

April 30, 2004

Mr. Joseph M. Solymossy  
Site Vice President  
Prairie Island Nuclear Generating Plant  
Nuclear Management Company, LLC  
1717 Wakonade Drive East  
Welch, MN 55089

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2 - REQUEST FOR  
ADDITIONAL INFORMATION ON THIRD 10-YEAR INSERVICE INSPECTION  
INTERVAL REQUEST FOR RELIEF NO. 16 (TAC NO. MC1775)

Dear Mr. Solymossy:

By letter to the U.S. Nuclear Regulatory Commission (NRC) dated January 7, 2004, the licensee, Nuclear Management Company (NMC), submitted Request for Relief No. 16 from the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Section XI, for Prairie Island, Unit 2.

A draft request for additional information was e-mailed to Mr. Jack Leveille (NMC) on April 8, 2004. This information was discussed with NMC representatives during a telephone conversation on April 14. The NRC staff reviewed the information provided by the licensee during this phone call and needed additional information to complete the review. The staff requested further discussions with the licensee. A follow-up telephone conference was held on April 20, 2004, during which it was determined that the enclosed information is required to complete the evaluation. During this phone call, a mutually agreeable response date of May 31, 2004, was established.

Please provide this information by the requested date in order for us to process this relief request in a timely manner. If you have any further questions, please call me at 301-415-8371.

Sincerely,

***/RA AMcMurtray for/***

Mahesh Chawla, Project Manager, Section 1  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-306

cc w/enclosure: See next page

REQUEST FOR ADDITIONAL INFORMATION  
ON THIRD 10-YEAR INSERVICE INSPECTION INTERVAL  
REQUEST FOR RELIEF  
FOR  
NUCLEAR MANAGEMENT COMPANY  
PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2  
DOCKET NO. 50-306

1. Regarding Table 1 on Page 10 of the submittal, what was the ultrasonic test (UT) coverage obtained from the previous interval? If, for any weld, the coverage was less than before, please explain.
2. For certain welds, including RC Weld W-6/2LSU on Page 3, the licensee stated, "the weld was included in the boundary examined by VT-2 during pressure testing..." Please provide a brief explanation to state what value has been added through the VT-2 examination. For example, if the results indicated leakage integrity of the component, etc.
3. For MS Weld W-36, the photo in the attached is not clear. Please provide drawings or sketches that include a cross section of the weld, basic dimensions of the weld and of the adjacent flange. Please explain in detail why UT examination can not be performed on the outside surface as well as on the inside surface when the valve was disassembled.
4. For MS Weld W-36, the licensee also stated that a VT-1 was performed on the inner diameter (ID) surface. Please explain how a VT-1 can be used to detect flaws. Please explain why surface examinations were not considered on the ID in lieu of VT-1 to detect flaws.
5. For MS Weld W-36, please provide its pre-service examination record and results.

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Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-306

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ADAMS Accession No.: ML041130662

\*See previous concurrence NRR-088

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Prairie Island Nuclear Generating Plant, Units 1 and 2

cc:

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