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Michael R. Kansler
President

April 14, 2004
JPN-04-010
NL-04-038
ENO 2.04.028

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

SUBJECT: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
Indian Point Nuclear Generating Units No. 2 and 3
Docket Nos. 50-247, and 50-286
Pilgrim Nuclear Power Station
Docket No. 50-293
Request to Use the 1998 Edition, 2000 Addenda of the American Society of Mechanical Engineers (ASME) Section XI Code Requirements for the Examination of Reactor Vessel Closure Studs

Reference:
1. USNRC letter from Robert A. Gramm to C. Lance Terry, dated September 3, 2003 regarding "Comanche Peak Steam Electric Station, Unit 2, Relief Request No. A-9, From the Requirements of the ASME Boiler and Pressure Vessel Code, Section XI, Concerning Second 10-Year Inservice Inspection Interval (TAC No. MB7946)."
2. Code of Federal Regulations, 10CFR50 (Revised as of January 1, 2003)

Dear Sir or Madam:

Pursuant to 10CFR50.55a(g)(4)(iv), Entergy Nuclear Operations, Inc. (ENO) hereby requests the Nuclear Regulatory Commission to approve the enclosed requests to use the 1998 Edition, 2000 Addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) Table IWB-2500-1, Category B-G-1, requirements for the examination of the reactor vessel closure studs, when removed.

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The Inservice Inspection Programs, under the current code of record (1989 Edition, No Addenda), require the reactor vessel closure studs be examined by both volumetric and surface examination methods. Entergy is requesting approval to use the 1998 Edition, 2000 Addenda of the ASME Section XI Code, which allows either the volumetric or the surface examination methods. The later ASME Code Edition / Addenda have been incorporated into 10CFR50.55a by reference (Reference 2).

Enclosed are four (4) requests for James A. FitzPatrick Nuclear Power Plant (JAF, Enclosure 1), Indian Point Nuclear Generating Unit No. 2 (IP2, Enclosure 2), Indian Point Nuclear Generating Unit No. 3 (IP3, Enclosure 3), and Pilgrim Nuclear Power Station (Enclosure 4).

Approval is requested by October 15, 2004 to support the IP2 outage. Due to the similarity of these requests, it is also requested that approval for JAF, IP3, and Pilgrim be granted at the same time.

A similar request was approved for Comanche Peak Steam Electric Station, Unit 2 (Reference 1).

There are no new commitments made in this letter. If you have any questions, please contact Ms. Charlene Faison at 914-272-3378.

Very truly yours,

Michael R. Kansler
President
Entergy Nuclear Operations, Inc.

Enclosures:
1. James A. FitzPatrick Nuclear Power Plant, Request No. R-33
2. Indian Point Generating Station Unit No. 2, Request No. R-71
3. Indian Point Generating Station Unit No. 3, Request No. R 3-40 (A)
4. Pilgrim Nuclear Power Station, Request No. R-41

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ENCLOSURE 1 TO JPN-04-010 / NL-04-038 / ENO 2.04.028

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

Request to Use
1998 Edition, 2000 Addenda of the ASME Section XI Code Requirements
For the Examination of Reactor Vessel Closure Studs

ENTERGY NUCLEAR OPERATIONS, INC.
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333
DPR-59

JAMES A. FITZPATRICK
THIRD TEN-YEAR INTERVAL INSERVICE INSPECTION PROGRAM
REQUEST No. R-33

Proposed Use of Subsequent ASME Code Edition and Addenda
In Accordance with 10CFR50.55a(g)(4)(iv)

1. ASME Code Component(s) Affected

Code Class:	1
References:	IWB-2500, Table IWB-2500-1
Examination Category:	B-G-1
Item Number:	B6.30
Parts Examined:	Reactor Vessel Closure Studs (when removed)

2. Applicable Code Edition and Addenda

The Code of Record for the Third Inservice Inspection Interval is ASME Section XI Code, 1989 Edition, No Addenda.

3. Proposed Subsequent Code Edition and Addenda (or Portion)

Entergy proposes to use the 1998 Edition, 2000 Addenda of the ASME Section XI, Table IWB-2500-1, Category B-G-1 Item B6.30 (Reactor Vessel Closure Studs, when removed) requirements for the examination of Closure Studs, when removed. The 1998 Edition, up to the 2000 Addenda of the ASME Section XI Code has been incorporated by reference in 10 CFR 50.55a(b) (reference 2003 Code of Federal Regulations, 10CFR50.55a(b)(2), revised as of January 1, 2003).

4. Related Requirements

The changes to Table IWB-2500-1 for Category B-G-1, Item B6.30, Reactor Vessel Closure Studs (when removed) examination criteria does not affect other parts of the Code. There are no limitations or modifications, which are associated with Category B-G-1, Item B6.30 in the 1998 Edition, 2000 Addenda of the ASME Section XI Code, addressed in 10 CFR 50.55a(b). There are no related requirements in the 1998 Edition, 2000 Addenda of the Code for Category B-G-1, Item B6.30 that would need to be implemented. Therefore, the proposed change will result in an acceptable level of quality and safety.

5. Duration of Proposed Alternative

It is proposed to use the alternative for the remainder of the Third Inservice Inspection Interval for James A. FitzPatrick Nuclear Power Plant.

ENCLOSURE 2 TO JPN-04-010 / NL-04-038 / ENO 2.04.028

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

Request to Use
1998 Edition, 2000 Addenda of the ASME Section XI Code Requirements
For the Examination of Reactor Vessel Closure Studs

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
DOCKET NO. 50-247
DPR-26

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2
THIRD 10-YEAR INSERVICE INSPECTION INTERVAL
REQUEST NO. R-71

Proposed Use of Subsequent ASME Code Edition and Addenda
In Accordance with 10CFR50.55a(g)(4)(iv)

1. ASME Code Component(s) Affected

Code Class:	1
References:	IWB-2500, Table IWB-2500-1
Examination Category:	B-G-1
Item Number:	B6.30
Parts Examined:	Reactor Vessel Closure Studs (when removed)

2. Applicable Code Edition and Addenda

The Code of Record for the Third Inservice Inspection Interval is ASME Section XI Code, 1989 Edition, No Addenda.

3. Proposed Subsequent Code Edition and Addenda (or Portion)

Entergy proposes to use the 1998 Edition, 2000 Addenda of the ASME Section XI, Table IWB-2500-1, Category B-G-1, Item B6.30 (Reactor Vessel Closure Studs, when removed) requirements for the examination of Closure Studs, when removed. The 1998 Edition, up to the 2000 Addenda of the ASME Section XI Code has been incorporated by reference in 10 CFR 50.55a(b) (reference 2003 Code of Federal Regulations, 10CFR50.55a(b)(2), revised as of January 1, 2003).

4. Related Requirements

The changes to Table IWB-2500-1 for Category B-G-1, Item B6.30, Reactor Vessel Closure Studs (when removed) examination criteria does not affect other parts of the Code. There are no related requirements in the 1998 Edition, 2000 Addenda of the ASME Section XI Code that would need to be implemented. Therefore, the proposed change will result in an acceptable level of quality and safety.

5. Duration of Proposed Alternative

It is proposed to use the alternative for the remainder of the Third Inservice Inspection Interval for Indian Point Nuclear Generating Unit No. 2 (IP2).

ENCLOSURE 3 TO JPN-04-010 / NL-04-038 / ENO 2.04.028

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

Request to Use
1998 Edition, 2000 Addenda of the ASME Section XI Code Requirements
For the Examination of Reactor Vessel Closure Studs

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
DOCKET NO. 50-286
DPR-64

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3
THIRD 10-YEAR INSERVICE INSPECTION INTERVAL
REQUEST NO. R 3-40 (A)

Proposed Use of Subsequent ASME Code Edition and Addenda
In Accordance with 10CFR50.55a(g)(4)(iv)

1. ASME Code Component(s) Affected

Code Class:	1
References:	IWB-2500, Table IWB-2500-1
Examination Category:	B-G-1
Item Number:	B6.30
Parts Examined:	Reactor Vessel Closure Studs (when removed)

2. Applicable Code Edition and Addenda

The Code of Record for the Third Inservice Inspection Interval is ASME Section XI Code, 1989 Edition, No Addenda.

3. Proposed Subsequent Code Edition and Addenda (or Portion)

Entergy proposes to use the 1998 Edition, 2000 Addenda of the ASME Section XI, Table IWB-2500-1, Category B-G-1 Item B6.30 (Reactor Vessel Closure Studs, when removed) requirements for the examination of Closure Studs, when removed. The 1998 Edition, up to the 2000 Addenda of the ASME Section XI Code has been incorporated by reference in 10 CFR 50.55a(b) (reference 2003 Code of Federal Regulations, 10CFR50.55a(b)(2), revised as of January 1, 2003).

4. Related Requirements

The changes to Table IWB-2500-1 for Category B-G-1, Item B6.30, Reactor Vessel Closure Studs (when removed) examination criteria does not affect other parts of the Code. There are no related requirements in the 1998 Edition, 2000 Addenda of the ASME Section XI Code that would need to be implemented. Therefore, the proposed change will result in an acceptable level of quality and safety.

5. Duration of Proposed Alternative

It is proposed to use the alternative for the remainder of the Third Inservice Inspection Interval for Indian Point Nuclear Generating Unit No. 3 (IP3).

ENCLOSURE 4 TO JPN-04-010 / NL-04-038 / ENO 2.04.028

PILGRIM NUCLEAR POWER STATION

Request to Use
1998 Edition, 2000 Addenda of the ASME Section XI Code Requirements
For the Examination of Reactor Vessel Closure Studs

ENTERGY NUCLEAR OPERATIONS, INC.
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293
DPR-35

PILGRIM NUCLEAR POWER STATION
THIRD 10-YEAR INSERVICE INSPECTION INTERVAL
REQUEST NO. R-41

Proposed Use of Subsequent ASME Code Edition and Addenda
In Accordance with 10CFR50.55a(g)(4)(iv)

1. ASME Code Component(s) Affected

Code Class:	1
References:	IWB-2500, Table IWB-2500-1
Examination Category:	B-G-1
Item Number:	B6.30
Parts Examined:	Reactor Vessel Closure Studs (when removed)

2. Applicable Code Edition and Addenda

The Code of Record for the Third Inservice Inspection Interval is ASME Section XI Code, 1989 Edition, No Addenda.

3. Proposed Subsequent Code Edition and Addenda (or Portion)

Entergy proposes to use the 1998 Edition, 2000 Addenda of the ASME Section XI, Table IWB-2500-1, Category B-G-1 Item B6.30 (Reactor Vessel Closure Studs, when removed) requirements for the examination of Closure Studs, when removed. The 1998 Edition, up to the 2000 Addenda of the ASME Section XI Code has been incorporated by reference in 10 CFR 50.55a(b) (reference 2003 Code of Federal Regulations, 10CFR50.55a(b)(2), revised as of January 1, 2003).

4. Related Requirements

The changes to Table IWB-2500-1 for Category B-G-1, Item B6.30, Reactor Vessel Closure Studs (when removed) examination criteria does not affect other parts of the Code. There are no related requirements in the 1998 Edition, 2000 Addenda of the ASME Section XI Code that would need to be implemented. Therefore, the proposed change will result in an acceptable level of quality and safety.

5. Duration of Proposed Alternative

It is proposed to use the alternative for the remainder of the Third Inservice Inspection Interval for Pilgrim Nuclear Power Station.