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DOCUMENTS THAT HAVE BEEN CLEARED BY NRC

Nuregs:

Nureg-0017 Calculation of Releases of Radioactive Materials in Gaseous and Liquid Effluents from Pressurized Water Reactors (PWR-GALE CODE).

Nureg-492 Fault Tree Handbook

Nureg-726: Preliminary Analysis of the Effect of Fatigue Loading and Crack Propagation (cleared by review of letter 4293)

Nureg-794 Protection of Unclassified Safeguards Information (by Bill Reckley 12/13)

Nureg-800, Standard Review Plan (SRP)

Nureg-908 Acceptance Criteria for the Evaluation of Nuclear Power Reactor Security Plans (by Bill Reckley 12/13)

Nureg-1275, vol. 13: Evaluation of Air-Operated Valves at US Light-Water Reactors

Nureg-1463: Regulatory Analysis for the Resolution of Generic Safety Issue 105: Interfacing System Loss-of-Coolant Accident in Light-Water Reactors (cleared by review of letter 4547)

Nureg-1465: Accident Source Terms for Light-Water Nuclear Plants (cleared by review by Margie Kotzalas, November 29, 2001)

Nureg-1576: Review of Fatigue Crack Growth of Pressure Vessel (cleared by review of letter 4293)

Nureg-CP-44, Vol. 1 & 2: Proceedings of the IAEA Specialists Meeting on Subcritical Crack Growth (cleared by review of letter 4293)

Nureg-CP-67, Vol. 1 & 2: Proceedings of the Second IAEA Specialists Meeting on Subcritical Crack Growth (cleared by review of letter 4293)

Nureg-CP-112: Proceedings of the Third International Atomic Energy Agency Specialists Meeting on Subcritical Crack Growth (cleared by review of letter 4293)

Nureg-CR-0136: Modeling of Crack Growth In Oxidized Zircaloy Cladding (cleared by review of letter 4293)

Nureg-CR-0969: Fatigue Crack Growth of A508 Steel (cleared by review of letter 4293)

Nureg-CR-1278 Handbook of Human Reliability Analysis with Emphasis on Nuclear Power Plant Applications (Cleared by R. Hsu 10/31/01)

Nureg-CR-1861: LWR Pressure Vessel Surveillance Dosimetry Improvement Program: PCA Experiments and Blind Test (cleared by review of letter 4183)

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Nureg-CR-2326 Calculations of Reactor Accident Consequences Version 1 CRAC1: Computer Code. User's Guide. 2/83. (Cleared by M. Kotzalas 12/26/01)

Nureg-CR-3059: Parametric Calculations of Fatigue Crack Growth (cleared by review of letter 4293)

Nureg-CR-3089: Specimen Geometry an Extended Crack Growth Effects (cleared by review of letter 4293)

Nureg-CR-3176: Crack Growth Evaluation for Small racks in Reactor Coolant Piping (cleared by review of letter 4293)

Nureg-CR-3230: Effects of Temperature on Fatigue Crack Growth (cleared by review of letter 4293)

Nureg-CR-3294: Fatigue Crack Growth Rates (cleared by review of letter 4293)

Nureg-CR-3546: The Temperature Dependence of Fatigue Crack Growth Rates (cleared by review of letter 4293)

Nureg-CR-3660, Vol. 2, 4: Probability of Pipe Failure In Reactor Coolant Loops of Westinghouse PWR Plants...(cleared by review of letter 4293)

Nureg-CR-3663, Vol, 2: Probability of Pipe Failure in the Reactor Coolant Loops (cleared by review of letter 4293)

Nureg-CR-3833: Behavior of Subcritical and Slow-Stable Crack Growth (cleared by review of letter 4293)

Nureg-CR-3945: Fatigue Crack Growth Rates (cleared by review of letter 4293)

Nureg-CR-4121: Effects of Sulfur Chemistry and Flow Rate...(cleared by review of letter 4293)

Nureg-CR-4221: An Evaluation of Stress Corrosion Crack Growth (cleared by review of letter 4293)

Nureg-CR-4422: A Review of the Models and Mechanisms for Environmentally Assisted Crack Growths (cleared by review of letter 4293)

Nureg-CR-4467: Relative Importance of Individual Elements to Reactor Accident Consequences Assuming Equal Release Fractions (cleared by review Margie Kotzalas, November 29, 2001)

Nureg-CR-4486 VISA II computer code for predicting the probability of reactor pressure vessel failures (cleared by review of letter #4413)

Nureg-CR-4573: Elastic-Plastic Finite Element Analysis of Crack Growth...(cleared by review of letter 4293)

Nureg-CR-4575: Predictions of J-R Curves with Large Crack Growth...(cleared by review of letter 4293)

Nureg-CR-4576: Evaluation of Surveillance Capsule and Reactor Cavity Dosimetry from H.B. Robinson Unit-2 Cycle 9 (cleared by review of letter 4183)

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- Nureg-Cr-4639 Nuclear Computerized Library for Assessing Reactor Reliability (NUCLARR) all volumes
- Nureg-CR-4723: Application of a Two-Mechanism Model for Environmentally Assisted Crack Growth (cleared by review of letter 4293)
- Nureg-CR-4724: Fatigue Crack Growth Rates in Pressure Vessel and Piping Steels...Final Report (cleared by review of letter 4293)
- Nureg-CR-4792: Probability of Failure in BWR Reactor Coolant Piping, Vol. 2 (cleared by review of letter 4293)
- Nureg-CR-4828: Fatigue Crack Growth (cleared by review of letter 4293)
- Nureg-CR-4840 Procedures for external event core damage frequency analysis for NUREG-1150 (ok per Ray Shu 11/16/01)- (This has generic info, whereas NUREG-1150 provides the detailed info and is **not** public)
- Nureg-CR-4863: Surface Spectroscopy of Pressure Vessel Steel Fatigue Fracture...(cleared by review of letter 4293)
- Nureg-CR-4952: Experimental Study of Fillet Weld Undercut Effects (cleared by review of letter 4293)
- Nureg-CR-4981 Safety Assessment of the Use of Graphite in NPP
- Nureg-CR-5020: Summary of Environmentally Assisted Crack-Growth Studies (cleared by review of letter 4293)
- Nureg-CR-5238: Development of an Engineering Definition of the Extent of J Singularity Controlled Crack Growth (cleared by review of letter 4293)
- Nureg-CR-5434: Anchor Bolt Behavior and Strength During Earthquakes (cleared by review of letter 4306)
- Nureg-CR-5563: A Technical Basis for Revision to Anchorage Criteria (cleared by review of letter 4306)
- Nureg-CR-6078: Analysis of Crack Initiation and Growth in the High Level Vibration Test at Tadotsu (cleared by review of letter 4293)
- Nureg-CR-6115: Pressure Vessel Fluence Calculation Benchmark Problem and Solutions (cleared by review of letter 4183)
- Nureg-CR-6176, and its errata sheets: Review of Environ Effects on Fatigue Crack Growth (cleared by review of letter 4293)
- Nureg-CR-6264: Validity Limits in J-Resistance Curve Determination...(cleared by review of letter 4293)
- Nureg-CR-6338: Resolution of the Direct Containment heating Issue for All Westinghouse Plants with Large Dry Containments or Subatmospheric Containments (cleared by review by Margie Kotzalas, November 29, 2001)
- Nureg-CR-6365 Steam generator tube failures (cleared by review of letter #4179)

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Nureg-CR-6440: The Effects of Cyclic and Dynamic Loading on the Fracture Resistance...(cleared by review of letter 4293)

Nureg-CR-6453: H.B. Robinson-2 Pressure Vessel Benchmark (cleared by review of letter 4183)

Nureg-CR-6454: Pool Critical Assembly Pressure Vessel Facility Benchmark (cleared by review of letter 4183)

Nureg-CR-6463 Review Guidelines for Software Languages for use in Nuclear Power Plant Safety Systems (cleared by review of ltr 4313)

Nureg-CR-6608 Summary and Evaluation of Low-Velocity Impact Test of Solid Steel Billet onto Concrete Pads (cleared by R. Hsu 11/13)

Nureg-CR-6717: Environmental Effects on Fatigue Crack Initiation...(cleared by review of letter 4293)

Nureg-CR-6721: Effects of Alloy Chemistry, Cold Work and Water Chemistry on Corrosion Fatigue..(cleared by review of letter 4293)

Nureg-CR-6728 "Technical Basis for Revision of Regulatory Guidance on Design Ground Motions: Hazard- and Risk-consistent Ground Motion Spectra Guidelines" has been reviewed and determined to be acceptable for public release 12/27/01.

**Generic Communications:**

IN-98-22 Deficiencies Identified During NRC Design Inspections (Cleared by R. Hsu 11/6/01)

**Regulatory Guides:**

Reg Guide 1.174

Reg Guide 1.175

Reg Guide 1.176      1.174 - 1.182 cleared by review, see Ltr 4175

Reg Guide 1.177

Reg Guide 1.178

Reg Guide 1.182

Reg Guide 1.183

DG-1102

**MISC:**

Capacity factors

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NRC Enforcement Manual - only paper copy available at this time

NRC Inspection Manual - all sections ok per Jerry Klingler 10/29

Secy-93-102 Review and Update of Options to Protect Against Malevolent Use of Vehicles and Related Threat Information (OK by R. Hsu 10/25/01)

January 14, 2002