ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam REV: 0 Scenario 04-1	
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
 EXAMINATION SCENARIO	DATE: 1/18/04 PAGE: 1 of	f 22
Written by: <u>Kenneth Masker</u> License Instructor	Date:1/18/04	_
Technical Review: <u>Doug Peterson</u> License Instructor	Date: <u>1/26/04</u>	
Time validated <u>80</u> minutes By: <u>K. Mas</u> Lice	<u>ker</u> Date: <u>2/2/04</u> nse Instructor	-
Date of exam:		
Examinees	Evaluators	
Final review Gumm License Instructor	Date: 3/29/04	_
Approved for use <u>Jame New</u> Supervisor License or Designee	Date: <u>3/29/04</u>	-

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 2 of 22

1. SCENARIO OVERVIEW

- 1.1 The plant is initially at 2-3% power during a startup following a mid cycle maintenance shutdown. Procedure O-1.2 is complete up to starting the first MFW pump and placing MFW in service. The TDAFW Governor has failed but this has not yet been detected.
- 1.2 The operators will place MFW system in service and secure AFW as per procedure O-1.2.
- 1.3 Przr Level Transmitter LT-427 fails low causing letdown isolation. The operators will defeat the channel per ER-INST.1 and restore Letdown Operation per S-3.2E.
- 1.4 The "A" S/G ARV controller fails causing the valve to open fully. The operators should take manual control of the ARV and close the valve.
- 1.5 A leak develops on a CRDM housing ramping up to 25 gpm. Shortly after the leak develops, the MRPI coil stack on the associate CRDM fails (shorts) due to the coolant leak. The operator should enter AP-RCS.1. After performing action to stabilize the RCS inventory, the operator should determine that a plant shutdown is required and transition to 0-2.1 or AP-TURB.5.
- 1.6 The leaking CRDM housing fails causing an 800 gpm LOCA and an ejected RCC. SI will auto actuate (possibly manual actuation). "A" SI Train Sequencer fails to actuate and "B" SI pump fails to auto start resulting in inadequate SI pump running. Both MDAFW pumps fail to auto start resulting in no AFW available. The operators should trip the Rx per AP-RCS.1. step 1 and enter E-0. They should start the failed ECCS equipment and verify other action per E-0. They should then transition to E-1 and then ES-1.2. The scenario is terminated when a cooldown is started as per ES-1.2.

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 3 of 22

2. SCENARIO OBJECTIVES

- 2.1 RO(s) Demonstrate the ability to monitor and operate the MCB, diagnose events and implement step of Normal, Abnormal and Emergency Procedures, interact with other members of the crew in a manner which facilitates the crews response to events which occur. Respond to the following events:
 - 1) Normal Operations
 - Start MFW pump and transition to MFW
 - 2) Instrument Component Malfunctions
 - Przr Level LT-427 fails low/letdown isolation
 - S/G ARV controller fails causing the ARV to fail open
 - RCS leak requiring shutdown
 - 3) Emergency Events
 - Ejected Rod/SB LOCA
 - ECCS equipment failures

SRO - Demonstrate the ability to supervise the Control Board operators, diagnose event and choose and implement response procedures, interact with other members of the crew in a manner which facilitate the crews response to events which occur, determine Tech Spec required actions.

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam REV: 0 Scenario 04-1	
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 4 of 22

3. CRITICAL TASKS (CTs)

<u>CT #1_E-O-I</u>

Task: Establish flow from at least two SI pumps before transition out of E-0.

- Cues: SI actuated
 - RCS pressure less than SI pump shutoff head
 - Less than two SI pumps indicating running

Indication: Manipulation of controls to establish flow from at least two SI pumps Feedback: Indications that two SI pumps are running

<u>CT #2 E-I-C</u>

Task: Trip all RCP within five minutes of reaching the Trip Criteria

Cues: Indications that RCP Trip Criteria are met

Indication: Manipulation of controls as required to trip both RCPs

Feedback: Indications all RCPs are tripped

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 5 of 22

<u>Notes</u>

4. INSTRUCTOR ACTIONS

Problem Time

	Actions
4.1 Set up	the simulator per OTG-3.2
For 200 4.1.1	04 License Exam saved as IC-171 Reset simulator to IC-21 MOL 10-8 Amp Xe-Free
4.1.2	Increase power to 2-3% and complete 0-1.2 up to step 5.4.3.18 and attachment MFW
4.1.3	Insert MAEF FDW12 ORPM
4.2 Initia	l Conditions
4.2.1	LT-427 Fails Low MALF PZR03B, 0%, 0 ramp Trigger 1
4.2.2	S/G A ARV Fails Open MALF STM04A 100%, 10 sec ramp Trigger 2
4.2.3	CRDM Leak/Coil Stack Failure MALF RCS02A, 25 gpm, 300 sec ramp
	Trigger 3 MALF ROD13C-G11, 300 sec TD Trigger 3
4.2.4	Ejected Rod/SI and AFW Pump Fail to Start MALF SIS02A, 1 no manual Trigger 4 MALF RPS07B
	Trigger 4 MALF RPS07K
	Trigger 4 MALE PPS07L
	Trigger 4
	ramp, 10 sec TD Trigger 4
	TTAACT 4

10 min 4.3 LT-427 Failure Trigger 1

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 6 of 22

20	min	4.4 S/G ARV Fails Open Trigger 2 If called to, manual isolate valve LOCA STM02, 0%, 120 sec Ramp, 120 sec TD
25	min	4.5 RCS Leak/MRPI Coil Stack Failure Trigger 3 When called to investigate AB System No leak found
30	min	4.6 Ejected Rod/Safeguard Equipment fails to Auto Start Trigger 4

4.7 Termination Criteria Terminate scenario when a cooldown is established in ES-1.2, unless directed otherwise by the Lead Examiner.

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 7 of 22

5. TURNOVER INFORMATION

5.1 The plant is at ~2% power during a startup following a week long maintenance shutdown. C_B - 1336 ppm Xenon Free. 0-1.2 Complete up to step 5.4.3.18 and MFW A Attachment up to step 9.0.

5.2 Equipment Out of Service

None

5.3 Work in Progress

Startup of A MFW Pump transfer to MFW securing AFW

5.4 Planned Work

Turbine Roll up and Synchronization

5.5 Significant Events

None

5.6 <u>Remarks</u>

Continue actions in O-1.2 to place plant on line.

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Ejected RCCA		
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 8 of 2	22
6. <u>EVALUATION</u> Event: <u>1</u> Event Title: _	Start MFW Pump			
Expected Respon	<u>se/Behavior</u>			
Response:			RATING	<u>n/a</u>
BOP/SRO	Take MFW out of Pull St	cop		
BOP/SRO	Ensure Recirc Valves on • AOV 4147 • AOV 4262 (Local call	pen AO)		
BOP/SRO	Start A MFW Pump			
BOP/SRO	Open MFW Discharge Valv	<i>r</i> e MOV 3977		
SRO	Notify AO to close 3977	7A (LOA FDW52)		

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
 EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 9 of 22
BOP/SRO	Verify MFW Regulating Operation • HCV-466 • HCV-476 • HCV-480 • HCV-481	Valve and Bypass Valve	
BOP/SRO	Place MFRV/Bypass Valvo desired	es in Automatic as	
BOP/SRO	Stop Both AFW Pumps		
BOP/SRO	Realign AFW for Normal	Operations	

ROCHESTER	GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0	
R. E. GIN	NA NUCLEAR POWER PLANT	TITLE: Ejected RCCA		
EX	AMINATION SCENARIO	DATE: 1/18/04	PAGE: 10 of	22
6. <u>EVALUATI</u> Event: <u>2</u> Event Title:	<u>ON</u> LT-427_Przr_Level Fails L	10W		
Expected Respo	nse/Behavior			
CUES: Przr Lo Letdown Przr He Chargin VCT Lev	w Level Alarm Isolation aters Trip g Pump Speed Alarm el Decreases			
Response:			RATING	<u>n/a</u>
RO/CRF	Recognize LT-427 has fa Letdown is isolated/Pra	ailed zr Heaters tripped		
CRF	Enter ER-INST.1			
RO/CRF	Monitor Przr Pressure			
RO/CRF	Place Charging to manua	al/minimize speed		
RO/CRF	Stop one charging pump			
RO/CRF	Adjust HCV-142 to Contr	col Lab Seal AP		
RO/CRF	Close AOV-427, 200A, B,	. 202		

ROCHESTER	GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 11 of 2
CRF	Determine if excess L/I service) should be placed in	
BOP/SRO	Defeat Channel per Atta	achment	
RO/SRO	Restore Przr Heaters		
RO/SRO	Restore Normal Letdown • PCV-135 TCV-130 Manual @ 40% • Open 427/371 • Open 200A or B • Return Controllers to	per S-3.2E Auto	
RO	Restore Charging to nor	mal	
SRO	Evaluate Tech Specs • 3.3.1-1 Function 8 • 3.3.3-1 Function 2		

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Ejected RCCA		
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 12 of	22
6. <u>EVALUATIO</u> Event: <u>3</u> Event Title: <u>-</u>	<u>N</u> "A" S/G ARV Fails Open			
Expected Respon CUES: Tavg – L A S/G Pr ARV indi	<u>se/Behavior</u> ecreasing essure Decreasing cates full open			
Response:			RATING	<u>n/a</u>
BOP/SRO	Recognize ARV full open		<u></u>	
SRO	Direct RO to close ARV m	anually		
BOP	Take ARV to manual close			
BOP/RO/SRO	Stabilize plant followin	g transient		

.

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Ejected RCCA		
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 13 of	22
7 <u>EVALUATIC</u> Event: <u>4</u>	<u>N</u>			
Event Title:	CRDM Leak			
Expected Respon CUES: CNMT Rad RCS leak MRPI Fai	<u>se/Behavior</u> liation Increasing age increasing (charging p lure Alarm on MRPI CRT	oump speed increasing)		
Response:			RATING	<u>n/</u> 2
RO/SRO	Recognize indications of	f increased leakage	- <u></u>	
SRO	Enter AP-RCS.1 RCS Leak			<u> </u>
RO/SRO	Monitor Przr Level • Increase Charging Flow • Isolate L/D if necess	w ary		
RO/SRO	Verify VCT Makeup System	m Alignment		
RO/SRO	Verify RCS Leakage in C • Rod Monitors • A Sump	NMT		
SRO	Send AO to check Aux. Bu	uilding		

	ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
	R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
2	EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 14 of 22

RO/SRO	Check CCW for Leak		
RO/SRO	Check CVCS for Leak • L/D Flow, Press • Charging/Seal Injection Flow • Aux. Building Radiation		
RO/SRO	Check PRT for Leak • Level • Pressure		
BOP/SRO	Check S/G for Leakage • S/G Radiation		<u></u>
RO/SRO	Check Accumulator Levels		
RO/SRO	Check RCP Seal Leakoff		
RO/SRO	Check RCDT Leak Rate		
SRO	Call AO to check Valve Leakoff Temperature	·····	
RO/SRO	Stabilize Plant Conditions		
SRO/RO/BOP	Check MCB Annunciators	·	

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 15 of 22

SRO Determine a Tech Spec Shutdown Required

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Ejected RCCA		
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 16 of	22
EVALUATIO	<u>N</u>			
vent: 5				
	WEDI Coil Stock Tails			
vent litle: _	MRPI COIL Stack Fails			
Expected Respon	se/Behavior '			
CUES: MRPI CRT	Alarm and Loss of Indicat	tion for Rod G-11		
esponse ·				
tesponse:			RATING	<u>N/</u>
RO/SRO	Recognize Condition			
,			······	
SRO	Enter AP-RCC.2 RCC/RPI	Malfunction		
RO/CRF	Check the following: • Dropped Rod Alarma C-	14 0-29		
	• Tavg Trend	14, C-23		
BOP/CRF	Check for			
	• G-15 Steam Dump Armed			
	- Generator Hoad > 15 M	177		
	.			
CRF	Determine that AP-TURB. entered	1 should not be		·
BOP/PO/SPO	Stabilize Plant			
LOF / RO/ SRU	SCADITIZE FIGUE			

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 17 of 22

RO/SRO	Verify Control Rod Alignment and QPTR	
RO/SRO	Determine that an MRPI has failed	<u> </u>
SRO	Review Tech Spec 3.1.7	

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Ejected RCCA		
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 18 of 22	
7. <u>EVALUATIC</u> Event: <u>6,7,8</u> Event Title: <u></u>	N Ejected Rod, SBLOCA			
Expected Respon	se/Behavior			
CUES: Rx Power Przr Pre CNMT Rac	r Increases ess and Level Decrease rap liation Increases	dly		
Response:			RATING N	/7
RO/SRO	Determine Leakage Exce	ed Charging Capacity	<u> </u>	<u></u>
RO	Trip Rx		<u></u>	
RO/BOP	Perform E-0 Immediate : • Trip Rx • Turbine Stop Valve • Vital Busses Energize • SI	Actions	. <u></u>	
RO/SRO	Determine SI needed Manually Actuate		<u> </u>	
			<u>SAT</u> U	NS
CT #1	Establish flow from at least transition out of E_{-0}	ast two SI pumps before		

ROCHESTER	GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0	
R. E. GIN	INA NUCLEAR POWER PLANT	TITLE: Ejected RCCA		
EX	AMINATION SCENARIO	DATE: 1/18/04	PAGE: 19 of 22	
RO/SRO	Manually Start • CNMT Recirc Fans • Both AFW Pumps			
RO/CRF	Verify CNMT Isolation		<u> </u>	<u> </u>
<u> </u>			<u>SAT</u> UI	ISAT
CT #2	<pre>Trip all RCP with five minu trip criteria • Two SI pumps running • RCS press - Highest S/G p (400 adverse)</pre>	utes of reaching the press < 175 psid		
RO/CRF	Verify CCW pump running			
RO/CRF	Verify SI Flow			
BOP/SRO	Verify AFW Valve Alignme	ent	<u>-</u>	
BOP/SRO	Control S/G Levels (Cont	cinuous Action)	<u> </u>	
RO/SRO	Check SI/RHR Alignment			
	ROCHESTER R. E. GIN EX RO/SRO RO/CRF CT #2 CT #2 RO/CRF BOP/SRO BOP/SRO BOP/SRO RO/SRO	ROCHESTER GAS & ELECTRIC CORPORATION R. E. GINNA NUCLEAR POWER PLANT EXAMINATION SCENARIO RO/SRO Manually Start • CINMT Recirc Fans • Both AFW Pumps RO/CRF Verify CNMT Isolation CT #2 Trip all RCP with five minitrip criteria • Two SI pumps running • RCS press - Highest S/G minitrip criteria • Two SI pumps running • RCS press - Highest S/G minitrip criteria • Two SI pumps running • RCS press - Highest S/G minitrip criteria • Two SI pumps running • RCS press - Highest S/G minitrip criteria • Two SI pumps running • RCS press - Highest S/G minitrip criteria • Two SI pumps running • RC/CRF Verify CCW pump running BOP/SRO Verify SI Flow BOP/SRO Verify AFW Valve Alignment BOP/SRO Control S/G Levels (Control S/G Levels (Control S/G Levels (Control RO/SRO RO/SRO Check SI/RHR Alignment	ROCHESTER GAS & ELECTRIC CORPORATION NO: 2004 NRC Initial Exam Scenario 04-1 R. E. GINNA NUCLEAR POWER PLANT TITLE: Ejected RCCA EXAMINATION SCENARIO DATE: 1/18/04 RO/SRO Manually Start • CRMT Recirc Fans • Both AFW Pumps RO/CRF Verify CNMT Isolation CT #2 Trip all RCP with five minutes of reaching the trip criteria • Two SI pumps running • RCS press - Highest S/G press < 175 psid (400 adverse) RO/CRF Verify CCW pump running RO/CRF Verify SI Flow BOP/SRO Verify AFW Valve Alignment BOP/SRO Control S/G Levels (Continuous Action) RO/SRO Check SI/RHR Alignment	ROCHESTER GAS & ELECTRIC CORPORATION NO:: 2004 NRC Initial Exam REV: 0 R. E. GINNA NUCLEAR POWER PLANT TTLE: Ejected RCCA DATE: 1/18/04 PAGE: 13 of 22 RO/SRO Manually Start • CNNT Recirc Fans

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 20 of 22
RO/SRO	Check Thermal Barrier Co	ooling	
BOP/SRO	Monitor Control Tavg • Stop dump steam • Control AFW • Shut MSIV's		
RO/SRO	Check PORVs and Block Va	alves	<u> </u>
SRO	Determine LOCA in progre Transition to E-1 • CNMT Radiation • CNMT Sump Level • CNMT Press	285	<u> </u>
BOP/SRO	Check Secondary Intact	(Pressure Stable)	
BOP/SRO	Control S/G levels (17-9	50%, 25-50 adverse)	
BOP/SRO	Check Secondary Radiatio	on Normal	<u> </u>
RO/BOP	Check PORVs		<u> </u>
RO/SRO	Reset SI and CI signals		

. ______

> ، بر____

			and the second	
ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0	
R. E. GINNA NUCLEAR POWER PLANT		TITLE: Ejected RCCA		
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 21 of 22	
BOP/SRO	Check SW System Direct AO to do SD-1			
BOP/SRO	Restore IA to CNMT			
BOP/SRO	Check Power Supplies fo	or Charging		
RO/SRO	Establish Charging Flow	Establish Charging Flow		
SRO	Check SI Termination Cr	riteria (not met)	<u></u>	
RO/SRO	Stop RHR pumps			
RO/BOP/ SRO	Check RCS and S/G Press or steamline break) Continue to step 16	ure (determine if LOCA	<u> </u>	
BOP/SRO	Stop Diesel Generators			
RO/SRO	Verify CNMT Sump Recirc Capability			
BOP/RO/ SRO	Evaluate Status		<u> </u>	

ROCHESTER GA	S & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-1	REV: 0
R. E. GINNA	NUCLEAR POWER PLANT	TITLE: Ejected RCCA	
EXAM	NATION SCENARIO	DATE: 1/18/04	PAGE: 22 of 22
SRO	Determine Post LOCA Cool Transition to ES-1.2 (Steps 1-2 are repeat of	down Required E-0 Actions)	
RO/SRO	Increase charging to 75	gpm	
BOP/SRO	Setup ARV for Pressure C intact S/G levels	ontrol and Control	
RO/SRO	Deenergize Przr Heater		
BOP/SRO	Initiate 100°F/hr Cooldow If < 100°F/hr cooldown no from the break	wn to CSD ot already present	<u> </u>

End Scenario



ROCHESTER GAS & ELECTRIC CORPORATION		NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Steam Generator Tube R	upture
EXAI		DATE: 1/18/04	PAGE: 1 of 23
Written by:	Kenneth Masker License Instructo	Date: r	1/18/04
Technical Revie	w: <u>Doug Peterson</u> License Instructo	Date:	1/26/04
Time validated	90_ minutes By: Lic	Ken Masker Date: ense Instructor	2/2/04
Date of exam:	······	_	
Examinees		Evaluators	
Final review	License Instructo	Date: 3/2	5/04
Approved for us	e <u>Supervisor License Tra</u>	Date: 3	29(04

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Rupt	ure
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 2 of 23

1. SCENARIO OVERVIEW

- 1.1 The plant is at 100% power MOL condition Xenon Equilibrium, C_B 845 ppm. The D CNMT Recirculation Fan is OOS. B and C charging pumps are running. A charging pump is OOS.
- 1.2 Turbine First Stage Pressure Transmitter PT-485 fails low, this results in rod stepping in at high speed. The operators should enter AP-RCC.1 Uncontrolled Rod Motion and place rods to manual. They should then defeat PT-485 per ER-INST.1.
- 1.3 A loss of off-site circuit results in loss of buses 16 and 17 until the diesel generator starts and restores the buses. The C Charging Pump is lost resulting in a partial loss of charging flow. The operators should respond using AP-ELEC.1.
- 1.4 A 10 gpm tube leak develops on the "B" S/G. The operator should recognize this condition from the increase in Air Ejector Radiation and increased RCS Makeup. They should respond per AP-SG.1 and initiate a plant shutdown.
- 1.5 Hotwell Level Controller LT-2001 Fails high causing hotwell rejection to the CSTs. The operators should recognize the failure and manual control hot well level utilizing LT-2002.
- 1.6 PCV-431C (Przr PORV) fails 50% open. The operator should recognize the rapidly decreasing pressurizer level and the valve mid position and enter AP-PRZR.1 Abnormal Przr Pressure. They should isolate the valve by closing its MOV block valve.
- 1.7 The SG Tube Leakage increases to 500 gpm (a SG Tube Rupture). The operator should trip the Rx and go to E-0 per the guidance of AP-SG.1. They should diagnose a SGTR and transition to E-3. When attempting to isolate the "B" MSIV, it fails to close requiring using the alternate isolation boundary in E-3.

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Rupt	ure
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 3 of 23

2. SCENARIO OBJECTIVES

RO(s) - Demonstrate the ability to monitor and operate the MCB, diagnose events and implement step of Normal, Abnormal and Emergency Procedures. Interact with other members of the crew in a manner which facilitates the crews response to the events listed below.

SRO - Demonstrate the ability to supervise the Control Board operators, diagnose events and choose and implement response procedures, implement Tech Spec requirements to ensure the plant is operated within its licensed basis, interact with other members of the crew in a manner which facilitates the crews response to the events listed below.

Instrument/Component Malfunction

- Loss of Circuit 751
- PT-485 Failure
- LT-2001 Failure
- PCV-431C Failure
- S/G Tube Leak

Major Malfunction

SGTR

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Rupture	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 4 of 23

- 3. CRITICAL TASKS (CTs)
- <u>CT #1 E-3--A</u>
- Task: Isolate feedwater flow into and steam flow from the ruptured steam generator before a transition to ECA-3.1 occurs.
- Cues: Indication of SGTR
 - Increasing SG water level
 - Steam Generator radiation
 - and
 - SI actuated

and

• Rx trip actuated

Indication: Manipulation of controls to isolate the rupture SG

- MSIV closed
- Local isolation by attachment Ruptured SG initiated
- ARV setpoint adjusted to 1050 in Auto
- SG blowdown and sample valve closed (verify)
- TDAFW steam supply valve from ruptured SG closed
- MDAFW and TDAFW discharge valve to ruptured SG closed
- Main feed reg and bypass valve to ruptured SG closed, main feedwater pumps tripped (verify)
- Feedback: Ruptured SG pressure stable or increasing. No indication of feed flow to ruptured SG.

<u>CT #2 E-3--B</u>

- Task: Establish/maintain RCS temperature so that a transition out of E-3 does not occur because the RCS temperature is in either of the following conditions:
 - Too high to maintain minimum subcooling or
 - Below the orange path PTS transition temperature (285°F)

Cues: Indication of SGTR

- Increasing SG water level
- SG radiation
- and
- Rx trip actuated
 - and
- SI actuated
- and
- Ruptured SG pressure greater than 300 psig

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Rupt	ure
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 5 of 23

3. <u>CRITICAL TASKS (CTs)</u> (continued)

Indications: Manipulation of intact SG ARV or condenser steam dump to establish the required RCS temperature.

Feedback: Indication of steam dump/ARV position Indication of RCS temperature decreasing Indications of RCS temperature less than target value

<u>CT #3 E-3--C</u>

Task: Depressurize RCS to SI termination criteria within one hour of SGTR initiation.

Cue: Indication of SGTR

• Ruptured SG level

• Ruptured SG radiation and

Rx trip and SI actuated

and

RCS cooled down to less than or equal to the target temperature

Indications: Manipulations of control to depressurize the RCS; PRZR spray, PORV or auxiliary spray.

Feedback: Indication of PRZR pressure decreasing Indication of PRZR level increasing

CT #4 CT E-3--D

- Task: Terminate SI within one hour of SGTR initiation and control RCS pressure and makeup so that primary and secondary inventory are stable before the end of the scenario.
- Cue: Indication of increasing SG level and high radiation RCS temperature at or below target cooldown value SI termination criteria met

Indications: SI pumps are stopped

Feedback: SI flow indicates zero

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Rupture	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 6 of 23

4. INSTRUCTOR ACTIONS

Problem Time		Action	<u>s</u>	<u>Notes</u>
	4.1 Setup s	imulato	or per OTG-3.2	
	4.2 Initial IC-172)	Condit	tions (Note: Saved as	
	4.2.1	Initia IC-19 100% M	lize the simulator to DL Xenon Eq.	
	4.2.2	Pull st Recirc Out 2 l	top and hold the D CNMT Fan nours for breaker	A-52.4 TS
	4.2.3	replace Insert	ement Malfunctions	
	4.2	3.1	"B" MSIV Fails Open	
	4.2.	.3.2	MALF SIMOSE 100% Turbine 1 st Stage Press PT-485 Fails low MALF TUR 16A, 0 Trigger 1	
	4.2.	3.3	LT 2006 Fails High MALF CND03A, 48, 0 Trigger 4	
	4.2.	3.4	Loss of 751 MALF EDS01A Trigger 2	
	4.2.	3.5	SG Tube Leak MALF SGN04B 10 gpm, Trigger 3	
	4.2.	3.6	PCV431C Przr PORV MALF PZR05B, 50% Trigger 5	
3 min.	4.3 Turbine Fail Lov Trigger	l st Sta W 1	age Press	
10 min.	4.4 Loss Ci: Trigger	rcuit 7 2	/51	

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Rupt	ure
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 7 of 23

- 20 min. 4.5 SG Tube Leak Trigger 3
- 20 min. 4.6 Hotwell Level Fails High Trigger 4
- 40 min. 4.7 Przr PORV 431C Fails Open Trigger 5
 - 4.8 SGTR Increase SGN04B to 500 gpm

Terminate when directed by the Lead Examiner.

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Rupture	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 8 of 23

5. TURNOVER INFORMATION

5.1 The plant is at 100% MOL Xenon Eq. $C_{\rm B}$ 845 ppm. The "D" CNMT Recirc Fan is OOS. See the A-52.4. "A" Charging Pump is OOS. See A-52.12

5.2 Equipment Out of Service

- D CNMT Recirc. Fan out for two hours
- A Charging Pump Belt replacement

5.3 Work in Progress

- D CNMT Recirc Fan Breaker
- A Charging Pump Belt replacement

5.4 Planned Work

None

5.5 Significant Events

CNMT Recirc Fan expected to be restored within 4 hours. A Charging Pump will be out at least 24 more hours.

5.6 <u>Remarks</u>

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Rupt	ure
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 9 of 23

- 6. EVALUATION
- Event: ____1

Event Title: <u>Turbine 1st Stage Pressure Fails Low (PT-485)</u>

Expected Response/Behavior

CUES: Tref Fails Low Control Rods step in at high speed

Response:

		RATING	N/A
SRO	Recognize rod motion, enter AP-RCC.1		
RO/SRO	Verify Turbine Power Stable, place rods in manual		
RO/SRO	Check Tavg. Manually operate rods to restore Tavg.		
RO/SRO/BOP	Check for Instrument Failure • Tavg • Turbine 1 st Stage Press		
SRO	Refer to ER-INST.1 for Channel Defeat		
SRO	Determine if Steam Dump should be placed in manual.		

ROCHESTER GAS & ELECTRIC CORPORATION	RIC CORPORATION NO.: 2004 NRC Initial Exam Scenario 04-2	
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	upture
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 10 of 23
BOP/SRO Monitor S/G Levels		<u> </u>

Defeat PT-485 per attachment First Stage Pressure PT-485 • Trip 485 Channel 2 in the white protection rack

- Verify AMSAC trip status light is extinguished
- Verify TL 400 Status Light extinguished (call AO to check)
- Call I&C to place TPS/485 to trip
- Verify AMSAC Auto Block life extinguished
- Delete point P0485 on PPCA

BOP/RO/SRO Check Systems in Auto

BOP

SRO Notify Plant Supervision

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	ipture	
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 11 of	23
6 <u>EVALUATIO</u> Event: <u>2</u> Event Title: <u>-</u> <u>Expected Respon</u> CUES: Circuit Buses 16 B D/G st	N Loss of Circuit 751 se/Behavior 751 Deenergized 7 17 lose power arts		v.	
Response: BOP/SRO	Recognize loss of 751		RATING	<u> N/A</u>
SRO	Enter AP-ELEC.1			
ro/sro	Verify Tavg > 350°F and	l both RCP running		
BOP/SRO	Verify B D/G running ar emergency buses energiz	nd both trains AC zed		
BOP/SRO	Verify SW pump running Manually start "C" or '	"ם"		
RO/SRO	Verify CCW pump running	3		
RO/SRO	Verify Charging Pump ru Determine none running	unning secure letdown	·	_

	ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
	R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	pture
.e.	EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 12 of 23
L				
	BOP/SRO	Verify Instrument Air a	available	
	BOP/SRO	Verify 11A/11B energize Valve Controlling	ed and MFW Regulating	
	RO/SRO	Check VCT Makeup System Charging Aligned to VCI		<u> </u>
	RO/SRO	Check VCT Makeup System Charging Aligned to VCI		
	RO/SRO	Start a Charging Pump		
	RO/SRO	Restore Letdown		
	RO/SRO	Restore Przr Heaters		
	RO/SRO	Restore Rod to Auto		
	RO/SRO	Stabilize Plant		
	BOP/SRO	Restore Power Alignment • Restore power to Bus • Restore power to buse	using ER-ELEC.1 12B (step 4.1.6) s 16/17 by holding	<u> </u>

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0	
R. E. GINNA NUCLEAR POWER PLANT		TITLE: Steam Generator Tube Ru	upture	
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 13 of 23	
BOP	Secure the B D/G per T	-27.4		
ro/sro	Verify at least two CNN	MT Recirc Fans running		
0/BOP/SRO	Check Control System in • 431K Przr Press • Heaters/Sprays • Charging Pumps • MFW Reg Valves • Turbine EH Control • Rods	n Auto		
SRO	Check Battery Chargers Dispatch AO			
RO/SRO	Secure one of two CCW p	pumps	<u></u>	
RO/SRO	Check/Start Radiation N	Monitor Sample Pump		
SRO	Send AO to check Seal (Dil System		
BOP/SRO	Close Fire Pump Breaker	c		
BOP/SRO	Check Aux. Building Ver	ntilation		
SRO	Dispatch AO to check Wa Compressors/SFP Cooling	aste Gas I/Reset UV Relav	<u></u>	

ROCHESTER (SAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINN	IA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	upture
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 14 of 23
5 <u>EVALUATIO</u> Event: <u>3</u> Event Title:	<u>N</u> S/G Tube Leak		
Expected Respon CUES: R-15/15 Compute: Charging VCT leve	<u>ise/Behavior</u> A increasing r alarms on S/G leakage (f: g Pump speed increases el decreases	rom R-15A)	
Response:			<u>RATING N/A</u>
RO/SRO	Determine S/G Tube Leak indications. Enter AP- Tube Leak	< has occurred from -SG.1 Steam Generator	·
RO/SRO	Monitor Przr Level adju stabilize pressurizer]	ist charging to level	
RO/BOP/SRO	Monitor Tube Leak Rate Determine that it is > Go to Step 8 (Rapid Pla	l gpm ant Shutdown)	

	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	Ipture	
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 15 of	23
6. <u>EVALUATIO</u> Event: <u>4</u> Event Title: _	N Hotwell Level Channel LT-	2001 Fails High		
Expected Respons	se/Behavior			
CUES: LT-2001 LT-2002 Hotwell Increasi	Offscale High Decreasing Level Controller Rejecting ng Condensate Storage Tan	g k Levels		
Response:			RATING	<u>n//</u>
BOP	Recognize symptom of LI	I-2001 failure	• <u> </u>	
BOP SRO/BOP	Recognize symptom of LT Place Hotwell Level cor Restore Hotwell Level	I-2001 failure ntrol to manual		
BOP SRO/BOP BOP	Recognize symptom of LT Place Hotwell Level cor Restore Hotwell Level Monitor Control Hotwell	I-2001 failure htrol to manual Level manually		
BOP SRO/BOP BOP SRO	Recognize symptom of LT Place Hotwell Level cor Restore Hotwell Level Monitor Control Hotwell Notify Scheduling	I-2001 failure htrol to manual Level manually		
BOP SRO/BOP BOP SRO BOP	Recognize symptom of LT Place Hotwell Level cor Restore Hotwell Level Monitor Control Hotwell Notify Scheduling Secure third Condensate	F-2001 failure htrol to manual Level manually		

ROCH	ESTER GAS	& ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0	
R.	E. GINNA	NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	Rupture	
	EXAMI	NATION SCENARIO	DATE: 1/18/04	PAGE: 16 of 2	3
5. <u>EVA</u> Event: _ Event Tit	LUATION 5 le: <u>P</u>	rzr PORV-431C Fails Open			
Expected_	Response	Behavior			
CUES: Pr PC PC	zr Press ORV Tail ORV-431C	sure Decreasing Pipe Temp Alarm Indicates mid position			
lesponse:				RATING	<u>N/2</u>
	RO	Recognize PORV-431C par Attempt to close/close	tly open block valve	<u> </u>	
	SRO	Enter AP-PRZR.1 Abnorma	l Przr Press		
RO/	CRF	Check Przr Press Channe	ls	<u> </u>	
RO/	CRF	Check RCS Parameters • Power • Przr Pressure			
RO/	SRO	Check Przr Press Contro • Przr Heaters • Normal Spray • Press Controller 431K	1		
RO/	SRO	Check PORVs. Determine	431C was open and the		

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	ipture
EXAMINATION SCENARIO		DATE: 1/18/04	PAGE: 17 of 23
RO/SRO	Check Przr Safety Valvo	es and Auxiliary Spray	
RO/SRO	Verify Przr Press retu step 16)	rning to normal (go to	

.

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	ipture
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 18 of 23
. EVALUATIO	N		
Ivent: <u>6,7</u>	_		
Ivent Title: _	SGTR		
-			
Typested Respon	se/Behavior		
	<u>SC/Denavier</u>		
CUES: Rapid De	creasing Przr Level		
SF/FF M1 Air Eiec	smatch tor R-15/15A Radiation in	creasing rapidly	
Response			
			RATING N/A
RO/SRO	Recognize Przr level de	ecreasing	
	Isolate Letdown	ing humb	
	Recognize that Przr lev	vel can't be maintained	
	Trip Rx and go to E-0		
RO/BOP	Check Immediate Actions	s (May manually	
	iniciace SI)		
RO/BOP/SRO	Check Status of Emerger	ncy Equipment	
10,201,010	• SI/RHR	ncy iquipment	······
	• CNMT Recirc Fans		
	• CNMT Spray (not requ:	ired)	
	• Steamline Isolation		
	MFW Isolation MDAEW Dumps		
	 MDAFW Pumps Service Water Dumps 		
	• CI/CVI		
	• CCW		
	• SI Flow		

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0	
R. E. GINNA NUCLEAR POWER PLANT EXAMINATION SCENARIO		TITLE: Steam Generator Tube Rupture		
		DATE: 1/18/04	PAGE: 19 of 23	
BOP/SRO	Check S/G levels (shou) S/G when level is > 50% Control intact S/G 17-5	ld secure feeding "B" %) 50%		
RO/SRO	Verify SI/RHR Alignment	:		
RO/SRO	Check CCW Cooling to RC	CPs		
RO/SRO	Secure TDAFW Pump		<u> </u>	
BOP/SRO	Control RCS Tavg to 547 (may need to limit AFW	7°F to A S/G)		
RO/SRO	Check PORVs and Sprays	Closed.	<u> </u>	
BOP/RO/ SRO	Monitor RCP Trip Criter	ria (should not be met)		

ROCHESTER	3AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0	
R. E. GINI	NA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube Ru	Rupture	
EXA		DATE: 1/18/04	PAGE: 20 c	of 23
BOP/SRO	Check for SGTR Transition to E-3 SGTR			. <u></u>
BOP/SRO	Identify "B" S/G as rup • Level Response • R-32 Alarm	otured		
<u>CT #1 E-3</u>	A Isolate feedwater flow i from the ruptured steam	into and steam flow generator before a	<u>SAT</u>	UNSAT
	CLAINSICION LO ECA-3.1 O	ccurs.		
	solate Rupture S/G			
	ARV 1050 psi in Auto		<u> </u>	
·	MON 25042 alegad			
	MOV 3504A Closed	alwag Classed		
	Try to close "B" MSTV W	hon it will not		
	Close perform the follow	vina.		
	- Close "A" MSTV	,111g :		
	= CIOSE + MSIV $= Set "A" ARV to 1005 pt$	sig in AUTO		
	- Idjust Condenser Steal	m Dump to 1050 pgi	_	
	in AUTO			
	- Place Steam Dump mode	switch to Manual		
	(Pressure Control Mode	e)		
	- Adjust reheat steam s	upply. ? to close		
	Reheat steam supply va	alves (will already be		
	closed)	_		<u></u>
	- Dispatch AO to complet	te Rupture S/G		
		d B for the B S/G)		<u> </u>
	Isolation (Parts A and			
	Isolation (Parts A and Check Rupture S/G Level >	5%		
•	Isolation (Parts A and Check Rupture S/G Level > - Close MOV 4008	5%		
	Isolation (Parts A and Check Rupture S/G Level > - Close MOV 4008 - Pull Stop MDAFW Pump '	5% "B"	<u></u>	
·	Isolation (Parts A and Check Rupture S/G Level > - Close MOV 4008 - Pull Stop MDAFW Pump ' - Close TDAFW Flow Conta	5% "B" rol Valve AOV 4298		
·	Isolation (Parts A and Check Rupture S/G Level > - Close MOV 4008 - Pull Stop MDAFW Pump ' - Close TDAFW Flow Contr - Verify MDAFW Cross-tie	5% "B" rol Valve AOV 4298 e Valves		

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube R	upture
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 21 of 23

BOP/SRO	 Check S/G Isolation MSIV Closed (instruct AO working on Attachment Rupture S/G to close Air Ejector and Flange Heat Steam Valves) Verify TDAFW Steam Supply Isolate Verify Rupture S/G Press > 300 psi 	
BOP/SRO	Adjust Intact S/G ARV to maintain S/G pressure	
RO/SRO	Reset SI	

		SAT	UNSAT
<u>CT #2_E-3B</u>	 Establish/maintain RCS temperature so that a transition out of E-3 does not occur because the RCS temperature is in either of the following conditions: Too high to maintain minimum subcooling or Below the orange path PTS transition temperature (285°F) 		
SRO	Determine required Core Exit Temp from Table		
BOP/SRO	Dump steam from the intact S/G ARV to cooldown To less than the required temperature		
BOP	Stabilize Core Exit Temperature		

 		······································		-
ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0	
R. E. GINNA NUCLEAR POWER PLANT		TITLE: Steam Generator Tube Ru	pture	
 EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 22 of 23	
BOP/SRO	Monitor Intact S/G Level (Note it will decrease b	l Control 17%-50% Delow 17%))		
RO/SRO	Check at least one PORV	available		
RO/SRO	Reset CI			
BOP/SRO	Check Normal Electrical busses	lineup to safeguard	. <u></u>	
RO/BOP/ SRO	 Restore Instrument Air Check adequate SW Dispatch AO to perform Buses 13/15 normal Feeds closed SW Isolation Valve to open Adequate Air Compresson A and B Instrument Or C Instrument Or Service Air Adequate IA Press Open AOV 5392 IA to CM 	A AH SD-1 the Turbine Building or running		
RO/SRO	Stop RHR pump			
RO/SRO	Establish 75 gpm Chargin	ng from RWST		
BOP/SRO	Check Ruptured S/G Press	Stable		

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-2	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: Steam Generator Tube R	upture
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 23 of 23

SRO Check for > 20°F core exit subcooling

		SAT	UNSAT
<u>CT #3 E-3C</u>	Depressurize RCS to SI termination criteria within one hour of SGTR initiation.		
RO/SRO	Depressurize using normal spray until termination criteria met.		
RO/SRO	Check RCS Press Increasing		
RO/BOP/ SRO	Check SI Termination Criteria		
		SAT	UNSAT
<u>CT #4 CT E-3-</u>	<u>-D</u> Terminate SI within one hour of SGTR initiation and control RCS pressure and makeup so that primary and secondary inventory are stable before the end of the scenario.		
		:	



ROCHESTER GAS & ELECTRIC O	CORPORATION NO.:	2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POW	ER PLANT	E: LOCA Outside CNMT	
	ARIODAT	E: 1/18/04	PAGE: 1 of 19
Written by: <u>Kenneth M</u> Lic	asker ense Instructor	Date:	1/21/04
Technical Review: <u>Dou</u> s SRO	g <u>Peterson</u> , SS	Date:	L/30/04
Time validated <u>90</u> minute	es By: <u>K. Masker</u> License In:	Date: structor	2/7/04
Date of exam:	····		
Examinees		Evaluators	
Final review Lice	ense Instructor		9/04
Approved for use Supervise or Design	or License Training	Date:3	29/04

ì

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 2 of 19

1. SCENARIO OVERVIEW

- 1.1 The plant is initially at 100% Power Xenon Equilibrium. Auto Rod Control is OOS due to Tavg input problems, I&C is investigating. The "C" Condensate Booster Pump is OOS due to a motor bearing failure. PT-429 failed low and has been defeated.
- 1.2 The running CCW pump trips. The standby pump fails to auto start. The operator needs to recognize the trip and take action per AP-CCW.1, Loss of CCW During Power Operations, prior to exceeding the operating limits for the RCPs.
- 1.3 The "A" Feed Regulating Valve fails closed in Auto. The operator needs to take manual control per AP-FW.1 and control S/G levels manually.
- 1.4 The "A" Charging Pump speed controller fails causing the pump to slow to minimum speed. The operator should start the standby pump and secure the "A" pump, then place the system back in auto control.
- 1.5 "B" Condensate Booster Pump Trips leaving only the "A" Pump Running. The operator should decrease load to prevent runout of the "A" pump per AP-TURB.5, Rapid Power Reduction.
- 1.6 A station blackout occurs causing a Turbine and Reactor Trip. Because of the PT-429 failure, an inadvertent SI occurs. The Diesel Generator fails to auto start requiring a manual start. When power is restored, MOV-852B opens on the SI signal and CV-853B fails allowing full RCS pressure to enter the RHR system. This results in an inter-system LOCA. The operators respond by implementing E-0, Reactor Trip or SI and transitioning to ECA-1.2, LOCA outside CNMT. The scenario is terminated when SI is terminated per procedure ES-1.1, SI Termination.

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 3 of 19

2. <u>SCENARIO OBJECTIVES</u>

RO(s) - Demonstrate the ability to monitor and operate the MCB, diagnose events and implement step of Normal, Abnormal and Emergency Procedures. Interact with other members of the crew in a manner which facilitates the crews response to the events listed below.

SRO - Demonstrate the ability to supervise the Control Board operators, diagnose events and choose and implement response procedures, implement Tech Spec requirements to ensure the plant is operated within its licensed basis, interact with other members of the crew in a manner which facilitates the crews response to the events listed below.

Instrument/Component Malfunctions

- Running CCW Pump Trip/Without Auto Start
- "A" FRV Failure of Auto Control
- "A" Charging Pump Speed Controller Failure
- PT-429 Failure Low
- Station Blackout
- Diesel Generator Fails to Auto Start

Reactivity Manipulations

• Power Reduction for Loss of Condensate Booster Pump

Major Malfunctions

CV-853B Fail during Inadvertent SI causing an inter-system LOCA

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 4 of 19

3. CRITICAL TASKS (CTs)

<u>CT #1 E-0--C</u>

- Task: Energize at least one AC emergency bus before transition out of E-0 unless the transition is to ECA-0.0, in which case the critical task must be performed before placing safeguards equipment handswitches in the pull-tolock position.
- Cues: Indication and/or annunciation that all AC emergency buses are de-energized • Circuit breaker position
 - Bus voltage
 - EDG status
- Indication: Manipulation of controls as required to energize at least one AC emergency bus
 - Circuit breaker position lamps indicate closed
 - Bus voltage indication shows nominal voltage present

Feedback: Indication that at least one AC emergency bus is energized

- Safeguards equipment sequences into the energized bus(es)
- EDG status
- CT #2 ECA-1.2--A
- Task: Isolate LOCA outside containment before transition out of ECA-1.2

Indication: MOV-852B is shut to isolate leak and is not reopened

- Feedback: RCS pressure increasing
 - Auxiliary Building radiation level decreasing

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 5 of 19

4. INSTRUCTOR ACTIONS

-

Problem_Time	<u>Action</u>	<u>s</u>	Notes
4.1 Init	ial Condi	tions	
4.2 Set	up Simulat	tor per OTG-3.2	
4.2.	1 IC-19 Equili	MOL 100% Xenon brium	
4.2.	2 Place Pump i:	"C" Condensate Booster n off and place hold Tag	
4.2.	Verify 3 Place "A" CC	"A" and "B" Pump Running "B" CCW in service secure W pump	
4.2.	4 Enter 1	Malfunctions	
4	.2.4.1	MALF CLG10	CCW Auto Start Failure
		MALF GEN08, 0 (both)	Both DG fail to Autostart
4	.2.4.2	MALF CLG2B Trigger 1	"B" CCW Pump Trip
4	.2.4.3	MALF FDW07A, 0%, 60 sec ramp Trigger 2	"A" FRV Fails Closed
4	.2.4.4	MALF CVC18A, 15 gpm, 0 ramp Trigger 2	"A" Charging Pump Speed Control Failure
4	.2.4.5	MALF CND01B Trigger 3	"B" CNDST Booster Pump Trips
4	.2.4.6	MALF EDS06, 0 Trigger 6	Station Blackout
		sec TD Trigger 4	LUCA U/S CNM
4.2.9	5 Insert Defeat	Malfunction PZR02A, 1600 PT-429 per ER-INST.1	Submit A-52.4 on PT-429 out 3 days

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 6 of 19

2 min	4.3 CCW Pump Trip Trigger 1
7 min	4.4 "A" FRV Fails Closed Trigger 2
7 min	4.5 Charging Speed Failure Trigger 2
25 min	4.6 CNDST BSTR Pump Trip Trigger 4
45 min	4.8 LOCA O/S CNMT Trigger 5

· ...

Terminate Scenario when SI has to be terminated or when instructed by the Lead Examiner.

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 6 of 19

- 7 min 4.4 "A" FRV Fails Closed Trigger 2
- 7 min 4.5 Charging Speed Failure Trigger 2
- 25 min 4.6 CNDST BSTR Pump Trip Trigger 4
- 45 min 4.8 LOCA O/S CNMT Trigger 5

Terminate Scenario when SI has to be terminated or when instructed by the Lead Examiner.

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	· · · · · · · · · · · · · · · · · · ·
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 7 of 19

5. TURNOVER INFORMATION

5.1 100% Power MOL C_B 845 ppm. Xenon Equilibrium.

5.2 Equipment Out of Service

- Auto Rod Control due to a problem with the Average Tavg signal to rod control
- "C" Condensate Booster Pump due to a motor bearing problem
- PT-429 Failed Low A-52.4 submitted

5.3 Work in Progress

- Rod Control Trouble Shooting
- "C" Condensate Booster Pump Motor Replacement

5.4 Planned Work

No Additional

5.5 <u>Significant Events</u>

Six hours ago rod began stepping out in Auto without a valid signal. I&C is investigating. Manual Rod Control is operable.

5.6 Remarks

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 8 of 19

- 6. EVALUATION
- Event: <u>1</u>

Event Title: Running CCW Pump Trips/No Auto Start

Expected Response/Behavior

CUES: CCW Pump "A" Tripped CCW Temp and Flow Alarms RCP Temp Alarms

Response:

		RATING	<u>N/A</u>
RO/SRO	Recognize Loss of CCW Pump Enter AP-CCW.2, Loss of CCW during Power Operations		
RO/SRO	Start the Standby CCW Pump		
RO/SRO	Check Monitor Plant Conditions • CCW Surge Tank Level • CCW to RCPs • Letdown		
RO/SRO	Check CCW Valve Alignment	<u> </u>	
RO/SRO	Check for CCW Leaks • Seal Water • CNMT • Aux Building		

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 9 of 19

SRO Determine that no CCW Leak exists

RO/SRO Verify Letdown in service

	ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0	
	R. E. GINN	A NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT		
	EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 10 of 1	9
6. Ever	EVALUATIO	<u>»N</u>			
Even	nt Title: _	"A" FRV Fails Closed			
<u>Expe</u> CUES	S: Feedwate S/G "A" S/G leve	er Flow decreases level decreases l deviation alarms			
Resp	ponse:			RATING	<u>N/</u>
	BOP/SRO	Recognize feed flow de Enter AP-FW.1 Abnormal	creasing Feedwater Flow		
	BOP/SRO	Check MFW Flow < Steam	Flow		
	BOP/SRO	Check both MFW Pumps Ru	unning		
	BOP/SRO	Check MFW Pump Suction	Press		
	BOP/SRO	Check MFW Regulating Va Determine "A" not contr manual control	alve rolling in Auto swap to		
	BOP	Restore "A" S/G Level i	in manual	<u> </u>	

. .

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 11 of 19
RO/BOP/ SRO	Stabilize Plant Condit: • Tavg • Przr Press • Przr Level • S/G Level • RIL Alarms	ions	<u></u>
BOP/SRO	Check MFW System • Pump Suction Press • Auto Control (not ava	ailable)	<u> </u>
BOP/SRO	Check Condensate System • CNDST Bypass Valve • Hotwell Level • Only two Condensate H • Trim Valves in Auto	n Pumps Running	
RO/BOP/ SRO	Establish Control Syste • Przr Press/Level	em in Auto	. <u></u>

		1		
ROCHESTER GA	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0	
R. E. GINNA	NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT		<u> </u>
EXAN	INATION SCENARIO	DATE: 1/18/04	PAGE: 12 of	19
6 <u>EVALUATION</u> Event: <u>3</u> Event Title: <u>0</u>	I Charging Pump Speed Contr	oller Failure		
Expected Respons	e/Behavior			
CUES: Charging Charging Przr Leve	Pump Speed Alarm (F-14) flow decreases al decreases			
Response:			RATING	<u>n/</u>
RO/SRO	Recognize abnormal ind: Determine "A" Charging to minimum	ication in CVCS pump speed has failed		
RO/SRO	Control Charging pump a control Przr Level (car	speed manually to nnot control in manual)		
RO/SRO	Swap Charging Pump Auto "B" or "C" pump	o Speed control to the		
SRO	Notify plant management	t of problem		

۰.

		······································		
ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0	
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT		
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 13 of	19
6. <u>EVALUATIC</u> Event: <u>4</u> Event Title: <u>-</u>	<u>N</u> "B" CNDST Booster Pump Tr:	ip		
Expected Respon CUES: Low Feed 4KV Moto Feedwate Condensa	<u>se/Behavior</u> Water Suction Pressure or Overload Alarm (6-8) er Pump NPSH Alarm ate Bypass Valve Opens			
Response:			RATING	<u>N/1</u>
BOP/SRO	Recognize Booster Pump Verify MFW Suction Pres	Trip s > 185 psi		
SRO	Determine power must be 30 minutes. (AR-AA-10)	e reduced to < 90% in		<u> </u>
SRO	Enter O-5.1 for power r	eduction	<u></u>	. <u></u>
SRO	Verify O-5.1 Initial Co	onditions		
RO/BOP/SRO	Make preparation for de • Notify Energy Operati • Przr Back Heaters on • Select rate on EHC • Notify plant personne	creasing load ons		

ROCHESTER GAS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXAMINATION SCENARIO	DATE: 1/18/04	PAGE: 14 of 19

RO/BOP/SRO

- Perform Load Reduction
- Rods in manual
- Add Boric Acid as necessary
- Lower Tube Load using EHC

ROCHESTER GAS	& ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0	
R. E. GINNA	NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT		
EXAMI	NATION SCENARIO	DATE: 1/18/04	PAGE: 15 of	19
6. <u>EVALUATION</u>				
Event: <u>6,7,8</u>	_			
Event Title:	Station Black out Rx Tr	ip, SI, LOCA O/S CNMT		
Expected Response	e/Behavior			
CUES: Rx Trip SI Signal	Present			
Response:			DATTNC	NT / 7
			RATING	<u>N/A</u>
SRO	Recognize Rx Trip, direc perform immediate action	et Board Operators to ns		
RO/BOP	Perform immediate actior	ns of E-O		
[SAT	UNSAT
<u>CT #1 E-0M</u>	Manually start at least Generator (see Section 3	one Emergency Diesel .0)		

SRO Determine SI Actuated, continue in E-0

R. E. GINNA NUCLEAN EXAMINATION SRO/RO/BOP Verif • SI/ • CNM • CNM • Mai • MFW • MDA • SW • CI • One • SI • AFW BOP/SRO BOP/SRO BOP/SRO RO/SRO Check	R POWER PLANT <u>SCENARIO</u> Ty Auto Actions RHR Pump Running T Recirc Fans Runn T Spray not requir n Steamline Isolat I Isolation FW Pump Running Pump Running Pump Running and CVI CCW Pump Running Flow Alignment or S/G Levels ol at 17-50%	TITLE: LOCA Outside CN DATE: 1/18/04	MT PAGE: 16 o
EXAMINATION SRO/RO/BOP Verif • SI/ • CNM • CNM • Mai • MFW • MDA • SW • CI • One • SI • AFW BOP/SRO BOP/SRO RO/SRO Check	SCENARIO y Auto Actions RHR Pump Running IT Recirc Fans Runn IT Spray not requir n Steamline Isolat Isolation FW Pump Running Pump Running Pump Running and CVI CCW Pump Running Flow Alignment or S/G Levels ol at 17-50%	DATE: 1/18/04	PAGE: 16 o
SRO/RO/BOP Verif • SI/ • CNM • CNM • Mai • MFW • MDA • SW • CI • One • SI • AFW BOP/SRO Monit Contr RO/SRO Check RO/SRO Check	y Auto Actions RHR Pump Running T Recirc Fans Runn T Spray not requir I Spray not requir I Solation FW Pump Running Pump Running and CVI CCW Pump Running Flow Alignment	ning red tion not required	
BOF/SRO Monit Contr RO/SRO Check RO/SRO Check	or S/G Levels ol at 17-50%		
RO/SRO Check RO/SRO Check			
RO/SRO Check	SI/RHR Alignment		
	RCP Cooling		
BOP/SRO Stop	TDAFW Pump		
BOP/SRO Contr • AFW • Ste	ol Tavg at 547°F aming (may need to	shut MSIVs)	
RO/SRO Check	Przr PORVs and Spi	ray	

.

ROCHESTER GAS & ELECTRIC CORPORATION		NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINN	A NUCLEAR POWER PLANT	TITLE: LOCA Outside CNMT	
EXA	MINATION SCENARIO	DATE: 1/18/04	PAGE: 17 of 19
SRO/BOP/ RO	Perform diagnostic step • S/G Intact • S/G Tubes Intact • No LOCA in CNMT Continue in E-0	2	
SRO	Determine SI cannot be	Terminated	
BOP/SRO	Control S/G Levels 17-5	50%	
BOP/SRO	Check Secondary Radiati	ion Levels	
RO/SRO	Reset SI and CI		
BOP/SRO	 Verify adequate SW Three pumps running Start third pump Send AO to do SD-1 At 	tachment	
BOP/SRO	Establish IA to CNMT • Close 13-14 and 15-16 • Open Turbine Building • Start Adequate IA Com "A" and "B" <u>Or</u> "C" <u>Or</u> Service Air	5 cross tie breakers g SW Isolation Valves mpressors	

ROCHESTER G	AS & ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0
R. E. GINNA NUCLEAR POWER PLANT TITLE: LOCA Outside CN EXAMINATION SCENARIO DATE: 1/18/04		TITLE: LOCA Outside CNMT	
		DATE: 1/18/04	PAGE: 18 of 19
SRO	Check Aux Building Radi Determine that it is ak CNMT Transition to ECA-	lation onormal from a LOCA O/S -1.2	<u> </u>
RO/SRO	Verify Proper Valve Ali • MOV 700, 701, 720, 72 • AOV 310, 296, 392A cl • MOV 313 and 371 close	ignments 21 closed Losed ed	
RO/SRO	Check for back flow int Close MOV 852A Monitorincrease. Determine proper MOV 852A	to RHR or for pressure ressure not increasing	
<u>CT #2 ECA-1.</u>	<u>2A</u> Isolate LOCA O/S CNM Of ECA-1.2	MT for Transition out	<u>SAT</u> <u>UNSAT</u>
		ia increasing	

• Leave 852B closed

SRO Verify pressure is increasing Transition to E-1, Loss of Reactor or Secondary Coolant

Note: Examinees will perform actions in E-1 until SI Termination Criteria is met (Step 12 or Foldout Page) just actions not previously performed are listed

ROCHESTER GAS	& ELECTRIC CORPORATION	NO.: 2004 NRC Initial Exam Scenario 04-3	REV: 0	
R. E. GINNA N	UCLEAR POWER PLANT	TITLE: LOCA Outside CNMT		
EXAMIN	ATION SCENARIO	DATE: 1/18/04	PAGE: 19 of 19	
RO/SRO	Establish Charging to Re • HCV-142 open • Align Charging Suction - 112B open - 112C closed • Start Charging Pumps	store Przr Level to the RWST		
SRO	Check SI Termination Criteria Determine it is met go to ES-1.1 SI Termination			
(Only step not	done in E-0 and E-1 appe	ar here)		
RO/SRO	Maintain Przr Press 1800	-2235 psig	<u> </u>	

RO/SRO Stop SI and RHR pumps

End Scenario