhanced, easily understood safety
- Assured accomplishment of safety functions with simple, passive features
- \* Limited consequences, even for beyond design basis accidents

SAR intended to provide full demonstration of safety

![](_page_25_Picture_6.jpeg)

## **Pre-Application Licensing Plan** for the **Gas Turbine - Modular Helium Reactor**

#### **Presented to NRC Staff 3 December 2001**

#### **Laurence L Parme**

**Manager, Reactor Safety & Licensing General Atomics** 

![](_page_26_Picture_4.jpeg)

# Objectives ol Pre-application Interactions

- \* Seek NRC licensability statement on
	- GT-MHR design licensability
	- Safety and licensing approach
- \* Establish foundation for future application
	- Combined License (COL) for a first module
	- Design Certification for the GT-MHR plant
	- NRC Advanced Reactor Policy guidance encourages earliest possible interaction
		- Most effective regulation for advanced reactors
		- Timely, independent assessment of the safety characteristics of advanced reactor designs

![](_page_27_Picture_10.jpeg)

Licensing Plan Outlines *Pre-application Activities*

- \* Identify and schedule pre-application activities
- \* Define objectives and expectations

*W*

- **Ensure planned activities facilitate future application**
- Record NRC staff / GT-MHR agreement on planned<br>activities

![](_page_28_Picture_5.jpeg)

# *Significant History ol HTGR Licensing Exists*

- Peach Bottom 1 ('67 -'74)  $\bullet$ 
	- Construction License
	- Operating License
	- Decommissioned
- \* Fort St Vrain ('79 '88)
	- Construction License
	- Operating License
	- Decommissioned
- i Large HTGR (mid '70s)
	- Summit 1 & 2 construction permit issued
	- Fulton I & 2 PSAR submitted

![](_page_29_Picture_12.jpeg)

![](_page_29_Picture_13.jpeg)

![](_page_29_Picture_14.jpeg)

![](_page_29_Picture_15.jpeg)

# MHTGR - Starting Point fol GT-MHR Interaction

- **Steam cycle MHTGR interaction with** NRC in mid-1 980s
- Key items submitted for review
	- Top-level regulatory criteria
	- Risk informed licensing bases
	- Probabilistic Risk Assessment
	- Prelim Safety Information Document
- \* Extensive review by NRC staff and national labs
- MHTGR evolved to Brayton cycle GT-MHR in early 1990s
	- Similar safety characteristics
	- Similar licensing approach

![](_page_30_Picture_11.jpeg)

![](_page_30_Picture_12.jpeg)

# 5 Areas Proposed for **Discussion**

- **Programmatic and Process Topics**
- Licensing Approach
- Technology Development
	- Fuel
	- Graphite
	- Metals
- \* Design Description
- \* Accident Analyses

![](_page_31_Picture_9.jpeg)

# **Pre-application Topics Address draft SAR**

![](_page_32_Figure_1.jpeg)

# Proposed Schedule of Activity Tied to Design

![](_page_33_Picture_69.jpeg)

وزكة بالاستنزاع وليشارك وأستحصر استوصاف والمتناه المتخالف المتحادية المتحادة المتحادث والمتحادث والمتحدثان والمحادث

![](_page_33_Picture_2.jpeg)

# Interaction Process during Pre-application Phase

- Topical Meeting
	- GT-MHR presentation on topic or issue
	- NRC comments or questions at meeting
- \* GT-MHR follows up meeting with written documentation of presentation
	- Letter
	- Report
	- Draft SAR format
- \* Written comments or queries from NRC
- \* GT-MHR response

Licensability statement is final comment

![](_page_34_Picture_11.jpeg)

 $\label{eq:2.1} \frac{1}{2} \sum_{i=1}^n \frac{$ 

# GT-MHR PLANT DESIGN **OVERVIEW\***

,.

www.c.c.com & proteintile intrate primes of international distance reservation it and the same of the de die provinsie in Grund (bie terstaatschapper) van die tuise van die eerste buite van die komponisie van die 

#### **Example 20 Presented to NRC Staff**

. . . . . . . . . . . . . . . . December 3, 2001

#### $\label{eq:2} \frac{1}{2} \int_{\mathbb{R}^3} \left[ \frac{1}{2} \int_{\mathbb{R}^3} \left( \frac{1}{2} \int_{\mathbb{R}^3} \frac{1}{2} \int_{\mathbb{R}^$ Arkal Shenoy **Director, Modular Helium Reactor** General Atomics

 $\mathcal{L}(\mathcal{S})$  and  $\mathcal{L}(\mathcal{S})$  are the set of the set of  $\mathcal{S}$  $\mathcal{L}^{\mathcal{A}}$  and  $\mathcal{L}^{\mathcal{A}}$  are the set of the s

M-273(1)<br>8-27-01 **CENERAL ATOMICS** 

 $\pmb{\cdot}$ 

# Presentation Outline

- \* Design goals & requirements
- \* Design description
- **International Program**
- \* Commercial Program
- Design status and deployment schedule
- **Conclusions**

![](_page_36_Picture_7.jpeg)

# **PLANT USER REQUIREMENTS**

- Plant sizes:300-1 200 MW(e) range
- Equivalent availability >80%-
	- \* Meet existing safety and licensing criteria with no release.requiring public sheltering greater than 5X10-7 per year
- \* 10% power cost advantage over Fossil Fuel
- 

M-273(5) *G* GENERAL *A TOMICS* 8-27-01

# ADDITIONAL REQUIREMENTS

- \* Utilize inherent characteristics for passive safety
- Do not require operator action (insensitive to operator error)
- \* Provide long time intervals for corrective action
- Actively support of users and suppliers during design and licensing

![](_page_38_Picture_5.jpeg)

8-27-01

### **MODULAR HTGR DEVELOPMENT FOCUSED** ON GEN IV GOALS (for past 15+ years)

- **Modular HTGR first conceptualized in early '80s to** provide simple, enhanced SAFETY
	-
- GT-MHR conceptualized in early '90s to provide enhanced ECONOMICS
	-
- Gas reactor TRISO coated particle fuel form ideal for spent fuel WASTE
	-
- Fissile fuel inventory, isotopic composition, and fuel form provides PROLIFERATION resistance

# **MHR** DESIGN **A** PPROA CH

- **Utilize inherent characteristics** 
	- Helium coolant inert, single phase
	- Refractory coated fuel high temp capability, low release
	- Graphite moderator high temp stability, long response times
- \* Utilize existing technology, successfully demonstrate components and experience
- **Develop simple modular design** 
	- Small unit rating per module
	- Below-grade installation
- Develop passively safe design
	- Annular core, large negative temperature coefficient
	- Passive decay heat removal system
	- Minimized powered reactor safety systems

![](_page_40_Picture_13.jpeg)

8-27-01

# SUMMA R Y DESCRIPTION OF **G T-MHR** DESIGN

- Plant Design  $\bullet$ 
	- Electrical output 286 MW(e) per module, efficiency =  $48\%$
	- Four identical reactor modules located below grade
	- Each module includes Reactor and Power Conversion System
- **Reactor System Design** 
	- -G00 MW(t), 102-column annular core.
	- $-$  Hexagonal prismatic blocks similar to FSV
	- TRISO ceramic particle fuel
	- Redundant reactivity control system
- **Power Conversion System Design** 
	- Generator, turbine, and two compressor sections on a single shaft
	- **Magnetic bearings**
	- Plate-fin recuperator
	- Cross-counterflow, water-cooled precooler and intercooler

M-273(26) **+ GENERAL ATOMICS** 

8-27-01

Report of the Mary Observal **GT-MHR COMBINES MELTDOWN-PROOF ADVANCED REACTOR WITH HIGH EFFICIENY GAS TURBINE** 

> **POWER LEVEL 600 MWt**

![](_page_42_Figure_2.jpeg)

![](_page_43_Figure_0.jpeg)

#### **U.S. AND EUROPEAN TECHNOLOGY BASES FOR MODULAR HIGH TEMPERATURE REACTORS**

is a clear superinted and an except of the control of the second of the second second and control of the second of the second of the second secon

#### BROAD FOUNDATION OF HELIUM REACTOR TECHNOLOGY

![](_page_44_Figure_2.jpeg)

**GENERAL ATOMICS** 

#### MODULAR HELIUM REACTOR CHARACTERISTICS **ATTRACTIVE FOR GEN IV GOALS**

![](_page_45_Picture_1.jpeg)

 $L-271(3)$ <br>6-6-95

- Helium gas coolant (inert)
- Refractory fuel (high temperature capability)
- Graphite reactor core (high)temperature stability)
- Low power density (order of magnitude lower than LWRs)
- . Demonstrated technologies
- EFFICIENT, RELIABLE PERFORMANCE WITH INHERENT SAFETY

![](_page_45_Picture_8.jpeg)

## CERAMIC FUEL RETAINS ITS INTEGRITY UNDER SEVERE ACCIDENT CONDITIONS

![](_page_46_Figure_1.jpeg)

![](_page_47_Figure_0.jpeg)

#### POSSIBLE HEAT REMOVAL PATHS WHEN NORMAL POWER CONVERSION SYSTEM IS UNAVAILABLE

A CARL A LINE OF THE LINE OF THE ALLEN CONTRACTOR COMPANY COMPANY - THE REAL OF STANDARD AND A CHARGAN CONTRACT ON A SURVEY OF THE BASIC COMPANY OF THE COMPANY OF

![](_page_48_Figure_1.jpeg)

**INHERENT CHARACTERISTICS** 

![](_page_48_Picture_3.jpeg)

 $L-266(2)$ 7-28-94

ka an ing mga mga kalalalang kalalang ang kululang tarak

## GT-MHR DEVELOPED FOR ENHANCED ECONOMICS

 $\bullet$ 

- Thermal Efficiency of HTGR Power Plants with Rankine (Steam) Cycle Limited to  $~28\%$
- **Direct Gas Turbine (Brayton) Cycle Long Time** Vision and Incentive for HTGR Development
- \* Gas Turbine Brayton Cycle Improves Economics
	- Significantly increases thermal efficiency
	- Significantly reduces plant equipment requirements

*.GENERAL* A.OMICS

# **HIGH TEMPERATURE GAS REACTORS HAVE** UNIQUE ABILITY TO USE BRAYTON CYCLE

![](_page_50_Figure_1.jpeg)

![](_page_50_Picture_2.jpeg)

 $L - 029(4)$ 6-9-94

![](_page_51_Figure_0.jpeg)

ERAL **ATOMICS** 

M-272(38)<br>8-27-01

### GT-MHR FLOW SCHEMA TIC

![](_page_52_Figure_1.jpeg)

L-271(12a)<br>8-14-94 A-36

# **TECHNOLOGY ADVANCEMENTS HAVE ENABLED THE GT-MHR**

- Small Passively Safe Modular Helium Reactor
	- turbine size requirements reduced
	- insensitive to turbine failure accidents
- **Large Gas Turbine Engines** 
	- significant increase in industrial applications
	- size now match modular reactor size
- **Magnetic Bearings** 
	- eliminates oil ingress concerns
	- improves performance and reliability
	- rapidly increasing industrial experience; larger sizes
- Compact Heat Exchangers
	- dramatically improves efficiency
	- size improves design integration
	- extensive fossil operating experience

![](_page_54_Figure_0.jpeg)

# CONTROL BASED ON-3600-RPM **SYNCHRONIZED GENERATOR**

- Reactor controlled to maintain constant turbine inlet temperature during normal power operation
- Turbine bypass valve provides short term control  $\overline{\mathbb{R}}$ . Rapid load following  $-$  Machinery protection
- Inventory (coolant pressure) control provides long term load control while maintaining high cycle efficiency
- Motorizing generator with frequency convertor provides startup and shutdown cooling capability

8-27-01

![](_page_55_Picture_5.jpeg)

a.

![](_page_56_Figure_0.jpeg)

# **GT-MHR OFFERS MAJOR ENVIRONMENTAL BENEFITS**

in to be completed and only about the abilities of the same was and the solution of the same of the best of the construction of the same

![](_page_56_Figure_2.jpeg)

![](_page_56_Figure_3.jpeg)

 $L-592(2)$ 11-14-95

#### GT-MHR IS PROLIFERA TION RESISTANT AND -SA TISFIES SAFEGUARD REQUIREMENTS

a

- GT-MHR utilizes low enriched fuel
	-
- Fuel particle refractory coatings make fissile material retrieval difficult
- Low fissile material volume fraction makes diversion of adequate heavy metal quantities difficult
- High spent fuel burnup degradation of Pu isotopic composition make it unattractive for weapons
- Neither a developed process nor capability anywhere in world for separating fissionable material from GT-MHR'spent fuel'
- $\clubsuit$  General atomics

# IN SUMMARY, GT-MHR MEETS ALL DESIGN REQUIREMENTS

- High level of inherent safety eliminating core melt without operator action
- Brayton cycle power conversion system for high<br>thermal efficiency (~50% higher than LWRs)
- Low electricity generation cost (reduced equipment capital and O&M; high thermal efficiency)
- Significantly reduced environmental impact
- Superior radionuclide retention for long-term spent disposal
- **High proliferation resistance**

![](_page_58_Picture_7.jpeg)

# **GT-MHR** Being Designed for Prototype in Russia for Disposition of WPu

- April 1993: Minatom/GA MOU on joint GT-MHR development for commercial units
- June 1994: Russia proposes to build GT-MHR at Seversk to burn Russian WPu
- July 1994: GA and Minatom each pledge \$1M for initial work $\mathbb{R}^n$  , and the set
- an de approprie de la contrata de la construcción ባለርስ አለምነት ምክሮም Jan 1996: Framatome & Fuji Electric join GT-MHR program
- Dec 1997: Team completes Conceptual Design
- Sept 1998: GT-MHR becomes an option within the US/RF Pu Disposition Strategy
- Jan 2000: Work started on Preliminary Design + GENERAL *ATOMICS*

# G T-MHR NOW BEING DE VEL OPED IN INTERNA TIONAL PROGRA M

- In Russia under joint US/RF agreement for management of surplus weapons Plutonium
- \* Sponsored jointly by US (DOE) and RF (Minatom); supported by Japan and EU

• Conceptual design completed; preliminary design complete early 2002

![](_page_60_Picture_4.jpeg)

.a.. ......... . . . . . -. = . .

# INTERNATIONAL GT-MHR PROGRAM'

![](_page_61_Figure_1.jpeg)

a a chuidhean an chomhair an t-air.<br>Tagairtí agus an chuid an chuid an chuid **GENERAL ATOMICS** 

# **COMMERCIALIZATION PROGRAM**

![](_page_62_Picture_1.jpeg)

والرهاجين والمتماطن للأسروق الكرواني والمراوي الممالح والمتواط المراق والمتعادلات والمتحدود والراد

#### Plant construction can start in 5 years

![](_page_62_Picture_3.jpeg)

![](_page_63_Figure_0.jpeg)