

April 23, 2004

Mr. Randall K. Edington
Vice President-Nuclear and CNO
Nebraska Public Power District
P. O. Box 98
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - REQUEST FOR ADDITIONAL INFORMATION
REGARDING RELIEF REQUEST RC-06 (TAC NO. MC2059)

Dear Mr. Edington:

By letter dated February 12, 2004, Nebraska Public Power District (the licensee) requested the Nuclear Regulatory Commission (NRC) staff grant relief from certain Inservice Inspection requirements, including RC-06. RC-06 requests approval to perform a VT-1 or VT-3 visual examination in lieu of a VT-2 visual examination following a repair or replacement.

The NRC staff has reviewed the information provided in the February 12, 2004, submittal and determined that additional information is required in order to complete the review and approval of RC-06. As agreed upon in a telephone call on April 22, 2004, the licensee will respond to the request for additional information (RAI) within 45 days. The RAI is enclosed.

Sincerely,

/RA/

Michelle C. Honcharik, Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosure: RAI

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

ISSUES RELATED TO RELIEF REQUEST RC-06

COOPER NUCLEAR STATION

1. The licensee requests relief from Subsubarticle IWE-5240 of the 1992 Edition and the 1992 Addenda of Section XI of the American Society of Mechanical Engineers Code. IWE-5240 (by reference to the requirements of Subsubarticle IWA-5240) requires the licensee to perform a VT-2 visual examination during a pressure test to detect evidence of leakage. The licensee requests relief from performing a VT-2 visual examination during pressure testing the containment pressure retaining components. Alternatively, the licensee will perform VT-1 or VT-3 visual examination, as appropriate. To get a clear understanding of how the licensee will determine the appropriateness of VT-1 and VT-3 visual examination, the licensee is requested to provide the following information:
 - (a) The intent of IWE-5240 is to detect the areas of leakage during pressure testing of the containment pressure boundary components. How would you perform the examination of a leaking component which is in part inaccessible and detect leakage area(s) (e.g., Type B testing of vent pipes between the drywell and the suppression chamber)?
 - (b) In some cases, IWE-5222 permits deferral of pressure test following minor repair. How would you perform the examination of the repaired areas to ensure that the repaired areas will not be leaking during the deferral period?

ENCLOSURE

Cooper Nuclear Station

cc:

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December 2003