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April 19, 2004

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Washington, DC 20555-0001

SUBJECT: PWR Containment Sump Evaluation Methodology – Baseline Evaluation

PROJECT NUMBER: 689

Dear Mr. Hannon:

During a March 23, 2004 public meeting to discuss Industry GSI-191 activities, we provided a status of activities to develop an evaluation methodology for use by PWR plants in their evaluation of containment sump performance.

The evaluation methodology presented at the March 23 meeting reflects significant modification of the methodology contained in an October 31, 2003 draft. The revised approach incorporates NRC comments and requests for additional information (RAI) received at the March 23 meeting.

An outline of the evaluation methodology process is shown in Figure 1. The methodology is intended to allow licensees to address and resolve GSI-191 issues in an expeditious manner through a process that starts with a conservative baseline evaluation. The baseline evaluation is expected to guide the analyst and provide a method for quick identification and evaluation of design features and processes that significantly affect the potential for adverse containment sump blockage for a given plant design. The baseline evaluation also facilitates the evaluation of potential modifications that can enhance the capability of the design to address sump debris blockage concerns and uncertainties and supports resolution of GSI-191.

The evaluation methodology currently allows for incorporation of either a deterministic evaluation process (Option A) or a risk-informed evaluation process (Option B). The risk-informed evaluation process, while in an early stage of development, acts upon the willingness of NRC staff, expressed in a March 4, 2004 letter, to utilize current work to risk-inform Title 10, *Code of Federal Regulations*

Mr. John N. Hannon

April 19, 2004

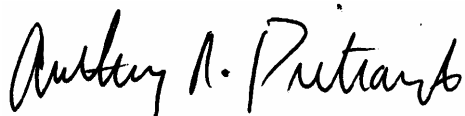
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Section 50.46, "Acceptance criteria for emergency core cooling system for light-water nuclear power reactors," as a suitable technical basis for defining a spectrum of break sizes for debris generation and containment sump strainer performance. We are currently attempting to arrange a meeting with NRC staff to discuss and hopefully come to agreement on a way to risk-inform GSI-191 evaluation activities that is compatible with the resolution schedule.

At the March 23 public meeting, we committed to provide the revised version of the Baseline Evaluation Methodology for NRC review by April 19, 2004 and the complete Evaluation Methodology and responses to the NRC RAIs by May 28, 2004. In accordance with this schedule, the Baseline Evaluation Methodology is provided as Enclosure 1. An outline of the complete document is provided as Enclosure 2.

Please contact John Butler 202-739-8108, jcb@nei.org, or me if you have any questions on this transmittal.

Sincerely,



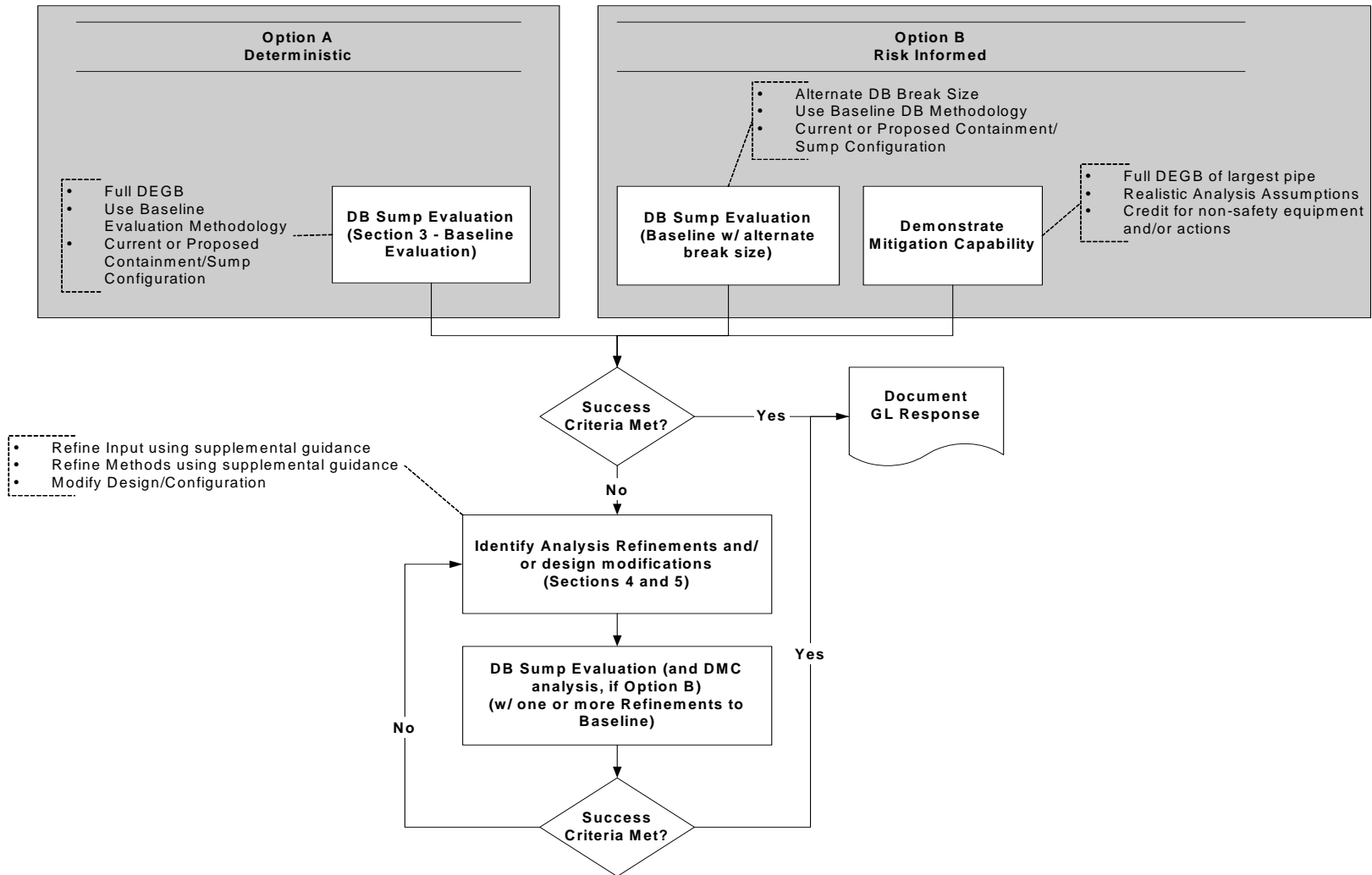
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Enclosure

c: Mr. Sunil D. Weerakkody, U. S. Nuclear Regulatory Commission
Mr. Ralph E. Architzel, U. S. Nuclear Regulatory Commission
Mr. Michael Marshall, U.S. Nuclear Regulatory Commission
Mr. John G. Lamb, U. S. Nuclear Regulatory Commission

Figure 1

PWR Containment Recirculation Sump Performance Evaluation Process Overview



Outline for PWR Sump Performance Evaluation Methodology

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- C - Derivation of Suitable Surface-to-Volume Ratio**
- D - Comparison of Evaluation Methodology to RG 1.82 Revision 3**
- E - Comparison of Nodal Network and CFD calculation**

* - Do not anticipate inclusion of supplemental guidance for this section beyond discussion of potential refinements of baseline analysis that could be developed and supported on a plant-specific basis.

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SECTION 3

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BASELINE EVALUATION

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3.1 INTRODUCTION

3.1.1 Purpose

The purpose of this baseline evaluation methodology is to provide licensees with a common and consistent approach for doing an initial scoping evaluation to evaluate the post-accident performance of the containment sump screen for a Pressurized Water Reactor (PWR). This common and consistent method is termed the “*Baseline Evaluation Method.*”

The Baseline Evaluation Method provides a conservative approach for evaluating the generation and transport of debris to the sump screen, and the resulting head loss across the sump screen. If a plant uses this method and guidance to determine that sufficient head loss margin exists for proper long-term Emergency Core Cooling (ECC) and Containment Spray (CS) function, no additional evaluation for head loss is required.

The same sumps may be used for both long-term ECC for heat removal from the core and long-term CS for heat removal from the containment environment. Revision 3 of Regulatory Guide 1.82 (Reference 3.1-1) as refers to sumps performing this combined or dual function as Emergency Core Cooling (ECC) sumps. This convention of referring to dual-function sumps as ECC sumps will be used here.

3.1.2 Background

The probability of a high-energy line break of PWR piping inside the Reactor Containment Building (containment) is extremely low. However, if the event were to occur, it could result in production of debris that, if transported to and deposited on the containment sump screens, could challenge the function of the ECC sumps. Specifically, debris that would accumulate on the sump screens would result in an increase in the head loss across the resulting debris bed and sump screen. This head loss may be sufficiently large such that the head loss may exceed the available net positive suction head (NPSH) margin of the ECC sumps.

3.1.2.1 General Accident Scenarios of Concern

Postulated accident scenarios of concern are those that require the plant to initiate recirculation flow from the containment sump to mitigate the event. Therefore, the primary design basis accident (DBA) that could present a challenge to the ECC sumps is the Loss-Of-Coolant-Accident (LOCA). However, for some plants, a main steam line break or feedwater break could challenge ECC sump function as well.

3.1.2.2 Accident Phenomena

Three (3) broad phenomena have been identified as governing post accident sump performance:

1. Debris Generation – The destruction of insulation, coatings and erosion of concrete due to the action of the jet resulting from the postulated pipe break.
2. Debris Transport – The movement of debris generated from the jet due to fluid movement associated containment pooling, washdown of containment sprays and from the erosion of submerged material, to the sump when the ECC and CS systems are realigned to draw suction from the containment sump.
3. Head Loss – The development of resistance to flow across the ECC sump screen due to the transport and collection of debris on the sump screen.

The Baseline Evaluation Method provides guidance for licensees to address each of these phenomena and to address post-accident sump screen performance.

3.1.2.3 Limits of Evaluation Method

The guidance presented in the Baseline Evaluation Method only addresses the phenomena and issues up to and including head loss across the sump screen. The application of the Baseline Evaluation Method will provide information which can be used to assess resultant effects on NPSH or pump suction inventory. The calculation of required NPSH of the ECC and CS systems, chemical corrosion effects occurring as a consequence of the postulated event, effects resulting from debris upstream of the sump screen, effects resulting from debris downstream of

1 the sump screen and/or sump screen structural integrity concerns are beyond the scope of the
2 Baseline Evaluation Method and are not addressed in this section.

3 4 **3.1.2.4 Guidance for Refinements to Baseline Evaluation**

5
6 The Baseline Evaluation Method presented in this section provides one suggested approach for
7 all utilities to perform an evaluation of the susceptibility of their ECC sumps to failure from
8 debris-induced screen blockage. In addition to the Baseline Evaluation Method and supporting
9 discussion, an example calculation applying the Baseline Method is also provided. The guidance
10 in this section provides a conservative approach for evaluating the generation and transport of
11 debris, and the resulting head loss across the sump screen.

- 12 • If a plant uses this Baseline Evaluation Method and determines that head loss margin
13 exists for proper ECC and CS function, no additional evaluation for head loss is required.
- 14 • If a plant determines that the results of the baseline approach are not acceptable, or
15 additional design margin is desirable, the refinement guidance provided in subsequent
16 sections may be used to further evaluate the post-accident performance of the ECC sump.

17 18 **3.1.3 Data Collection to Support Baseline Evaluation**

19
20 In order to perform the sump performance evaluation according to the guidance in this
21 document, gather the appropriate plant information. The information needed to support the
22 baseline sump performance evaluation is similar to that needed to perform a containment
23 condition assessment walkdown as described in NEI 02-01, “Condition Assessment Guidelines:
24 Debris Sources Inside PWR Containments” (Reference 3.1-2). Therefore, the information
25 primarily documents the configuration of containment and the potential debris sources contained
26 therein.

27
28 The information required to perform the assessment can be categorized as follows:

- 29 1. General Containment Design Information
 - 30 • Topographical containment layout drawings

- 1 • Piping isometric drawings
- 2 • Process diagrams
- 3 • Accident analysis of record and associated licensing basis for post-LOCA
- 4 recirculation including ECC and CS recirculation flows for various break sizes, spray
- 5 sequence and flows, time duration, sump water temperature profile, etc.

6 2. Insulation Details

- 7 • What insulation was used inside containment (insulation specifications),
- 8 • Volume of insulation material installed,
- 9 • Where it was used on equipment, in penetrations, on piping, etc. (drawings),
- 10 • How it was installed; encapsulated, banded, etc. (drawings),
- 11 • Inspection records, if appropriate or available, and
- 12 • Design changes that may have changed insulation used (specifications and
- 13 drawings).

14 3. Penetration Details

- 15 • Penetration plan (elevation and azimuth)
- 16 • Drawings of insulation material used in penetrations.

17 4. Fire Barrier Details

- 18 • What material was used inside containment (material specifications),
- 19 • Where it was used inside containment (drawings),
- 20 • How it was installed (drawings),
- 21 • Inspection records, if appropriate or available, and
- 22 • Design changes that may have changed fire barrier material or location inside
- 23 containment (specifications and drawings).

24 5. Protective Coatings Details

- 25 • What coatings were applied,

- 1 • Where they were applied,
- 2 • QA program requirements,
- 3 • Coatings application specification(s),
- 4 • Coatings inspection records,
- 5 • What coatings were applied to purchased equipment and the coatings program used
- 6 to apply them, and
- 7 • A copy of the “Exempt” or “Unqualified” coatings log, if used at the site.

8 6. Other Potential Debris Sources

- 9 • Foreign materials exclusion program documentation
- 10 • Latent debris observed to be inside containment
- 11 • Tagging and labeling procedures or technical instructions
- 12 • References for use of cable ties inside containment

13 The above listing of information is intended to be as complete as possible to support a plant-
14 specific baseline evaluation. However, plant-specific features may suggest that additional
15 information be collected and supporting documents be reviewed in support of performing the
16 baseline evaluation.

17

18 **3.1.4 References**

19

- 20 3.1-1 Regulatory Guide 1.82, Revision 3, “Water Sources for Long-Term Recirculation
- 21 Cooling Following a Loss-Of-Coolant Accident,” US Nuclear Regulatory Commission,
- 22 November 2003
- 23 3.1-2 NEI 02-01, Revision 1, “Condition Assessment Guidelines: Debris Sources Inside PWR
- 24 Containments,” September 2002

25

3.2 METHOD DESCRIPTION

3.2.1 Break Selection

Discussed in this section are the considerations and guidance for selecting an appropriate postulated break size and evaluating the location of the postulated break that presents the greatest challenge to post-accident sump performance.

3.2.1.1 Introduction

The break selection is the first step in assessing post-accident sump screen performance. Break selection consists of two considerations:

1. The size of the break, and,
2. The location of the break.

The objective of the break selection process is to determine the break size and location that results in debris generation that is evaluated to determine the maximum head loss across the sump screen. Since this location is not known prior to performing the evaluation, the term break selection refers to a process of evaluating a number of break locations for a given size break to identify the location that presents the greatest challenge to post-accident sump performance.

3.2.1.2 Discussion

The objective of the break selection process is to evaluate and identify the break locations that provide for the following two results:

1. The maximum amount of debris that is transported to the sump screen, and,
2. The worst combination of debris mixes that are transported to the sump screen.

The locations that provide for these conditions are identified as “limiting break locations” for the purpose of evaluating post-accident sump screen performance.

1 The criterion used to define the limiting break location is the head loss across the sump screen
2 resulting from deposition of debris on the sump screen; the limiting break location results in the
3 maximum head loss. As noted above, the limiting break location is not known prior to
4 performing the evaluation, but is determined by evaluating a number of postulated break
5 locations. To perform this evaluation, it is necessary to perform the debris generation, debris
6 transport, and head loss calculations for each postulated break location. Therefore, the selection
7 of the limiting break site is an iterative process that requires rigor.

8
9 The guidance below documents the process for determining the limiting break location.

11 **3.2.1.3 Postulated Break Size**

12
13 A double-ended guillotine break (DEGB) of piping, including the primary system piping, may be
14 used as the postulated break size. This approach provides for the prediction of large volumes of
15 debris from insulation and other materials that may be within the region affected by the fluid
16 escaping through the postulated break. NRC has accepted this as an acceptable approach in the
17 resolution of ECCS strainer blockage concerns for Boiling Water Reactor (BWR) plants. This
18 method is applicable to all PWR designs.

19
20 Some plant designs require recirculation of containment spray for long term containment cooling
21 after a main feedwater line break or a main steam line break. Either the same considerations as
22 for LOCA or the plant's current licensing basis for those breaks may be used for break selection
23 and size characterization.

25 **3.2.1.4 Identifying Break Locations**

26
27 Postulation of the break location is somewhat more complex than postulation of the break size.
28 All Reactor Coolant System (RCS) piping, and connected piping, must be considered in the
29 evaluation. Since many break locations are to be considered, a wide range of results is to be
30 expected. Some plant designs require plants to eventually recirculate coolant from the sump for

1 pipe ruptures other than a LOCA. If this is a part of the plant under consideration's licensing
2 basis, then these lines must also be considered for debris generation.

3 4 **3.2.1.4.1 General Guidance**

5
6 It is recommended that pipe break locations considered are postulated based on the following
7 criteria:

- 8 1. For postulated LOCAs, break exclusion zones are disregarded for this evaluation. In other
9 words, pipe breaks must be postulated in pre-existing break exclusion zones. For main
10 steam and feedwater line breaks, licensees should evaluate the licensing basis and include
11 potential break locations in the evaluation, if necessary.
- 12 2. NRC Branch Technical Position MEB 3-1 shall not be used as a basis for determining
13 potential LOCA break locations. The purpose of the analysis is to determine the worst
14 possible break with respect to ECCS sump concerns. Therefore, the location of the pipe
15 break is not chosen based on the stress distribution and or fatigue characteristics of the
16 piping system.
- 17 3. For the plants for which main steam line breaks and/or feedwater line breaks must be
18 considered, then the break locations should be consistent with the plant's current
19 licensing basis.
- 20 4. Pipe breaks shall be postulated at locations such that each location results in a unique
21 debris source term (i.e. multiple identical locations need not be examined).
- 22 5. Pipe break shall be postulated in locations containing high concentrations of problematic
23 insulation (micro-porous insulation, calcium-silicate, fire barrier material, etc.).
- 24 6. Pipe breaks shall be postulated with the goal of creating the largest quantity of debris and
25 or the worst-case combination of debris types.
- 26 7. Piping attached to the RCS that is small (< 2 inches in diameter) need not be considered.
27 Breaks of this size are sufficiently small (and bounded by the larger breaks) that
28 quantities of debris large enough to challenge the post-accident operability of the
29 containment sump are not generated.

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3.2.1.4.2 Piping Runs to Consider

As a minimum, LOCA breaks in the following lines should be considered:

1. Hot leg, cold leg, intermediate (crossover) leg and surge line
2. Piping attached to the reactor coolant system. Examples include, but are not limited to Charging Lines and/or RHR lines.

Some plant designs require plants to eventually recirculate containment coolant from the sump for pipe ruptures other than a LOCA. Two such events are main feedwater breaks and steam line breaks. If this is a part of the licensing design basis for the plant under consideration, then these lines must also be considered for this evaluation.

3.2.1.4.3 Other Considerations for Selecting Break Locations

Section 3.2.1.2, "Discussion," identified the objective of break selection as of identifying a limiting break for post-accident sump performance consideration. Listed below are additional guidelines to use in selecting break locations that support that objective.

1. Identify locations for postulated large breaks that result in the generation of two or more different types of debris. These locations are determined by considering the location of materials (insulation, coatings, etc.) inside containment relative to the break location and Zone of Influence. Specifically, look for locations where problematic insulation (for example, micro-porous insulation) may be combined with particulate debris. Note that the location of materials inside containment should have been identified during the application of NEI-02-01 (Reference 3.2.2-1).
2. Identify locations for which postulated breaks generate an amount of fibrous debris that, after transport to the sump screen, creates a uniform fibrous bed of equal to or greater than 1/8-inch layer to filter particulate debris.
3. If the insulation does not result in the generation of significant particulate debris (for example, the insulation in the ZOI is RMI, there is no micro-porous insulation inside

1 containment, and fibrous insulation is not affected by the postulated break), particular
2 attention should be given to the characterization of latent debris sources as this source
3 may present the limiting debris loading condition with respect to either fiber, particulates,
4 or both.

6 **3.2.1.4.4 Selecting the Initial Break Location**

7
8 To start the break selection evaluation, select an initial break location using the guidance given in
9 Sections 3.2.1.4.1 through and including Section 3.2.1.4.3. Multiple breaks will be examined to
10 demonstrate the limiting break location was considered. However, using the guidance identified
11 in these sections, it is possible to identify locations that may be considered to be likely
12 candidates for the limiting location. Thus, it is suggested that an initial postulated break location
13 be chosen with the following characteristics:

- 14 1. Pick the initial break location to be near a large quantity of potential debris and/or is near
15 a combination of potential debris types that are known to challenge post-accident sump
16 operation. It is suggested that results from a containment condition assessment, similar to
17 that described in NEI 02-01 (Reference 3.2.2-1) would be useful in assessing such
18 locations.
- 19 2. The location is a convenient place to start a sequence of breaks (e.g. at the physical end
20 of a length of pipe when multiple locations on that length of pipe are being evaluated).

21 Given the above, it is suggested that a candidate location for the initial break location is the
22 junction of the primary piping and the steam generator. Two general industry observations
23 support this suggestion:

- 24 1. As a consequence of their size, steam generators have a larger volume of insulation
25 applied to them than does primary system piping.
- 26 2. It has been observed that steam generators often have several different types of insulation
27 applied to them.

28 Therefore, the selection of a break location at the junction of primary piping and the steam
29 generator is a reasonable starting point to address the criteria of evaluating both the maximum

1 amount of debris that is transported to the sump screen, and, the worst combination of debris
2 mixes that are transported to the sump screen.

3

4 **3.2.1.5 Evaluation of Break Consequences**

5

6 The evaluation of break consequences is the determination of the head loss across the sump
7 screen as a result of the generation, transport and accumulation of debris on the sump screen that
8 is calculated to occur as a result of the postulated break, and the consequential head loss across
9 the sump screen as emergency core cooling and containment spray recirculation water attempts
10 to pass through the debris bed.

11

12 **3.2.1.5.1 Purpose of Break Consequence Evaluation**

13

14 The purpose of evaluating the consequences of a postulated break is to determine the head loss
15 associated with that break and its effect on available NPSH for the recirculation pumps. To
16 accomplish this, the following additional evaluations must be performed for each break location
17 considered:

18 1. Evaluation of the Zone of Influence (ZOI), the region inside containment that is affected
19 by the fluid escaping from the postulated break location, resulting in the generation of
20 debris. As the ZOI is moved about to different break locations, robust structures (walls)
21 may affect the geometry of the ZOI.

22 2. Evaluation of the debris source term.

23 3. Evaluation of debris transport to the sump screen.

24 4. Evaluation of head loss across the sump screen resulting from debris that has been
25 (transported to and) deposited on the containment sump screen.

26 Evaluating break consequences in this way provides for the evaluation of sump screen head loss
27 as a function of postulated break size and break location.

28

29

30

3.2.1.5.2 Selection of Intervals for Additional Break Locations

Having evaluated the initial break location, additional locations are evaluated and the results compared to each other. The purpose of this comparison of results is to determine the limiting location for the break size used.

For primary piping, it is suggested that the break location be moved at 3-foot increments along the pipe being considered. This break frequency provides for an acceptable determination of the limiting break location with respect to both:

1. The maximum volume of debris that may be generated and transported to the sump screen, and,
2. The worst combination of debris that may be generated and transported to the sump screen.

It is expected that as the plant specific analysis develops it would be determined and documented by inspection that the number of cases requiring detailed analysis can be limited based on debris inventory, similarity of transport paths and piping physical characteristics.

The same strategies need not be applied when considering main steamline or feedline breaks. A sufficient number of breaks, consistent with the plant-specific design and licensing basis, should be considered to reasonably bound variations in debris generation by the size, quantity, and type of debris. This may result in break intervals as small as 3 feet, depending on plant configuration.

For attached piping, only the length of pipe run up to the flow isolation point need be considered. This approach will account for debris generation from postulated pipe breaks, including single-sided breaks, from attached piping. There is no need to consider pipe breaks in attached piping beyond the isolation points as such breaks, should they occur, will not result in the plant evolving to recirculation from the containment sump to mitigate the event.

1 **3.2.1.6 Break Size for Sample Calculation**

2

3 For the purposes of the sample calculation presented in this document, the break is taken as ten
4 (10) inches. The break size of 10 inches is based on the assumption that the 10-inch break
5 represents the maximum size of attached piping to the primary system. A single sample debris
6 generation calculation, debris transport evaluation and subsequent head loss is presented. A
7 number of break locations should be identified, as described here, and evaluated to assure that
8 the potential for debris generation, transport and sump screen head loss are adequately evaluated
9 for a given plant.

10

11 **3.2.1.7 References**

12

13 3.2.1-1 NEI-02-01, “Condition Assessment Guidelines: Debris Sources Inside PWR
14 Containments,” Revision 1, September 2002.

15 3.2.1-2 Regulatory Guide 1.82, “Water Sources for Long-Term Recirculation Cooling
16 Following a Loss-Of-Coolant Accident”, Revision 3 November 2003.

17

3.2.2 DEBRIS GENERATION

3.2.2.1 Introduction

Following identification of postulated break locations, the next step taken in evaluating post-accident sump performance is to determine an appropriate zone of influence (ZOI) within which the resultant break jet would have sufficient energy to generate debris. It is noted that not all debris that is evaluated to be generated is in a form that may be transported to the sump. Thus, evaluation of debris generation from a postulated break is a two-step process:

1. The first step is to evaluate an appropriate ZOI in which debris is generated.
2. The second step is to evaluate the characteristics of the debris generated.

Included in this second step is the identification of transport characteristics of the debris generated by the postulated break. Thus, the evaluation of debris generation for a given break location is an exercise of establishing an appropriate size and shape of the ZOI, mapping that ZOI volume over the spatial layout of insulated piping and calculating the volume of insulation within that ZOI. The final step to evaluating debris generation is the application of a size distribution to the debris generated within the ZOI volume that will be used to evaluate debris transport.

The identification of the ZOI and resulting debris generation for postulated pipe breaks (LOCA, main steamline or feedwater) is both plant- and break-specific. Presented in this section is guidance on establishing the appropriate ZOI and resulting debris characteristics for LOCA.

3.2.2.2 Zone of Influence

The zone of influence is defined as the volume about the break in which the fluid escaping from the break has sufficient energy to generate debris from insulation, coatings, and other materials within the zone. For the baseline calculation, it is recommended that the boundary of the ZOI be assumed to be spherical, with the center of the sphere located at the break site. The use of a spherical ZOI is intended to encompass the effects of jet expansion resulting from impingement

1 on structures and components. Use the guidance in Sections 3.2.2.2.1 and 3.2.2.2.2 to determine
2 the Zone of Influence for a postulated pipe break.

3
4 Guidance on the identification of other, more realistic ZOI's is given in the Supplemental
5 Guidance.

7 **3.2.2.2.1 Recommended Size of Zone of Influence**

8
9 To determine the radius of the spherical ZOI needed to represent the effects of the jet originating
10 from a postulated pipe break, the ANSI/ANS 58.2-1988 standard (Reference 3.2.2-1) was used.
11 Appendices B, C, and D of Reference 3.2.2-1 provide the guidance necessary to determine the
12 geometry of a freely-expanding jet. Guidance is provided for jets originating from a variety of
13 reservoir conditions, including subcooled conditions.

14
15 The guidance in Reference 3.2.2-1 was used to determine the geometry of a jet originating from
16 a postulated break in a PWR piping system. A subcooled reservoir and flashing break flow were
17 assumed for the calculations as detailed below. The following steps were followed in
18 performing the calculations:

- 19 1. The mass flux from the postulated break was determined using the Henry-Fauske model,
20 as recommended in Appendix B, for subcooled water blowdown through nozzles, based
21 on a homogeneous non-equilibrium flow process. No irreversible losses were
22 considered.
- 23 2. The initial and steady-state thrust forces were calculated based on the guidance in
24 Appendix B of Reference 3.2.2-1, and the postulated reservoir conditions detailed below.
- 25 3. The jet outer boundary and regions were mapped using the guidance in
26 Appendix C, Section 1.1, of Reference 3.2.2-1 for a circumferential break with full
27 separation. The input to the equations from Appendix C for the thermodynamic
28 conditions at the asymptotic plane was calculated using principles of thermodynamics
29 and the postulated conditions in the reservoir.

- 1 4. A spectrum of isobars was mapped using the guidance in Appendix D of Reference
2 3.2.2-1. Several isobars were considered of interest, including the 10 psi isobar. The 10
3 psi isobar was of interest as NEDO-32686 (Reference 3.2.2-2) identified 10 psi as the
4 destruction pressure of jacketed Nukon® insulation with standard bands or unjacketed
5 Nukon®.
- 6 5. The volume encompassed by the various isobars was calculated using a trapezoidal
7 approximation to the integral. A study was performed to ensure that the results of the
8 volume calculations are not sensitive to the resolution of the trapezoidal approximation.
9 Since the volume result only represents the volume encompassed by the isobars in a free
10 jet, the volume encompassed by results were doubled to represent the Double Ended
11 Guillotine Break (DEGB).
- 12 6. The radius of an equivalent sphere was calculated to encompass the same volume as
13 double the volume of a feely expanding jet calculated from Step 5, above.

14 The radius calculated in Step 6, above, is taken to be the radius of the ZOI that will be used to
15 calculate the volume of debris generated from a postulated break.

16
17 The jet expansion calculations were based on the following conditions:

- 18 1. A circular break geometry was used for the calculations. This break geometry is
19 representative of both a postulated DEGB of primary piping as well as the DEGB of
20 piping attached to the RCS. The complete breaking of a pipe, either primary piping or
21 piping attached to the RCS, provides for a maximum debris generation volume as there
22 are two ends of the break to release fluid.
- 23 2. Fluid reservoir conditions of 2250 psia and 540 °F were used for the calculations. The
24 corresponding stagnation enthalpy and subcooling used in the calculations are
25 547.2 BTU/lbm and 102.7 °F, respectively. These conditions are intended to represent a
26 PWR in hot standby conditions and provide for a conservatively large ZOI compared to
27 hot leg conditions at power operations.

- 1 3. Ambient pressure of 14.7 psia was used. This is conservative as no credit is taken for
2 containment backpressure (the increase in containment pressure that would result from
3 the release of mass and energy into the containment as a result of the postulated break).

4 The ZOI is expressed as the ratio of the radius of the equivalent ZOI sphere to break size
5 diameter. This allows the ZOI to be expressed independent of the break size.

6
7 The use of a spherical ZOI is conservative compared to jet impingement evaluations previously
8 reviewed and approved by NRC. It is noted that, for a number of plants, a 10 D value is assumed
9 for the limit of jet damage. This is based on NUREG/CR-2913 dated January, 1983. As an
10 example, the acceptability of this approach is documented in the Supplement 6 of the Watts Bar
11 Safety Evaluation Report (SER);

12 “The applicant has given the staff information requiring the analysis of jet
13 impingement loads for postulated breaks. In FSAR section 3.6A.1.1.2, test data
14 and analysis developed in NUREG/CR-2913, “Two Phase Jet Loads,” dated
15 January 1983, are used to establish the criterion that unprotected components
16 located more than 10 diameters from a pipe break are without further analysis
17 assumed undamaged by a jet of steam or subcooled liquid that flashes at the
18 break. The staff has previously reviewed the methodology used in NUREG/CR-
19 2913 for determining the effects of such a jet on components at a distance greater
20 than 10 diameters and has found it acceptable.”

21 The 10D value is associated with a 10 degree half angle jet or a total jet spread of 20 degrees.
22 Thus, a spherical ZOI of 10D is conservative on volume by a factor of approximately 16 for a
23 single ended rupture and 8 for a double ended rupture as described in the following example;

24 Assume a 3 foot diameter pipe. If using a jet with a 10° half angle, with a damage
25 inducing length of 30 feet (10D). This would produce a cone with a volume of
26 approximately 853 ft^3 . The volume of a cone is $\frac{1}{3} \pi r^2 h$. A triangle 30 feet high
27 with sides a and b of equal length, the angle opposite side c of 20 degrees gives a
28 base of about 10.5 feet. One half of this value is r used in the cone equation. The
29 volume of a sphere is $\frac{4}{3} \pi r^3$. The “r” in the sphere equation for a 3 foot diameter
30 pipe is 15. This gives a volume of 14,137 ft^3 .

1 Dividing the volume of the cone into the volume of the sphere gives the amount of conservatism.
2 For a double ended rupture assuming that the two jets expanded without contacting each other
3 results in the factor of eight. Thus, the use of a spherical representation generally provides a
4 conservatively large approximation to the region affected by a jet.

6 **3.2.2.2.1.1 Insulation**

7
8 Equivalent spherical ZOI calculations were performed and documented (Reference 3.2.2-3) for
9 values of isobars corresponding to destruction pressures of several types of insulation. Table
10 3.2.2-1 summarizes these insulation types and the applicable ZOI, expressed as the ratio of the
11 ZOI radius to the break diameter, for which the calculations were performed. The calculations
12 summarized in Table 3.2.2-1 make no changes in insulation destruction pressures based on the
13 differences between dry or saturated steam jets and flashing jets. The reasons for this are as
14 follows:

- 15 1. The stagnation pressure of the jet is taken to correspond to the destruction pressure of
16 insulation.
- 17 2. It is also noted that there is only anecdotal test data to suggest modification of the
18 destruction pressures of insulation that were determined in support of the BWR strainer
19 blockage resolution.

20 Therefore, noting that the values for ZOI values listed in Table 3.2.2-1 maximize the ZOI, and
21 the use of a spherical representation generally provides a conservatively large approximation to
22 the region affected by a jet, no addition changes to insulation destruction pressures were made.

24 **3.2.2.2.1.2 Protective Coatings**

25
26 The criteria for DBA-qualification, or designation as “Acceptable,” of protective coatings
27 (paints) applied to systems, structures and components in PWR containments do not provide data
28 concerning coatings exposed to direct impingement of fluids. As such, the ZOI for DBA-
29 qualified coatings or coatings determined to be “Acceptable,” applied to PWR containment
30 surfaces, which results from fluid impingement from the break jet, has not been clearly defined.

1

2 However, an extensive body of data exists related to removal of industrial protective coatings by
3 high-pressure and ultra-high-pressure waterjetting. Examination of this data and associated
4 industry standards, compiled since the mid-1980's, reveals that industrial protective coating
5 systems, identical to the DBA-qualified and "Acceptable" coatings applied to systems, structures
6 and components in Pressurized Water Reactor (PWR) containments, require a water jetting
7 pressure of at least 7,000 psig to initiate destruction of sound coatings. This ability of coatings to
8 withstand high and ultra-high pressure has been reviewed and documented in a paper prepared
9 by a recognized industry coatings expert (Reference 3.2.2-5) and is included as Attachment A.

10

11 Based on evaluation presented, a destruction pressure of 1000 psi is chosen for coatings that
12 meet DBA-qualified or "Acceptable" criteria. This is conservative for the following reasons:

- 13 1. The value of 1000 psi is seven to eight times lower than the pressures that have been
14 observed in industrial practice to remove coatings using waterjet technology.
- 15 2. The initial reactor coolant system pressure of 2250 psi is about ¼ the pressures that have
16 been observed in industrial practice to remove coatings using waterjet technology.
- 17 3. Industrial experience with waterjet technology to remove coatings requires application of
18 the high-pressure jet at close proximity of the surface to which the coating is applied (<
19 12 inches from the jet nozzle discharge) for extended periods of time (> 60 seconds).
- 20 4. The blowdown of a PWR RCS due to a large LOCA is in the order of 30 seconds
- 21 5. The break discharge pressure decreases over the duration of the blowdown period.

22 Thus, it is concluded that the use of a value of 1000 psi as the destruction pressure for DBA-
23 qualified and "Acceptable" protective coatings is both appropriate and conservative. The
24 recommended ZOI to be used to evaluate protective coatings debris for the baseline containment
25 sump evaluation is listed in Table 3.2.2-1.

26

27 This same industrial experience suggests that the mechanism of coatings removal by waterjets is
28 erosion. The observed coatings debris sizes are in the range of 10 microns to 50 microns, not
29 flakes or chips. Thus, it is recommended that the coatings debris generated within the ZOI

1 representing 1000 psi be treated as fine particulate debris. It is further recommended that these
2 coatings debris be considered highly transportable.

3 4 **3.2.2.2 Selecting a Zone of Influence**

5
6 For the baseline calculation, the ZOI for insulation is selected based on the insulation inside
7 containment with the minimum destruction pressure. This ZOI is then applied to all insulation
8 types. As discussed in the previous section, this approach provides for the calculation of a
9 conservatively large value for debris generation.

10 11 **3.2.2.3 The ZOI and Robust Barriers**

12
13 For a given break location, the boundary of the spherical ZOI is drawn about the break. It is
14 possible that this boundary will extend beyond robust barriers such as walls and components.
15 Such barriers will terminate further expansion of the ZOI.

- 16 1. In the case of a wall, the sphere will be truncated at the intersection of the sphere and
17 wall.
- 18 2. For a component or structural components such as supports, a pressurizer, steam
19 generator, reactor coolant pump or jet shields, the area in the shadow of the component or
20 structure will be free from damage.

21 There is sufficient conservatism in drawing the sphere that it is not reasonable that a jet reflected
22 off of a wall or structure would extend further than the unrestrained sphere. Furthermore, there
23 is precedence for this conclusion. When evaluating targets for jet impingement, jets were
24 terminated when a robust barrier was encountered. Reflected jets were not considered as they
25 were bounded by the conservatism in the approach taken.

1 **Table 3.2.2-1: ZOI Radii for Common PWR Insulation and Coatings Materials**

2

Insulation Types	Destruction Pressure (psi)	ZOI Radius/Break Diameter	
		Calculated Value	Recommended Value
Protective Coatings (paints)	1000 ^(Ref. 3.2.2-5)	0.29	1.0
Transco RMI Darchem DARMET	190 ^(Ref. 3.2.2-6)	1.20	1.3
Jacketed Nukon® with Sure-hold® bands Mirror® with Sure-hold bands®	150 ^(Ref. 3.2.2-6)	1.59	1.6
Cal-Sil (Al. cladding, SS bands, seam @ 180°)	64 ^(Ref. 3.2.2-7)	2.84	2.9
Cal-Sil (Al. cladding, SS bands, seam @ 0°)	50 ^(Ref. 3.2.2-7)	3.25	3.3
K-wool	40 ^(Ref. 3.2.2-6)	3.69	3.7
Cal-Sil (Al. cladding, SS bands, seam @ 45°)	24 ^(Ref. 3.2.2-7)	5.37	5.4
Temp-Mat with stainless steel wire retainer	17 ^(Ref. 3.2.2-6)	7.6	7.7
Unjacketed Nukon®, Jacketed Nukon® with standard bands Knauf	10 ^(Ref. 3.2.2-6)	11.96	12.0
Koolphen-K	6 ^(Ref. 3.2.2-6)	16.81	16.9
Min-K Mirror® with standard bands	4 ^(Ref. 3.2.2-6)	21.47	21.5

3

4

5 **3.2.2.2.4 Simplifying The Determination of the ZOI**

6

7 Given the complexity of the analysis as a whole, it may be desired to make conservative

8 assumptions with the goal of simplifying the analysis. For example, for some breaks it may be

1 only slightly more conservative and much simpler to assume that an entire subcompartment (but
2 not outside the subcompartment) becomes the ZOI.

3
4 Once the boundary of the ZOI has been defined, proceed with determining the amount of debris
5 that is generated within the ZOI.

7 **3.2.2.2.5 Evaluating Debris Generation Within the ZOI**

8
9 Once the ZOI has been determined, calculate the amount of debris generated within the ZOI.
10 Information about the type, location and amount of debris sources within the containment is
11 obtained from plant drawings and the results of a condition assessment walk-down such as
12 described in NEI 02-01 (Reference 3.2.2-4). The characterization of the debris (transport
13 characteristics) is evaluated using the guidance of the following section.

15 **3.2.2.2.6 Sample Calculation**

14
17 The following is a sample calculation of a ZOI and the debris that would be generated within that
18 ZOI.

- 19 1. For the purposes of a sample calculation, a single break size and break location will be
20 assumed and evaluated.
- 21 2. The break will be assumed to be at the base of the steam generator.¹ The reason for this
22 choice is that often, more than one type of insulation is applied to steam generators.
23 Figure 3.2.2-1 shows a sample schematic of a reactor coolant system.
- 24 3. It will also be assumed that walk-down data for the plant is available and documented in
25 sufficient detail to support this evaluation. For the purposes of this evaluation, it will be
26 assumed that the walk-down was performed by dividing the containment into zones and
27 recording the amount of insulation in each zone. The debris generation zones defined
28 from the walk-down are also shown on Figure 3.2.2-1 and are labeled as Zone A1, Zone

¹ A 10-inch break is an idealization for the steam generator. It is used here to illustrate the calculation method.

1 A2, etc. Note that the plant layout and engineering judgment are used to define the zones
 2 used to record the location of insulation inside containment.

- 3 4. The postulated break will be assumed to occur in Zone A4. For this sample calculation, it
 4 is assumed that the walk-down records show the amount of insulation in Zone A4 to be:

- 5 • 300 ft³ of Nukon insulation
- 6 • 15,000 ft² of RMI

- 7 5. A 10-inch break of piping attached to the RCS is assumed. The corresponding ZOI is
 8 evaluated as follows:

9 The diameter of the break is taken as:

$$10 \quad D_{BREAK} = 10 \text{ inches}$$

11 Using the criteria identified above, the minimum destruction pressure for insulation is
 12 used to determine the ZOI. From Table 3.2.2-1, the recommended ratio of ZOI radius
 13 to break diameter is;

$$14 \quad \frac{r_{ZOI}}{D_{BREAK}} = 12$$

15 The radius of the spherical ZOI is calculated as:

$$16 \quad r_{ZOI} = D_{BREAK} \times \frac{r_{ZOI}}{D_{BREAK}} = 10 \text{ inches} \times 12 = 120 \text{ inches} = 10 \text{ ft}$$

17 Thus, the radius of the ZOI is determined to be 10 feet. This ZOI is conservatively
 18 applied to all insulations types in the region within the ZOI for the baseline evaluation.

- 19 6. A ZOI having a 10 foot radius is superimposed at the base of the steam generator in Zone
 20 A4 of Figure 3.2.2-1. From the figure, it is observed that the ZOI includes a substantial
 21 portion of the steam generator and associated reactor coolant system piping within Zone
 22 A4. Insulation is applied to these components. Therefore, for the purposes of this sample
 23 calculation, a ZOI with a radius of 10 ft is conservatively evaluated to result in the
 24 destruction of all the insulation within Zone A4. This results in the following volumes of
 25 insulation debris:

- 300 ft³ of Nukon insulation
- 15,000 ft² of RMI.

7. Using the recommended ratio of ZOI radius to break diameter for coatings of 1.0 that is given in Table 3.2.2-1, the radius of the coatings ZOI is evaluated as:

$$r_{ZOI} = D_{BREAK} \times \frac{r_{ZOI}}{D_{BREAK}} = 10 \text{ inches} \times 1.0 = 10 \text{ inches} = 0.833 \text{ ft.}$$

From Figure 3.2.2-1, it is clearly observed that the coating ZOI will not be in contact with either walls or floors. Furthermore, with a small ZOI for coatings, coated structures or components may not be within the ZOI. However, a conservative estimate of the square footage of coatings debris is estimated by using the surface area of the sphere of the coatings ZOI:

$$A_{COATINGS DEBRIS} = 4 \pi r^2 = 4 \pi (0.833)^2 = 8.72 \text{ sq. ft.}$$

Thus, the amount of coatings debris generated by the postulated 10-inch break is conservatively estimated to be 8.72 ft².

The transport characteristics of the debris volumes calculated above are evaluated using the guidance of the following section. The transport of the debris evaluated using the guidance of Section 3.2.4, and the resulting head loss evaluated using the guidance of Section 3.2.5. The debris generation evaluation is repeated using the guidance of Section 3.2.1 until the limiting head loss is evaluated.

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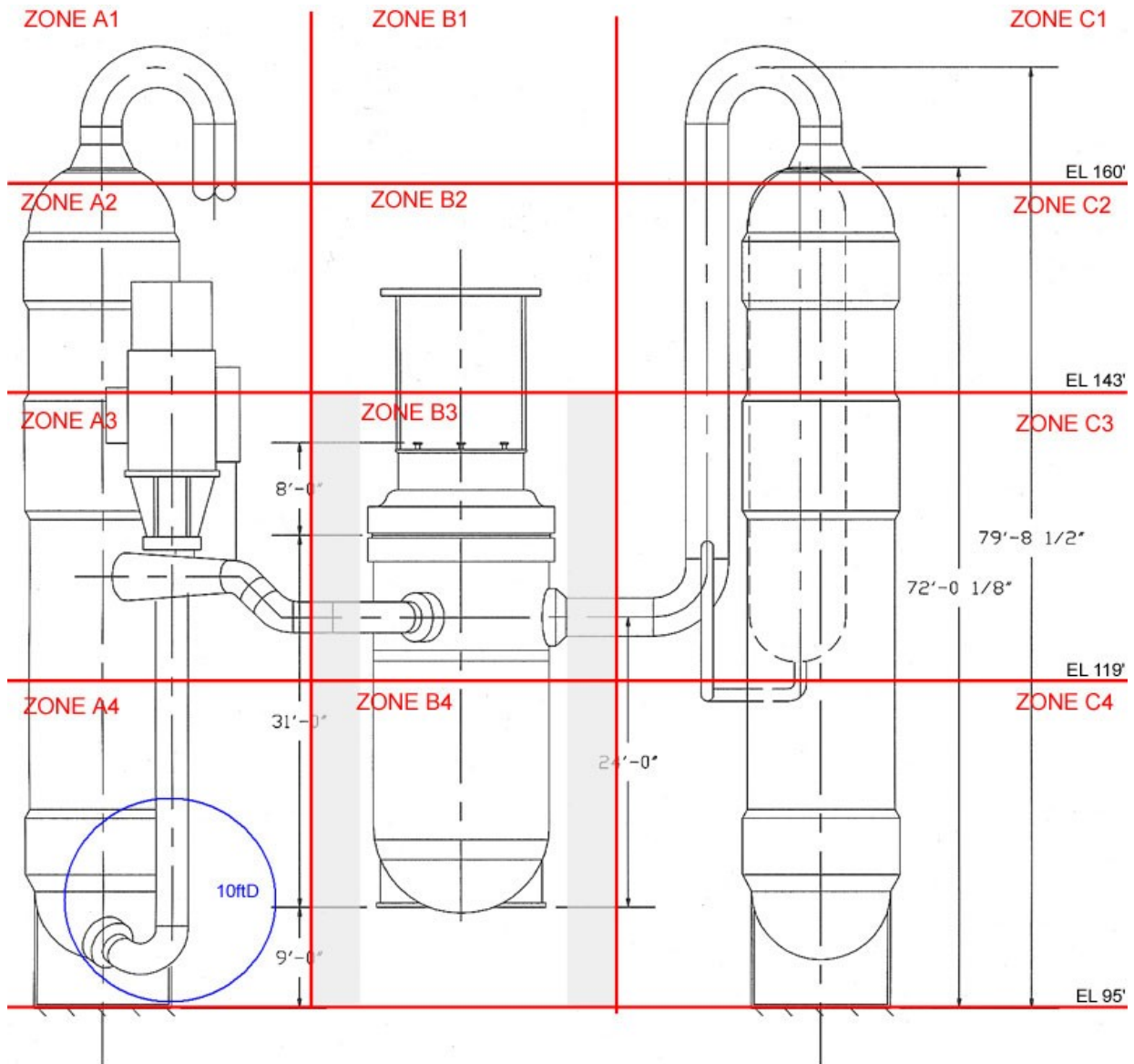


Figure 3.2.2-1: Schematic Showing Reactor Coolant System and Walkdown Zones

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3.2.2.3 Quantification of Debris Characteristics

3.2.2.3.1 Definition

Debris characteristics are:

- The post-accident (LOCA and/or secondary pipe breaks where applicable) size distribution size distribution of a material, and
- The debris material size and shape as well as the micro-density (i.e. material density) and macro-density (i.e. as fabricated density).

Debris characteristics are used in transport and head loss calculations. The debris generation section provides the following items as inputs to this section:

- The volume of insulation material in a ZOI
- The surface area of the ZOI for coatings
- The total quantity of indeterminate² and unqualified coating inside containment.
- The total quantities of indeterminate and unqualified coating that have been applied to piping that are covered by undamaged insulation.

3.2.2.3.2 Discussion

The first order debris characteristic is the size distribution of the material inside the Zone of Influence (ZOI) of a postulated pipe break. Following a postulated pipe break, all material inside containment may also be subjected to containment spray or immersed in the post-accident pool, and additional debris would be generated, hence the characteristics of the debris generated post-blowdown also need to be identified.

² For definitions of DBA-qualified / acceptable, DBA-unqualified / unacceptable and indeterminate coatings, see ASTM D-5144-00, "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants."

1 There have been numerous schemes developed for classifying debris size distribution of material
2 inside a ZOI. Most of the classifications schemes developed were for low-density fiberglass
3 blankets manufactured by Performance Contracting Inc. (PCI) and Transco. NUREG/CR-6369
4 (Ref. 3.2.2-8) employed 5 fibrous debris size classification schemes, with 3 to 6 size designations
5 (e.g. large, medium, and small). NUREG/CR-6224 (Ref. 3.2.2-9) adopted a classification
6 scheme of 7 size categories for fiber. As noted in NUREG/CR-6369, the BWROG URG adopted
7 a fiber classification scheme of 2 sizes: fines and large.

8
9 The Air Jet Impact Tests conducted by the BWROG indicated a dependence of the size
10 distribution of the debris as a function of distance from the nozzle, i.e., the higher the pressure
11 the larger the quantity of small debris. As discussed in NUREG/CR-6808 (Ref. 3.2.2-6) Section
12 3.3, an analytical model could be applied that correlates the size distribution to the spherical ZOI.
13 This type of modeling requires the understanding of the damage distribution based on applicable
14 experimental data. Unfortunately there is a paucity of applicable debris generation test data
15 applicable for PWR conditions. In the absence of directly applicable experimental data, i.e. tests
16 conducted with prototypical PWR conditions, for a wide variety of material, the NEI Guideline
17 adopts a two size distribution for material inside of the ZOI of a postulated break: small fines and
18 large pieces. Small fines will be defined as any material that could transport through gratings,
19 trash racks, or radiological protection fences by blowdown, containment sprays, or post-accident
20 pool flows. Furthermore, the small fines are assumed to be the basic constituent of the material
21 for fibrous blankets, (i.e. individual fibers) and pigments for coatings. This guideline assumes
22 the largest openings of the gratings, trash racks, or radiological protection fences to be less than a
23 nominal 4 inches by 4 inches (less than 20 square inches total open area). The remaining
24 material that cannot pass through gratings, trash racks, and radiological protection fences is
25 classified as large pieces.

26
27 Some material in the post-DBA environment will be eroded by the water flows. Additionally,
28 some debris material may be disintegrated by the water flow. The classification for fibrous
29 material in the ZOI adopted by this guidance assumes that all fibrous material classified as small
30 fines are essentially reduced to the individual fibers. As such, the debris classification implicitly
31 considers the erosion and disintegration of the debris by conservatively assuming that they are

1 already of a characteristic size that cannot be further decreased by erosion or disintegration. For
2 fibrous insulation material, the large pieces are assumed to be jacketed/canvassed. According to
3 NUREG/CR-6369 jacketed pieces are not subjected to further erosion. The same conservatism
4 was applied for coatings in the ZOI where this guideline assumes that all coatings in the coating
5 ZOI are considered to be small fines of the size of the original pigment, hence not capable of
6 being subjected to erosion or disintegration. For material outside the ZOI, all insulation material
7 that is jacketed is assumed not to undergo erosion or disintegration by containment spray or
8 break flow. This assumption is based also on NUREG/CR-6369 tests that showed no erosion of
9 damaged jacketed material, hence the same applies to un-damaged jacketed material.
10 Additionally PCI has conducted tests on undamaged NUKON[®] blankets to demonstrate that they
11 do not subject to erosion in a post-DBA environment. The NRC issued an SER on the tests
12 accepting the PCI test results.

13

14 The main source of data on debris size distribution of material subjected to simulated pipe break
15 conditions are those reported in the BWROG URG AJIT tests and the NRC debris transport set
16 of experiments described in NUREG/CR-6339. This NEI Guideline selected the test of the
17 insulation that had the most data points (NUKON[®]) that produced the smallest fines and adopted
18 this point as the bounding values of fines production for un-jacketed fibrous blankets. The data
19 of size distribution following exposure to simulation of a pipe break close to PWR prototypical
20 conditions is depicted in Table 3-7 of NUREG/CR-6808 for a low-density fiberglass tested at
21 Ontario Power Generation. That test indicates 52% were of the category defined as small fines
22 adopted by this guideline. This test suggests that the size distribution for NUKON[®] blankets in
23 this guideline to be conservative for PWR applications. For fibrous insulation materials that
24 underwent testing at AJIT, this guideline adopted the NUKON[®] blanket size distribution for
25 fibrous blankets whose destruction pressure was the same or higher than for NUKON[®] blankets.
26 If a material has a higher destruction pressure it signifies that the material has a higher resistance
27 to damage. As such, the size distribution would tend to be larger than a more fragile material
28 indicated by a lower destruction pressure. Therefore it is conservative to adopt the NUKON[®]
29 blanket size distribution for material with a higher destruction pressure. For material with an
30 equivalent destruction pressure as NUKON[®] blankets, engineering judgment suggests that the
31 fraction of fines should be no worse than for NUKON[®] blankets.

1
2 The calculation of the quantities for each size category for each of the materials entails
3 multiplying the volume of each material calculated to be in the ZOI by the percentage of the two-
4 size distribution recommended below.

6 **3.2.2.3.3 Size Distribution**

8 **3.2.2.3.3.1 Fibrous Material in a ZOI**

9
10 The fibrous classification of “small fines” adopted in this guideline can be correlated to the
11 combination of “small” and “medium” classification of Table 3-7 of NUREG/CR-6369 Vol. 2,
12 the combination of “small” and “large” classification of Table 2-5 of NUREG/CR-6369 Vol. 1,
13 Classes 1 – 6 of NUREG/CR-6224, and the combination of “Fines” and “Large” classification
14 of the BWROG URG Air Jet Impact Test (AJIT). The classification of “large pieces” adopted in
15 this guideline can be correlated to the “large” category of Table 3-7 of NUREG/CR-6369 Vol. 2,
16 the “large canvassed” of Table 2-5 of NUREG/CR-6369 Vol. 1, Class 7 and “non-transportable”
17 of NUREG/CR-6224, and the combination of “canvas” of the BWROG URG Air Jet Impact
18 Test.

19
20 The following are the material-specific size distribution values adopted by this guideline:

- 21
- 22 a. NUKON® Fiber Blankets. This guideline adopts the value of 60% for small fines
23 and 40% for large pieces as the size distribution of NUKON® (jacketed or
24 unjacketed) inside a pipe break ZOI. As noted previously, these values were selected
25 from the BWROG URG Air Jet Impact Test of NUKON® that generated the largest
26 quantity of small fines and is consider being applicable to PWR conditions based on
27 the Ontario Power Generation test reported in NUREG/CR-6808.
 - 28
 - 29 b. Transco Fiber Blankets. This guideline adopts the value of 60% for small fines and
30 40% for large pieces as the size distribution of NUKON® inside a pipe break ZOI.
31 Transco blankets were not tested by the BWROG at the CEESI Air Jet Impact test

1 facility. Transco blankets were used, however, by the NRC at the CEESI Air Jet
2 Impact test facility as documented in NUREG/CR-6369. The study shows that the
3 Transco blankets tested behaved similar to the NUKON[®]. Given these experimental
4 data, engineering judgment suggests that Transco low density fiberglass blankets
5 would behave similarly to the NUKON[®] fiberglass blankets when subjected to
6 prototypical PWR DEGB DBA conditions, hence the size distribution adopted for
7 Transco fiberglass blankets in this guideline is conservative since the size distribution
8 adopted for NUKON[®] fiberglass blankets was the most conservative size distribution
9 of any of the AJIT tests of NUKON[®] fiberglass blankets.

- 10
- 11 c. Knauf. Knauf was tested by the BWROG at the CEESI Air Jet Impact test facility
12 and shown to have the same destruction pressure as NUKON[®]. Since the destruction
13 pressure is the same as NUKON[®], engineering judgment suggests that the size
14 distribution should be no worse than NUKON[®]. Hence this guideline adopts the
15 same size distribution for Knauf as NUKON[®]: 60% for small fines and 40% for large
16 pieces.
- 17
- 18 d. Temp-Mat. Temp-Mat was tested by the BWROG at the CEESI Air Jet Impact test
19 facility and shown to have a higher destruction pressure than NUKON[®]. Since the
20 destruction pressure is higher than NUKON[®], engineering judgment suggests that the
21 size distribution should be no worse than NUKON[®]. Hence this guideline adopts the
22 same size distribution for Knauf as NUKON[®]: 60% for small fines and 40% for large
23 pieces.
- 24
- 25 e. K-Wool. K-Wool was tested by the BWROG at the CEESI Air Jet Impact test
26 facility and shown to have a higher destruction pressure than NUKON[®]. Since the
27 destruction pressure is higher than NUKON[®], engineering judgment suggests that the
28 size distribution should be no worse than NUKON[®]. Hence this guideline adopts the
29 same size distribution for K-Wool as NUKON[®]: 60% for small fines and 40% for
30 large pieces.
- 31

- 1 f. Min-K. Absent applicable experimental data, a value of 100% small fines is adopted
2 by this guideline for Min-K in a ZOI.
- 3
- 4 g. Generic Low-Density Fiberglass. Absent applicable experimental data a value of
5 100% small fines is adopted by this guideline for generic fiberglass in a ZOI.
- 6
- 7 h. Generic High-Density Fiberglass. Absent applicable experimental data a value of
8 100% small fines is adopted by this guideline for generic high-density fiberglass in a
9 ZOI.
- 10
- 11 i. Generic Mineral Wool. Absent applicable experimental data a value of 100% small
12 fines is adopted by this guideline for any type of mineral wool in a ZOI.
- 13

14 **3.2.2.3.3.2 Reflective Metallic Insulation (RMI) in a ZOI**

15

16 NEI guideline adopts one size distribution classification scheme for all types of RMI insulation
17 after exposure to the conditions within a PWR ZOI, since their ensuing transport and head loss
18 guidelines do not differentiate between different types of RMI (i.e. stainless steel or aluminum).

19

20 RMI. The NEI guideline adopts the value of 75% for small fines and 25% for large
21 pieces as the size distribution of any type of RMI inside a pipe break ZOI. These
22 values are based on the size distribution of less than 4 inches as listed in Figure 3-7 of
23 NUREG/CR-6808 based on the two phase testing of a Diamond Power RMI cassette.
24 The size of 4 inches was selected as a conservative upper bound of an RMI debris
25 size that would go through gratings, trash racks, or radiological protection fences by
26 blowdown, containment sprays, or post-accident pool flows. BWROG URG Air Jet
27 Impact Tests (AJIT) of other types of RMI suggests a significantly larger destruction
28 pressure and a consequently smaller quantity of small size debris. Engineering
29 judgment suggests that the 75% adopted for the RMI small-size category in this
30 guideline is conservative in that it is based on the test that resulted in the largest
31 quantity of small RMI debris for a type of RMI that has the lowest AJIT destruction
32 pressure.

1
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3 **3.2.2.3.3 Other Material in ZOI**
4

- 5 a. Calcium Silicate. There is a wide variety of calcium silicate type insulation installed
6 in PWRs. Some include fiberglass fibers as re-enforcement, some others use organic
7 fibers, and some of the Cal-Sil used up to the late 1950s used asbestos fibers. The
8 Cal-Sil solubility also varies from manufacture to manufacture with some Cal-Sil
9 dissolving promptly in hot water whereas some dissolve at a significantly lower rate.
10 The only publicly available size distribution data on the reaction of an unspecified
11 Cal-Sil to a two-phase jet are found in Table 3-6 of NUREG/CR-6808. Test 5
12 indicated that the size categories adopted by this guideline would be 50% for small
13 fines and 50% for large Cal-Sil pieces. Given the uncertainties in the subsequent
14 erosion by the post-DBA water this guideline assumes 100% of Cal-Sil in a ZOI is
15 destroyed as small fines.
16
- 17 b. Microtherm. Absent applicable experimental data, a value of 100% small fines is
18 adopted by this guideline for Microtherm in a ZOI.
19
- 20 c. Koolphen. Absent applicable experimental data, a value of 100% small fines is
21 adopted by this guideline for Koolphen in a ZOI.
22
- 23 d. Fire Barrier. Absent applicable experimental data or qualification documentation, a
24 value of 100% small fines is adopted by this guideline for all types of fire barrier
25 material in a ZOI.
26
- 27 e. Lead Wool. Absent applicable experimental data, a value of 100% small fines is
28 adopted by this guideline for all types of lead wool material in a ZOI.
29
- 30 f. Coatings. All coatings within the Coatings ZOI are considered in this guideline to fail
31 when subjected to DBA conditions. Guidance concerning the determination of the

1 Coatings ZOI is contained in Reference 3.2.2-5. Absent applicable experimental
2 data, a coating debris size value of 100% small fines (10 µm IOZ equivalent) is
3 adopted by this guideline for all types of coating material in the ZOI.
4

5 **3.2.2.3.3.4 Material Outside the ZOI**

6

7 Material outside the ZOI can be subjected to containment spray and/or be immersed in the post-
8 DBA pool. Under these circumstances some material could become debris and become subject to
9 transport to the sump screen. Material and components that meet equipment qualification
10 requirements (i.e. material and components on the Environmental Qualification list) have been
11 demonstrated not to degrade in a post-DBA environment so they will not contribute to the post-
12 DBA debris load.
13

14 a. Covered (Jacketed) Undamaged Insulation. NUKON[®] blankets are EQ qualified and as
15 such will not be damaged by the post-DBA environment outside the ZOI. The few
16 publicly available data for reaction of jacketed fibrous insulation material to post-DBA
17 conditions that exist were performed by the NRC. The NRC tests were performed on low
18 density fiber (Transco blankets) and reported in NUREG/CR-6369 Volume 1. Both series
19 of tests were conducted with pieces of blankets that had been subjected to the air jet
20 impact tests at the AJIT facility. No intact blankets were tested. NUREG/CR-6369
21 concluded that partially torn insulation blankets that retained their cover were unlikely to
22 be eroded by water flow from washdown and spray. Based on these tests and the EQ
23 qualification of NUKON[®] blankets, this guideline adopts the position that covered
24 (jacketed) undamaged insulation material outside the ZOI will not generate transportable
25 debris (covered or jacketed insulation is any insulation that the raw material, e.g.
26 fiberglass bats, are covered or encapsulated by another material).
27

28 b. Other material outside the ZOI.

- 29 • Fire Barrier. Applying the same logic as was concluded in NUREG/CR-6339 for
30 partially torn insulation that retained their covers/jackets, all jacketed or covered
31 fire barriers are presumed not to degrade by the post-accident environment,

1 hence not generate debris. Fire barrier materials that are unjacketed are
2 presumed to fail as small fines.

- 3 • Lead Wool. The lead wool blankets have the same general covers as the
4 NUKON[®] and Transco blankets. As such the conclusion of the NRC experiments
5 are applicable. The NEI Guideline considers that all lead wool blankets outside
6 the ZOI will not be damaged by the post-DBA environment.
- 7 • Unjacketed insulation. All material outside the ZOI that is unjacketed, e.g.
8 fiberglass bats without any covering are presumed to fail to small fines.
- 9 • Coatings. DBA-qualified / acceptable coatings³ located outside the Coatings ZOI
10 are considered in this guideline not to fail when subjected to containment spray
11 or immersed in the post-DBA pool. All indeterminate and DBA-unqualified /
12 unacceptable coatings are considered in this guideline to fail. This baseline
13 guideline considers all indeterminate and DBA-unqualified / unacceptable
14 coatings as a single category of coating, producing debris of the same
15 characteristic independent of the type of coating, when subjected to containment
16 spray or immersed in the post-DBA pool. All types of coatings on piping or
17 components covered with undamaged insulation are considered in this guideline
18 not to contribute to the post-DBA debris source term.

20 **3.2.2.3.4 Calculate Quantities of Each Size Distribution**

21
22 The total quantity of each size distribution for each material is the summation of the size
23 distribution for the debris size quantity in the ZOI added to the debris size quantity outside the
24 ZOI. To calculate the quantity of debris size for a material⁴ the process is:

³ For definitions of DBA-Qualified / acceptable, DBA-unqualified / unacceptable and indeterminate coatings, see ASTM D5144-00, "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants."

⁴ Plant specific information size distribution based on qualification testing should be used in lieu of the general recommendation of the NEI Guideline.

- 1 1. To obtain the quantity of small fines, multiply the volume of a material in the ZOI
2 computed in the debris generation section by the recommended value of the small size
3 percentage.
- 4 2. To obtain the quantity of large fines, multiply the volume of a material in the ZOI
5 computed in the debris generation section by the recommended value of the large size
6 percentage.
- 7 3. Recent surveys of US PWR containments per NEI 02-01 have determined that the
8 majority of the coatings on structures, systems and components within containment can
9 be classified into three major categories:
 - 10 A. inorganic zinc primers,
 - 11 B. epoxy primers and topcoats, and,
 - 12 C. epoxy phenolic primers and topcoats.

13 Plant specific information should be used to estimate the thickness of the
14 coatings. For those plants that do not have detailed plant specific information, the
15 following guidance is provided. For coatings within the ZOI, multiply the area of
16 the coating ZOI as determined in the debris generation section by the thickness of
17 the coating system: 3 mils inorganic zinc primer plus 6 mils epoxy / epoxy
18 phenolic topcoat⁵ to obtain the quantity (volume) of coating debris small fines
19 from a ZOI (Ref. 3.2.2-10). Coatings within the ZOI will be reduced, worst case,
20 post-DBA to small (10 μm ⁶), pigment-sized particles (see Table 3.2.2-3).

21
22 To obtain the quantity (volume) of coating debris outside the ZOI, multiply the
23 total area of DBA-unqualified / unacceptable and indeterminate coatings⁷ in
24 containment by the worst case of 3 mils inorganic zinc primer (Ref. 3.2.2-10).

⁵ Typical dry film thickness values for inorganic zinc primers and epoxy / epoxy phenolic primers and topcoats are taken from coating manufacturer's product data sheets (for instance, Carboline CZ 11, Carboline Phenoline 305 primer and finish, Ameron D-6, Ameron D-9, Ameron Amercoat 66) for coating products currently installed in US PWR containments.

⁶ The 10 micron size is conservative (i.e. more transportable and causes higher head losses) than the larger sizes suggested in Section 3.2.1.3 and Attachment A.

⁷ For definitions of DBA-unqualified / unacceptable and indeterminate coatings, see ASTM D5144-00, "Standard Guide for Use of Protective Coating Standards in Nuclear Power Plants."

1 Note that epoxy and epoxy phenolic coating failure outside the ZOI will result, in
 2 all likelihood, in debris that are relatively larger, highly cohesive, no smaller in
 3 the worst case than 25 μm . Unfortunately there are no applicable experimental
 4 data as to the size distribution of failed DBA-unqualified / unacceptable and
 5 indeterminate coatings when subjected to a post-DBA environment. As such, the
 6 assumption that an equivalent volume of inorganic zinc particulate debris (particle
 7 size 10 μm) is conservative.

8 9 **3.2.2.3.5 Sample Calculation**

10
11 Material in the ZOI:

12
13 Total volume of NUKON[®] blankets in ZOI: 300 cu ft

14 Quantity of small fines of NUKON[®] in the ZOI: 300 cu ft * 60% = 180 cu ft

15 Quantity of large pieces of NUKON[®] in the ZOI: 300 cu ft * 40% = 120 cu ft

16
17 Total area of RMI material in ZOI: 15,000 sq ft

18 Quantity of small fines of RMI in the ZOI: 15,000 sq ft * 75% = 11,250 sq ft

19 Quantity of large pieces of RMI in the ZOI: 15,000 sq ft * 25% = 3,750 sq ft

20
21 Coatings:

22
23 The baseline sample plant has not conducted a detailed containment coating walkdown. From
 24 the debris generation, the coating ZOI has a radius of 10 inches. The surface area of a 10 inch
 25 sphere is 8.7 sq ft. The total quantity of failed coatings from the ZOI can be calculated as: 8.7
 26 sq ft * 7.5 E-4 ft⁸ = 0.007 cu ft of IOZ equivalent debris.

27
28 From the plant Appendix R, the total quantity of coatings in containment is 190,000 sq ft. From
 29 the plant construction records a total of 160,000 sq ft can be shown to be DBA qualified. Hence

⁸ 9 mills = 7.5 E-04 ft

1 the total quantity of DBA-unqualified / unacceptable and indeterminate coatings is estimated to
 2 be 30,000 sq ft. The total quantity of small fines coating from outside ZOI: (30,000 sq ft of total
 3 quantity of unqualified and undetermined coating in containment less the 0 sq ft of unqualified
 4 and undetermined coating on piping that is covered by undamaged insulation.) * $2.5E-4 \text{ ft}^9 = 7.5$
 5 cu ft of IOZ equivalent debris.

7 **3.2.2.3.6 Debris Characteristics for Use in Debris Transport and Head Loss**

8
 9 The debris characteristics for the Small Fines size adopted by this guideline are those in the
 10 following tables labeled as characteristic size. The next sections describe the characteristics of
 11 common fibrous, coatings and particulate debris.

12
 13 The characteristic sizes listed are the most conservative values that can be associated with debris
 14 transport and head loss since they are the size that will have the highest transport factor and
 15 cause the highest head loss. Other small debris characteristic sizes can be adopted in lieu of those
 16 listed for materials that have applicable transport and head loss experimentally determined
 17 characteristic sizes. Plant-specific data can supersede these where necessary and appropriate.

19 **3.2.2.3.6.1 Mass Insulation**

20
 21 This class of insulation includes low-density fiberglass ($\sim 2.4 \text{ lbm/ft}^3$), medium-density
 22 fiberglass, and preformed fiberglass, as well as fiber felt materials. It also includes microporous
 23 insulation such as MinK and Microtherm, as well as Calcium Silicate insulation.

24
 25 There are three principal types of mass insulation in PWR containments:

- 26 • Fibrous Insulation (including Asbestos)
- 27 • Granular Insulation (Calcium Silicate & Microporous)
- 28 • Cellular Insulation

29
⁹ 3 mills = 2.5 E-04 ft

1 The characteristic densities and sizes for thermal insulation materials that have been identified as
2 potential debris in nuclear containments are listed in Table 3.2.2-2. Some are listed by trade
3 names and some by generic names, whereas others are listed as a system and still others as
4 simply an insulation material. For materials not listed the manufacturer should be contacted to
5 obtain the type of information listed in Table 3.2.2.-2.

6
7 Fibrous insulation materials include fibrous glass wool such as Performance Contracting's
8 NUKON®, Transco Products' Thermal Wrap®, pre-formed fiberglass pipe (made by Owens-
9 Corning, Knauf, and Johns-Manville), and fiberglass pipe and tank wrap (from the same three
10 manufacturers).

11
12 The NRC refers to the insulation fillers in NUKON®, Thermal Wrap®, and Knauf-ET as "Low
13 Density Fiber Glass" (LDFG). The LDFG materials are soft, loose and contain minimal binders.
14 There is extensive test data for LDFG. There are also some glass fiber felt mat insulation
15 materials and these include Temp-Mat® and Insulbatte® insulations, both made by JPS Corp., as
16 well as some by other trade names such as AlphaMat® by Alpha Inc. Again, these are relatively
17 soft and loose. Other fibrous materials include ceramic felt mat insulation, two of which are
18 Kaowool® and Cerawool®, both by Thermal Ceramics, Inc.

19
20 Finally, there are mineral wool insulation products with a number of different trade names,
21 forms, and densities. Major North American manufacturers are Rock Wool Manufacturing,
22 Roxul, Fibrex, IIG, and Thermafiber. These materials have higher densities and are generally
23 stiffer, having more binder and particulate. While mineral wool has been widely used in Europe,
24 mineral wool has limited use in North American nuclear containments. Mineral wool was the
25 original drywell piping insulation at the Barseback Plant that was blown off by a lifted steam
26 relief valve and which subsequently blocked a couple of ECCS strainers. In general, mineral
27 wool is available in densities that are at least twice those of comparable fibrous glass wool
28 insulations, up to ~10 pcf.

29
30 Asbestos insulation may be encountered at some plants. It is typically used as a structural fiber in
31 calcium silicate insulation and sold under the trade name Unibestos.

1
2 Granular insulation materials include calcium silicate and microporous insulation. All the
3 calcium silicate insulation in North America has been manufactured without the use of asbestos
4 since about 1972. Produced by various manufacturers over the years, today all calcium silicate is
5 manufactured by IIG, a joint venture between Calsilite Corp. and Johns-Manville Corp., at three
6 factories. The only microporous insulation manufactured in North America is MinK®,
7 manufactured by Thermal Ceramics, Inc. today but by Johns-Manville for many years.
8 Microtherm®, manufactured in the UK, is also available in North America.

9
10 The only cellular insulation in Table 3.2.2-2 is cellular glass. Most of what has been installed in
11 US nuclear plants has been manufactured by Pittsburgh Corning Corporation and is known by its
12 trade name, Foamglas®. This is an inorganic, rigid, and brittle cellular insulation typically used
13 in containments on chilled water lines. However, for reference, there are numerous other types
14 of cellular insulations available which are organic compounds. These include melamine,
15 polystyrene, polyisocyanurate, phenolic, polyimide, polyolefin, flexible elastomeric, and
16 polyurethane foams. There are numerous trade names by which these are known. The best
17 known is Dow Chemical's Styrofoam, which is polystyrene foam insulation.

18 19 **3.2.2.3.6.2 Failed Coatings**

20
21 To properly characterize coatings debris for the head loss evaluation, the type, mass, application
22 thickness, particle sizes, and surface area or volume are necessary inputs, and these should be
23 specified to the extent practicable in the debris generation and debris transport calculations. The
24 quantity of a failed coating is adequately specified by the mass of the coating and its density.
25 Alternatively, the surface area of the failed coating, along with its thickness and the density can
26 be used to determine the mass.

27
28 Unless replaced by plant-specific information of higher value, Table 3.2.2-3 lists the bulk density
29 and the characteristic size and shape for various types of coatings debris, and these can be used
30 for the evaluation. The actual size distributions of these materials in a post-DBA environment

1 are not known. Thus, the table lists particle sizes that are conservative (i.e. small) for head loss
2 evaluations. Plant-specific data, if available, can supersede these data.

3
4 The following types of coatings are commonly found within PWR containments: Inorganic Zinc
5 (IOZ), Epoxy, Epoxy-Phenolic and Alkyd. The densities for the epoxy, epoxy-phenolic and
6 alkyd coatings listed in Table 3.2.2-3 are based on specific gravities presented in the
7 “Performance of Containment Coatings During a Loss of Coolant Accident.” (Ref. 3.2.2-11).
8 The density for IOZ is 437 lbm/ft³ as reported by Carboline for the zinc dust used in the
9 formulation of CarboZinc-11 (Ref. 3.2.2-12).

10
11 This guidance assumes complete destruction of coatings within the coating ZOI. In the absence
12 of specific experimental data about the debris particle size distribution for IOZ, alkyds, epoxy
13 and epoxy-phenolic coating debris generated by high pressure water/steam jets in the ZOI, a
14 diameter of 10 μm is assumed as the characteristic size of coating debris generated within the
15 ZOI. The 10 μm characteristic diameter is the nominal diameter of unbound zinc particles and
16 also the alkyd pigment particles of failed coatings. Coatings outside the ZOI that have not been
17 demonstrated to be DBA-qualified or “acceptable,” or whose qualification is “indeterminate,”
18 are assumed to fail as chips. A typical lower bound for epoxy and epoxy-phenolic coating chip
19 thickness is 1-mil (25.4 μm). A ten-micron (10 μm) diameter is assumed as the characteristic
20 size of debris from IOZ and alkyd coatings outside the ZOI that have not been demonstrated to
21 be DBA-qualified or “acceptable,” or whose qualification is “indeterminate.”

1 **Table 3.2.2.-2: Mass Insulation Material Debris Characteristics¹⁰**

2

Debris Name	Insulation Material Description	As-Fabricated Density (lbs/ft ³)	Material Density (lbm/ft ³)	Characteristic Size ¹¹	
				µm	inch
PCI's NUKON [®] Blankets	Removable / reusable blankets with woven glass fiber cloth covering fibrous glass insulating board (referred to by the NRC as a "LDFG")	2.4 ^{3.2.2-15, 3.2.2-19}	159 ^{3.2.2-19}	7.0 fiber diameter	28E-05 ^{3.2.2-15, 3.2.2-19}
Fiberglass – preformed pipe	Knauf fibrous glass wool preformed into cylindrical shapes	4.0 +/- 10% ^{3.2.2-14} or	159 ^{3.2.2-14}	7.5 fiber diameter	30E-05 ^{3.2.2-14}
Fiberglass – preformed pipe	Owens-Corning fibrous glass wool preformed into cylindrical shapes	3.5 to 5.5 ^{3.2.2-19}	159 ^{3.2.2-19}	8.25 fiber diameter	33E-03 ^{3.2.2-19}
Fiberglass – pipe and tank wrap	Fibrous glass wool wrap, using perpendicularly oriented fibers, adhered to an All Service Jacketing (ASJ) facing (made by Knauf, Owens-Corning, & others)	3.0 +/- 10%	159 ^{3.2.2-14}	6.75 fiber diameter	27E-05 ^{3.2.2-14}
Transco's Thermal Wrap [®] Blankets	Removable / reusable blankets with woven glass fiber cloth covering fibrous glass insulation)	2.4 ^{3.2.2-14, 3.2.2-25}	159 ^{3.2.2-14}	5.5 fiber diameter	22E-05 ^{3.2.2-14}
Knauf	Knauf ET Panel (LDFG similar to Nukon)	2.4	159	5.5 fiber diameter	22E-05
Temp-Mat [®] and Insulbatte [®]	Glass fibers needled into a felt mat; these are trade names of insulation products made by JPS Corp.	11.8 ^{3.2.2-16}	162 ^{3.2.2-16, 3.2.2-17}	9.0 fiber diameter	36E-05 max. average ^{3.2.2-24}
Cellular Glass	Foamglas [®] is the trade name for this cellular glass product made by Pittsburgh Corning Corporation	6.1 to 9.8 (mean value of 7.5) ^{3.2.2-26}	156 ^{3.2.2-26}	NA	0.05 to 0.08 pore size ^{3.2.2-26} , grain size unknown
Kaowool [®]	Needled insulation mat made from ceramic fibers; Kaowool is a trade name for a family of ceramic fiber products made by Thermal Ceramics, Inc.	3 to 12 ^{3.2.2-18}	160 to 161 ^{3.2.2-27}	2.7 to 3.0 fiber diameter	10.8 to 12.0 E-05
Cerawool [®]	Needled insulation mat made from ceramic fibers; Cerawool is a trade name for a family of ceramic fiber products made by Thermal Ceramics, Inc.	3 to 12 ^{3.2.2-18}	156 to 158 ^{3.2.2-27}	3.2 to 3.5 ^{3.2.2-27} fiber diameter	12.8 to 14.0E-05
Mineral Wool	Generic name for families of products made by Rock Wool Mfg., Roxul, Fibrex, IIG, and others	4, 6, 8, 10 ^{3.2.2-20} pcf are standard	90 ^{3.2.2-20}	5 to 7 ^{3.2.2-20} fiber diameter	20 to 28 E-05
K [®]	Trade name of microporous insulation products made by Thermal Ceramics, Inc. from fumed silica, glass fibers, and quartz fibers	8 to 16 pcf ^{3.2.2-28}	NA	< 0.1 ^{3.2.2-29}	< 4E-06
Calcium Silicate	Manufactured by IIG in three locations (2 use diatomaceous earth, 1 uses expanded perlite)	14.5 ^{3.2.2-21}	144 ^{3.2.2-22}	5 µm mean particle size (2 to 100 µm range) ^{3.2.2-22}	20E-05
Microtherm	Microporous Insulation	5 to 12 pcf ^{3.2.2-23}	NA	< 0.2	< 4.0E-06
Asbestos	Structural fiber used in Cal-Sil type ins.	7 to 10	153	1 to 8	4 to 32E-05

¹⁰ For materials not listed the manufacturer should be contacted to obtain the type of information listed in Table 3.3.6-1.

¹¹ The sizes listed are to be used in the NUREG/CR-6224 head loss correlation as an initial value absent applicable experimental data.

1
2
3
4**Table 3.2.2-3: Coating Debris Characteristics**

Generic Coating Material	Material Density (lbs/ft³)	Characteristic Size (μm)	Characteristic Size (Ft)
Inorganic Zinc (IOZ)	457	10 ⁽¹⁾	3.28E-05 ⁽¹⁾
Epoxy and Epoxy Phenolic Coating Chip (outside ZOI)	94	25 ⁽²⁾	8.20E-05 ⁽²⁾
Epoxy and Epoxy Phenolic Coating Particles (in ZOI)	94	10 ⁽¹⁾	3.28E-05 ⁽¹⁾
Alkyd Coating	98	10 ⁽¹⁾	3.28E-05 ⁽¹⁾
Aluminum	90	10 ⁽²⁾	3.28E-05 ⁽²⁾

5
6
7
8
9
10

Note 1: Spherical Particle Diameter

Note 2: Flat Plate Thickness

3.2.2.4 References

- 1
2
- 3 3.2.2-1 ANSI/ANS 58.2-1988, “American National Standard Design Basis for Protection of
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5 1988
- 6 3.2.2-2 NEDO-32686, “Utility Resolution Guidance for ECCS Suction Strainer Blockage,”
7 Revision 0, Boiling Water Reactor Owner’s Group, November 1996
- 8 3.2.2-3 Calculation Note CN-CRA-04-12, “Jet Expansion Calculation Using ANSI/ANS 58.2-
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- 11 3.2.2-4 NEI-02-01, “Condition Assessment Guidelines: Debris Sources Inside PWR
12 Containments,” Revision 1, September 2002
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14 Size for DBA-Qualified and Acceptable Coatings in Pressurized Water Reactor (PWR)
15 Containments,” J. R. Cavallo, April 2, 2004, included as Attachment A.
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17 Reactor Emergency Core Cooling Sump Performance,” US Nuclear Regulatory
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22 Report; Vol 2: Drywell Debrsi Transport Study: Experimental Work-Final Report,
23 February 1998
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4 in Inorganic Zinc Coatings, October 23, 2002.
- 5 3.2.2-13 Thermal Insulation Handbook, by William C. Turner and John F. Malloy, Robert E.
6 Krieger Publishing Company, Inc., 1981
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9 2003
- 10 3.2.2-16 Product literature provided by JPS Glass Fabrics on Insulbatte[®] and Tempmat[@]
11 insulation products (www.jpsglass.com)
- 12 3.2.2-17 Phone conversation with Joan Kirby, JPS Glass Fabrics on July 7, 2003
- 13 3.2.2-18 Product literature provided by Thermal Ceramics, Inc. on Kaowool Blanket Products
14 (www.thermalceramics.com)
- 15 3.2.2-19 E-mail from Chris Crall, Owens Corning Corporation, July 7, 2003
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- 17 3.2.2-21 Telephone conversation with Jeremy Haslam of IIG, Inc., July 7, 2003
- 18 3.2.2-22 Certificate of Analysis from Tech-Flo, supplier of Perlite Filteraids, provided by IIG,
19 Inc.
- 20 3.2.2-23 Product literature provided by Microtherm, Ltd. (<http://www.microtherm.uk.com>)
- 21 3.2.2-24 Military Specification MIL-I-16411F, Section 3.3
- 22 3.2.2-25 Telephone conversation with Mike Sollie, Transco Products, Inc., July 9, 2003
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- 24 3.2.2-27 E-mail from Garrick Ackart, Thermal Ceramics, Inc., July 10, 2003
- 25 3.2.2-28 Telephone conversation with Frank Duchon, Thermal Ceramics, Inc., July 10, 2003

1 3.2.2-29 Microporous Theory, Technical Notes on MinK, Document Number TN01301,
2 provided by Thermal Ceramics, Inc

3

1 **3.2.3 LATENT DEBRIS**

3 **3.2.3.1 Definition**

5 Latent debris is defined as dirt, dust, paint chips, fibers, pieces of paper (shredded or intact),
6 plastic, tape, or adhesive labels, and fines or shards of thermal insulation, fireproof barrier, or
7 other materials that are present in the containment prior to a postulated break in a high-energy
8 line inside containment. Dust and dirt includes miscellaneous particulates that are present in the
9 containment. Potential origins for this material include activities performed during outages and
10 foreign particulates brought into containment during outages.

12 **3.2.3.2 Discussion**

14 The potential for latent debris in containment during plant operation which may impact head loss
15 across the Emergency Core Cooling sump screens should be considered. Therefore, it is
16 necessary to determine the types, quantities, and locations of latent debris sources.

18 Due to the variations in containment design and size from unit to unit, many miscellaneous
19 sources should be evaluated on a plant-specific basis. It is not appropriate for the licensees to
20 say that their Foreign Materials Exclusion (FME) programs can entirely eliminate sources of
21 miscellaneous debris unless plant-specific walkdowns verify this. Plant-specific walkdown
22 results can be used to determine a conservative amount of dust and dirt to be included in the
23 debris source term. The walkdown will not be able to directly measure this type of debris.
24 However, it is possible to quantify the amount of debris with additional steps.

26 It is recommended that the following activities are performed to quantify the amount of latent
27 debris inside containment:

- 28 • Calculate the horizontal and vertical surface areas inside containment. This calculation
29 will determine the total area with the potential for accumulation of debris.

- 1 • Evaluate the resident debris buildup. It is necessary to determine the amount of debris
2 present on surfaces inside containment.
- 3 • Define the debris characteristics. This information will be used in subsequent steps of the
4 sump performance evaluation.
- 5 • Calculate the total quantity and composition of debris. This information will also be used
6 in subsequent steps of the sump performance evaluation, such as evaluation of the
7 transport of latent debris to the sump screen and the resulting head loss.

8 Detailed guidance for accomplishing the recommended activities for quantification of the
9 amount of latent debris is provided below.

11 **3.2.3.3 Baseline Approach**

13 Latent debris is a contributor to head loss across the sump screen and should be evaluated
14 accordingly. Information is provided in the guidance below to evaluate the quantity of latent
15 debris with sufficient rigor to eliminate excessive conservatism. Note, however, that in many
16 cases the contribution to head loss by latent debris will be small in comparison to that caused by
17 debris from other sources such as insulation materials. In these cases, latent debris will not
18 determine the course of action for mitigating ECCS sump strainer issues.

20 The impact on the results of the sump performance evaluation as a whole should be considered
21 before performing an extremely rigorous analysis of latent debris loading. While it is possible to
22 evaluate the effects of latent debris to a high degree of detail, use of conservative strategies is
23 recommended. Furthermore, the use of conservative strategies in the evaluation of latent debris
24 effects can provide for more head loss analysis margin and can improve operational flexibility if
25 sump modifications are made.

27 **3.2.3.3.1 Calculate horizontal and vertical surface area inside containment.**

29 Horizontal and vertical surfaces are considered in this calculation. Vertical surfaces such as walls
30 and sides of equipment are considered although a significant amount of debris does not typically

1 collect on vertical surfaces in the absence of things that promote adhesion of solids to the
2 surface.

3

4 The following surfaces are included in the calculations:

- 5 • Floor area
- 6 • Walls
- 7 • Cable trays
- 8 • Equipment (such as valve operators, air handlers, etc.)
- 9 • Other surfaces, as appropriate (junction boxes, etc.).

10

11 Use the following guidance in the calculations:

12 1. Flat surfaces are considered to be floors, cable trays, AOV diaphragms, and other flat or
13 nearly-flat surfaces. The bases for this are:

- 14 • Unless the surface is highly convoluted (e.g., a heat exchanger or similar device),
15 assuming a flat surface will not have a significant effect on the surface area
16 calculation. Furthermore, the area projected onto the horizontal plane by the surface
17 would be the key determining factor for the settling and accumulation of debris. For
18 example, while a series of heat exchanger fins may have a large surface area, a
19 significant percentage of that area could be vertical which would preclude
20 accumulation of debris on much of the surface area.

- 21 • The surface area calculations are greatly simplified if the intricacies of surfaces are
22 not explicitly accounted for.

23 2. Half of the surface area of round surfaces such as conduits and ladder rungs is used. The
24 basis for this assumption is that the lower half of the surface area is either inverted or
25 tangent to the vertical plane, so accumulation of debris in this area does not occur. In
26 reality it is likely that the percentage of surface area susceptible to debris accumulation is
27 less than half, because it is unlikely that debris would remain on the regions of the
28 surface that are nearly vertical.

- 1 3. Ten percent of the vertical surfaces inside containment is used. The basis for this
2 assumption is that accumulation of debris on vertical surfaces will typically not occur, but
3 is considered for conservatism. Although walls are considered, the containment dome
4 itself is not considered. Debris accumulation on this surface is precluded because it is
5 inverted or tangent to the vertical plane.
- 6 4. Perform thorough calculations to determine the surface area to be considered for each
7 area of containment. The information needed to perform the calculations can be obtained
8 through plant drawings (plans) and photographic evidence obtained during containment
9 walkdowns.
- 10 5. If exact dimensions are unavailable, use estimated dimensions. Acceptable sources of
11 estimated dimensions are plant drawings (plans) that do not include explicit dimensions
12 for the component in question (i.e., a representation of the component is shown but not
13 detailed) and photographic evidence. Conservatively large values shall be used when
14 dimensions are estimated and bases for the values used shall be provided.

15

16 **3.2.3.3.2 Evaluate resident debris buildup.**

17

18 Although recent sampling of surfaces inside containment at a number of plants indicated that it is
19 likely that the maximum mass of latent debris inside containment is less than 200 pounds, it is
20 recommended that a survey of containment is performed, with the objective of determining the
21 quantity of latent debris.

22

23 Surveying the containment for latent debris will ensure that higher-than-average debris loads are
24 accounted for and will allow plants to take advantage of smaller latent debris loading if lower
25 quantities are present.

26

27 Note that it will be necessary to perform periodic surveys (as part of outage efforts) to validate
28 that there has been no significant change in the latent debris load inside containment. This
29 evaluation of the presence of foreign material is described in NEI-02-01 (Reference 3.2.3-3). The
30 necessary rigor of these surveys is dependent on the effectiveness of the licensee's FME and
31 housekeeping programs with respect to containment cleanliness. If the licensee has rigorous

1 programs in place to control the cleanliness of containment and documents the condition of
2 containment following an outage, it is adequate to perform inspections and limited sampling of
3 surfaces. If the cleanliness of containment is not controlled through rigorous programs, or if the
4 programs in place do not address all areas of containment, it is necessary to perform more
5 comprehensive surveys.

6 7 **3.2.3.3.2.1 Evaluate the resident debris buildup on surfaces.**

8
9 To quantify the amount of latent debris on horizontal surfaces in containment, determine the
10 thickness of the debris layer on a surface and the surface area the layer covers. This information
11 can be used with the macroscopic debris density (with respect to volume) to determine the mass
12 of debris present.

13
14 Use the following steps to evaluate the resident debris buildup on horizontal surfaces:

- 15 1. Divide containment into areas based on the presence of robust barriers. This will allow
16 differing (from section to section) latent debris concentrations and compositions to be
17 adequately represented and will facilitate subsequent debris transport calculations.

18 Examples of appropriate areas include:

- 19 • Accumulator rooms
- 20 • In-core instrumentation room
- 21 • Loop subcompartments
- 22 • Steam generator or pressurizer subcompartments

- 23 2. Determine representative surfaces for each section of containment. For each section this
24 involves defining survey areas of known dimensions. The number of sampling areas
25 examined per section of containment must be determined on a plant-specific basis. Use
26 the following guidance to select representative surfaces:

- 27 • If the worst surface in a given section can be readily identified, it is acceptable to use
28 that surface to represent the entire section. For example, if little or no debris is

1 present on the surfaces in a section except for one, that one surface can be used to
2 represent the debris accumulation in the entire section.

- 3 • If multiple surfaces have debris accumulation with different compositions and
4 thicknesses, it is necessary to sample each of the surfaces to adequately represent the
5 latent debris load for that section.
- 6 • If the area has a uniform and homogeneous latent debris load, a convenient surface
7 can be chosen as the representative surface.

8 3. Survey the representative surfaces in each section to measure the debris quantity. Take
9 care to ensure all health physics procedures are followed for any samples collected. Two
10 strategies are recommended.

- 11 • Collect the debris using equipment that will allow measurement of the quantity of
12 debris at a later time. The volume of debris collected is then divided by the surface
13 area to determine the thickness of the debris layer.

14 The collection method should allow estimation of the debris layer thickness and not
15 change the macroscopic density of the debris that is collected. An acceptable
16 method for collection is the use of swipes to remove the debris from the area in
17 question. Since there is the potential to damage samples during the collection
18 process, take care to not destroy or otherwise change the physical properties of the
19 debris.

- 20 • Measure or estimate the thickness of the debris layer directly. Since it is unlikely
21 that a measurement device (such as calipers) can determine the layer thickness
22 directly, it is recommended that the layer thickness be determined by comparison to
23 an object of known or measurable thickness. Since the debris layers are expected to
24 be quite thin (mils or fractions of mils), comparison to objects like sheets of paper or
25 very thin sheets of metal is recommended.

26 While it is possible to determine the thickness of the debris layer to an acceptable
27 degree of accuracy, it may be difficult to accomplish, even if the debris layer is of
28 uniform thickness and homogeneous composition. Therefore, care should be taken
29 in the measuring process to achieve the most accurate results possible.

- 1 4. Calculate the thickness of the debris layer, based on the quantity of debris collected and
2 the surface area of the sampling area.

3
4 **3.2.3.3.2.2 Evaluate the quantity of other miscellaneous debris.**

5
6 In addition to determining the amount of latent debris accumulation on surfaces, other
7 miscellaneous debris sources are to be accounted for in the debris source term. The survey of
8 containment for these materials is to be performed consistent with the guidance in NEI 02-01
9 (Reference 3.2.3-3). Use the following guidance for each source to be considered:

- 10 • Equipment tags: Determine the number and location of equipment tags of each material
11 type (paper, plastic, metal) within containment. Evaluate the transport of tags to the
12 sump screen when performing the Debris Transport analysis (Section 3.2.4). Although
13 paper tags may dissolve in the post-accident containment environment, it is conservative
14 to assume that they remain intact and available for transport to the sump screen. This
15 assumption shall be used unless there is information that indicates the tags will not
16 remain intact.
- 17 • Tape: Determine the amount and location of each type of tape within containment.
18 Evaluate the transport of tape to the sump screen when performing the Debris Transport
19 analysis (Section 3.2.4). Although FME and housekeeping programs will remove most
20 of the tape used during outage and construction activities, there may still be quantities
21 present in containment. These pieces of tape could be in inaccessible areas or attached to
22 components in plain view. Pieces of tape that have partially disintegrated from being in
23 containment during plant operation should be considered in the latent debris source term.
24 Additionally, tape affixed to surfaces such as ladder rungs in order to improve grip shall
25 be assumed to fail and become transportable debris.
- 26 • Stickers or placards affixed by adhesives: Include items such as stickers and signs that are
27 not mechanically attached to a structure or component in the latent debris source term.
28 Evaluate the transport of these materials to the sump screen when performing the Debris
29 Transport analysis (Section 3.2.4). It is likely that adhesives would fail in post-accident

1 conditions. Assume that all stickers and placards affixed by adhesives fail and become
2 transportable debris.

4 **3.2.3.3.3 Define debris characteristics.**

6 Debris characteristics can be defined using two methods:

- 7 • Analyze debris samples to determine composition and physical properties.
- 8 • Assume composition and physical properties of the debris, using conservative values.

9 Because of the additional rigor and complexity as well as the additional time required to perform
10 detailed analysis of the samples, it is recommended that conservative characteristics (with
11 respect to head loss, as documented in Section 3.2.5) are assumed for the latent debris. The
12 following debris characteristics should be used:

- 13 • Use a fiber/particulate mix that will cause the thin-bed effect to occur.

14 Use the guidance for calculation of head loss to evaluate the required fiber quantity for
15 the thin bed effect to occur. Assign the fiber/particulate mix such that just enough fiber
16 is available to cause thin-bed formation, and classify the remaining debris as particulates.
17 This approach is conservative with respect to head loss and provides potential for the thin
18 bed effect to occur, even if no fibrous insulation is present in containment.

- 19 • Fiber Density = 62.4 lbm/ft^3

20 The basis for this value is that it effectively makes the fiber neutrally buoyant, which
21 results in maximum transport to the sump screen.

- 22 • Particle Density = 100 lbm/ft^3

23 The basis for this value is that most particulate material can be categorized as “dirt”. A
24 representative material would likely be soil or sand, brought into containment during
25 outage activities or construction. According to Reference 3.2.3-1, the densities of “Earth”
26 and “Sand” are both 95 lbm/ft^3 . Therefore, 100 lbm/ft^3 is recommended.

- 27 • Particle Diameter = $10 \mu\text{m}$

1 Based on typical diameter of dust particles [Reference 3.2.3-2], a diameter of 10 μm is
2 suggested. This diameter is conservatively small with respect to transport to the sump
3 screen, since the diameter of “dirt” particles such as earth or sand is larger than that of dust.
4 Furthermore, the diameter of 10 μm is consistent with the size of particles of failed coatings
5 (Reference 3.2.3-4).

6
7 Note that ongoing research efforts by NRC and Los Alamos National Labs may provide
8 additional information regarding the physical characteristics of latent debris.

9
10 If it is decided to analyze the debris samples to determine the composition and physical
11 properties, the work should be performed by a laboratory experienced in material
12 identification, analysis of the macroscopic and microscopic properties of material samples,
13 and handling of radioactive materials. Note that there are challenges to effectively
14 determining the debris characteristics by analysis:

- 15 • It is likely that thorough analysis of samples would be extremely expensive, possibly with
16 little benefit.
- 17 • It is potentially impractical or impossible to separate the debris from the media or device
18 used to capture it.
- 19 • It is possible that the macroscopic density of the debris as well as other characteristics
20 will be changed during the sampling process or transportation to the analysis facility.
21 These changes in characteristics would result because it is likely that the debris is not a
22 homogenous solid; therefore it is possible for the debris to be compacted, damaged, or
23 otherwise manipulated.

24 25 **3.2.3.3.4 Determine fraction of surface area susceptible to debris accumulation.**

26
27 Not all areas are susceptible to accumulation of debris. For example, housekeeping activities at
28 some plants may involve cleaning floors with special wipes, vacuum cleaners, or other methods.
29 In these cases, the areas that are within the scope of the cleaning program could have essentially
30 no debris accumulation, whereas inaccessible areas of the same surface could have an

1 accumulation of debris. A single debris layer thickness would not accurately represent the entire
2 surface.

3

4 It is appropriate to conservatively assume that the entire surface area is susceptible to debris
5 accumulation. If it is unreasonable to use this assumption, in addition to determining the total
6 horizontal surface area inside containment (per Section 3.2.3.3.1) it is necessary to determine the
7 fraction of the surface area of each component and surface that is susceptible to debris
8 accumulation. To accomplish this, evaluate the fraction of the surface area susceptible to debris
9 accumulation a component-by-component or surface-by-surface basis using the results from
10 Sections 3.2.3.3.1 and 3.2.3.3.2 as input. Use the following guidance:

- 11 1. Assume 100% of the surface area is susceptible to debris accumulation for inaccessible
12 areas as well as accessible areas that are not thoroughly cleaned and documented as clean
13 per plant procedures prior to restart (e.g., cable trays, junction boxes, and valve
14 operators), and floors with gratings sitting on flat surfaces.
- 15 2. Evaluate the fractional area susceptible to debris accumulation for smooth floor areas and
16 other surfaces cleaned per plant procedures prior to restart on a case-by-case basis.
17 Considerations include the method of cleaning (e.g. pressure washing vs. vacuuming) and
18 accessibility of areas. Because of wide variations in containment design and
19 effectiveness of housekeeping and FME programs, evaluations must be performed on a
20 plant-specific basis.

21 For all cases in which the area susceptible to debris accumulation is reduced, a
22 conservatively large fractional area susceptible to accumulation must be determined, and
23 bases must be provided for the fractions used. Use the following guidance:

- 24 • Calculate the total surface area of the surface being considered.
- 25 • Calculate the area of the surface that is clean. Use simplifying assumptions that will
26 result in a conservatively small clean area.
- 27 • Calculate the ratio of potentially dirty area to the total area.

28

29 **3.2.3.3.5 Calculate total quantity and composition of debris.**

1
2 The final step in determining the quantity of latent debris located inside containment is to
3 compute the total quantity of latent debris using the results from Sections 3.2.3.3.1, 3.2.3.3.2, and
4 3.2.3.3.3 as input.

5
6 Use the following guidance when performing the final calculations:

- 7 1. The calculations should be performed on an area-by-area basis (consistent with Sections
8 3.2.3.3.1, 3.2.3.3.2, and 3.2.3.3.3). Performing the calculations in this way will facilitate
9 adequate representation of the debris densities and characteristics in the different areas
10 inside containment.
- 11 2. Compute the total quantity of debris for each area by multiplying the total surface area
12 susceptible to debris accumulation by the debris layer thickness for the area of
13 containment being considered.
- 14 3. Include quantities of other types of latent debris such as tape, equipment tags, and
15 stickers.
- 16 4. Categorize and catalog the results for input to the debris transport analysis.

17

18 **3.2.3.4 Sample Calculation**

19

20 The sample calculation considers the bottom level of containment. Equipment tags, tape, and
21 stickers have been excluded from this example since minimal calculations are required for these
22 items and guidance is included in Reference 3.2.3-3. The following surfaces are included in the
23 calculation:

- 24 • Floor areas
- 25 • Cable trays
- 26 • Sump drain pumps

27 For an actual calculation, more detail and rigor are required to document all the surface area on a
28 given level of containment. Since this is a sample calculation, only representative examples were
29 used.

1

2 Section 3.2.3.4.1 documents the calculation of the horizontal areas for complex rooms and cable
3 trays. Section 3.2.3.4.2 documents the calculation of the amount of debris present in the area
4 being considered.

5

6 **3.2.3.4.1 Calculate horizontal surface area**

7

8 The examples below show the calculation of a number of complex floor areas. Rooms of simpler
9 geometry are calculated with less effort and therefore examples of those calculations have not
10 been shown.

11

12 1. Calculate area between containment shell and Steam Generator (SG) compartments

13 The floor area between the containment shell and SG compartments roughly looks like
14 the region between the octagon and circle in the figure below:

15

16

17

18

19

20

21

22

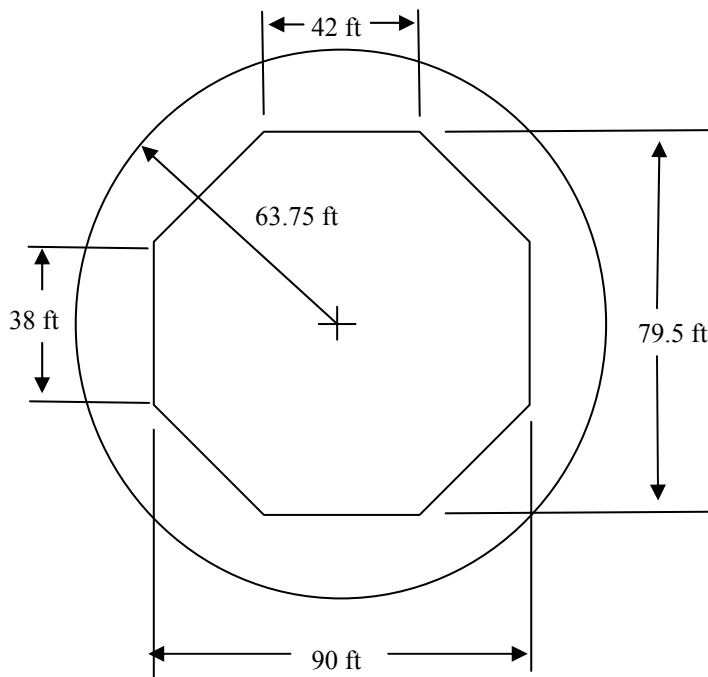
23

24

25

26

27



28

Therefore the area of the octagon is calculated as:

29

$$A = (90 \text{ ft}) (75 \text{ ft}) - (4) (0.5) [(0.5) (79.5-38)] [(0.5) (90-42)]$$

30

$$A = 5754 \text{ ft}^2$$

31

1 Subtract area of octagonal region from round region:

$$2 \quad A = \pi (63.75 \text{ ft})^2 - 5754 \text{ ft}^2$$

$$3 \quad A = 7014 \text{ ft}^2$$

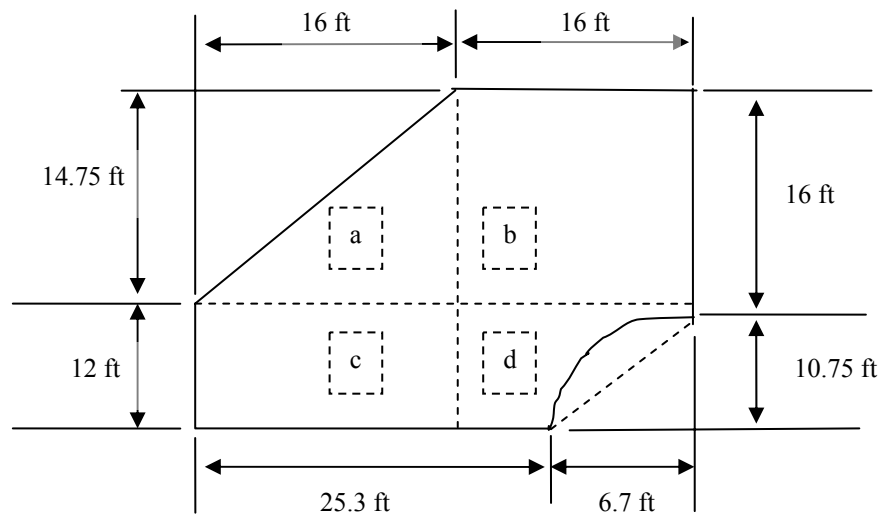
4
5 Subtract area of the Reactor Coolant Drain Tank (RCDT) room and Excess Letdown
6 Heat Exchanger room (these areas protrude from the rough octagonal shape):

$$7 \quad A = 7014 \text{ ft}^2 - 56 \text{ ft}^2 - 94.6 \text{ ft}^2$$

$$8 \quad A = 6914 \text{ ft}^2$$

9
10 2. Calculate area inside SG compartments

11 Each SG compartment has a shape and dimensions roughly like the shape with the solid
12 border below. To simplify the calculations, the room was divided into four regions and
13 the round wall was assumed to be straight:



$$24 \quad A = a + b + c + d$$

$$25 \quad a = 0.5(16 \text{ ft})(14.75 \text{ ft}) = 118 \text{ ft}^2$$

$$26 \quad b = (16 \text{ ft})(14.75 \text{ ft}) = 236 \text{ ft}^2$$

$$27 \quad c = (12 \text{ ft})(16 \text{ ft}) = 192 \text{ ft}^2$$

$$28 \quad d = (16 \text{ ft})(12 \text{ ft}) - (0.5)(10.75 \text{ ft})(6.7 \text{ ft}) = 156 \text{ ft}^2$$

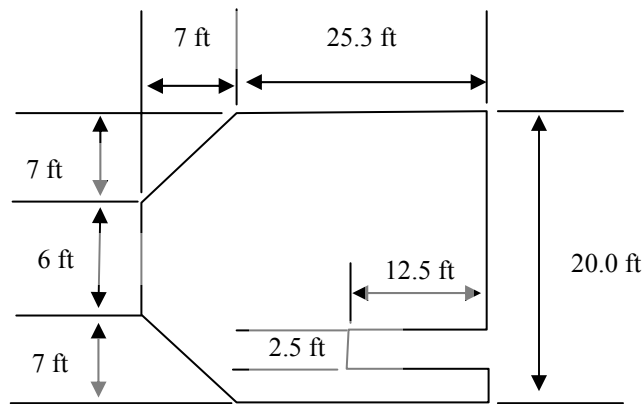
$$29 \quad A = 466 \text{ ft}^2$$

$$A_{\text{total}} = 4(A) \quad (\text{since there are 4 steam generators})$$

$$= 1864 \text{ ft}^2$$

3. Calculate area inside seal table room

The geometry of the seal table room is as shown in the figure below. One simplifying assumption was with regard to the six foot long wall. It is actually curved and protrudes into the room, but was assumed to be straight. This assumption results in prediction of a conservatively large floor area.



$$A = (32.3 \text{ ft}) (20 \text{ ft}) - (2) (0.5) (7.0 \text{ ft}) (7.0 \text{ ft}) - (2.5 \text{ ft}) (12.5 \text{ ft})$$

$$A = 563.8 \text{ ft}^2$$

4. Calculate area of cable trays and other components.

For this sample calculation, 300 linear feet of cable trays was assumed. It was also assumed that the trays were 1 foot wide, resulting in a total surface area of 300 ft². For all cable trays, the length and width should be documented and used to calculate the horizontal surface area.

The other example of component surface area in this sample calculation is the rectangular cover on the sump drain pumps, as shown in the spreadsheet below. It is noteworthy that

1 the covers over the sump were documented as part of the floor area, since there is no
2 floor area considered below them.

3
4 Other components were not examined in detail for this sample calculation. Components
5 that should be examined include, but are not limited to:

- 6 • RCS piping and other piping
- 7 • Pressurizer relief tank
- 8 • Excess letdown heat exchanger (depending on location)
- 9 • Air handling units
- 10 • RCS draindown tank and associated heat exchanger
- 11 • Junction boxes

12 13 **3.2.3.4.2 Calculate quantity of debris**

14
15 This section documents sample calculations of the quantity of debris in the area considered.
16 The calculations are relatively straightforward. To calculate the mass of debris in a given
17 area:

$$18 \quad \text{Volume} = (\text{Debris Layer Thickness}) * (\text{Surface Area})$$

$$19 \quad \text{Mass} = (\text{Volume}) * (\text{Density})$$

20 Example results are presented in Table 3.2.3-1. It is noteworthy that the results are for
21 demonstration only and are based on hypothetical debris survey results.

22 23 **3.2.3.5 References**

24
25 3.2.3-1 "ASHRAE Handbook of Fundamentals," American Society of Heating,
26 Refrigerating, and Air-Conditioning Engineers, Inc., 1972

27 3.2.3-2 Strok & Koral, "Handbook of Air-Conditioning, Heating, and Ventilation,"
28 Second Edition, Industrial Press, 1965

- 1 3.2.3-3 NEI-02-01, "Condition Assessment Guidelines: Debris Sources Inside PWR
- 2 Containments," Revision 1, September 2002
- 3 3.2.3-4 Bostelman, Jan and Zigler, Gilbert, "Failed Coatings Debris Characterization,"
- 4 BWRG Containment Coatings Committee, July 10, 1998

Table 3.2.3-1: Sample Calculation of Debris Quantity

Description	Length ft	Width ft	Surface Area ft ²	Layer Thickness in	Percent Clean %	Debris Volume ft ³	Fiber by Volume %	Fiber			Particulates		
								Volume ft ³	Density lb/ft ³	Mass lb	Volume ft ³	Density lb/ft ³	Mass lb
Floor Areas													
1 Area between SG rooms and cont. shell			6914.0	1.00E-03	25.0	0.43	50.0	0.22	62.40	13.48	0.22	100.00	21.61
2 SG rooms (4 rooms)			1864.0	1.00E-03	25.0	0.12	50.0	0.06	62.40	3.63	0.06	100.00	5.83
3 RCDT room	24.00	8.00	192.0	1.00E-03	0.0	0.02	50.0	0.01	62.40	0.50	0.01	100.00	0.80
4 RCDT HX room	20.00	6.75	135.0	1.00E-03	0.0	0.01	50.0	0.01	62.40	0.35	0.01	100.00	0.56
5 RCDT HX room anteroom	13.30	11.25	149.6	1.00E-03	0.0	0.01	50.0	0.01	62.40	0.39	0.01	100.00	0.62
6 Excess letdown HX rm	22.25	4.25	94.6	1.00E-03	0.0	0.01	50.0	0.00	62.40	0.25	0.00	100.00	0.39
7 Seal table room			563.8	1.00E-03	0.0	0.05	50.0	0.02	62.40	1.47	0.02	100.00	2.35
Equipment													
1 Sump drain pump cover	6.00	4.00	24.0	1.00E-03	0.0	0.00	50.0	0.00	62.40	0.06	0.00	100.00	0.10
2 Cable trays	300.00	1.00	300.0	1.00E-03	0.0	0.03	50.0	0.01	62.40	0.78	0.01	100.00	1.25
Totals						0.67		0.34		20.91	0.34		33.51
Notes: Sump top plate surface area included in Floor Area #1 Calculations for floor areas #1, 2, 7 documented separately Debris layer thicknesses are hypothetical, not based on actual survey data.													

1
2

3
4
5

3.2.4 DEBRIS TRANSPORT

3.2.4.1 Definition

Debris transport is the estimation of the fraction of debris that is transported from debris sources (break location) to the sump screen. The four major debris transport modes considered in the NEI Guidance are:

- Blowdown Transport – the transport of debris by the break jet
- Washdown Spray Transport – the vertical transport by the containment sprays/break flow
- Pool Fill-up Transport – the horizontal transport of the debris by break and containment spray flows to active and inactive areas of basement pool
- Recirculation Transport – the horizontal transport of the debris in the active portions of the basement pool by the recirculation flow through the ECCS system

3.2.4.2 Discussion

For the NEI Guidance the methodology used to determine the amount of debris transported is based on the methodology reported in Section 4.2 of NUREG/CR-6762 Vol. 4 (Reference 3.2.4-1). Figure 3.2.4-1 depicts the generic transport logic tree for use in the NEI Guidance.

Transport fractions for each branch are provided for debris from the ZOI as well as debris outside the ZOI. These transport fractions are provided for three general types of containments:

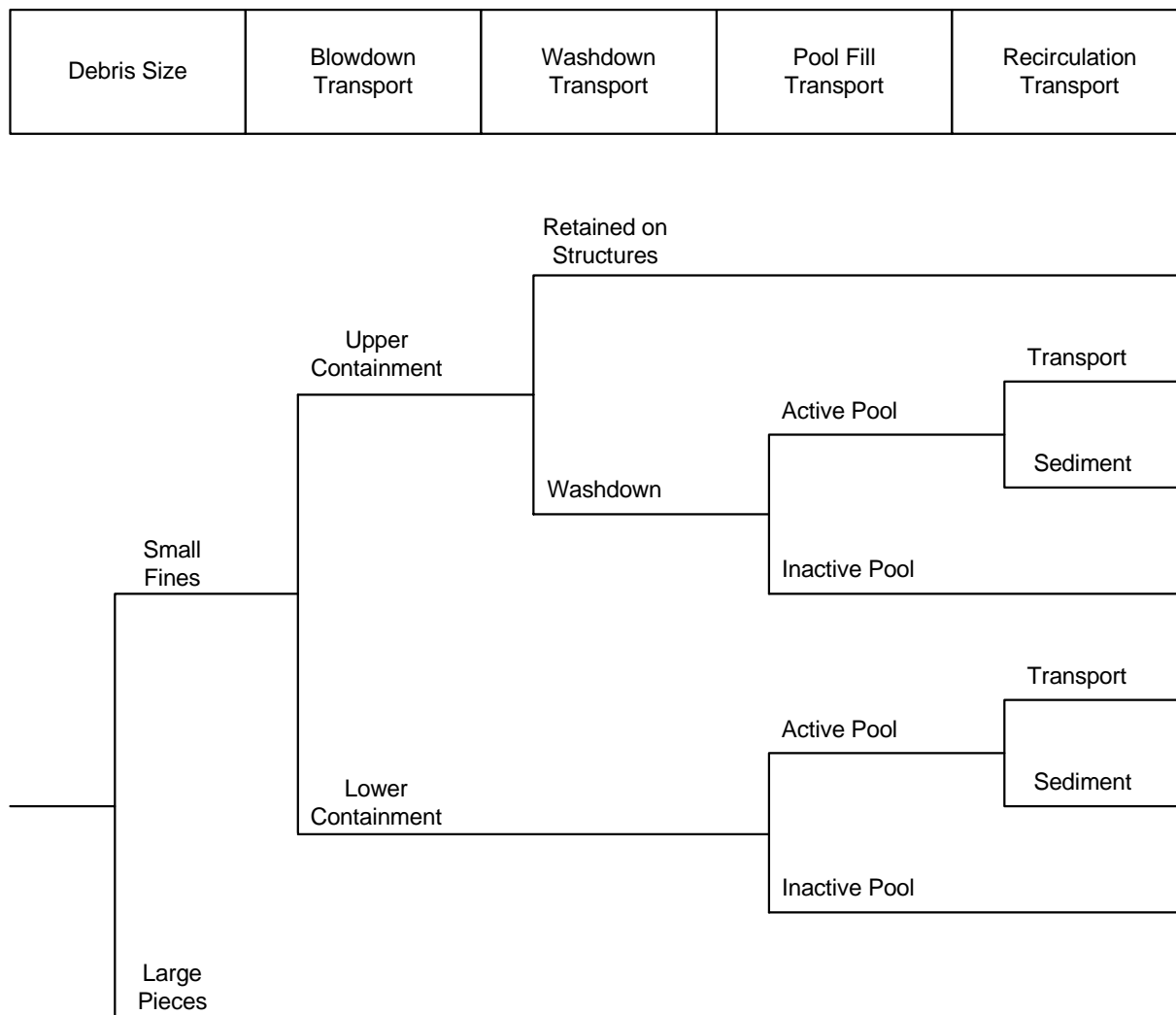
- Highly compartmentalized containments
- Mostly un-compartmentalized containments
- Ice condenser containments

Highly compartmentalized containments are those that have distinct robust structures/compartments totally surrounding the major components of the RCS, e.g. steam generator and pressurizer. Typical examples of these containments are Westinghouse 3 loop plants and earlier CE plants with dry ambient atmosphere containments. Mostly un-

1 compartmentalized containments are those that have partial robust structures surrounding the
 2 steam generators. Typical examples are the B&W dry ambient atmosphere plants. All of the 7 ice
 3 condenser plants are 4 loop Westinghouse plants with no compartmentalization in the lower
 4 containment. For breaks that are not inside a defined compartment the transport fractions of the
 5 mostly un-compartmentalized containments should be used.

6
7
8

Figure 3.2.4-1: Unquantified NEI Guidance Logic Tree



9
10
11
12

3.2.4.3 Debris Transport

Guidance is provided to calculate the debris transport values for each of the three major types of containments, for the major categories of debris: fibrous insulation in the break ZOI, RMI insulation in the break ZOI, other material in the ZOI, and debris outside the ZOI. The type of material found in each classification is provided in the debris characteristic section and the latent debris section.

The debris characteristic terminologies employed herein are those from the debris characteristics section. Small fines are defined as any material that could transport through gratings, trash racks, or radiological protection fences by blowdown, containment sprays, or post-LOCA pool flows. This guideline assumes the largest openings of the gratings, trash racks, or radiological protection fences to be less than 4 inches by 4 inches. The remaining material that cannot pass through gratings, trash racks, and radiological protection fences is classified as large pieces. For fibrous insulation material, the large pieces are assumed to be jacketed/canvassed, hence not subjected to further erosion.

The Baseline Evaluation guidance considers two transport modes for the containment bottom floor: pool fill transport and recirculation transport. During pool formation the break and containment spray water will preferentially fill the “inactive sumps” – those volumes that are below the containment bottom floor elevation. All plants have a calculation determining the water level in containment following a DBA. This calculation provides estimates of the volume of each compartment that are considered to be flooded by the DBA. Using this calculation and a layout of the containment elevation an analyst can determine which of the volumes are below the containment bottom floor. The analyst then needs to review all the lower compartments to ensure that those volumes do not have drains from the upper part of the containment (e.g. refueling pool) that may cause them to participate in the active volumes. This guideline considers that all volumes at the containment bottom floor elevation will participate in the recirculation flow path from the containment sprays and break flow to the sump.

1 All the debris that is on the containment bottom floor during pool formation will tend to be
2 preferentially washed into the inactive sumps by the thin sheets of fast moving water. Only when
3 the inactive sumps are filled will the water level in the containment bottom floor begin to
4 increase and the pool turbulence decrease. During this fill process of the containment bottom
5 floor pool, as depicted in Figure 1-4 and 1-5 of NUREG/CR-6808 (Reference 3.2.4-2), the
6 switchover to recirculation has not occurred hence there is no preferential direction for water to
7 flow to the sump. In the pool fill transport, this NEI guidance considers that all debris in the
8 containment bottom floor is uniformly distributed throughout the entire volume of water in
9 containment. This guidance then considers that the debris transported to the inactive sumps is
10 strictly based on the ratio of the volume of the inactive sumps to the total water volume in
11 containment at the start of recirculation. This assumption is clearly conservative since it ignores
12 the preferential sweeping of the debris on the containment bottom floor to the inactive sumps by
13 the thin sheets of high velocity water. To add to the conservatism, the NEI guidance then
14 considers that all debris classified as “small fines” or “small RMI pieces” are transported to the
15 sump during recirculation. Plants can deviate from the Baseline Evaluation guidelines to account
16 for plant specific features. Such deviations from the Baseline Evaluation guidance are
17 considered refinements to the baseline methodology. Additionally, plants may consider
18 implementing refinements identified in Sections 4 and 5 of this guide.

19 20 **3.2.4.3.1 Highly Compartmentalized Containment**

21
22 This guidance assumes that the pipe break in a highly compartmentalized containment occurs at
23 the bottom of the compartment. For breaks that are not located in the bottom of the compartment
24 or on upper portion of a compartment, e.g. a main steam line break, the mostly un-
25 compartmentalized containment values should be used.

26 27 **Fibrous Insulation in the ZOI:**

28 The following guidance is provided for all types of fibrous debris in the ZOI.

29 **Blowdown Transport:**

30 Debris transport during blowdown is assumed to cause the small fines
31 debris from the compartment where the break is postulated to occur to be

1 distributed to all horizontal surfaces outside the compartments and the
2 dome. Most of the break locations in a compartment are located in the
3 bottom of the compartment. For conservatism it is assumed that only 25%
4 of the small fines debris is ejected upward, the rest going to the
5 containment bottom floor. This fraction is derived as a conservative
6 estimate of the free volume in a compartment above the lower portion of
7 the compartment not occupied by components such as steam generators.
8 The large debris pieces from the ZOI are assumed to fall to the
9 compartment floor and not be transported.

10
11 Washdown Transport:

12 Debris transport by the containment spray is assumed to cause all the
13 small fines to be transported to the containment bottom floor and be
14 evenly distributed on the floor. No transport of the large pieces is assumed
15 to occur by containment spray.

16
17 Pool Fill Transport:

18 Debris transport in the containment bottom floor pool during fill up will
19 transport all the small fines. Some of the small fines will be transported to
20 the inactive volumes of the pool that will not participate in the
21 recirculation flow, i.e. the cavity under the reactor vessel. The transport
22 factor to the inactive pools is calculated by calculating the ratio of the
23 volumes of the inactive pool to the total pool volume. No transport of the
24 large pieces is assumed to occur during pool fill up.

25
26 Recirculation Transport:

27 Debris transport in the containment bottom floor pool during recirculation
28 is assumed to transport 100% of the small fines in the active volumes of
29 the pool to the sump. No transport of the large pieces is assumed to occur
30 during recirculation.

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RMI Insulation in the ZOI:

The following guidance is provided for all types of RMI debris in the ZOI.

Blowdown Transport:

Debris transport during blowdown is assumed to cause the small RMI debris pieces from the compartment where the break is postulated to occur to be distributed to all horizontal surfaces outside the compartment. For conservatism it is assumed that only 25% of the small RMI debris is ejected upward, the rest going to the containment floor. This fraction is derived as a conservative estimate of the free volume in a compartment above the lower portion of the compartment not occupied by components such as steam generators. The large RMI debris pieces from the ZOI are assumed to fall to the compartment floor and not be transported.

Washdown Transport:

Debris transport by the containment spray is assumed to cause none of the small RMI debris that are not on the containment bottom floor and are in containment spray pathway to be transported to the containment bottom floor. The flow velocities and the very shallow pool depths are not conducive to transport of small RMI debris. No transport of the large pieces is assumed to occur by containment spray.

Pool Fill Transport:

Debris transport in the containment bottom floor pool during fill up will transport all the small RMI debris. Some of the small fines will be transported to the inactive volumes of the pool that will not participate in the recirculation flow, i.e. the cavity under the reactor vessel. The transport factor to the inactive pools is calculated by calculating the ratio

1 of the volumes of the inactive pool to the total pool volume. No transport
2 of the large RMI pieces is assumed to occur during pool fill up.

3
4 **Recirculation Transport:**

5 Debris transport in the containment bottom floor pool during recirculation
6 is assumed to transport 100% of the small RMI debris in the active
7 volumes of the pool to the sump. No transport of the large RMI pieces is
8 assumed to occur during recirculation.

9
10
11 **Other Material in the ZOI:**

12
13 All other material in the ZOI, including coatings within the Coatings ZOI will be assumed to
14 transport similar to the small fines of fibrous material.

15
16 **Debris from Materials Outside the ZOI:**

17
18 All debris from materials outside the ZOI is considered to be in the active volumes of the pool at
19 the start of recirculation and 100% transported by the active volumes of the pool to the sump.
20 Latent debris is also considered to be to be in the active volumes of the pool at the start of
21 recirculation and 100% transported by the active volumes of the pool to the sump. This is
22 conservative since debris from outside the ZOI is not considered to be transported to the inactive
23 sump.

24
25 **3.2.4.3.2 Mostly Un-compartmentalized Containment**

26
27 The following guidance is provided for all types of fibrous debris in the ZOI.

28
29 **Fibrous Insulation in the ZOI:**

30 **Blowdown Transport:**

1 Debris transport during blowdown is assumed to cause the small fines
2 debris from the compartment where the break is postulated to occur to be
3 distributed to evenly to all horizontal surfaces outside the compartments
4 and the dome. The large debris pieces from the ZOI are assumed to fall to
5 the containment bottom floor and not be transported.

6
7 **Washdown Transport:**

8 Debris transport by the containment spray is assumed to cause all the
9 small fines to be transported to the containment bottom floor and be
10 evenly distributed on the floor. No transport of the large pieces is assumed
11 to occur by containment spray.

12
13 **Pool Fill Transport:**

14 Debris transport in the containment bottom floor pool during fill up will
15 transport all the small fines. Some of the small fines will be transported to
16 the inactive volumes of the pool that will not participate in the
17 recirculation flow, i.e. the cavity under the reactor vessel. The transport
18 factor to the inactive pools is calculated by calculating the ratio of the
19 volumes of the inactive pool to the total pool volume. No transport of the
20 large pieces is assumed to occur during pool fill up.

21
22 **Recirculation Transport:**

23 Debris transport in the containment bottom floor pool during recirculation
24 is assumed to transport 100% of the small fines in the active volumes of
25 the pool to the sump. No transport of the large pieces is assumed to occur
26 during recirculation.

27
28 **RMI Insulation in the ZOI:**

29 The following guidance is provided for all types of RMI debris in the ZOI.

30

1 Blowdown Transport:

2 Debris transport during blowdown is assumed to cause the small RMI
3 debris pieces from the compartment where the break is postulated to occur
4 to be distributed to all horizontal surfaces outside the compartments. For
5 conservatism it is assumed that all the small RMI debris is deposited on
6 the containment bottom floor. The large RMI debris pieces from the ZOI
7 are assumed to fall to the containment bottom floor and not be transported.

8
9 Washdown Transport:

10 There is no debris transport by the containment spray of the small RMI
11 pieces since all small RMI debris is assumed to be transported by the
12 blowdown to the containment bottom floor. Also, no transport of the large
13 pieces is assumed to occur by containment spray.

14
15 Pool Fill Transport:

16 Debris transport in the containment bottom floor pool during fill up will
17 transport all the small RMI debris. Some of the small fines will be
18 transported to the inactive volumes of the pool that will not participate in
19 the recirculation flow, e.g. the cavity under the reactor vessel. The
20 transport factor to the inactive pools is calculated by calculating the ratio
21 of the volumes of the inactive pool to the total pool volume. No transport
22 of the large RMI pieces is assumed to occur during pool fill up.

23
24 Recirculation Transport:

25 Debris transport in the containment bottom floor pool during recirculation
26 is assumed to transport 100% of the small RMI debris in the active
27 volumes of the pool to the sump. No transport of the large RMI pieces is
28 assumed to occur during recirculation.

29
30

Other Material in the ZOI:

All other material in the ZOI, including coatings within the Coatings ZOI will be assumed to transport similar to the small fines of fibrous material.

Debris from Materials Outside the ZOI:

100% of debris from materials outside the ZOI is considered to be in the active volumes of the pool at the start of recirculation and 100% transported by the active volumes of the pool to the sump. Latent debris is also considered to be to be in the active volumes of the pool at the start of recirculation and 100% transported by the active volumes of the pool to the sump. This is conservative since debris from outside the ZOI is not considered to be transported to the inactive sump.

3.2.4.3.3 Ice Condenser Containment**Fibrous Insulation in the ZOI:**

The following guidance is provided for all types of fibrous debris in the ZOI.

Blowdown Transport:

Debris transport during blowdown is assumed to cause most of the small fines debris from the lower containment where the break is postulated to occur to be transported to the upper compartment and the dome through the ice condenser baskets. Ten percent of the small fines debris is retained in the upper compartment and the ice condensers, the rest returning back to the lower containment floor by the melting ice. Steam and water with entrained debris will all go through the ice condenser cavities. Some of the debris will be entrained in the baskets. At the end of blowdown at least 50% of the ice will have melted. Ten percent is a conservative average value of the open area in the ice condenser. The large debris pieces from

1 the ZOI are assumed to fall to the lower containment floor and not be
2 transported.

3

4 Washdown Transport:

5 All the small fines that were transported to the upper containment by the
6 blowdown will be conservatively assumed to be all transported by the
7 containment sprays from the upper containment to the lower containment
8 bottom floor and be evenly distributed on the lower containment bottom
9 floor. No transport of the large pieces is assumed to occur by containment
10 spray.

11

12 Pool Fill Transport:

13 Debris transport in the lower containment bottom floor pool during fill up
14 will transport all the small fines. Some of the small fines will be
15 transported to the inactive volumes of the pool that will not participate in
16 the recirculation flow, i.g. the cavity under the reactor vessel. The
17 transport factor to the inactive pools is calculated by calculating the ratio
18 of the volumes of the inactive pool to the total pool volume. No transport
19 of the large pieces is assumed to occur during pool fill up.

20

21 Recirculation Transport:

22 Debris transport in the containment bottom floor pool during recirculation
23 is assumed to transport 100% of the small fines in the active volumes of
24 the pool to the sump. No transport of the large pieces is assumed to occur
25 during recirculation.

26

27 **RMI Insulation in the ZOI:**

28 The following guidance is provided for all types of RMI debris in the ZOI.

29 Blowdown Transport:

1 Debris transport during blowdown is assumed to cause most of the small
2 RMI debris from the lower containment where the break is postulated to
3 occur to be transported to the upper compartment and the dome through
4 the ice condenser baskets. For conservatism it is assumed that only 10% of
5 the small RMI debris is transported to the upper compartment, the rest
6 returning back to the lower containment bottom floor by the melting ice.
7 Steam and water with entrained debris will all go through the ice
8 condenser cavities. Some of the debris will be entrained in the baskets. At
9 the end of blowdown at least 50% of the ice will have melted. Ten per cent
10 is a conservative average value of the open area in the ice condenser. The
11 large debris pieces from the ZOI are assumed to fall to the lower
12 containment bottom floor and not be transported.

13 14 Washdown Transport:

15 Debris transport by the containment spray is assumed to cause none of the
16 small RMI debris that are on the upper containment bottom floor and are
17 in containment spray pathway to be transported to the containment bottom
18 floor. The flow velocities and the very shallow pool depths in the upper
19 containment floor are not conducive to transport of small RMI debris. No
20 transport of the large pieces is assumed to occur by containment spray.

21 22 Pool Fill Transport:

23 Debris transport in the lower containment bottom floor pool during fill up
24 will transport all the small RMI debris. Some of the small fines will be
25 transported to the inactive volumes of the pool that will not participate in
26 the recirculation flow, e.g. the cavity under the reactor vessel. The
27 transport factor to the inactive pools is calculated by calculating the ratio
28 of the volumes of the inactive pool to the total pool volume. No transport
29 of the large RMI pieces is assumed to occur during pool fill up.

30 31 Recirculation Transport:

1 Debris transport in the containment bottom floor pool during recirculation
2 is assumed to transport 100% of the small RMI debris in the active
3 volumes of the pool to the sump. No transport of the large RMI pieces is
4 assumed to occur during recirculation.

6 **Other Material in the ZOI:**

7
8 All other material in the ZOI, including coatings within the Coatings ZOI will be assumed to
9 transport similar to the small fines of fibrous material.

11 **Debris from Materials Outside the ZOI:**

12
13 All of debris from materials outside the ZOI is considered to be in the active volumes of the pool
14 at the start of recirculation and 100% transported by the active volumes of the pool to the sump.
15 Latent debris is also considered to be in the active volumes of the pool at the start of recirculation
16 and 100% transported by the active volumes of the pool to the sump. This is conservative since
17 debris from outside the ZOI is not considered to be transported to the inactive sump.

19 **3.2.4.4 Calculate Transport Factors**

20
21 The calculation of the transport factors for each type of debris is done by using the unquantified
22 logic tree as a guide. A logic tree should be developed for each of the debris types and using the
23 previously discussed values for the appropriate containment type. The summation of the two
24 “Transport” branches is the cumulative transport fraction for the debris type.

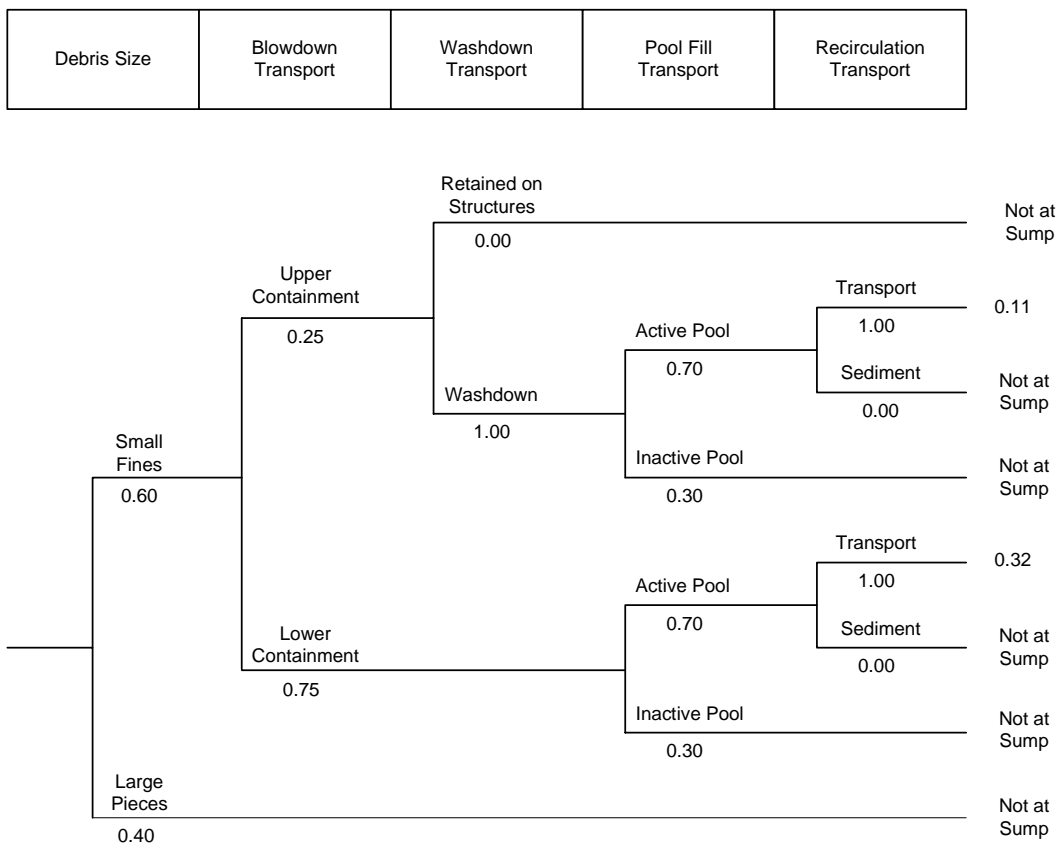
26 **3.2.4.4.1 Sample Calculation**

27
28 The baseline sample plant is classified as a highly compartmentalized containment. From the
29 post-DBA water level calculations we have that the inactive pools account for 30% of the total
30 post-DBA water volume in containment.

1 From the debris classification section there are two types of debris from the ZOI for the baseline
 2 sample plant: NUKON[®] and RMI. Using the recommended transport fractions we have:

3
 4 NUKON[®]

5 The following is a quantified logic tree for NUKON[®]:



8
 9

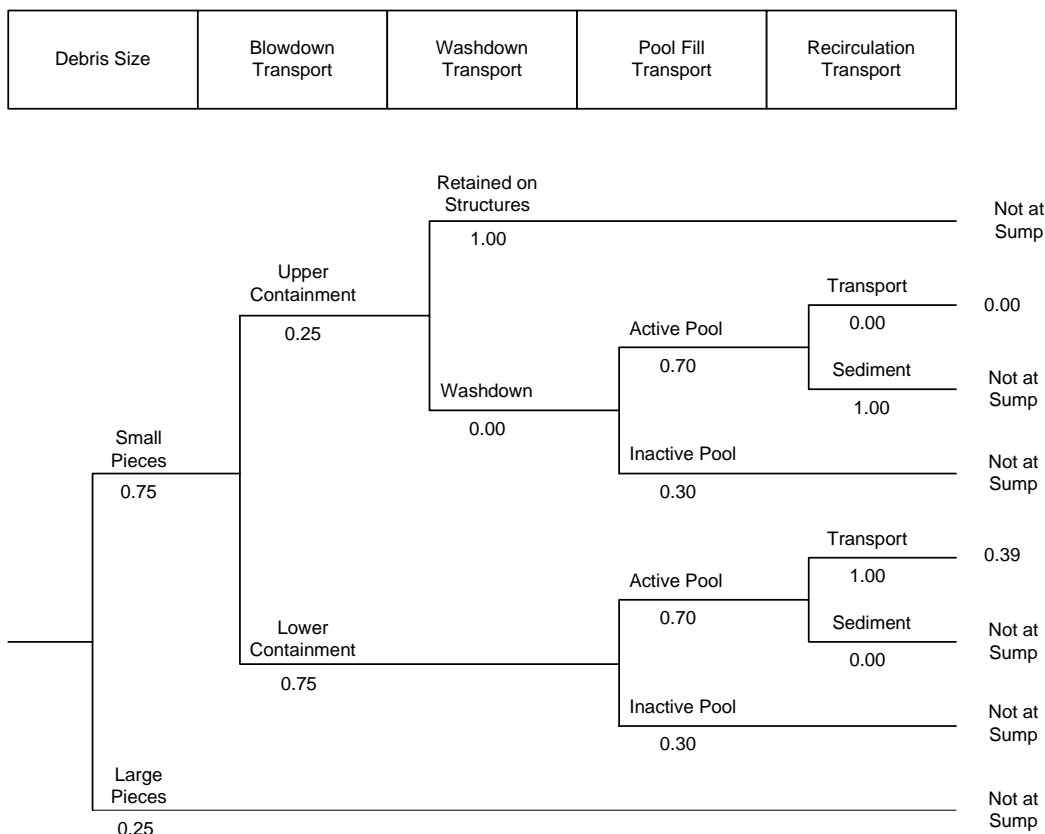
10
 11 **Figure 3.2.4-2: NUKON[®] Transport Logic Tree**
 12 **(Sample Problem)**

13
 14 Adding the two paths that reach the sump, the total cumulative transport factor for NUKON[®]
 15 fines reaching the sump is the $0.11 + 0.32 = 0.43$. As such 43% of the volume of NUKON[®] in the
 16 ZOI reaches the sump in the form of small fines. No large pieces of NUKON[®] will be
 17 transported to the sump.

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RMI:

The following is a quantified logic tree for the RMI:



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Figure 3.2.4-3: RMI Transport Logic Tree (Sample Problem)

From inspection of the logic tree 0.39 is the transport factor for RMI. As such 39 % of the volume of RMI in the ZOI reaches the sump in the form of small pieces. No large RMI pieces will be transported to the sump.

1 Coating debris material from both from within the coatings ZOI and from outside the coatings
 2 ZOI will all be transported to the sump. All debris material outside the ZOI, including latent
 3 debris, will also be transported to the sump.

4
 5

6 From the debris generation sample calculations we have:

7 Total volume of NUKON[®] blankets in ZOI: 300 cu ft

8 Total quantity of RMI material in ZOI: 15,000 sq ft

9

10 From the debris characterization section we have:

11 Total Quantity of small fines coating:

12 0.007 cu ft from the ZOI + 7.5 cu ft from outside the ZOI = 7.5 cu ft

13

14 From the latent debris section we have:

15 Latent fiber: 20.91 lbs @ 62.4 lbs/cu ft = 0.34 cu ft

16 Latent particulates: 33.51 lbs @ 100 lbs/cu ft = 0.34 cu ft

17

18 Using the transport fractions derived above, the following quantities of debris are transported to
 19 the sump:

20 Fibers: small fines: $300 * 0.43 + 0.34 = 129.34$ cu ft

21 RMI small pieces: $15,000 * 0.39 = 5,850$ sq ft

22 Coating small fines (IOZ equivalent): = 7.5 cu ft

23 Latent Particulates: = 0.34 cu ft

24

1 **3.2.4.5 References**

2

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4 Commission (2002)

5 Volume 1: D. V. Rao, B. Letellier, C. Shaffer, S. Ashbaugh, and L. Bartlein, “GSI-
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7 Recirculation Sump Performance,” LA-UR-01-4083, 2002

8 Volume 2: D. V. Rao, B. Letellier, K. W. Ross, L. Bartlein, and M. T. Leonard, “GSI-
9 191 Technical Assessment: Summary and Analysis of U.S. Pressurized Water
10 Reactor Industry Survey Responses and Responses to GL 97-04,” LA-UR-01-1800,
11 2002

12 Volume 3: C. J. Shaffer, D. V. Rao, and S. G. Ashbaugh, “GSI-191 Technical
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14 Parametric Evaluation,” LA-UR-01-6640, 2002

15 Volume 4: S. G. Ashbaugh, and D. V. Rao, “GSI-191 Technical Assessment:
16 Development of Debris Transport Fractions in Support of the Parametric Evaluation,”
17 LA-UR-01-5965, 2002

18 3.2.4-2 NUREG/CR-6808, LA-UR-03-0880, D.V. Rao, Clinton J. Shaffer, M.T. Leonard,
19 “Knowledge Base for the Effect of Debris on Pressurized Water Reactor Emergency
20 Core Cooling Sump Performance,” U.S. Nuclear Regulatory Commission, February
21 2003

1 **3.2.5 HEAD LOSS**

2

3 **3.2.5.1 Introduction/Scope**

4

5 The methodology presented within this chapter details how to calculate the head loss from a
6 debris bed that could be formed on the ECCS sump screen(s). The sump screen parameters and
7 the thermal/hydraulic conditions required for this analysis will first be discussed. The types,
8 total quantities and characteristics of debris that are generated in the containment and transported
9 to the sump screen are also primary design inputs for this methodology.

10

11 The methodology will provide the user with the head loss (feet-of-water) for the debris bed on
12 the sump screen. The user then has to add the estimated clean sump screen head loss to obtain
13 the total head loss across the sump screen. The ability to sustain this head loss is then assessed
14 by comparison to the NPSH Margin. Sample problems are provided to illustrate the
15 methodology.

16

17 **3.2.5.2 Inputs for Head Loss Evaluation**

18

19 **3.2.5.2.1 Sump Screen Design**

20

21 The sump screen design is an important consideration in the evaluation of debris head loss. Plant
22 drawings should provide details as to the screen construction, the orientation and the mesh size
23 (or hole-size and pitch for perforated plates). Typical PWR sump screen configurations are
24 illustrated in Figure 3.2.5-1. Newer designs, such as those installed in the BWRs, typically have
25 more surface area and different geometries.

26

27

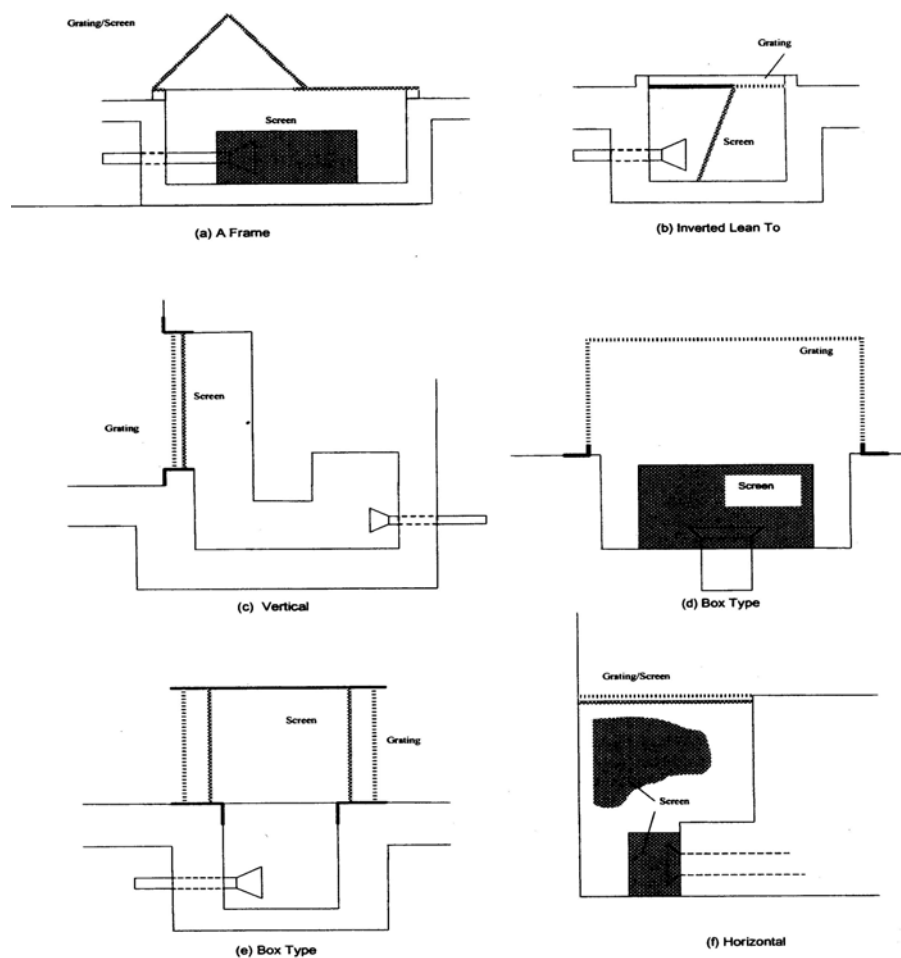
28

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Figure 3.2.5-1: Typical PWR Sump Screen Configurations

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4 Derived from plant drawings, the sump screen area (A) is the total area of the sump screen
 5 (without any correction for the solid area of the mesh or wire screen) over which debris
 6 accumulates. Curbs are ignored when determining the screen area. For flat screens, the sump
 7 screen is simply the total circumscribed area of the screen or perforated plate. Framing and/or
 8 significant structures that block flow through the screen should be subtracted from the total area
 9 to get a net screen area. For alternate geometries, particularly in the case of star or stacked disc
 10 designs, the initial strainer surface area available for debris deposition is the total perforated plate
 11 surface area, decreasing to the circumscribed area as debris fills in the voids and gaps between
 12 the ridges and disks.

13

1 If the screen is completely submerged, the net screen area is used. If the screen is partially
2 submerged, the wetted area should be determined based on the height of the containment floor
3 water pool at the time the head loss is calculated.

4
5 The sump screen opening size (or hole-size and pitch for perforated plate screens) is obtained
6 from plant drawings. The opening size is usually the size needed to keep out debris of a size
7 greater than the minimum size of openings in the ECCS (e.g. spray nozzles, valve throats and
8 pump cooling lines). The sump screen opening size is used in determining the clean strainer
9 head loss. The debris-bed head loss calculation methodology adopted in this chapter is largely
10 independent of the sump screen opening size.

11
12 The Clean Strainer Head Loss (CSHL) is the head loss of the sump screen assembly in a clean,
13 unfouled condition. The CSHL is a required input for the overall head loss evaluation and is
14 highly dependent on plant-specific sump screen construction details and thermal hydraulic
15 conditions. Calculating the head loss of the sump screen assembly in a clean condition involves
16 calculating the head loss across the screen itself taking submergence of the screen into
17 consideration. The CSHL will mainly depend on the screen mesh size (or hole size and pitch for
18 perforated plates), the flow through the screen, and the water temperature using standard
19 methods of fluid mechanics. This baseline methodology does not provide details on how to
20 calculate clean strainer head loss as this information is available from other sources. Clean sump
21 screen head loss information is typically available from the manufacturer of the raw screen
22 material itself. Note that existing plant calculations often document CSHL. In some cases the
23 head losses due to the attendant support structures, mechanical configuration of the bracing and
24 other structures in the sump (such as vortex suppressors) cannot be neglected, and these losses
25 should normally be included in the CSHL calculation.

1 **3.2.5.2.2 Thermal-Hydraulic Conditions**

2

3 **3.2.5.2.2.1 Recirculation Pool Water Level**

4

5 For conservatism, the minimum water level of the recirculation pool should be used to estimate
6 the head loss across the debris bed accumulated on a screen. The minimum level will yield the
7 smallest surface area (thus potentially greater head loss) for those screens that are not completely
8 submerged in the pool as well as the lowest available NPSH to the ECCS pumps.

9

10 **3.2.5.2.2.2 ECCS Flow Rate**

11

12 For conservatism, the highest flow rate (Q) should be used in calculating the head loss across a
13 screen. In this regard, the Baseline Methodology recommends that maximum pump flows, as
14 identified in current NPSH calculations, be used for the ECCS flow rates. For multiple sump
15 screens, the flow rate for the head loss calculation is the flow through each of the screens.

16

17 **3.2.5.2.2.3 Temperature**

18

19 The recirculation sump water temperature should be documented in the plant design basis
20 calculations and is an important parameter in the head loss calculation.

21

22 The Baseline Evaluation Methodology recommends the following:

23

1. The temperature at which the head loss is evaluated should be consistent with the
24 temperature used for the NPSH evaluation.

25

2. However, it is not clear which temperature is limiting overall, therefore, multiple times,
26 temperatures and flows during the accident may need to be evaluated. (For example, use
27 of 250°F gives a head loss of 8.8-feet for the sample problem of this section, whereas
28 using 120°F gives 33.9-feet).

- 1 3. As a conservative simplification, the maximum expected sump temperature may be used
2 for the NPSH analysis, whereas the lowest expected temperature during ECCS operation
3 may be taken for the head loss analysis
4

5 **3.2.5.2.2.4 Debris Types, Quantities and Characteristics**

6

7 Fibrous insulation debris, RMI debris, coatings debris, and miscellaneous debris such as concrete
8 debris, dust, dirt, other latent debris, rust, etc. all have to be considered if they are present inside
9 the containment. Therefore, the types, quantities (mass or volume) and characteristics of all
10 potential debris materials need to be specified in the design input for a sump screen head loss
11 evaluation. For fibrous materials, the insulation volume is the main parameter needed. For
12 particulate materials, the mass and the density are the main parameters required. For RMI, the
13 main parameter needed is the total foil area of the damaged RMI.
14

15 The composition and characteristics of the debris bed on the sump screen are important inputs
16 into the head loss model. The debris types, quantities (i.e. mass or volume), and characteristics
17 (e.g. shape and thickness) reaching the sump screen are needed to calculate the pressure drop
18 across the debris bed. The debris types and potential quantities at the sump screen are
19 determined by the debris generation and transport calculations.
20

21 **3.2.5.2.3 Head Loss Methodology**

22

23 The head loss model assumes that the screen is initially clean and that the floor pool contains a
24 homogenous mixture and concentration of debris (i.e., fibrous, particulate, etc.). Upon
25 switchover of suction from the refueling water storage tank (RWST) to the recirculation sump,
26 debris begins to be transported to the sump and accumulates on the sump screen. Initially, some
27 portion of the debris whose size is smaller than the screen mesh size (or hole-size of the
28 perforated plate) passes through the sump screen. Fibers will quickly start to form a fiber mat in
29 the cases where there is no RMI debris transported to the sump screen. (If RMI is present at the
30 screen, refer to Section 3.2.5.2.3.1.3). As the fiber mat forms it will start trapping particulate
31 debris reaching the sump screen. With sufficient fibers reaching the screen, a uniform fiber mat

1 bed will be formed at which time the head loss across the debris will start increasing. The head
2 loss across the debris bed will continue to rise as more debris is deposited on the screen, reaching
3 steady state when all of the available debris is deposited on the screen.

4
5 Most analysts are interested in the head loss across the sump screen when all debris reaching the
6 sump screen accumulates on the screen. The head loss methodology herein provides the ability
7 to compute the sump screen head loss given the total quantity and type of debris over a specified
8 surface area at a given ECCS pump flow.

10 **3.2.5.2.3.1 General Theoretical/Empirical Formulas**

12 **3.2.5.2.3.1.1 Fibrous Debris Beds with Particulate**

13
14 For general use with fiber and particulate debris beds, the NUREG/CR-6224 correlation is
15 recommended for determination of the head loss. The refinement guidance of Section 4 provides
16 a discussion of factors associated with estimating debris head losses and presents several debris
17 head loss correlations developed over the last few years.

18
19 The NUREG/CR-6224 head loss correlation is described and validated in detail in Appendix B
20 of that report and is a semi-theoretical head loss model. The correlation is based on the
21 theoretical and experimental research for the head loss across a variety of porous and fibrous
22 media carried out since the 1940s. The NUREG/CR-6224 head loss correlation has been
23 thoroughly validated for fibrous debris and ferrous sludge found in BWRs for a variety of flow
24 conditions, water temperatures, and in different experimental facilities. The types of fibrous
25 insulation material tested include NUKON™ and Temp-Mat®. The particulate matter debris
26 tested includes iron oxide particles from 1 to 300 μm in characteristic size, plus inorganic zinc
27 and paint chips. In these cases, with the appropriate selection of particle sizes as described in
28 Tables 3.3.2.3.6-1 and 3.3.2.3.6-2 of this document, the NUREG/CR-6224 head loss correlation
29 bounds the experimental results.

1 US NRC Regulatory Guide 1.82 Revision 3 states that estimates of head loss caused by debris
 2 blockage should be developed from empirical data based on the sump screen design (e.g., surface
 3 area and geometry), postulated combinations of debris (i.e., amount, size distribution, type), and
 4 approach velocity. Therefore, there may be materials and combinations of materials for which
 5 the empirical head loss data does not exist. In these cases, the following options are available:

- 6 • Characterization of the material with Scanning Electron Microscopy (SEM) analysis, and
 7 establishing a size distribution;
- 8 • Choosing an alternative material that conservatively represents the material in question,
 9 via similitude arguments;
- 10 • Head loss testing of the particular material to establish a correlation or else validate an
 11 existing correlation for that material; or
- 12 • Utilize other data which may exist to establish head loss for the material in question.
 13 (The refinement guidance presented in Section 4 summarizes some of the industry test
 14 data. More data are possibly available, some of which are currently the property of
 15 individual utilities.)

16 The NUREG/CR-6224 head loss correlation, applicable for laminar, transient and turbulent flow
 17 regimes through mixed debris beds (i.e., debris beds composed of fibrous and particulate matter)
 18 is given by:

$$20 \quad \Delta H = \Lambda [3.5 S_v^2 \alpha_m^{1.5} (1 + 57 \alpha_m^3) \mu U + 0.66 S_v \alpha_m / (1 - \alpha_m) \rho U^2] \Delta L_m \quad (3.2.5-1)$$

21
 22 where:

23 ΔH is the head loss (feet-of-water)

24 S_v is the surface-to-volume ratio of the debris (ft^2/ft^3)

25 μ is the dynamic viscosity of water (lbm/ft/sec)

26 U is the fluid approach velocity (fps)

27 ρ is the density of water (lbm/ft³)

1 α_m is the mixed debris bed solidity (one minus the porosity)

2 ΔL_m is the actual mixed debris bed thickness (inches)

3 Λ is a conversion factor –

4 $\Lambda = 1$ for SI units, and

5 $\Lambda = 4.1528 \times 10^{-5}$ (ft-water/inch)/(lbm/ft²/sec²) for English units.

6 The fluid approach velocity, U , is given simply in terms of the volumetric flow rate and the
7 effective screen surface area as:

8

$$9 \quad U = \frac{Q}{A}$$

10

11 where:

12 Q is the total volumetric flow rate through the screen, (ft³/sec) and

13 A is the effective screen surface area (ft²).

14 The screen surface area, A , is the submerged (wetted) effective surface area of the screen as
15 described in Section 3.2.5.2.1 above. As noted previously, the available surface area may change
16 with time, particularly in the case of star or stacked disc designs. For these particular alternate
17 geometry screens, given sufficient debris reaching the screen, the effective surface area may
18 eventually decrease to the circumscribed area. At the limit, the head loss for alternate geometry
19 screens may be calculated using the circumscribed area and the debris load equal to the total
20 debris load transported to the screen less the quantity of debris required to fill in the
21 volumes/gaps of the alternate geometry screen.

22

23 The mixed debris bed solidity (α_m) is given by:

24

$$25 \quad \alpha_m = \left(1 + \frac{\rho_f}{\rho_p} \eta \right) \alpha_o c \quad (3.2.5-2)$$

26

1 where:

2 α_o = is the solidity of the original fiber blanket (i.e. the “as fabricated” solidity)

3 η = m_p/m_f , the particulate-to-fiber mass ratio in the debris bed

4 m_p = Σm_i is the total particulate mass, (lbm)

5 ρ_f = the fiber density (lbm/ft³)

6 ρ_p = the average particulate material density (lbm/ft³) = $\Sigma \rho_i V_i / \Sigma V_i$

7 c = the head-loss-induced volumetric compression of the debris (inches/inch)

8

9 For debris deposition on a flat surface of a constant size, the compression (c) relates the actual
10 debris bed thickness, ΔL_m , and the theoretical fibrous debris bed thickness, ΔL_o , (inches), via the
11 relation:

12

$$13 \quad c = \frac{\Delta L_o}{\Delta L_m} \quad (3.2.5-3)$$

14

15 Compression of the fibrous bed due to the pressure gradient across the bed is also accounted.

16 The relation that accounts for this effect, which must be satisfied in parallel to the previous
17 equation for the head loss, is given by (valid for ratios of $\Delta H/\Delta L_o > 0.5$ ft-water/inch-insulation):

18

$$19 \quad c = 1.3 * K * (\Delta H / \Delta L_o)^{0.38} \quad (3.2.5-4)$$

20

21 Here, ‘K’ is a constant that depends on the insulation type. It is 1.0 for Nukon[®] fiber. Test data
22 or a similitude analysis is required to determine ‘K’ for fibrous materials that are dissimilar to
23 Nukon. It should be noted that this formulation for debris bed compression may over predict
24 compression significantly in the case of very thick debris layers, roughly 6-inches or more.

25 Thus, in these cases, it is conservative.

26

1 For very large pressure gradients and for cases where very little fiber is present, the compression
 2 has to be limited such that a maximum solidity is not exceeded. In NUREG/CR-6224, this
 3 maximum solidity is defined to be:

$$4 \quad \alpha_m = 65 \text{ lbm/ft}^3 / \rho_p \quad (3.2.5-5)$$

6
 7 which is equivalent to having a granular debris layer with a bulk density of 65 lbm/ft^3 . Note that
 8 65 lbm/ft^3 is the macroscopic, or bulk density of a granular media such as sand or gravel and clay
 9 (Reference 3.2.5-1). Based on NUREG/CR-6224 (Reference 3.2.5-2), the above value is also
 10 appropriate for ferrous sludge. For a sludge particle density of $\sim 324 \text{ lbm/ft}^3$, the maximum
 11 solidity is $\sim 20\%$, and this value has been determined from test data to yield acceptable results
 12 with the NUREG/CR-6224 head loss correlation. In general, solidity is defined as:

$$13 \quad \alpha_m = \rho_b / \rho_p \quad (3.2.5-5a)$$

14
 15 where ρ_b is the bulk, or macroscopic density, and ρ_p is the particle, or grain density. Since the
 16 solidity depends on the material properties, different materials may require testing to establish
 17 appropriate values. In practice, however, the limiting value of solidity specified above works
 18 well for many particulate mixtures.

19
 20
 21 Each constituent of debris has a surface-to-volume ratio associated with it based on the
 22 characteristic shape of that debris type. For typical debris types, we have:

23 Cylindrically-shaped debris: $S_v = 4/\text{diam};$

24 Spherically-shaped debris: $S_v = 6/\text{diam};$

25 Flakes (flat-plates): $S_v = 2/\text{thick};$

26 where 'diam' is the diameter in feet of the fiber or spherical particle, and 'thick' is the thickness
 27 in feet of the flake/chip. Other debris not listed above would have its surface-to-volume ratio
 28 calculated similarly based on one of the above characteristic shapes. Clearly, the above relations
 29 are simplified approximations. Generally, what is done is to select a characteristic size, for
 30 example, small spheres to represent irregularly shaped particulate debris, small cylinders to

1 represent fiber, etc. Whatever modeling approach is used, a comparison to test data then has to
 2 be made to assess the validity of the approximation for that particular material, with the
 3 characteristic sizes adjusted as required for the head loss correlation to conservatively match the
 4 data. For debris not yet tested and for which similitude arguments cannot be made, SEM
 5 analysis and/or plant-specific testing may be required.

6
 7 The following is a method for calculating the average surface to volume ratio for two different
 8 types of debris constituents (Reference 3.2.5-3).

$$S_v = \text{SQRT} [(S_{v1}^2 * v_1 + S_{v2}^2 * v_2)/(v_1 + v_2)], \quad (3.2.5-6)$$

11
 12 where v_1 and v_2 are the microscopic volumes of constituents '1' and '2,' respectively.

13
 14 Clearly, this result can be extended to more than two such fiber species as follows:

$$S_v = \text{SQRT} [\Sigma(S_{vn}^2 * v_n)/\Sigma(v_n)], \quad (3.2.5-7)$$

17
 18 where the subscript 'n' refers to the nth constituent.

19
 20 The above procedure is developed in detail in Attachment C of this document. Averaging in the
 21 above manner will yield a higher pressure drop as more than one type of debris is added to the
 22 mixture.

23
 24 Tables 3.2.2-2 and 3.2.2-3 list recommended values of fiber and particle sizes based on the data
 25 currently available, from which values of S_v may be derived. Where values are not given or
 26 where uncertainty otherwise exists, it is best to err on the small side for conservative values of
 27 S_v . In some cases, further measurements to establish debris sizes, SEM analysis, and
 28 comparisons to head loss correlations and test data may be required to establish appropriate
 29 values.

1 To obtain an aggregate density for both particulate and fibrous debris, a simple volume
2 averaging procedure is appropriate, as indicated in association with Equation 3.2.5-2, since, for a
3 well-mixed debris bed, the individual species can reasonably be expected to see the same
4 porosity.

5
6 Summarizing the computation process:

- 7 • Fiber and particulate debris are handled with the general form of the NUREG/CR-6224
8 correlation, Equation 3.2.5-1.
- 9 • Material properties are necessary – see Section 3.2.2.3 (Debris Characteristics) for
10 material properties of material commonly encountered in PWRs.
- 11 • Knowing the debris quantities that are calculated to reach the sump screen, the mass ratio
12 of particulates-to-fiber (η), the fiber density (ρ_f), and the average particulate density (ρ_p),
13 and the theoretical bed thickness (ΔL_o) are determined.
- 14 • A compression factor [c] must be specified. This is an iterative process, with a value of
15 2.0 being a reasonable first approximation. (Adjust ‘c’ thereafter in the direction of
16 convergence. Alternatively, the bed thickness may be assumed and ‘c’ derived from
17 this.)
- 18 • The mixed bed solidity (α_m) is next calculated from Equation 3.2.5-2.
- 19 • An overall, average value of S_v must be determined for the fibrous materials, each of the
20 particulates and then an average for the overall debris mixture by Equation 3.2.5-7. If
21 multiple fiber types are present, then each type should be included in the averaging
22 process.
- 23 • The water properties (ρ and μ) are specified at the sump temperature at the time the head
24 loss across the debris bed is calculated. Alternatively a conservative approach would be
25 to calculate the head loss using the lowest sump water temperature calculated over the
26 entire time frame that the ECCS needs to function.
- 27 • The approach velocity will be known from the sump screen area and the ECCS flows
28 through the screen.

- Substitution of all of the above information into Equation 3.2.5-1, in combination with iterative solution of Equations 3.2.5-3 and 3.2.5-4, yields the sump screen head loss and the actual debris-bed thickness, ΔL_m .

The head loss across a debris bed consisting of fibrous debris (no particulates) can be calculated with the general form of the NUREG/CR-6224 correlation, Equation 3.2.5-1, where the mass ratio of particulates-to-fiber (η) is set to zero. Given the presence of particulates from dirt/dust and possibly unqualified coatings, it would be unusual to have to analyze pure fiber bed head loss for a PWR. However, this case has application when interpreting experimental results, so it is mentioned for completeness.

3.2.5.2.3.1.2 RMI Debris Beds

The head loss for a RMI debris bed on the sump screen surface depends mainly on the accumulation at the sump screen and the type and size distribution of RMI debris. The key parameter needed to evaluate pure RMI head loss is the surface area of the RMI bed on the screen. The commonly accepted empirical correlation for RMI (Reference 3.2.5-1) is:

$$\Delta H = [1.56E-05/(K_t)^2] U^2 A_{\text{foil}}/A_c \quad (3.2.5-8)$$

where:

K_t is the interfoil gap thickness (ft)

ΔH is the head loss, (feet-of-water)

U is the sump screen approach velocity, (ft/sec)

A_{foil} is the RMI foil surface area, (ft²)

A_c is the sump screen surface area, (ft²).

Extracted from Table 7-2 of NUREG/CR-6808, some values of K_t are listed below. Other values of K_t are listed in Appendix K of the SER to the URG.

Table 3.2.5-1: Values of K_t from NUREG/CR-6808

Foil Type and Bed Type	K_t (feet)
2.5-mil SS (NRC large pieces)	0.014
2.5-mil SS (NRC small pieces)	0.010
1.5-mil Al (debris bed)	0.008
1.5-mil Al (debris bed)	0.006
2.5-mil SS (STUK flat pieces)	0.007
2.5-mil SS (1-mm dimple)	0.003

In Appendix K of the NRC SER to the BWROG URG, the NRC concluded that a value of K_t of 0.012 in the above general equation bounds the head loss data reasonably well for 2.5-mil SS RMI. Substituting this value of K_t into Equation 3.2.5-8, one obtains:

$$\Delta H = 0.108 U^2 A_{\text{foil}}/A_c \quad (3.2.5-9)$$

Equation 3.2.5-9 accounts for experimental uncertainties, test repeatability variations, and debris size and material types. As such, for 2.5-mil, SS foil, Equation 3.2.5-9 predicts the head loss across a pure RMI debris bed for PWR sump screens. The refinement guidance given in Section 4 will provide further discussion of RMI head loss correlations.

3.2.5.2.3.1.3 Mixed Debris Beds (RMI, Fiber and Particulates)

A mixed debris bed of RMI, fiber and particulates is handled by superposition (Reference 3.2.5-1). First, the fiber-and-particulate head loss is determined using the methodology of Section 3.2.5.2.3.1.1. Next, the RMI head loss is determined using the methodology of Section 3.2.5.2.3.1.2. These two head losses are then added together to estimate the total head loss of a RMI, fiber, and particulate bed. This procedure is conservative, and the user need not be concerned with how the debris bed is formed.

The superposition of RMI and fiber may be overly conservative for cases where relatively large amounts of RMI and trace amounts of fiber (e.g. latent fiber) are estimated to be transported to

1 the sump screen. Experiments have shown that fiber can become caught either within the voids
2 of the RMI bed or at the surface of the RMI bed (which can have a significantly larger surface
3 area and a lower approach velocity than the sump screen itself). For plants that are essentially all
4 RMI, a relatively small amount of latent fiber could provide the quantity necessary to develop a
5 thin bed, causing unacceptable or unrealistic results when added algebraically to the RMI head
6 loss. More realistic methods for trace amounts of either RMI or fiber will be addressed in the
7 refinement guidance of Section 4.

8 9 **3.2.5.2.3.1.4 Calcium Silicate Insulation**

10
11 Calcium silicate (Cal-Sil) is a granular insulation. It consists of fine particulate material that is
12 chemically bonded and is also held together by a fine fibrous matrix. Experiments thus far
13 indicate that it is best treated as a particulate material for head loss calculations. Test data will
14 be required for specification of the appropriate particle sizes and surface-to-volume ratios to use
15 in head loss analysis. At present, most of the head loss test data for Cal-Sil are privately held,
16 the exception being the NRC/LANL/UNM Cal-Sil Test Report whose issuance is pending.
17 Based on current information, the NUREG/CR-6224 correlation can be used according to the
18 methods of Section 3.2.5.2.3.1.1 if the application is limited to particulate mixtures containing up
19 to about 20% Cal-Sil by mass. Additional head loss data for Cal-Sil is anticipated to be released
20 by the NRC in the near future. The Supplemental Guidance will provide additional background
21 regarding the insights gained in the very limited series of head loss experiments available for
22 review through April 2004.

23
24 Cal-Sil is used in many of the PWRs and has different compositions. For example, it may
25 contain diatomaceous earths, perlite and/or asbestos fibers, and plant-specific characterization
26 (via SEM analysis, at a minimum) is warranted to identify the specific composition, particle size
27 range, and source of this material.

28 29 **3.2.5.2.3.1.5 Microporous Insulation**

1 Microporous insulation (e.g. MinK and Microtherm) is also a granular insulation and has been
2 used in PWRs. The analyst is cautioned to ensure that the applicable material properties are
3 used, since there may be significant variations in material properties from those suggested in the
4 Debris Characteristics section. The Supplemental Guidance will provide additional background
5 regarding the insights gained in the very limited series of head loss experiments available for
6 review through April 2004.

7 8 Microporous and Fiber Debris 9

10 A limited series of head loss tests was performed with microporous debris in the presence of
11 fibrous debris. These tests showed that the NUREG/CR-6224 correlation bounded the
12 experimental data for all cases where the microporous-to-fiber mass ratio was less than about 20
13 percent. For mass ratios higher than about 20 percent, the NUREG/CR-6224 correlation was
14 found to be potentially non-conservative.

15
16 The computation of the head loss of mixed microporous and fiber debris beds (where the
17 microporous to fiber mass ratio is less than 20 percent) is the same as described for a fiber and
18 particulate bed (Section 3.2.5.2.3.1.1). The currently available experimental database does not
19 support a correlation for estimating the head loss across a debris bed composed of micro-porous
20 and fibrous insulation where the microporous to fiber mass ratio is more than 20 percent.

21
22 In the event that a debris bed composed of microporous and fibrous insulation (or calcium
23 silicate and fiber, where the microporous-to-fiber (or the Cal-Sil-to-fiber) mass ratio is more than
24 20 percent), is calculated to form on the screens, the alternatives currently available for
25 improving the sump screen performance include:

- 26 • Removal of microporous or calcium silicate insulation until the debris generation and
27 transport analysis yields a debris mixture in which the particulate-to-fiber mass ratio is
28 less than 20 percent;
- 29 • Use of a head loss correlation other than NUREG/CR-6224 (See the refinement
30 guidance of Section 4 for potentially applicable head loss correlations.); or

- Conduct of head loss experiments using plant-specific debris mixtures, sump screen configuration, and thermal hydraulic conditions.

Microporous or Calcium Silicate Debris Only

Based on results from a very limited series of experiments, microporous insulation debris or calcium silicate debris by itself has been shown to induce significant head losses. Tests have determined that the NUREG/CR-6224 correlation is unreliable for predicting the head loss of microporous insulation debris alone. The currently available experimental database does not support a correlation for estimating the head loss across a debris bed composed solely of microporous insulation debris.

Calcium silicate by itself has also been shown to induce high head losses (Ref. 3.2.5-4). Preliminary indications are that the NUREG/CR-6224 correlation may fit the data if appropriate physical parameters are used in the correlation. Further instruction in this regard is deferred to the Supplemental Guidance and pending the release of the NRC/LANL/UNM Cal-Sil test report.

The alternatives currently available for improving sump screen performance for a debris bed on the screen composed of only microporous or calcium silicate insulation include:

- Removal of all granular insulation (e.g. Cal-Sil, MinK, Microtherm, etc.);
- Use of a head loss correlation other than NUREG/CR-6224 (The guidance on refinements given in Section 4 will address potentially applicable head loss correlations);
- Conduct of head loss experiments using plant specific debris mixtures, sump screen configuration, and thermal hydraulic conditions.

Granular Insulation and RMI Debris

Reference 3.2.5-4 suggests that the head loss for an RMI and calcium silicate debris bed will be relatively low, with increased head loss as the quantity of Cal-Sil debris quantities increases.

1 The expectation is that the same would also occur for all types of granular insulation (Min-K,
2 Microtherm and calcium silicate) and RMI debris beds. Mixtures of granular insulation, RMI,
3 fiber, and other debris should be treated the same as mixed debris bed treatment of 3.2.5.2.3.1.3
4 with the limitations noted in 3.2.5.2.3.1.4 and 3.2.5.2.1.3.5 above.

6 **3.2.5.2.3.2 Methodology Application Considerations**

8 **3.2.5.2.3.2.1 Total Sump Screen Head loss**

10 The total strainer head loss (TSHL) is the sum of the debris-bed head loss (DBHL) and the clean
11 strainer head loss (CSHL).

$$13 \quad \text{TSHL} = \text{CSHL} + \text{DBHL}$$

15 **3.2.5.2.3.2.2 Evaluation of Breaks with Different Combinations of Debris**

17 It is important to identify the break location that produces the highest debris bed head loss, i.e.,
18 the limiting break. The limiting break is not necessarily the break that generates the largest total
19 quantity of debris. For example, a break that generates enough fiber that, after the transport
20 considerations, deposits enough fiber on the screen to cause a thin bed may yield higher head
21 losses in the presence of particulate than the break that generates more fiber (for the same
22 quantity of particulate). As such, the analyst needs to evaluate a spectrum of breaks with
23 different combinations of debris types to ensure that the mixture of debris on the screen that
24 causes the highest head loss is identified.

26 **3.2.5.2.3.2.3 Thin Fibrous Beds**

28 For the reasons discussed in Section 3.2.5.2.3.2.2 and as suggested in Revision 3 of RG 1.82, this
29 methodology recommends that the head loss for a one-eighth-inch-thick fiber debris-bed
30 (including particulates) be evaluated for existing PWR sump screens.

1
2 For conditions of fiber and particulate present in the post-LOCA containment floor pool, as the
3 fiber-bed is deposited on the screen, particulate material will be trapped by the fiber, increasingly
4 so as the fiber bed thickens. Once a fiber bed of approximately one-eighth-inch thickness is
5 formed, if there is sufficient particulate debris, a low permeability granular layer of debris on top
6 of the fiber bed will be formed. The head loss associated with the accumulation of mostly
7 particulate debris on thin fibrous beds can be quite high, and surprisingly enough, greater than
8 the head losses associated with much larger quantities of fiber and much thicker beds of debris.
9 This apparently counter-intuitive head loss phenomenon is known as the Thin Bed Effect (TBE).
10 The Supplemental Guidance will provide further discussions on the TBE.

11
12 It only takes a small quantity of fiber to facilitate TBE occurrence, and since it is difficult to
13 make a defensible case that no fibers whatsoever are present in the containment, the possibility
14 of forming a thin fibrous bed generally has to be evaluated for existing PWR screens.
15 Additionally, given the uncertainties of debris generation and transport calculations, the total
16 quantities of fiber calculated to reach the sump screens may be on the high side, hence the impact
17 of a smaller quantity of fiber reaching the sump screen should be examined, i.e. the transport of
18 only the fiber necessary to form a thin bed potentially being the limiting case. This methodology
19 recommends that the head losses given a one-eighth-inch fiber bed (plus particulate) be
20 calculated as a sensitivity analysis.

21
22 To analyze a thin fiber bed, a fiber quantity sufficient to form a bed one-eighth-inch thick should
23 be determined to be available and if present could be deposited on the sump screen. The
24 requisite quantity is easily calculated as 0.010-foot times the sump screen net area. The head
25 loss computations are the same as described for fiber and particulate beds (Section 3.2.5.2.3.1.1)
26 using the full value of particulate matter transported to the sump screen. (This would include
27 latent debris such as dirt and concrete dust. It would also include any other fine particulate
28 debris such as rust, inorganic zinc, epoxy fine material, etc.) It should be noted that the
29 particulate layer is characterized by a very high sludge-to-fiber ratio; hence a limiting value for
30 the compression is used. If under these conditions, the thin-bed head loss should exceed the

1 NPSH Margin, then the allowable particulate loading can be evaluated by reducing the
2 particulate quantity until the calculated head loss is within the NPSH margin.

3 4 **3.2.5.2.3.2.4 Sump Screen Submergence**

5
6 For submerged screen sumps the head loss computation methods presented herein are directly
7 applicable. Submerged screens are characterized by having the ambient pressure on one side of
8 the screen, and the flow is driven by the pump. The limiting criterion for submerged screens
9 occurs when the combined clean sump and debris bed head loss exceeds the NPSH Margin.

10
11 For partially submerged screen sumps the head loss computation methods presented herein are
12 also directly applicable. Partially submerged screens are characterized by having the ambient
13 pressure on both sides of the screen. In this case the flow driver is the difference in fluid
14 elevation between the two sides of the screen. As debris accumulates on the screen, the water
15 level behind the screen falls in order to generate a pressure drop to allow the flow rate to be
16 achieved. The limiting criterion for a partially submerged screen is when the debris bed
17 accumulation on the screen reduces the flow to less than the flow requirements for the sump.
18 Numerical simulations confirm that an effective head loss across a debris bed approximately
19 equal to one-half of the pool height is sufficient to prevent adequate water flow. As such, for
20 partially submerged sump screens the methodology described herein should be used to estimate
21 the pressure drop due to debris across the submerged sump screen area. The partially submerged
22 sump screen will operate properly if the estimated head loss (in feet-of-water across the debris
23 bed, when added to the clean screen head loss) is less than one-half the pool height.

24 25 **3.2.5.2.3.2.5 Buoyant Debris**

26
27 For fully submerged screens, buoyant debris is not considered a problem since it would not reach
28 the sump screens. However, for partially submerged screens, the effects of buoyant debris
29 should be considered. Note that the transport analysis may indicate that the quantity of buoyant
30 debris reaching the sump screen is negligible, since trash racks and gates may largely prevent
31 this.

1
2 For buoyant debris that is determined to reach a partially submerged screen, this baseline
3 methodology recommends that the effective screen area be reduced by the thickness of the
4 buoyant debris layer times the length of the covered perimeter, to the extent that it fully
5 envelopes the screen. This is very conservative, since floating debris will have gaps and large
6 pore space among pieces that will admit flow.

7

8 **3.2.5.2.3.3 Methodology Limitations and Other Considerations**

9

10 **3.2.5.2.3.3.1 Flat Screen Assumption**

11

12 The NUREG/CR-6224 correlation adopted in this methodology was developed mainly using data
13 obtained in a closed loop that contained a vertical pipe section which housed a horizontally
14 mounted flat screen. The flat screens yielded conservative data for the development of the
15 NUREG/CR-6224 correlation because all debris was forced onto a very small screen in a small-
16 scale test apparatus. In the case of alternate design screens (stacked disc, star, large-passive, etc.)
17 direct application of the NUREG/CR-6224 correlation may yield overly conservative results
18 (Reference 3.2.5-2). For these alternate geometry screens, independent head loss correlations
19 should be developed based on actual design configurations, debris loads, and test data to reduce
20 conservatism.

21

22 **3.2.5.2.3.3.2 Non-Uniform Deposition on Sump Screen Surfaces**

23

24 PWR sump screens can have vertical and inclined orientation. On a vertical screen, there is
25 greater chance for non-uniform deposition of debris, which will usually lead to lower head losses
26 because of thin spots in the debris bed. Body forces also tend to shear the bed from the screen,
27 also a mitigating factor. For these reasons, using the uniform deposition assumption for vertical
28 screens is a conservative approach. Similar statements can be made for curved surfaces such as
29 horizontally oriented, cylindrical strainer designs, since body forces in the debris bed essentially
30 act in the opposite direction to the suction forces over a significant portion of the strainer area.

1 An inclined, flat surface is less limiting than a horizontal surface, therefore, the uniform
2 deposition assumption again should be conservative.

3.2.5.2.3.3 Very Thin Fiber Beds

6 This section pertains to the regime where fiber loading is less than that required to form a thin-
7 bed. The NUREG/CR-6224 head loss correlation was developed and validated for debris that is
8 uniformly distributed on the screen surface. However, experiments have shown that very thin
9 fibrous beds (with a thickness of less than one-eighth inch) are characterized by large scale non-
10 uniformities on the screen and negligible head losses. For fibrous debris bed less than one-eighth
11 inch thick, the NUREG/CR-6224 head loss correlation significantly over predicts the
12 experimentally determined head loss and should not be used. Instead, it is appropriate to
13 consider the head loss across fibrous debris beds less than one-eighth inch to be negligible.

3.2.5.2.4 Sample Calculation

17 The following examples demonstrate the use of the head loss equations with the debris sources
18 specified in Section 3.2.2.3 of this document and typical plant conditions. These calculations
19 assume steady-state conditions at final debris loading with steady ECCS flows and a simple, flat-
20 plate strainer geometry.

3.2.5.2.4.1 Fiber and Particulate Debris Bed

Flow Conditions:

26 These are obtained from plant design documents and NPSH calculations.

28 ECCS Flow Rate (Q) = 9000 gpm

29 Temperature (T) = 170 °F

30 Fluid Density (ρ) = 60.80 lbm/ft³

31 Fluid Viscosity (μ) = 2.51E-04 lbm/ft/sec

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31

Screen Parameters:

These are obtained from screen design drawings and ECCS flow rate.

$$\begin{aligned} \text{Effective Surface Area (A)} &= \underline{\quad 300 \quad} \text{ ft}^2 \\ \text{Screen Approach Velocity (U)} &= \underline{\quad 0.067 \quad} \text{ ft/s} \end{aligned}$$

Debris Types/Quantities at Screen:

These are obtained from Debris Characteristics (Section 3.2.2), Latent Debris (Table 3.2.3-1) and the Transport Analysis (Section 3.2.4).

$$\begin{aligned} \text{NUKON Fiber} &= \underline{\quad 129 \quad} \text{ ft}^3 \\ \text{Latent Fiber} &= \underline{\quad 8.84 \quad} \text{ ft}^3 \leftarrow 62.4/2.4 * 0.34 \text{ ft}^3 \\ \text{Latent Dirt-Dust} &= \underline{\quad 33.51 \quad} \text{ lbm} \\ \text{Qual-Epoxy} &= \underline{\quad 329 \quad} \text{ lbm} \\ \text{Unqual. Coatings} &= \underline{\quad 2625 \quad} \text{ lbm} \end{aligned}$$

Debris Characteristics:

- NUKON

$$\begin{aligned} \text{Theoretical Packing Density } (\rho_f) &= \underline{\quad 2.4 \quad} \text{ lbm/ft}^3 \\ \text{Fiber Diameter (D)} &= \underline{\quad 2.33 * 10^{-5} \quad} \text{ ft (use LDFG)} \\ \text{Surface to Volume Ratio } (S_v) &= \underline{\quad 1.717 * 10^5 \quad} \text{ ft}^{-1} \leftarrow 4 / 2.33 * 10^{-5} \text{ ft}^3 \\ \text{Mass of Fiber } (m_f) &= \underline{\quad 309.6 \quad} \text{ lbm} \leftarrow 129 \text{ ft}^3 * 2.4 \text{ lbm/ft}^3 \\ \text{Fiber Density} &= \underline{\quad 175 \quad} \text{ lbm/ft}^3 \\ \text{Fiber Volume} &= \underline{\quad 1.77 \quad} \text{ ft}^3 \leftarrow 309.6 \text{ lbm} / 175 \text{ lbm/ft}^3 \end{aligned}$$

- Latent Fiber

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Theoretical Packing Density (ρ_f)	=	<u>2.4</u>	lbm/ft ³ (assume same as LDFG)
Fiber Diameter (D)	=	<u>$2.33 * 10^{-5}$</u>	ft
Surface to Volume Ratio (S_v)	=	<u>$1.717 * 10^5$</u>	ft ⁻¹ $\Leftarrow 4 / 2.33 * 10^{-5}$ ft ³
Mass of Fiber (m_f)	=	<u>21.22</u>	lbm $\Leftarrow 8.84$ ft ³ * 2.4 lbm/ft ³
Fiber Density	=	<u>62.4</u>	lbm/ft ³ (Table 3.2.3.4.2-1)
Fiber Volume	=	<u>0.34</u>	ft ³ $\Leftarrow 21.22$ lbm / 62.4 lbm/ft ³

- Latent Dirt/Dust

Particle Density	=	<u>100</u>	lbm/ft ³
Particle Diameter (D)	=	<u>$3.28 * 10^{-5}$</u>	ft
Surface to Volume Ratio (S_v)	=	<u>$1.829 * 10^5$</u>	ft ⁻¹ $\Leftarrow 6 / 3.28 * 10^{-5}$ ft ³
Particle Volume	=	<u>0.335</u>	ft ³ $\Leftarrow 33.51$ lbm / 100 lbm/ft ³

With respect to qualified coatings in the ZOI, a relatively high damage pressure has been justified in earlier sections of this document. However, the demonstration calculations will use a spherical ZOI with radius of 10-feet, for a surface area of 1256.6 ft². The qualified coatings thickness is taken to be 0.009". For unqualified coatings, a thickness of 0.003" is used, and 30,000 ft² is the assumed coverage. In both cases, the coatings particles are conservatively assumed to be spherical with diameter equal to 10 μ m. The coatings material is assumed to be inorganic zinc (IOZ) in both cases.

- Qualified Epoxy

Particle Density	=	<u>350</u>	lbm/ft ³ (IOZ-equivalent)
Particle Diameter (D)	=	<u>$3.28 * 10^{-5}$</u>	ft
Surface to Volume Ratio (S_v)	=	<u>$1.829 * 10^5$</u>	ft ⁻¹ $\Leftarrow 6 / 3.28 * 10^{-5}$ ft ³
Particle Volume	=	<u>0.94</u>	ft ³ $\Leftarrow 329$ lbm / 350 lbm/ft ³

1 • Unqualified Epoxy

2

3 Particle Density = 350 lbm/ft³ (IOZ-equivalent)

4 Particle Diameter (D) = 3.28 * 10⁻⁵ ft

5 Surface to Volume Ratio (S_v) = 1.829 * 10⁵ ft⁻¹ ⇐ 6 / 3.28 * 10⁻⁵ ft³

6 Particle Volume = 7.50 ft³ ⇐ 2625 lbm / 350 lbm/ft³

7

8 • Average Fiber

9

10 Total Fiber Volume = 2.11 ft³

11 Total Fiber Mass = 330.82 lbm

12 Ave Fiber Density = 156.86 lbm/ft³

13 Ave Surface to Volume Ratio (S_v) = 1.717 * 10⁵ ft⁻¹

14

15 • Average Particulate

16

17 Total Particle Volume = 8.775 ft³

18 Total Particle Mass = 2987.5 lbm

19 Ave Particle Density = 340.46 lbm/ft³

20 Ave Surface to Volume Ratio (S_v) = 1.829 * 10⁵ ft⁻¹

21

22 • Average Debris

23

24 Total Particle Volume = 8.775 ft³

25 Ave Surface to Volume Ratio (S_v) = 1.829 * 10⁵ ft⁻¹

26

27 Total Fiber Volume = 2.11 ft³

28 Surface to Volume Ratio (S_v) = 1.717 * 10⁵ ft⁻¹

29

30 Ave Debris Surface to Volume Ratio (S_v) = 1.8078*10⁵ ft⁻¹

31

1 Debris Bed Equations:

- 2
- 3 • Theoretical Debris Bed Thickness (ΔL_o)

4 Total Volume of Fiber divided by Screen Area = 5.51 inches

- 5
- 6 • Particulate to Fiber Mass Ratio (η)

7 Mass of Particles divided by Mass of Fiber = 9.03

- 8
- 9 • Actual Bed Thickness (ΔL_m) = 2.72 inches

10

11 Assume a value for the bed thickness and iterate until Equations 3.2.5-3 and 3.2.5-4

12 converge on approximately the same number. Computer solution may be required.

13

14 Eq. 3.2.5-1: Head Loss Across Debris Bed (ΔH) = 17.80 feet H₂O

15

16 Eq. 3.2.5-2: Mixed Debris Bed Solidity (α_m) = 0.16

17

18 Eq. 3.2.5-3: Head Loss Volumetric Compression (c) \approx 2.03

19

20 Eq. 3.2.5-4: Head Loss Volumetric Compression (c) \approx 2.03

21

22 Equations 3.2.5-3 and 3.2.5-4 have converged within <1% of each other, which is

23 considered and acceptable convergence. Therefore, the head loss is calculated as 17.80

24 feet-of-water.

25

26 The mixed debris bed solidity should be less than or equal to 0.20, therefore OK.

27

28 **3.2.5.2.4.2 Fiber Debris Bed**

29

30 No sample calculation is provided since a pure fiber debris bed would be unusual, given the

31 coatings particulate debris in the ZOI, latent debris, the presence of dirt/dust and other possible

1 sources of particulates such as ablated concrete. However, should a fiber-only debris-bed head
 2 loss need to be calculated, the process would be the same as for fiber and particulate except that
 3 the particulate quantities would be set to zero.

4 5 **3.2.5.2.4.3 RMI Debris Bed**

6 7 Flow Conditions:

8
9 These are obtained from plant design documents and NPSH calculations.

10			
11	ECCS Flow Rate (Q)	=	<u>9000</u> gpm
12	Temperature (T)	=	<u>170</u> °F
13	Fluid Density (ρ)	=	<u>60.80</u> lbm/ft ³
14	Fluid Viscosity (μ)	=	<u>2.51E-04</u> lbm/ft/sec

15 16 Screen Parameters:

17
18 These are obtained from screen design drawings and ECCS flow rates.

19			
20	Effective Surface Area (A)	=	<u>300</u> ft ²
21	Screen Approach Velocity (U)	=	<u>0.067</u> ft/s

22 23 Debris Types/Quantities:

24
25 These are obtained from the Debris Characteristics (Section 3.2.2) and Debris Transport
 26 Analysis (Section 3.2.4).

27			
28	2.5-mil SS RMI	=	<u>4387.5</u> ft ² \Leftarrow 11,250 ft ² * 0.39 T.F.

29 30 Debris Bed Equations:

31

1 The head loss correlation for RMI is taken from Section 3.2.5.2.3.1.2

$$2 \quad \Delta H = 0.108 U^2 (A_{\text{foil}} / A_c)$$

4 where,

6 ΔH = the head loss across the RMI bed (ft-water),

7 U = the approach velocity to the screen (ft/s),

8 A_{foil} = the surface area of the RMI foils (ft² – nominal), and

9 A_c = the strainer circumscribed area (ft²).

10
11 Substituting the above plant specific parameters,

$$12 \quad \Delta H = 0.108 (0.067)^2 (4387.5 / 300)$$

$$13 \quad = 0.007 \text{ ft-water} \cong 0.01 \text{ ft-H}_2\text{O}$$

14 15 16 17 **3.2.5.2.4.4 Mixed Debris Beds (RMI, Fiber, and Particulates)**

18
19 The head loss of a mixed fiber, particulate, and RMI debris bed is the addition of the fiber-and-
20 particulate head loss to the RMI head loss. For example, if the quantities of debris were as in the
21 totals of Sections 3.2.5.2.4.1 and 3.2.5.2.4.2, then the total mixed RMI and fibrous debris bed
22 head loss would be:

$$23 \quad \Delta H_{\text{RMI}} = 0.01 \text{ ft-water}$$

$$24 \quad \Delta H_{\text{Fiber + Particulate}} = 17.80 \text{ ft-water}$$

25
26
27 hence,

$$28 \quad \Delta H_{\text{RMI+ Fiber + Particulate}} = 17.81 \text{ ft-water. (We can neglect the RMI in this case).}$$

3.2.5.2.4.5 Thin-Bed of Fiber and Particulate Debris

Flow Conditions:

These are obtained from plant design documents and NPSH calculations.

ECCS Flow Rate (Q)	=	<u>9000</u>	gpm
Temperature (T)	=	<u>170</u>	°F
Fluid Density (ρ)	=	<u>60.80</u>	lb/ft ³
Fluid Viscosity (μ)	=	<u>2.51E-04</u>	lb/ft/sec

Screen Parameters:

These are obtained from screen design drawings and ECCS flow rate.

Effective Surface Area (A)	=	<u>300</u>	ft ²
Screen Approach Velocity (U)	=	<u>0.067</u>	ft/s

Debris Types/Quantities:

As a starting point, use plant-specific quantities of fine particulate and latent debris.

Provided that sufficient fiber is available, the fiber quantity is specifically selected to create a Thin-Bed. Nukon[®] is assumed for this example, although latent fiber could just as well be used if a sufficient amount is present.

NUKON Fiber	=	<u>3.125</u>	ft ³	$\Leftarrow 0.125''/12 * 300 \text{ ft}^2$
Dirt-Dust	=	<u>33.51</u>	lbm	
Qual-Epoxy	=	<u>329</u>	lbm	
Unqualified Coatings	=	<u>2625</u>	lbm	

Debris Characteristics:

1 • NUKON

2

3 Theoretical Packing Density (ρ_f) = $\frac{2.4}{\text{ft}^3}$ lbm/ft³

4 Fiber Diameter (D) = $2.33 * 10^{-5}$ ft

5 Surface to Volume Ratio (S_v) = $1.717 * 10^5$ ft⁻¹ $\Leftarrow 4 / 2.33 * 10^{-5}$ ft³

6 Mass of Fiber (m_f) = 7.5 lbm $\Leftarrow 3.125$ ft³ * 2.4 pcf

7 Fiber Density = 175 lbm/ft³

8 Fiber Volume = 0.043 ft³ $\Leftarrow 7.5$ lbm / 175 lbm/ft³

9

10 • Latent Dirt/Dust

11

12 Particle Density = 100 lbm/ft³

13 Particle Diameter (D) = $3.28 * 10^{-5}$ ft

14 Surface to Volume Ratio (S_v) = $1.829 * 10^5$ ft⁻¹ $\Leftarrow 6 / 3.28 * 10^{-5}$ ft³

15 Particle Volume = 0.335 ft³ $\Leftarrow 33.51$ lbm / 100 lbm/ft³

16

17 • Qualified Epoxy

18

19 Particle Density = 350 lbm/ft³ (IOZ-equivalent)

20 Particle Diameter (D) = $3.28 * 10^{-5}$ ft

21 Surface to Volume Ratio (S_v) = $1.829 * 10^5$ ft⁻¹ $\Leftarrow 6 / 3.28 * 10^{-5}$ ft

22 Particle Volume = 0.94 ft³ $\Leftarrow 329$ lbm / 350 lbm/ft³

23

24 • Unqualified Coatings

25

26 Particle Density = 350 lbm/ft³ (IOZ-equivalent)

27 Particle Diameter (D) = $3.28 * 10^{-5}$ ft

28 Surface to Volume Ratio (S_v) = $1.829 * 10^5$ ft⁻¹ $\Leftarrow 6 / 3.28 * 10^{-5}$ ft

29 Particle Volume = 7.5 ft³ $\Leftarrow 2625$ lbm / 350 lbm/ft³

30

1 • Average Particulate

2

3 Total Particle Volume = 8.775 ft³

4 Total Particle Mass = 2987.5 lbm

5 Ave Particle Density = 340.46 lbm/ft³

6 Ave Surface to Volume Ratio (S_v) = 1.829 * 10⁵ ft⁻¹

7

8 • Average Debris

9

10 Total Particle Volume = 8.775 ft³

11 Ave Surface to Volume Ratio (S_v) = 1.829 * 10⁵ ft⁻¹

12

13 Total Fiber Volume = 0.043 ft³

14 Surface to Volume Ratio (S_v) = 1.717 * 10⁵ ft⁻¹

15

16 Ave Debris Surface to Volume Ratio (S_v) = 1.82847 * 10⁵ ft⁻¹

17

18 Debris Bed Equations:

19

20 • Theoretical Debris Bed Thickness (ΔL_o)

21 Total Volume of Fiber divided by Screen Area = 0.125-inch

22

23 • Particulate to Fiber Mass Ratio (η)

24 Mass of Particles divided by Mass of Fiber = 398.34

25

26 • Actual Bed Thickness (ΔL_m) = 1.764-inches

27 Sum the fiber and particulate volumes. Multiply by 12

28 and divide by the product of the (Solidity * Screen Net Area)

29 Limiting solidity value of 0.20 is recommended.

30

31 Eq. 3.2.5-1: Head Loss Across Debris Bed (ΔH) = 19.27 ft-H₂O

1
2 Eq. 3.2.5-2: Mixed Debris Bed Solidity (α_m) = 0.20
3

4 The calculated head loss is 19.27 feet of water via iterative solution. Computational tools may
5 be required. Since the calculated head loss of the thin-bed exceeds the NPSH Margin at most
6 plants, parametric calculations can be performed to determine the allowable particulate quantities
7 at the sump screen(s).
8

9 **3.2.5.2.4.6 Microporous Insulation**

10
11 As noted in 3.2.5.2.3.1.5 above, the currently available experimental data can only support the
12 head loss calculations of microporous insulation debris in the presence of fibrous debris provided
13 the mass ratio of microporous insulation-to-fiber is less than 20 percent. In these cases the
14 microporous insulation debris is treated as a particulate and the equations and methods for
15 fibrous and particulate head loss are used (see example of Section 3.2.5.2.4.1 above).
16

17 **3.2.5.2.4.7 Determination of Requisite Sump Screen Size**

18
19 If, through the evaluation of the debris head loss, the existing screen does not provide sufficient
20 surface area, the calculations provided within this methodology can be utilized with little or no
21 modification to determine the amount of surface area required.
22

23 The key assumption in the head loss correlations provided is homogeneous debris accumulation
24 on a flat plate. As noted in Section 3.2.5.2.3.3.1, different screen orientations and configurations
25 can provide different debris accumulation profiles and take advantage of uneven debris
26 distribution and flow redistribution. In these cases, the head loss correlations provided in this
27 methodology may yield overly conservative results. As such, adjustments to the head loss
28 correlation could be made based on experimental test data applicable to the actual sump screen
29 orientation and configuration. Some test data exist for vertical screens; (see Ref. 3.2.5-6), but
30 applicability of the test data always has to be assessed. In some cases, plant-specific testing may

1 be required to reduce conservatism. Suggested refinements are further outlined in the
2 Supplemental Guidance.

3 4 **3.2.5.2.5 Calcium Silicate**

5
6 Informal results on the NRC/LANL calcium silicate testing at UNM were presented in February
7 2003 (Reference 3.2.5-4). This presentation did not provide any quantitative guidance with
8 respect to use of the NUREG/CR-6224 correlation with Cal-Sil debris mixtures. A recent
9 LANL/NRC/UNM paper [Reference 3.2.5-5] has provided more detailed test results. The formal
10 NRC test report on this program is not yet available.

11
12 Reference 3.2.5-5 has been reviewed, and some observations are provided. With respect to the
13 calcium silicate tests with Nukon fiber, the principal comment is that these results will have to be
14 applied very carefully on a plant-specific basis. For example, the researchers operated their test
15 apparatus at very high flow rates, which induced high approach velocities that compressed the
16 debris beds to the compression limit of the granular debris. When the flow was reduced, the
17 compressed bed did not relax, nor did it release the trapped particles. Hysteretic effects were
18 observed, for which head losses were actually greater at lower flows. Then, the surface-to-
19 volume ratio was adjusted such that the NUREG/CR-6224 correlation conservatively predicted
20 the hysteretic effects. For some plants, this testing does not represent prototypical behavior, and
21 it is excessively conservative. It also suggests the troubling conclusion that there is no benefit to
22 throttling the ECCS flows to reduce sump screen head loss with Cal-Sil, which for some plants
23 again may not be true.

24
25 Based on the research procedures described above, Reference 3.2.5-5 concludes that $S_V =$
26 $550,000 \text{ ft}^{-1}$ is an appropriate value for the specific type of Cal-Sil that was tested. The
27 researchers further recommend that this value be conservatively enhanced for safety analyses.
28 Our observation is that this procedure will be excessively conservative for many plants,
29 depending on the type(s) of Cal-Sil present in these plants and on the sump screen approach
30 velocities. Therefore, the results of Reference 3.2.5-5 should be applied with extreme caution.

31

1 The researchers applied similar techniques to the test with fiber, dirt and concrete dust in
 2 Reference 3.2.5-5. Therefore, the recommended value of $S_v = 190,000 \text{ ft}^{-1}$ is also considered too
 3 conservative.

4
 5 Reference 3.2.5-5 itself mentions that the LANL Test Report, LA-UR-03-0471, should be
 6 consulted for final recommendations once it is issued. On April 17, 2004, the PWR Industry
 7 became aware that Los Alamos published LA-UR-04-1227, "GSI-191: Experimental Studies of
 8 Loss-of-Coolant-Accident-Generated Debris Accumulation and Head Loss with Emphasis on the
 9 Effects of Calcium Silicate Insulation". A review of that document has been initiated for the
 10 purpose of assessing what, if any, further guidance regarding treatment of Calcium Silicate might
 11 be supported by the tests reported.

13 **3.2.5.3 References**

14
 15 3.2.5-1 NUREG/CR-6808, LA-UR-03-0880, D.V. Rao, Clinton J. Shaffer, M.T. Leonard,
 16 "Knowledge Base for the Effect of Debris on Pressurized Water Reactor Emergency
 17 Core Cooling Sump Performance," U.S. Nuclear Regulatory Commission, February
 18 2003

19 3.2.5-2 NUREG/CR-6224, G. Zigler, J. Bridaeu, D. V. Rao, C. Shaffer, F. Souto, W.
 20 Thomas, "Parametric Study of the Potential for BWR ECCS Strainer Blockage Due
 21 to LOCA Generated Debris," Final Report, U.S. Nuclear Regulatory Commission,
 22 October 1995

23 3.2.5-3 C.J Shaffer, et al., "BLOCKAGE 2.5 Reference Manual," NUREG/CR-6371,
 24 December 1996

25 3.2.5-4 Leonard, M. T., Arup Maji, et. al., "Debris Accumulation & CalSil Head Loss
 26 Testing—Findings & Preliminary Conclusions," Presentation at GSI-191 Public
 27 Meeting, Albuquerque, NM, March 5, 2003

28 3.2.5-5 Shaffer, C. J. et. al., "Debris Accumulation and Head-Loss Data for Evaluating the
 29 Performance of Vertical Pressurized-Water-Reactor Recirculation Sump Screens",

- 1 presented at the NEA/NRC Workshop on Debris Impact on Emergency Coolant
2 Recirculation, Albuquerque, NM, February 25-27, 2004
- 3 3.2.5-6 NEA/CSNI/R (95) 11, “Knowledge Base for Emergency Core Cooling System
4 Recirculation Reliability,” Prepared by U.S. Nuclear Regulatory Commission for the
5 Principal Working Group 1 (PWG-1), International Task Group, Committee on the
6 Safety of Nuclear Installations, Organization for Economic Cooperation and
7 Development (OECD) Nuclear Energy Agency (NEA), February 1996
- 8 3.2.5-7 LA-UR-04-1227, “GSI-191: Experimental Studies of Loss-of-Coolant-Accident-
9 Generated Debris Accumulation and Head Loss with Emphasis on the Effects of
10 Calcium Silicate Insulation,” Los Alamos, April, 2004
- 11

1 **Attachment A**

2
3 **WHITE PAPER**

4 **DEFINING THE ZONE OF INFLUENCE (ZOI) AND MINIMUM COATING DEBRIS**
5 **SIZE FOR DBA-QUALIFIED AND ACCEPTABLE COATINGS**
6 **IN PRESSURIZED WATER REACTOR (PWR) CONTAINMENTS**

7
8 **PREPARED FOR**

9
10 **NUCLEAR ENERGY INSTITUTE (NEI)**

11
12 **PREPARED BY**

13
14 **JON R. CAVALLO, PE, PCS**

15 **VICE PRESIDENT**

16 **CORROSION CONTROL CONSULTANTS & LABS, INC.**

17
18 **APRIL 2, 2004**

1 EXECUTIVE SUMMARY

2
3 This White Paper has been prepared to provide a conservative approach for determining the Zone
4 of Influence (ZOI) and minimum coating debris size for DBA-qualified and Acceptable coatings
5 in Pressurized Water Reactor (PWR) containments.
6

7 The criteria for DBA-qualification or designation as “Acceptable” of coatings applied to
8 systems, structures and components in PWR containments do not provide data concerning
9 coatings exposed to direct impingement of fluids. As such, the ZOI for DBA-qualified coatings
10 or coatings determined to be “Acceptable,” applied to PWR containment surfaces, which results
11 from fluid impingement from the break jet, has not been clearly defined.
12

13 An extensive body of data exists related to removal of industrial protective coatings by high-
14 pressure and ultra-high-pressure waterjetting. Examination of this data and associated industry
15 standards, compiled since the mid-1980’s, reveals that industrial protective coating systems,
16 identical to the DBA-qualified and “Acceptable” coatings applied to systems, structures and
17 components in Pressurized Water Reactor (PWR) containments, require a water jetting pressure
18 of at least 7,000 psig to initiate destruction of sound coatings.
19

20 In the writer’s opinion, protective coating systems which have been successfully DBA-qualified
21 will have a minimum waterjet destruction pressure of no lower than 7,000 psig. This 7,000 psig
22 waterjet destruction pressure can be conservatively used to define a bounding “coating ZOI” for
23 DBA-qualified coatings as 1,000 psig about the DBA pipe rupture.
24

25 All currently-available industrial high-pressure waterjetting equipment systems are equipped
26 with 10 µm filtration to remove particles from discharge water to protect the environment. Based
27 upon the acceptance of this lower-bound of filtration by environmental regulatory bodies
28 throughout the United States, it can be conservatively assumed that debris generated from the
29 destruction of industrial coatings identical to those coatings used in PWR containments to be no
30 less than 10 µm in minor dimension.
31

1 **DISCUSSION**

2

3 The criteria for DBA-qualification of coating systems applied to systems, structures and
4 components in PWR containments are contained in ANSI N101.2, "Protective Coatings (Paints)
5 for Light Water Nuclear Reactor Containment Facilities (Reference A-1)," and its successor
6 document, ASTM D 3911, "Standard Test Method for Evaluating Coatings Used in Light-Water
7 Nuclear Power Plants at Simulated Design Basis Accident (DBA) Conditions (Reference A-2)."
8 Both of these national standards are essentially identical to their requirements for DBA-
9 qualification of coatings:

10

11 1. Fully-cured coated panels are placed in an autoclave chamber. Note that, by test method
12 requirements, the panels are positioned such that they are not subject to direct steam
13 impingement.

14

15 2. Using saturated steam, the autoclave pressure and temperature are adjusted to produce
16 conditions approximating the DBA environment of a utility's containment structure.

17

18 3. After completion of the DBA cycle, each panel is examined. Any disbondment of the
19 coating (including cracking, peeling, delamination and/or complete detachment) is rated
20 unacceptable.

21

22 The only technical difference between ANSI N101.2 and ASTM D3911 is the acceptance criteria
23 for intact blistering, which has no bearing on coating debris production.

24

25 The US Nuclear Regulatory Commission has reviewed ASTM D 3911 and found it
26 "...acceptable to the NRC staff for the...qualification... of protective coatings applied in nuclear
27 power plants," as stated in Regulatory Guide 1.54 Revision 1 (July 2000) (Reference A-3).

28

29 Nuclear plants licensed prior to the issuance of ANSI N101.2 selected and tested coating systems
30 for use in containment by virtue of sound engineering practices. Containment coatings by this

1 pre-ANSI N101.2 process are designated as “Acceptable” as defined in ASTM D5144-00
2 (Reference A-4):

3
4 “*Acceptable Coating or Lining System* – A safety-related coating or lining system for
5 which a suitability for application review which meets the plant licensing requirements
6 has been completed and there is reasonable assurance that, when properly applied and
7 maintained, the coating or lining will not detach under normal or accident conditions.”

8
9 In most cases, the coating products and systems applied to PWR containment structures, systems
10 and components in pre-ANSI N101.2 plants are identical to those used in post-ANSI N101.2
11 plants. These coating materials were system combinations of inorganic zinc, epoxy and epoxy
12 phenolic primers; and epoxy and epoxy phenolic topcoats supplied by Ameron, Carboline, and
13 Keeler & Long. As such, the performance of “Acceptable” coating systems can be equated with
14 the performance of “DBA-qualified” coating systems within the scope of this White Paper.

15
16 As part of the overall regulatory investigation of GSI-191, Savannah River Technology Center
17 (SRTC) was engaged by the US Nuclear Regulatory Commission Division of Engineering
18 Technology Office of Regulatory Research to “...investigate the potential for degradation and
19 failure of such coating systems (safety-related coatings located inside containment, ed.) when
20 subjected to DBA conditions, and to characterize failed coating debris...” (Reference A-5), and,
21 “...investigate the performance and potential for debris formation of Service Level I (safety-
22 related coatings located inside containment, ed.) used in nuclear power plant containment...”
23 (Reference A-6). The two major findings concerning the performance of DBA-qualified coatings
24 in PWR containment service which resulted from the SRTC research are presented in References
25 A-4 and A-5 as follow:

26
27 Reference 5:

28
29 “Properly applied coatings that would contain only minor defects and that have not been
30 subjected to irradiation of 10^9 rads, can be expected to remain fully adhered and intact on

1 a **concrete substrate** (emphasis added, ed.), following exposure to simulated DBA-
2 LOCA conditions.”

3
4 Reference 6:

5
6 “Properly applied coatings that have not been subjected to irradiation of 10^9 rads, can be
7 expected to remain fully adhered and intact on a **steel substrate** (emphasis added, ed.),
8 following exposure to all simulated DBA-LOCA conditions.”

9
10 Thus, the independent research by SRTC on behalf of the USNRC validates the DBA test
11 methodology for containment coatings contained in ANSI N101.2 and ASTM D 3911.

12
13 The test method defined in ANSI N101.2 and ASTM D 3911 does not provide data concerning
14 coatings exposed to direct impingement of fluids, however. As such, the ZOI for DBA-qualified
15 coatings or coatings determined to be “Acceptable,” applied to PWR containment surfaces,
16 which results from fluid impingement from the break jet, was not clearly defined in the past.
17 Understanding the performance of DBA-qualified protective coatings in fluid impingement
18 conditions is necessary to quantify coating debris generation in DBA conditions and its
19 contribution to the overall debris source term.

20
21 An extensive body of data and experience exists related to removal of industrial protective
22 coatings by high-pressure and ultra-high-pressure waterjetting. National standards and related
23 commentary related to coating removal by high-pressure waterjetting have been published under
24 the auspices of SSPC: The Society for Protective Coatings and are contained in the following
25 documents:

- 26
27 1. Good Painting Practice; SSPC Painting Manual Volume 1 (Reference A-7)
28
29 2. Systems and Specifications; SSPC Painting Manual Volume 2 (Reference A-8)
30

1 Reference A-7 contains an entire section on water-process paint and coating removal (“Chapter
2 2.7, Wet Abrasive Blast and Pressurized Water Cleaning (Waterjetting)”), which provides the
3 following information concerning the effects of a high-pressure water jet on industrial coatings:
4

5 A. “At 10,000 psig, the velocity of the water is close to 1,100 ft. /sec., or a fluid jet. The
6 velocity then starts to change the amount of cutting... from a hydraulic action to an
7 erosion action.” This permits correlation with the effects of a high-pressure single-phase
8 water jet as used in industrial surface preparation with a two-phase steam-water jet which
9 might be encountered in a DBA pipe break on industrial coatings.
10

11 B. “Typical cleaning applications for (sic) at various pressures include:
12

- 13 • **10,000-24,000 psig:** ...most paints ...” This sets the lower-bound damage
14 pressure for sound industrial coatings, and similarly DBA-qualified PWR
15 containment coatings, at 10,000 psig.
16

17 Reference A-8 contains the technical standards for high-pressure water blasting. The applicable
18 standard, “Joint Surface Preparation Standard SSPC-SP 12/NACE No. 5, Surface Preparation
19 and Cleaning of Steel and other Hard Materials by High- and Ultrahigh-Pressure Water Jetting
20 Prior to Recoating (© 1995),” provides the following information applicable to determining the
21 lower- bound damage pressure for sound industrial coatings, and similarly DBA-qualified PWR
22 containment coatings:
23

24 A. “2.1.5 **High-Pressure Water Jetting (HP WJ):**

25 HP WJ is cleaning performed at pressures from 70 to 170 MPa (10,000 to 25,000 psi).”
26

27 B. “D1.2 Typically, the water jet nozzle should be held 5 to 25 cm (2 to 10 in.) from the
28 surface being cleaned...”
29

30 The industrial practice requirements related to high-pressure waterjetting at 10,000 psig to
31 25,000 psig fluid pressure for holding the waterjetting nozzle relatively close to the surface to be

1 cleaned reflects industry experience of a relatively small pressure drop between the nozzle tip
2 and the substrate, and provides additional correlation with an conservative lower-bound
3 destruction pressure for industrial coatings of 10,000 psig.

4
5 Limited information is available concerning the use of high-pressure waterjetting for removal of
6 PWR containment coatings. One utility has performed trials of high-pressure waterjetting for
7 removal of standard PWR containment coating systems applied to concrete and steel substrates,
8 and has determined a threshold waterjetting destruction pressure for coatings of 7,000 to 8,000
9 psi (Reference A-9). This information correlates well with the generally available industrial
10 coating removal data.

11
12 Modern waterjetting nozzles are designed to produce turbulent, high energy water droplet flow,
13 either by a single orifice with a diffusion pattern or a multiple orifice nozzle rotating up to 3,000
14 rpm (Reference A-7) to produce optimum coating destruction. As such, the effect of single-
15 phase waterjetting on coatings can be conservatively equated to the effect of the two-phase
16 impingement of fluid from a DBA pipe rupture in a PWR containment with regard to coating
17 destruction.

18
19 In the writer's opinion, protective coating systems which have been successfully DBA-qualified
20 using ANSI N101.2 or ASTM D3911 test methods, and "Acceptable" coating systems essentially
21 identical to those later DBA-qualified, will have a minimum waterjet destruction pressure of no
22 lower than 5,000 psig. This 5,000 psig waterjet destruction pressure can be conservatively used
23 to define a "coating ZOI" for DBA-qualified coatings of 1,000 psig about the DBA pipe rupture.

24
25 All currently-available industrial high-pressure waterjetting equipment systems are equipped
26 with 10 μm filtration to remove particles from both discharge water and thus to protect the
27 environment (Reference A-10). Based upon the acceptance of this lower-bound of filtration by
28 environmental regulatory bodies throughout the United States, it can be conservatively assumed
29 that debris generated from the destruction of industrial coatings identical to those used in PWR
30 containments to be no less than 10 μm in minor dimension. This 10 μm debris minimum
31 dimension would apply to coatings within the 10,000 psig coating ZOI.

1
2 In the writer's opinion, any coating debris produced within the coating ZOI by the destruction of
3 DBA-qualified coatings can be conservatively assumed to be no less than 10 μm in minor
4 dimension.

5
6
7

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