

2003 NRC EXAM

JPM BASIS INFORMATION

ASK: 1250440201 Perform Event Control Actions for a Control Room Fire CASK STANDARD: Diesel Generator "B" supplying loads on PBB-S04 L/A: 4.2-068-AA1.10 K/A RATING: RO: 3.7 SRO: 3.9 L/A: 4.2-068-AA1.31 K/A RATING: RO: 3.9 SRO 4.0 LAPPLICABLE AO/RO/SRO VALIDATION 25 min. OSITION(S): TIME: TIME CRITICAL - 15 MINUTES FROM THE TIME D/G "B" IS STARTED UNTIL SP "B" IS STARTED LEFERENCES: 40AO-9ZZ19, Control Room Fire	
UGGESTED TESTING ENVIRONMENT: SIMULATOR PLANT X	_
APPROVAL	
DEVELOPER: T. Stahler TECH REVIEW: LEVISION DATE: 04/08/03 APPROVAL:	
TESTING METHOD	_
CTUAL TESTING ENVIRONMENT: SIMULATOR PLANT	
TESTING METHOD: SIMULATE PERFORM	
EVALUATION	
XAMINEE NAME:	
VALUATOR NAME: (print)	
(print)	
ATISFACTORY UNSATISFACTORY	
Time Start Time Stop	
EMEDIAL TRAINING REQUIRED? YESNO	



2003 NRC EXAM

1. SIMULATOR SETUP:

A. IC#: N/A

B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.		
2.		
3.		
4.		

- C. SPECIAL INSTRUCTIONS:
- None
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

• None



2003 NRC EXAM

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operation of in-plant equipment is to be **SIMULATED ONLY**, **DO NOT OPERATE** any equipment.
- Inform the control room staff of any discovered deficiencies.
- Vocalize the following items when simulating equipment operation.
 - a. Condition / indication you would check to determine equipment status.
 - b. Component you would operate indicating direction of travel.
 - c. Expected change in indications as a result of operation.
- You may use any source of information normally available.

INITIATING CUE:

- The control room has been evacuated due to a fire.
- There has been a loss of offsite power. No automatic start and loading of the Emergency Diesel Generators, or load shed has occurred.
- The CRS directs you to complete Appendix E of 40AO-9ZZ19 as the D/G AO to manually start and load the "B" Diesel Generator.
- Assume you have a portable lantern.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- The complete load shed and manual sequencing of loads will not be performed.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Supervisor of in-plant JPM performance.
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Elements and Standards are met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.

SAFETY CONSIDERATIONS:

None



STEP	ELEMENT	STANDARD
1.	 Direct the Lower Auxiliary Building Operator to Perform Appendix H, step 	Same as Element.
	2.	If requested cue: Lower Auxiliary Building Operator reports Appendix H, step 2 has been completed.
SAT	UNSAT (UNSAT r	equires comments)
STEP	ELEMENT	STANDARD
2.	Place the following Control Room Disconnect Switches in 'LOCAL' (100 ft	Examinee simulates placing the following handswitches in the "local" position:
	Control Bldg Switchgear Room B).	NOTE: See CUEs below as switches are being manipulated.
	 PHB-M3209, Battery Charger D PKD- H14. 	PHB-M3209, Battery Charger D PKD-H14 handswitch 'C' is in 'Local'
	 PHB-M3205, Control Room Circuits Disconnect Switches (4 switches). 	PHB-M3205 is in 'Local' PHB-M3205 is in 'Local' PHB-M3205 is in 'Local' PHB-M3205 is in 'Local'
		If requested CUE: Evaluator may cue switches in "local" position either individually as manipulated, or as a group when complete.
SAT	UNSAT (UNSAT r	equires comments)
COMME	ENTS:	



	ELEMENT	STANDARD
STEP 3.	Ensure that Both of the following breakers are open:	Examinee simulates moving breaker switch to open
	 PHB-M3209, Battery Charger D PKD-H14 PHB-M3210, To Voltage Regulator for 120VAC Vital Dist Panel PND-V28 	 If requested cue: PHB-M3209 breaker is in open position; all lights are out. PHB-M3210 breaker is in open position; all lights are out
SAT	UNSAT (UNSAT r	requires comments)
CEPP	ELEMENT	STANDARD
STEP 4. *	Place all of the disconnects switches on DGB-C01, DG Disconnect cabinet in	Examinee simulates placing the following handswitches in the "local" position:
	'LOCAL' 100 ft Control Bldg Swicthgear room B)	If requested CUE: Evaluator may cue switches in "local" position either individually as manipulated, or as a group when complete.
		J-DGB-HS-2A in local
		J-DGB-HS-2B in local
		E-PEB-HS-2 in local
		J-HDB-HS-14A in local
		J-DFB-HS-22C in local
SAT	UNSAT (UNSAT I	requires comments)
COMME	ENTS:	



STEP	ELEMENT	STANDARD
5. *	Ensure that the disconnect switches for ALL of the following breakers on PBB-S04 are in 'LOCAL'.	Examinee simulates placing the following handswitches in LOCAL:
	PBB-S04S	Disconnect (CS-3) in LOCAL If requested CUE: PBB-S04S disconnect is in LOCAL
	PBB-S04N	Disconnect (CS-3) in 'LOCAL'. If requested CUE: PBB-S04N disconnect is in LOCAL.
	PBB-S04M	Disconnect (CS-3) in LOCAL If requested CUE: PBB-S04M disconnect is in LOCAL.
	PBB-S04L	Disconnect (CS-3) in LOCAL. If requested CUE: PBB-S04L disconnect is in LOCAL.
	PBM-S04K	Disconnect (CS-3) in. If requested CUE: PBB-S04K disconnect is in LOCAL.
	PBB-S04J	Disconnect (CS-3) in LOCAL. If requested CUE: PBB-S04J disconnect is in LOCAL.
	PBB-S04H	Disconnect (CS-3) in LOCAL. If requested CUE: PBB-S04H disconnect is in LOCAL.
	PBB-S04G	Disconnect (CS-3) in LOCAL. If requested CUE: PBB-S04G disconnect is in LOCAL.
COMMEN	TS:	Disconnect (CS-3) in LOCAL.



PBB-S04F	If requested CUE: PBB-S04F disconnect is in LOCAL.
PBB-S04C	Disconnect (CS-3) in LOCAL. If requested CUE: PBB-S04C disconnect is in LOCAL.
PBB-S04B	Disconnect (CS-3) in LOCAL. If requested CUE: PBB-S04B disconnect is in LOCAL.
SATUNSAT	(UNSAT requires comments)
COMMENTS:	



STEP	ELEMENT	STANDARD
6. *	Ensure that ALL of the following breakers	For any closed Breaker Examinee simulates
	on PBB-S04 are open:	turning the Local hand switch to Open.
	PBB-S04B	PBB-S04B Breaker is open. If requested cue: PBB-S04B Breaker Red
		Light off, Green Light on.
	PBB-S04C	PBB-S04C Breaker is open.
		If requested cue: PBB-S04C Breaker Red
		Light off, Green Light on.
	PBB-S04F	PBB-S04F Breaker is open.
		If requested cue: PBB-S04 F Breaker Red
		Light off, Green Light on.
	PBB-S04G	PBB-S04G Breaker is open.
		If requested cue: PBB-S04G Breaker Red
		Light off, Green Light on.
	PBB-S04K	PBB-S04K Breaker is open.
		If requested cue: PBB-S04K Breaker Red
		Light off, Green Light on.
	PBB-S04L	PBB-S04L Breaker is open.
		If requested cue: PBB-S04L Breaker Red
		Light off, Green Light on.
	PBB-S04M	PBB-S04M Breaker is open.
		If requested cue: PBB-S04M Breaker Red
		Light off, Green Light on.
	PBB-S04S	PBB-S04S Breaker is open.
		If requested cue: PBB-S04S Breaker Red
		Light off, Green Light on.
SAT	UNSAT (UNSAT requ	uires comments)
COMMENT		
COMMENT	5:	



STEP		ELEMENT		STANDARD
7.	*	Place all of the following disconnect	t	Disconnect switches are placed in 'LOCAL'
/ .		switches in 'LOCAL'	•	Disconnect switches are placed in EOCILE
		Switches in LOCAL		If requested cue:
		- CC 2/D2 - DCD 12/D1		CS-2/B2 on PGB-L36B1 is in 'local'
		• CS-2/B2 on PGB-L36B1		
		• CS-1/B2 on PGB-L34B1		CS-1/B2 on PGB-L34B1 is in 'local'
		 CS-1/B2 on PGB-L32B1 		CS-1/B2 on PGB-L32B1 is in 'local'
		 CS-1/C4 on PGB-L32C1 		CS-1/C4 on PGB-L32C1 is in 'local'
SAT		UNSAT	(UNSAT requ	ires comments)
STEP		ELEMENT		STANDARD
8.	*	Open PBG-L32C4, Charging Pump	2	Examinee simulates opening PBG-L32C4 by
•		CHB-P01.		depressing the stop pushbutton.
		CIID I VI.		depressing the stop pushoutton.
				When requested cue: PBG-L32C4: Green light
				is on and Red light is off.
SAT		UNSAT	(LINCAT room	ires comments)
SA1		UNSAT	(UNSAT Tequ	ires comments)
STEP		ELEMENT		STANDARD
9.	*	Manually start D/G 'B' by pressing		Examinee simulates starting 'B' D/G by
		EMERGENCY START (SIMULAT	ED	depressing DGB-HS-31.
		LOP), DGB-HS-31, push button.		
				Inform CUE: D/G 'B' started.
				NOTE: Time critical portion starts here.
				1,0 120 1 mile of them position distribution.
				START Time
SAT		UNSAT	(IJNSAT regu	ires comments)
		01(0111	(CINDIII IOQU	nes comments)
COM	MEN	TC.		
COMI	VILIN	15:		



	2000 11110 211	
STEP	ELEMENT	STANDARD
10.	Check that Both of the following conditions	Examinee identifies indication of Diesel
	for closing the DG Output Breaker are met:	Generator control cabinet.
	• Discal Consenter Divoltage is 2740	
	 Diesel Generator B voltage is 3740 – 4580 	Inform Cue: Diesel Generator B voltage is
	D: 10 . D0 . 700	4220 V; frequency is 60 Hz
	Diesel Generator B frequency is 58.8- 61.2 Hz.	
	01.2 112.	
SAT	UNSAT (UNSAT re	equires comments)
	(67,6777	vquin vo voiminanus)
STEP	ELEMENT	STANDARD
11.	Close breaker PBB-S04B, Diesel Generator	Examinee simulates closing PBB-S04B.
	PEB-G02, using the local control switch.	C
	-	INFORM CUE: Nothing changed when
		Handswitch was taken to 'close'. PBB-S04B
		Green light is lit at normal brightness, Red light
		is out.
		If Requested Cue: All indicating lights indicate
		as you see them. (note: this cue should indicate
		to the examinee that there is not a DC control
		problem.)
		IAD ALC CDC II
		If Requested Cue: CRS directs you to
		manually close PBB-S04B.
SAT	UNSAT (UNSAT ro	equires comments)
	CHOITI (CHOITI	squires comments)
COMME	ENTS:	



STEP 12.	*	If b	EMENT reaker PBB-S04B, Diesel Generator 3-G02 will be closed manually, en perform the following: Check the closing spring indicates		STANDARD Note the indications for this step are located inside the breaker cubical. The evaluator may ask for a description of component locations or, with control room concurrence, locate components on the spare breaker. If Requested Cue: The closing spring indicates		
			"CHGD"		charged.		
		b)	Press the 'MANUAL CLOSE' plung for the breaker.	ger	If Requested Cue: PBB-S04B Closed.		
SAT			UNSAT (U	NSAT requii	res comments)		
COM	MENT	S:					



STEP 13.	*	ELEMENT Start Spray Pond Pump 'B' by using control switch CS-1 at PBB-S04C.	STANDARD Examinee simulates positioning 'local' breaker PBB-S04C CS-1 handswitch to START.
			If requested CUE: PBB-S04C indicates Red light ON, green light OFF
			NOTE: Spray pond pump must be started within 15 minutes following D/G start with no load.
			NOTE: TIME CRITICAL PORTION ENDS HERE.
			FINISH Time
			Time recorded in step 9 till finish time must be less than 15 minutes.
			Inform Cue: Another operator will complete this appendix.
SAT		UNSAT (UNSAT re	equires comments)
			NORMAL TERMINATION POINT
COM	MEN	TS:	



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RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
2	10/10/96	6	New Format
3	10/11/96	3,6	More format changes per OTG-02
04	01/03/97	6	Task Standard Change
05	04/08/03	3	Procedure change.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



2003 NRC EXAM

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

- Operation of in-plant equipment is to be **SIMULATED ONLY**, **DO NOT OPERATE** any equipment.
- Inform the control room staff of any discovered deficiencies.
- You may use any source of information normally available.

INITIATING CUE:

- The control room has been evacuated due to a fire.
- There has been a loss of offsite power. No automatic start and loading of the Emergency Diesel Generators, or load shed has occurred.
- The CRS directs you to complete Appendix E of 40AO-9ZZ19 as the D/G AO to manually start and load the "B" Diesel Generator.
- Assume you have a portable lantern.

SAFETY CONSIDERATIONS:

• None

THIS JPM CONTAINS A TIME CRITICAL ELEMENT.



JPM BA	ASIS INFOR	VIATION		
TASK: 1100020401, Operate CEDMCS TASK STANDARD: Transfer CEA's to K/A: 3.1 001-A2.14 K/A: APPLICABLE POSITION(S): RO/SRO REFERENCES: 40AL-9SF01 Local Alarn SUGGESTED TESTING ENVIRONMEN	the hold bus K/A RATIN K/A RATIN VALI Panel J-SFN-C01	DATION TIME D Responses	SRO: 15 minutes	3.9 X
	APPROVAI			
DEVELOPER: W. Drey REVISION DATE: 7/13/00	TECH RI APPROV			
TE	STING MET	HOD		
ACTUAL TESTING ENVIRONMENT:	SIMULATOR _		PLANT	
TESTING METHOD: SIMULATE	P	ERFORM		
]	EVALUATIO)N		
EXAMINEE NAME:				
EVALUATOR NAME:		(print)		
		(print)		
SATISFACTORY UNS	SATISFACTORY			
Time Start Time Stop		_		
REMEDIAL TRAINING REQUIRED? (SEE 15TD-0TR03)	YES	NO		



NRC EXAM 2003

1. SIMULATOR SETUP:

A. IC#: NA

B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	N/A	
2.		
3.		
4.		

- C. SPECIAL INSTRUCTIONS:
- None
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

• Copy of 40AL-9SF01.



NRC EXAM 2003

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operation of in-plant equipment is to be **SIMULATED ONLY**, **DO NOT OPERATE** any equipment.
- Inform the control room staff of any discovered deficiencies.
- Vocalize the following items when simulating equipment operation.
 - a. Condition / indication you would check to determine equipment status.
 - b. Component you would operate indicating direction of travel.
 - c. Expected change in indications as a result of operation.
- You may use any source of information normally available.

INITIATING CUE:

- With the plant at 100% power the control room has received a CEDMCS Trouble alarm. Continuous Gripper High Voltage is indicated. The CRS directs you to place the affected subgroup on the hold bus.
- This is a time critical JPM.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW NUREG-1021.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Manager of in-plant JPM performance.
- Comply with the REP. Do not enter contaminated, airborne, or high radiation areas. You may be required to discuss actions to be taken.

SAFETY CONSIDERATIONS:

- Proper personal protective equipment
- Slip/Fall hazard on stairways.
- Pinch points at doorways.
- Equipment is located in high noise areas.



STEP	ELEMENT	STANDARD
1.	Proceed to CEDMCS room and obtain alarm response procedure.	Examinee goes to CEDMCS room and obtains copy of 40AL-9SF01.
		TIME START:
		NOTE: Start time is when examinee has entered the RCA.
SAT	UNSAT (UNSAT	requires comments)
STEP 2.	ELEMENT Evaluate supervisory panels for alarms and indications.	STANDARD Examinee evaluates local panel for indications and alarms.
		Inform Cue: When Examinee looks at SFN-C01C for subgroup 16 CUE: Red LED's 17 and 19 for continuous gripper high voltage on subgroup 16 are lit. No other LED's are lit.
SAT	UNSAT (UNSAT	requires comments)
STEP 3.	ELEMENT Notify the Reactor Operator of the alarm.	STANDARD Examinee contacts control room and notifies them of alarm.
		Inform Cue: Reactor Operator acknowledges communication and instructs you to continue with alarm response and place the affected subgroup on the hold bus.
SAT	UNSAT (UNSAT	requires comments)
COMME	ENTS:	



STEP	ELEMENT	STANDARD
4.	Determine number of CEDM subgroups	Examinee determines subgroup 16 is the only
-1.	affected by checking for lit CGHV LED's	subgroup affected.
	on the bays of each cabinet where CEA	suogroup universu.
	subgroup power assemblies exist.	If requested CUE: Red LED's 17 and 19 for
	subgroup power assembles exist.	continuous gripper high voltage on subgroup
		16 are lit. No other LED's are lit.
		To are he two other LED's are he.
SAT	UNSAT (UNS	SAT requires comments)
	(6116	arr requires comments)
STEP	ELEMENT	STANDARD
5.	It is determined that only one subgroup is	Examinee determines only subgroup 16 is
	affected.	affected.
		If requested CUE, CDS instructs you to
		If requested CUE: CRS instructs you to place subgroup 16 on the hold bus IAW the
		alarm response procedure.
		alar in response procedure.
SAT	UNSAT (UNS	SAT requires comments)
	CINDIII (CIND	arr requires comments)
STEP	ELEMENT	STANDARD
6. *		Examinee instructs RO to place CEDMCS in
0.	CEDMCS in standby.	standby.
	CEDIVICS III standby.	standoy.
		If requested CUE: RO acknowledges
		communication and reports CEDMCS is in
		standby.
		standby.
SAT	UNSAT (UNS	SAT requires comments)
		T
COMME	ENTS:	



STEP 7.	ELEMENT Obtain keys for CEDMCS cabinet back panels and the Hold Bus panel.	STANDARD Examinee obtains key.
		INFORM CUE: Key is in Examinee's possession as Area 3.
SAT	UNSAT (UNSAT	requires comments)
STEP 8.	ELEMENT Check that no other subgroups are on the Hold Bus.	STANDARD Examinee determines that no other subgroups are on the hold bus.
		When examinee checks Hold Bus, INFORM CUE: All subgroups (except for subgroup 16) lights are extinguished on the Hold Bus panel.
SAT	UNSAT (UNSAT	requires comments)
STEP 9. *	ELEMENT Unlock and open hold bus control panel.	STANDARD Examinee unlocks and opens hold bus control panel.
		When Requested CUE: Hold bus control panel is open.
SAT	UNSAT (UNSAT	requires comments)
COMMI	ENTS:	



STEP 10.	ELEMENT Check for Hold Bus voltage of greate 50 volts.	STANDARD Examinee simulates verifying Hold Bus voltage greater than 50 volts. When Hold Bus voltage is checked, When requested CUE: Hold Bus voltage 70 VDC.
SAT	UNSAT	(UNSAT requires comments)
STEP 11.	* ELEMENT* Select subgroup 16 by adjusting the SG/SEL thumbwheel to 16.	STANDARD Examinee selects subgroup 16 on the SG/SEL thumbwheel.
		If Requested CUE: SG/SEL thumbwheel indicates 16.
SAT	UNSAT	(UNSAT requires comments)
STEP 12.	* Operate transfer switch to latch the subgroup selected.	STANDARD Examinee simulates placing transfer switch in transfer position and notes red "Transfer" light illuminates as well as the affected subgroup light. If Requested CUE: Transfer switch is in the
		transfer position. Transfer light and subgroup 16 lights are illuminated.
SAT	UNSAT	(UNSAT requires comments)
COM	MENTS:	



STEP 13.	*	ELEMENT One at a time, open the four individual CEA breakers on the appropriate Subg Power Switch Assembly for the Subgreath that was placed on the Hold Bus. Verificorrect breaker is opened by ensuring a CEA's drop.	roup oup y the no	Examinee simulates opening the following breakers and verifies with the Control Room that no CEA's drop. • XESFNCEA55 • XESFNCEA58 • XESFNCEA61 • XESFNCEA64 When Requested CUE: No CEA's drop. End Time CEA's must be transferred to hold bus within 10 minutes. NOTE: End time is when final breaker is open.
SAT		UNSAT(UNSAT requi	res comments)
STEP 14.		ELEMENT Notify the Reactor Operator that subgr 16 is on the Hold Bus.		STANDARD Examinee notifies RO that subgroup 16 is on the Hold Bus. Inform CUE: Another AO will investigate the problem with I&C assistance.
SAT		UNSAT(UNSAT requi	res comments)
				NORMAL TERMINATION POINT
COM	MENT	S:		



NRC EXAM 2003

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	7/13/00	6	New JPM

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



NRC EXAM 2003

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operation of in-plant equipment is to be **SIMULATED ONLY**, **DO NOT OPERATE** any equipment.
- Inform the control room staff of any discovered deficiencies.
- Vocalize the following items when simulating equipment operation.
 - d. Condition / indication you would check to determine equipment status.
 - e. Component you would operate indicating direction of travel.
 - f. Expected change in indications as a result of operation.
- You may use any source of information normally available.

INITIATING CUE:

- With the plant at 100% power the control room has received a CEDMCS Trouble alarm. Continuous Gripper High Voltage is indicated. The CRS directs you to place the affected subgroup on the hold bus.
- This is a time critical JPM.

SAFETY CONSIDERATIONS:

- Proper personal protective equipment
- Slip/Fall hazard on stairways.
- Pinch points at doorways.
- Equipment is located in high noise areas.



NRC EXAM 2003

JPM BASIS INFORMATION

TASK: 1250270201 Establish Control of the Plant at the Remote Shutdown Panels TASK STANDARD: SG levels are being maintained at 35 - 80% WR using AFA. K/A: 4.2-068-AA1.02 K/A RATING: RO: 4.3 SRO: 4.5 APPLICABLE POSITION(S): RO VALIDATION TIME: 10 min REFERENCES: 40AO-9ZZ18, Shutdown Outside Control Room SUGGESTED TESTING ENVIRONMENT: SIMULATOR PLANT X					
		APPROVAL			
DEVELOPER: P. Capehart TECH REVIEW: REVISION DATE: 10/16/98 APPROVAL:					
	TES	STING METHOD			
ACTUAL TESTING ENVIRONMENT: SIMULATOR PLANT TESTING METHOD: SIMULATE PERFORM					
EVALUATION					
EXAMINEE NAME: (print) EVALUATOR NAME:					
	(print)				
SATISFACTORY UNSATISFACTORY					
Time Start Time Stop					
REMEDIAL TRAINING REQUIRED? YESNO					



NRC EXAM 2003

1. SIMULATOR SETUP:

A. IC#: NA

B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	N/A	
2.		
3.		
4.		

- C. SPECIAL INSTRUCTIONS:
- None
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

• None



NRC EXAM 2003

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operation of in-plant equipment is to be **SIMULATED ONLY**, **DO NOT OPERATE** any equipment.
- Inform the control room staff of any discovered deficiencies.
- Vocalize the following items when simulating equipment operation.
 - a. Condition / indication you would check to determine equipment status.
 - b. Component you would operate indicating direction of travel.
 - c. Expected change in indications as a result of operation.
- You may use any source of information normally available.

INITIATING CUE:

- The Unit 1 Control Room is evacuated due to a bomb threat.
- "B" train Class Auxiliary Feed Pump (AFB-P01) is under clearance and unavailable.
- The CRS directs you to start the "A" train Class Auxiliary Feed Pump (AFA-PO1), and maintain steam generator levels between 35 and 80% WR in accordance with 40AO-9ZZ18, Shutdown Outside the Control Room, Appendix E.
- Current level in both steam generators is 25% WR

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW NUREG-1021.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Manager of in-plant JPM performance.

SAFETY CONSIDERATIONS:

- Slip/Fall hazard on stairways
- Pinch points at doorways
- Equipment may auto start at any time
- Equipment is located in high noise areas
- Hot piping and components are located within the JPM performance area.



STEP	ELEMENT	STANDARD
1.	Obtain procedure 40AO-9ZZ18, Shutdown	Examinee obtains procedure 40AO-9ZZ18,
	Outside the Control Room, Appendix E.	Shutdown Outside the Control Room, Appendix
		E.
CAT	LINICAT (LINICA	T == ==i=== ===========================
SAT	UNSAT (UNSA	T requires comments)
STEP	ELEMENT	STANDARD
2. *	Adjust AFA-SK-52B, AFA speed to	Examinee simulates adjusting speed to
2. "	minimum	minimum
		If requested CUE: Speed is at minimum
		•
SAT	UNSAT (UNSA	T requires comments)
STEP	ELEMENT	STANDARD
3. *	Place AFA-HS-52A, speed control transfer,	Examinee simulates putting AFA-HS-52A,
	to the remote shutdown position.	speed control transfer to the remote shutdown
		position
		If requested CUE: AFA-HS-52A is in the
		"Remote Shutdown" position
		F. S.
SAT	UNSAT (UNSA	T requires comments)
STEP	ELEMENT	STANDARD
4. *	Open one of the following steam supply	Examinee simulates opening one of the valves.
	valves	A LOVE CO. VIVAND CO.
	CCA 107 124	If requested CUE: SGA-UV-134B or SGA-
	• SGA-UV-134	UV-138B is open.
	• SGA-UV-138	NOTE: Either valve is acceptable.
	- 56/1 6 V 156	THO TEXT DAMAGE NAME OF MOTOR
SAT	UNSAT (UNSA	T requires comments)
	<u> </u>	-
COMMEN'	TS:	



STEP	ELEMENT	STANDARD
5. *	Adjust turbine speed using AFA-SK-52B to 3590 -3600 rpm for Unit's 1 & 3, 3560-3570 for Unit 2.	Examinee simulates adjusting turbine speed using AFA-SK-52B to 3590 - 3600 rpm
	3300 3370 IOI OIM 2.	If requested CUE: Speed is 3600 rpm (if on Unit 1 or 3) Speed is 3570 rpm (if on Unit 2).
SAT	UNSAT (UNSAT	requires comments)
STEP	ELEMENT	STANDARD
6. *	Open both of the following AFW pump A to SG down stream valves:	Examinee simulates placing handswitches to OPEN.
	• AFA-UV-37	If requested CUE: AFA-UV-37 and AFC-UV-36 have the red light on and the green
	• AFC-UV-36	light off.
SAT	UNSAT (UNSAT	requires comments)
STEP	ELEMENT	STANDARD
7. *		Examinee simulates placing handswitches to OPEN and establishes AFW flow to SGs.
	• AFA-UV-32	NOTE: S/G levels are indicated on "B" Train.
	• AFC-UV-33	Inform CUE: Feed flow to SG 1 and SG 2 using AFW Pump "A" is 250 GPM.
		Inform CUE: SG 1 and SG 2 level is being maintained between 35 – 80% WR
SAT	UNSAT (UNSAT	requires comments)
		NORMAL TERMINATION POINT
COMMI	ENTS:	



NRC EXAM 2003

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
14	09/24/96	6	New Format
15	11/07/97	3	New procedure changed steps
16	01/21/98	6	Update step 6
17	05/08/98	6	Update Task Statement
18	10/16/98	6	Updated step 4 to include Unit difference and to correct administrative items.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



NRC EXAM 2003

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operation of in-plant equipment is to be **SIMULATED ONLY**, **DO NOT OPERATE** any equipment.
- Inform the control room staff of any discovered deficiencies.
- Vocalize the following items when simulating equipment operation.
 - a. Condition / indication you would check to determine equipment status.
 - b. Component you would operate indicating direction of travel.
 - c. Expected change in indications as a result of operation.
- You may use any source of information normally available.

INITIATING CUE:

- The Unit 1 Control Room is evacuated due to a bomb threat.
- "B" Train Class Auxiliary Feed Pump (AFB-P01) is under clearance and is unavailable.
- The CRS directs you to start the "A" Train Class Auxiliary Feed Pump (AFA-PO1), and maintain steam generator levels between 35 and 80% WR in accordance with 40AO-9ZZ18, Shutdown Outside the Control Room, Appendix E.
- Current level in both steam generators is 25% WR

SAFETY CONSIDERATIONS:

None



2003 NRC EXAM

JPM BASIS INFORMATION

TASK: 1250010301 Respond to a condition requiring emergency boration instructions and contingencies. TASK STANDARD: In order to Emergency Borate, operate HPSI pumps in lieu of Charging pumps. K/A: 4.2.024AK3.02 K/A RATING: RO: 4.2 SRO: 4.4 K/A: K/A RATING: RO: SRO: APPLICABLE POSITION(S): RO VALIDATION TIME: 15 min REFERENCES: 40AO-9ZZ01, Emergency Boration SUGGESTED TESTING ENVIRONMENT: SIMULATOR X PLANT NOTE: Alternate Path JPM					
	APPROV	AL			
DEVELOPER: T. Stahler REVISION DATE: 4/15/03		REVIEW: OVAL:			
TESTING METHOD					
ACTUAL TESTING ENVIRONM	MENT: SIMULATO	₹	PLANT		
TESTING METHOD: SIMUL	ATE	PERFORM			
	EVALUAT	ION			
EXAMINEE NAME:					
EVALUATOR NAME:		(print)			
		(print)			
SATISFACTORY	UNSATISFACTO	RY	<u> </u>		
Time Start Time	me Stop				
REMEDIAL TRAINING REQUIRED? YESNO					



2003 NRC EXAM

1. SIMULATOR SETUP:

- A. IC#: 4 Mode 5
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	RST 4	Reset to IC-4 Mode 5
2.	MRF B302:CHBP01 Rack_Out	'B' Charging Racked Out
3.	MRF B302:CHEP01 Rack_Out	'E' Charging Racked Out
4.	When CHN-UV-501 is closed:	Charging Pump 'A' trips.
	Then: IMF DP06:CHAP01.	

C. SPECIAL INSTRUCTIONS:

- Perform events 1,2,3.
- Place Charging Pumps 'B' & 'E' in Pull-To-Lock.
- ACK any alarms.
- FREEZE Simulator.
- Provide Initiating Cue.
- Go to RUN on Simulator.
- IMF DP06:CHAP01 at appropriate time during JPM. (after CHN-UV-501 is closed)

D. REQUIRED CONDITIONS:

- 'B' & 'E' Charging Pumps inoperable
- 'A' Charging Pump running
- 'B' HPSI pump available. (breaker indicating lights on)
- 'A' HPSI is unavailable. (breaker indicating lights are out)

2. SPECIAL TOOLS/EQUIPMENT:

None



2003 NRC EXAM

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room
- Vocalize what indications you are checking and why.
- You may use any source of information normally available.

INITIATING CUE:

- The unit is in Mode 5 with 'B' LPSI on Shutdown cooling.
- An "Emergency Boration" due to inadequate shutdown margin is required.
- Charging pumps B&E are inoperable.

The CRS directs you to Emergency Borate per 40AO-9ZZ01, Section 3 Emergency Boration.

• Inform the CRS when adequate flow is established.

SAFETY CONSIDERATIONS:

None

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.

SAFETY CONSIDERATIONS:

• None



STEP 1.	ELEMENT Obtains 40AO-9ZZ01, Emergency Boration and refers to section 3 Emergency Boration.	STANDARD Correct procedure referenced.
SAT	UNSAT (UNSA	T requires comments)
STEP 2.	ELEMENT Check that a Charging Pump is available for Emergency Boration.	STANDARD 'A' Charging Pump is running.
SAT	UNSAT (UNSA	T requires comments)
STEP 3.	 ELEMENT Check Both of the following: RWT level is greater than 73% RWT is available for Emergency Boration 	STANDARD RWT level is approximately 90%. IF REQUESTED CUE: The CRS has determined that the RWT is available for Emergency Boration.
SAT	UNSAT (UNSA	T requires comments)
COMME	NTS:	



STEP 4. *	 ELEMENT Perform the following to align CHN-F. 536, RWT to Charging Pump Suction: Refer to Appendix F, Simplified Drawings for a basic flow view. Ensure the BAMP(s) are stopped Open CHE-HV-536. Close CHN-UV-501, Volume Con Tank Outlet. 	BAMP(s) are off. Examinee Opens CHE-HV-536.
SAT	UNSAT ((UNSAT requires comments)
STEP 5.	ELEMENT Ensure CHN-UV-527, VCT Bypass, is closed.	STANDARD Examinee checks CHN-UV-527 Closed. Note: Charging Pump 'A' will trip during this step or at the end of the previous step.
SAT	UNSAT ((UNSAT requires comments)
STEP 6.	ELEMENT Charging pump 'A' trip.	STANDARD The examinee acknowledges the 'A' Charging Pump trip.
		INFORM CUE: The CRS has assigned another operator to investigate the Charging Pump Trip. The CRS directs you to establish Emergency Boration to the RCS.
SAT	UNSAT ((UNSAT requires comments)
COMME	ENTS:	



2003 NRC EXAM

5TEP 7.	ELEMENT Examinee reevaluates step 1. With no Charging Pumps now available he goes to section 4 HPSI Pump.	Examinee goes to HPSI Pump, section 4.
SAT	UNSAT (UNSAT	requires comments)
STEP 8.	ELEMENT Refer to Appendix F, simplified Drawings for a basic flow view.	STANDARD May refer to Appendix. Inform Cue: CRS directs using 'B' HPSI pump.
SAT	UNSAT (UNSAT	requires comments)
STEP 9.	ELEMENT Check that the HPSI 'B' Pump breaker is racked in.	STANDARD HPSI 'B' Pump has breaker indication on the control board.
		If Requested Cue: HPSI 'B' Pump breaker is Racked in.
SAT	UNSAT (UNSAT	requires comments)
STEP 10.	ELEMENT Check that the HPSI 'B' pump lockout relay is reset.	STANDARD Normal green indication at breaker indication If Requested Cue: HPSI Pump Lockout relay is reset.
SAT	UNSAT (UNSAT	requires comments)
COMME	ENTS:	



2003 NRC EXAM

STEP 11.	ELEMENT Check that the HPSI 'B' Pump UC fuses are 'ON'	STANDARD Normal green indication at breaker indication.
		If Requested Cue: HPSI Pump breaker UC fuses are in the 'ON' position.
SAT	UNSAT (UNSAT I	requires comments)
STEP 12.	 ELEMENT If HPSI Pump 'B' will be used for emergency boration. Then perform the following: a) Ensure all of the following valves are closed: SIB-UV-668, LPSI Pump B Miniflow 	STANDARD
	 Recirc. SIB-UV-665, CS Pump B Miniflow Recirc. SIB-HV-609, HPSI Pump B Long Term Recirc Isolation. All Train 'B' HPSI Cold Leg Injection Valves. 	Examinee closes SIB-UV-665. All other valves are verified CLOSED.
	 b) Ensure ALL of the following valves are open: CHB-HV-530, RWT to Train B Safety Injection. SIB-UV-667, HPSI Pump B Miniflow recirc. SIB-UV-659, Train B Pumps Combined Recirc. 	All valves are verified OPEN.
SAT	UNSAT (UNSAT I	requires comments)
COMME	NTS:	
		_
		_



2003 NRC EXAM

STEP 13.	*	ELEMENT Start The appropriate HPSI Pump		STANDARD Examinee starts HPSI pump 'B'
SAT _		UNSAT	(UNSAT requ	uires comments)
STEP 14.	*	ELEMENT Throttle open one of the HPSI Cold Injection Valves to obtain 75 gpm of		STANDARD Flow of greater than 75 GPM established through a 'B' train HPSI Cold Leg Injection Valve.
				NOTE : Approx. 50% valve position will give flowrate of between 75 - 100 gpm.
				NOTE: From Initiating cue, the examinee should inform the CRS Emergency Boration Flow is established at this point.
				Inform Cue: Another RO will complete the remaining actions.
SAT		UNSAT	(UNSAT requ	uires comments)
				NORMAL TERMINATION POINT
COMN	MENT	rs:		



2003 NRC EXAM

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
12	05/30/97	6	New Format

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



2003 NRC EXAM

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available.

INITIATING CUE:

- The unit is in Mode 5 with 'B' LPSI on Shutdown cooling.
- An "Emergency Boration" due to inadequate shutdown margin is required.
- Charging pumps B&E are inoperable.

The CRS directs you to Emergency Borate per 40AO-9ZZ01, Section 3 Emergency Boration.

• Inform the CRS when adequate flow is established.

SAFETY CONSIDERATIONS:

• None



JPM BASIS INFORMATION

JI WI D	PASIS INFORM	ATION		
TASK: 1240021701 Direct Containment TASK STANDARD: 'B' Hydrogen Rec recombiner 'B'	Hydrogen Control ombiner Control Board	Alignment comp	leted, AO directed	to start
K/A: 3.5-028-A4.03 K/A: APPLICABLE POSITION(S): RO/SRO	K/A RATING K/A RATING O VALID		SRO: SRO: 10 min	3.3
REFERENCES: 40EP-9EO10, Standard Ap			10 111111	
SUGGESTED TESTING ENVIRONME	NT: SIMULATO	PR X	PLANT	
	APPROVAL			
DEVELOPER: T. Stahler	TECH REV	VIEW:		
REVISION DATE: 4/15/03	APPROVA	L:		
TI	ESTING METH	OD		
ACTUAL TESTING ENVIRONMENT:	SIMULATOR		PLANT	
TESTING METHOD: SIMULATE	PEI	RFORM		
	EVALUATION	N		
EXAMINEE NAME:				
EVALUATOR NAME:		(print)		
		(print)		
SATISFACTORY UN	SATISFACTORY _		_	
Time Start Time Stop	p	<u>.</u>		
REMEDIAL TRAINING REQUIRED? (SEE OTG-04)	YES	NO		



1. SIMULATOR SETUP:

- A. IC#: 20
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.		
2.		
3.		
4.		

C. SPECIAL INSTRUCTIONS:

- Go to run on simulator. (if being performed with other JPM's simultaneously, the following actions are not required.)
- IMF th01a at 3%
- Secure RCPs
- Allow CSAS to actuate. (Run time about 4 minutes)
- Acknowledge alarms.
- FREEZE simulator.
- Provide initiating CUE
- Go to RUN on simulator

D. REQUIRED CONDITIONS:

- RCS LOCA.
- RCPs secured.
- CSAS actuated.

2. SPECIAL TOOLS/EQUIPMENT:

• none



TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- A Reactor Trip due to a Large LOCA has occurred
- You are in Unit 1 Control Room.

The Control Room Supervisor has directed you to place the Hydrogen Recombiner "B" in service using 40EP-9EO10, Standard Appendix 19.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.

SAFETY CONSIDERATIONS:

None



	ust is not
desired.	
SAT UNSAT (UNSAT requires comments)	
STEP 2. Inform Radiation Protection that Recombiner "B" is being aligned for use. Recombiner "B" is being aligned for use. When requested CUE: RP has informed.	
SAT UNSAT (UNSAT requires comments)	
STEP 3. * Request that I&C Maintenance perform Attachment 19-G. When requested CUE: I&C h informed and Attachment 19- completed.	as been
SAT UNSAT (UNSAT requires comments)	
COMMENTS:	



STEP	ELEME	NT		STANDARD
4.	* Direct an 19-F.	operator to perform Attach	nment	Operator directed to perform Attachment 19-F. When requested CUE: Attachment 19-F has been completed. The AO reports Recombiner "B" is ready to start.
SAT	UI	NSAT	(UNSAT requi	res comments)
STEP	ELEME	NT		STANDARD
5.		ll of the following condition	ns are	Examinee informs CRS.
	• The of 19-F ready THEN In	chment 19-G is complete. operator performing Attach has reported that Recombinate to Start. form the CRS that Recombination operation.	ner B is	If Requested Cue: The CRS directs starting Recombiner B.
SAT	U	NSAT	(UNSAT requi	res comments)
STEP	ELEME	NT		STANDARD
6.		P that Recombiner B will b	e	Examinee informs RP.
SAT	Uì	NSAT	(UNSAT requi	res comments)
STEP 7.		NT B-UV-2, Control System " olation valve.	B"	STANDARD Examinee overrides and opens HPB-UV-2.
SAT	Uì	NSAT	(UNSAT requi	res comments)
COM	MENTS:			



STEP		ELEMENT		STANDARD
8.	*	Open HPB-UV-4, Control System	"B" to	Examinee overrides and opens HPB-UV-4
0.		Recombiner Isolation Valve.		
SAT		UNSAT	(UNSAT requ	ires comments)
			(01,01111040	
STEP		ELEMENT		STANDARD
9.	*	Open HPB-UV-6, Control System	"B"	Examinee overrides and opens HPB-UV-6.
7.		From Recombiner Isolation Valve.	2	Enumero o verriado una openo en El el el ver
SAT		UNSAT	(UNSAT requ	ires comments)
_		 -	• `	,
STEP		ELEMENT		STANDARD
10.	*	Direct the Operator to start Recomb	oiner	Operator directed to start Recombiner "B".
10.		"B".		
				When requested CUE: The Recombiner "B"
				has been started.
SAT		UNSAT	(UNSAT requ	ires comments)
				NORMAL TERMINATION POINT
				NORWAL TERMINATION FOINT
COM	MEN	ΓS:		



RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
7	10/10/96	6	New Format
8	10/11/96	6	More format changes per OTG-02
9	4/15/03	3	Minor enhancements

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- A Reactor Trip due to a Large LOCA has occurred
- You are in Unit 1 Control Room.

The Control Room Supervisor has directed you to place the Hydrogen Recombiner "B" in service using 40EP-9EO10, Standard Appendix 19.

SAFETY CONSIDERATIONS:

• None



JPM BASIS INFORMATION

	JPM BA	515 INFUR	MATION		
TASK: 1250800201 Utask Standard: Cok/A: 3.4.045 APPLICABLE POSITION REFERENCES: 40AO-92 SUGGESTED TESTING:	ompletes Appendix A I(S): RO ZZ25 ECC Directe	A Steps 1-9. K/A RATI VAI d Turbine Unlo	NG: RO: 2.7 LIDATION TIME:	SRO: 15 minutes PLANT	2.6
	1	APPROVA	L		
DEVELOPER: T St REVISION DATE: 3/26	ahler /03	TECH R APPRO	EVIEW: VAL:		
	TES	TING MET	THOD		
ACTUAL TESTING ENV	TRONMENT: S	SIMULATOR		PLANT	
TESTING METHOD:	SIMULATE	1	PERFORM		
	E	VALUATIO	ON		
EXAMINEE NAME:			(print)		
EVALUATOR NAME:			(print)		
			(print)		
SATISFACTORY	UNSA	TISFACTORY		_	
Time Start	Time Stop				
REMEDIAL TRAINING	REQUIRED?	YES	NO		



1. SIMULATOR SETUP:

A. IC#: 20

B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	None	
2.		
3.		
4.		

C. SPECIAL INSTRUCTIONS:

- Simulator in Run
- Acknowledge alarms
- D. REQUIRED CONDITIONS:
- Verify Main Generator Gross MW at about 1320 MW on ERFDADS point MAJ1.

2. SPECIAL TOOLS/EQUIPMENT:

• none



Task JS3 PVNGS JOB PERFORMANCE MEASURE NRC 2003 TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available.

INITIATING CUE:

The following plant conditions exist:

- You are in Unit 1.
- Reactor Power is 100%.
- ECC just requested that Palo Verde reduce generator output.

The CRS directs you to align the unit to prepare for turbine unloading using 40AO-9ZZ25 Appendix A steps 1 through 9.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.

SAFETY CONSIDERATIONS:

• None



STEP	ELEMENT Candidate refers to 40AO-9ZZ25 Ap	endix STANDARD Obtains 40AO-9ZZ	725 Annendiy A
1.	A.	Clidix Obtains 40AO-722	23 Appendix A.
SAT	UNSAT	UNSAT requires comments)	
STEP	ELEMENT	STANDARD	
2.	Inform Chemistry of the intention to steam to the condenser.	ump Contacts chemistry	
	steam to the condenser.		Chemistry acknowledges team to the condenser.
SAT	UNSAT	UNSAT requires comments)	
STEP	ELEMENT	STANDARD	
3.	Record Main Generator Gross MW.		n Generator Gross MW. IC ately 1320 MW
		If requested CUE : ERFDADS/PMS p	
SAT	UNSAT	UNSAT requires comments)	
STEP	ELEMENT	STANDARD	
4.	Record position of the Load Limit Potentiometer.	Records position.	
	i dendonicei.	Note: Load Limit F approximately 7.72	
SAT	UNSAT	UNSAT requires comments)	
COMMI	ENTS:		



STEP 5.	ELEMENT Override and energize pressurizer backup heaters.	STANDARD Examinee energizes all pressurizer backup heaters.
		If requested CUE: CRS directs energizing all pressurizer backup heaters.
SAT	UNSAT (UNSA	AT requires comments)
STEP 6.	ELEMENT Lower the setpoint on RCN-PIC-100, Pressurizer Pressure Controller to 2220 psia.	STANDARD Setpoint lowered. Main Spray valve modulate open slightly.
SAT	UNSAT (UNSA	AT requires comments)
STEP 7. *	ELEMENT Place CEDMCS in a mode other than Auto Sequential.	STANDARD CEDMCS removed from Auto Sequential. If Requested Cue: CRS concurs with any recommended position or directs Manual Sequential.
SAT	UNSAT (UNSA	AT requires comments)
STEP 8. *	ELEMENT Place SGN-PIC-1010, SBCS Master Control in Local Auto.	STANDARD Examinee places SGN-PIC-1010 to Local Auto.
SAT	UNSAT (UNSA	AT requires comments)
COMMI	ENTS:	



STEP 9.	*	ELEMENT Lower the Local Auto setpoint (black on SGN-PIC-1010, SBCS Master to PSIG above the indicated actual pressure(red pen).	s pen) F 20 I N to a S	Black pen is set to approximately 1005 psig in C 20. Note: Critical nature of step is to lower set-point to less than auto setpoint (black and white pen) above system pressure (as verified by GN-PV-1001 remaining closed when given a termissive in the following step).
SAT		UNSAT	(UNSAT require	es comments)
STEP 10.	*	ELEMENT Give both of the following a Manual Permissive: SGN-PV-1001, Valve 1 SGN-PV-1004, Valve 4	F	EXAMDARD Examinee places both valves in Manual Permissive. Both valves should remain closed.
SAT		UNSAT	(UNSAT require	es comments)
				NORMAL TERMINATION POINT
COM	MENT	rs:		



RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	04/01/03	6	New JPM, T.E.S.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



Task JS3 PVNGS JOB PERFORMANCE MEASURE NRC 2003 INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

The following plant conditions exist:

- You are in Unit 1.
- Reactor Power is 100%.
- ECC just requested that Palo Verde reduce generator output.

The CRS directs you to align the unit to prepare for turbine unloading using 40AO-9ZZ25 Appendix A steps 1 through 9.

SAFETY CONSIDERATIONS:

• None



JPM BASIS INFORMATION	
TASK: 0100010401 Operate the Pressurizer Pressure Control System TASK STANDARD: Pressurizer pressure restored to 2250 ± 25 psia K/A: 4.2-027-A1.01 K/A RATING: RO: 4.0 SRO: 3.9 APPLICABLE POSITION(S): RO/SRO VALIDATION TIME: 10 Minutes REFERENCES: 40AL-9RK4A, Panel B04A Alarm Responses SUGGESTED TESTING ENVIRONMENT: SIMULATOR XX PLANT	_
APPROVAL	
DEVELOPER: L. Wilhelm TECH REVIEW: REVISION DATE: 06/12/02 APPROVAL:	
TESTING METHOD	
ACTUAL TESTING ENVIRONMENT: SIMULATOR PLANT	
TESTING METHOD: SIMULATE PERFORM	
EVALUATION	
EXAMINEE NAME:	
(print) EVALUATOR NAME:	
(print)	
SATISFACTORY UNSATISFACTORY	
Time Start Time Stop	
REMEDIAL TRAINING REQUIRED? YESNO	



1. SIMULATOR SETUP:

- A. IC#: Any normal operating pressure IC (IC 20 preferred).
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	IMF RC02A 0	Fails Pressurizer Spray Valve 100E Closed
2.	IMF RC02B 0	Fails Pressurizer Spray Valve 100F Closed
3.	IOR ZDRCNHS100 ASIS	Fails PPCS selector switch to the "X" position
4.	IMF TR01:RCNPT100X 1500	Fails Pressurizer Pressure Control Channel "X" to 1500 psia

C. SPECIAL INSTRUCTIONS:

- Reset to any normal operating pressure IC (IC 20 preferred).
- Go to run on the SIM.
- Ensure PPCS on channel X.
- Insert the Malfunctions and Overrides
- Acknowledge alarms and FRZ the SIM when PZR pressure is > 2285 psia and the Pressurizer pressure alarm is in.
- Provide initiating CUE and go to RUN.

D. REQUIRED CONDITIONS:

- RCS Pressure > 2285 psia
- Pressurizer Spray Valves RCE-100E and 100F are failed Closed
- Pressurizer Pressure RCN-PT100X is faill to 1500 psia.

2. SPECIAL TOOLS/EQUIPMENT:

NONE



TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- Pressurizer Pressure is >2285 psia and increasing.
- The CRS directs you to restore pressurizer pressure to 2250 psia, in accordance with 40AL-9RK4A Window 4A01B.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- NOTE: Alternate Path JPM
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Supervisor of in-plant JPM performance.

SAFETY CONSIDERATIONS:

• NONE



STEP	ELEMENT	STANDARD
1.	Obtain Annunciator Alarm Response	Examinee obtains 40AL-9RK4A. Goes to
	Manual 40AL-9RK4A, Window	4A01B Group B for PZR PRESS High.
	4A01B, Group B.	
SAT	UNSAT (UNSAT	requires comments)
CEED	EV EL CONTE	CELVE
STEP 2 *	ELEMENT	STANDARD
2. *	Trip reactor if high pressure trip is	Examinee determines Pressurizer Pressure
	impending (≥ 2383 psia) and proceed	< 2383 psia.
	to 40EP-9EO01.	TEDMINIATE IDM. IF DEACTOR IC
		TERMINATE JPM, IF REACTOR IS
		TRIPPED. JPM would be UNSAT.
SAT	UNSAT (UNSAT	requires comments)
	``	,
STEP	ELEMENT	STANDARD
3.	Verify pressurizer pressure high alarm	Examinee determines actual high pressure
	by observing RCN-PIC-100X and/or	condition exists.
	RCN-PIC-100Y on recorder RCN-PR-	
	100 (B04).	
SAT	UNSAT (UNSAT	requires comments)
CTED	DI DAMBAKE	CT AND ADD
STEP	ELEMENT Varify controlling channel transmitter	STANDARD Examinee determines CH "X" is
4.	Verify controlling channel transmitter has not failed.	
	has not raned.	inaccurate.
SAT	UNSAT (UNSAT	requires comments)
		,
COMME	ENTS:	



STEP	ELEMENT		STANDARD
5.	Switch to unaffected channed RCN-HS-100.	nel using	Examinee selects Channel "Y"
			NOTE: Will have no affect due to switch failure.
SAT	UNSAT	(UNSAT	requires comments)
STEP	ELEMENT		STANDARD
6.	Manually initiate pressurize flow using RCN-PIK-100, Spray Control to reduce present band.	Pressurizer	Examinee determines normal spray doesn't respond.
SAT	UNSAT	(UNSAT	requires comments)
STEP	ELEMENT		STANDARD
7. *	Initiates Aux Spray to redu		Examinee initiates Aux Spray Flow and
	pressure to normal band us		observes pressure lowering.
	HS-205 and/or CHB-HS-2	03 on B03.	Controls Pressurizer Pressure at 2250 +/-25 psia.
SAT	UNSAT	(UNSAT	requires comments)
COMME	NITC.		
COMME	115.		



STEP 8.	ELEMENT Deenergize Pressurizer Heaters a required to limit pressure increas	STANDARD De-energize Pressurizer heaters as required to limit pressure increases.
		When Pressurizer Pressure is controlled at approximately 2250 PSIA:
		Inform Cue: Another RO will take actions to maintain Pressurizer Pressure at approximately 2250 PSIA.
SAT	UNSAT	(UNSAT requires comments)
		NORMAL TERMINATION POIN
COMME	ENTS:	



RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
13	07/25/96	3,6	New Format per OTG-02
14	10/10/96	6	More Format changes per OTG-02
15	06/18/02	3	Procedure revised

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- Pressurizer Pressure is >2285 psia and increasing.
- The CRS directs you to restore pressurizer pressure to 2250 psia, in accordance with 40AL-9RK4A Window 4A01B.



	JPM BA	ASIS INFORM	IATION		
TASK: 0780031301 Transformer TASK STANDARD:	MAN-X02 Transfers 13.8KV Transformer MAN	bus S01 From 13.8 I-X02	KV Bus S03 to	the Unit Auxilia	ry
K/A: 3.6-062-K1.04 APPLICABLE POSIT REFERENCES: 41OP SUGGESTED TESTII	ION(S): RO/SRO -1NA01, 13.8KV Ele	ectrical System (NA	OATION TIME:	15 Minutes	4.2
		APPROVAL			
DEVELOPER: REVISION DATE:		TECH REV APPROVA			
	TES	STING METH	OD		
ACTUAL TESTING I	ENVIRONMENT:	SIMULATOR		PLANT	
TESTING METHOD:	SIMULATE _	PE	RFORM		
	I	EVALUATION	V		
EXAMINEE NAME:			(: 0		
EVALUATOR NAME	E:		(print)		
			(print)		
SATISFACTORY	UNS	ATISFACTORY		_	
Time Start	Time Stop		-		
REMEDIAL TRAININ	•	YES	NO		



1. SIMULATOR SETUP:

- A. IC#: 20 or any IC with the Main Generator on line at $\geq 20\%$
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	None	

C. SPECIAL INSTRUCTIONS:

- Go to RUN on Simulator; ensure Main Generator on line at \geq 20%.
- Ensure NAN-S01 and S02 are transferred to S03 and S04 respectively, then acknowledge alarms.
- FREEZE Simulator and provide initiating cue.
- Go to RUN on Simulator.

D. REQUIRED CONDITIONS:

• Main Generator on line at ≥ 20%; NAN-S01 and S02 are powered from offsite power

2. SPECIAL TOOLS/EQUIPMENT:

• None



TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- The Unit's power is being supplied from offsite power (startup transformers).
- The CRS directs you to transfer Unit loads to the Unit Auxiliary Transformer MAN-X02 in accordance with 40OP-9NA03.
- All prerequisites have been performed.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW NUREG-1021.
- Step sequence is not critical unless noted or will prevent achieving the task standard.

SAFETY CONSIDERATIONS:

• None



STEP 1.	ELEMENT Obtain procedure 40OP-9NA03 and goes to section 4.8 or section 4.9.	STANDARD 40OP-9NA03 obtained. Note: If examinee goes to section 4.9 first, then start at JPM step 8.
SAT	UNSAT (UNSAT	requires comments)
STEP 2. *	ELEMENT Turn the Synch Switch for 13.8KV Bus NAN-S01 Supply Breaker, NAN-SS-S01A to ON, and check for proper synchronization.	STANDARD Examinee places synch switch NAN-SS-S01A to ON and verifies proper synchronization.
SAT	UNSAT (UNSAT	requires comments)
STEP 3. *	ELEMENT Close the 13.8 KV Bus NAN-S01 Supply Breaker, NAN-S01A by turning handswitch NAN-HS-S01A to CLOSE.	STANDARD NAN-S01A is closed.
SAT	UNSAT (UNSAT	requires comments)
STEP 4.	ELEMENT Check that the 13.8KV Bus NAN-S03- 1S01 Supply Breaker, NAN-S03B, automatically opens when handswitch NAN-HS-S01A is released.	STANDARD Examinee verifies NAN-S03B opens when NAN-HS-S01A is released.
SAT	UNSAT (UNSAT	requires comments)
COMME	ENTS:	



STEP	ELEMENT		STANDARD
5.	Check that 13.8KV Bus NAN-S01 voltage is between 12.42 - 14.49KV.		Examinee verifies voltage between 12.42 and 14.49 KV.
SAT	UNSAT	(UNSAT re	equires comments)
STEP 6.	ELEMENT Turn Synch Switch for 13.8KV Bus NAN-S01 Supply Breaker, NAN-SS-S01A to OFF.		STANDARD NAN-SS-S01A is placed to off. Key removed.
SAT	UNSAT	(UNSAT re	equires comments)
STEP 7.	ELEMENT Evaluate need to perform Appe	endix D.	STANDARD Examinee evaluates the step as N/A. Only one unit off Start-up Transformers X03Y and X01Z.
SAT	UNSAT	(UNSAT re	equires comments)
STEP 8.	* Turn Synch Switch for 13.8KV NAN-S02 Supply Breaker, NA to ON and check for proper synchronization.		STANDARD Examinee places NAN-SS-S02A to ON and verifies proper synchronization.
SAT	UNSAT	(UNSAT re	equires comments)
STEP 9.	* Close the 13.8 KV Bus NAN-S Breaker, NAN-S02A by turning NAN-HS-S02A to CLOSE.		STANDARD NAN-S02A is closed.
SAT	UNSAT	(UNSAT re	equires comments)
COM	MENTS:		



STEP 10.	ELEMENT Check that the 13.8KV Bus NAN-S0 NAN-S02 Supply Breaker NAN-S04 automatically opens when handswite NAN-HS-S02A is released.		
SAT	UNSAT	UNSAT requires comments)	
STEP 11.	ELEMENT Check that 13.8KV Bus NAN-S02 V is between 12.42 - 14.49 KV.	STANDARD Examinee verifies voltage be 14.49KV.	etween 12.42 and
SAT	UNSAT	UNSAT requires comments)	
STEP 12.	ELEMENT Turn Synch Switch for 13.8KV Bus NAN-S02 Supply Breaker, NAN-SS to OFF.	STANDARD NAN-SS-S02A is placed to 0	OFF.
SAT	UNSAT	UNSAT requires comments)	
STEP 13.	ELEMENT Evaluate need to perform Appendix	STANDARD Examinee evaluates the step Only one unit off Start-up Tr MAN-X03Y and MAN-X01	ransformers
SAT	UNSAT	UNSAT requires comments)	
		NORMAL TEI	RMINATION POINT
COMME	ENTS:		
			_



RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
12	07/25/96	3	OTG-02
13	10/15/96	6	New format per OTG-02
14	07/14/98	6	Changed a few Inform CUE's to If Requested Steps 6 and 11
15	07/24/98	6	Typo corrections and added information to Initiating CUE
16	10/13/98	6	Correct K & A reference and administrative items.
17	06/07/01	3	Ensure JPM steps comply with procedure
18	06/18/02	3	Procedure revised
19	04/16/03	6	Removed if requested cues.
20	5/14/03	3	Added evaluation if Appendix D needs to be performed.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- The Unit's power is being supplied from offsite power (startup transformers).
- The CRS directs you to transfer Unit loads to the Unit Auxiliary Transformer MAN-X02 in accordance with 40OP-9NA03.
- All prerequisites have been performed.

SAFETY CONSIDERATIONS:



JPM BASIS INFORMATION

TASK: 1030011001 Place BOP ESFASTASK STANDARD: FBEVAS 'A' BOPK/A: 3.7.016.A4.01 APPLICABLE POSITION(S): ROREFERENCES: 400P-9SA01, BOP ESFORGESTED TESTING ENVIRONMENT	PESFAS module is in Bypass. K/A RATING: RC VALIDATION (AS Modules Operation.	D: 3.9 SRO: 2.8 TIME: 15 minutes X PLANT
	APPROVAL	
DEVELOPER: T Stahler REVISION DATE: 04/01/03	TECH REVIEW: APPROVAL:	
TF	ESTING METHOD	
ACTUAL TESTING ENVIRONMENT:	SIMULATOR	PLANT
TESTING METHOD: SIMULATE	PERFORM	I
	EVALUATION	
EXAMINEE NAME:		
EVALUATOR NAME:	(print)	
	(print)	
SATISFACTORY UN	SATISFACTORY	
Time Start Time Stop		
REMEDIAL TRAINING REQUIRED?	YES NO)



1. SIMULATOR SETUP:

A. IC#: 20

B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.		
2.		

C. SPECIAL INSTRUCTIONS:

- Simulator in Run
- Place FBEVAS 'B' in bypass.
- Acknowledge alarms.
- Acknowledge BOP ESFAS alarms on B05 as they come in
- D. REQUIRED CONDITIONS:
- FBEVAS 'B' is in bypass
- Lamp test both BOP ESFAS panels and replace any burnt out bulbs.

2. SPECIAL TOOLS/EQUIPMENT:



JS6 PVNGS JOB PERFORMANCE MEASURE NRC EXAM 2003 TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

The following plant conditions exist:

- The CRS has directed you to place BOP ESFAS FBEVAS 'A' in bypass in accordance with 40OP-9SA01 SECTION 4.6, due to RU-31 power supply degradation.
- The CRS and STA have reviewed applicable LCO's and ODCM requirements.
- Prerequisites have <u>NOT</u> been preformed.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Supervisor of in-plant JPM performance.

SAFETY CONSIDERATIONS:

None



STE 1.	P	*	Perform prerequise CRS has directed p Checks redundant not in bypass.	performance.	3' is	Examinee identifies Train 'B' FBEVAS is in bypass. If requested cue: CRS directs you to remove Train 'B' FBEVAS from bypass, then continue to bypass FBEVAS 'A'
SAT			UNSAT		(UNSAT requ	ires comments)
STE 2.	P		ELEMENT Examinee goes to	section 4.7		STANDARD Section 4.7 removing BOP ESFAS Modules From Bypass is entered.
SAT			UNSAT		(UNSAT requ	ires comments)
STE 3.	P		ELEMENT Examinee identifie module is not tripp		AS	STANDARD Examinee identifies prerequisites are met. Goes to step 4.7.3.1.
SAT			UNSAT		(UNSAT requ	ires comments)
STE 4.	P		ELEMENT Perform a lamp tes	st on BOP ESFAS	'B'	STANDARD Lamp test is performed No intentional burnt out bulbs.
SAT			UNSAT		(UNSAT requ	ires comments)
CO	MN	MENT	°S:			



STEP	ELEMENT		STANDARD
5.	Perform the appropriate section belo	ow:	Identifies section 4.7.4. as the appropriate
	• 4.7.4 Removing FBEVAS, CRI	EFAS	section.
	and CPIAS Modules from Bypa		
SAT	UNSAT	(UNSAT regi	uires comments)
		` 1	,
STEP	ELEMENT		STANDARD
6.	If the BOP ESFAS module(s) is not		Determines FBEVAS Train 'B' not tripped and
	tripped, them GO TO step 4.7.4.		goes to step 4.7.4.
SAT	UNSAT	(UNSAT regi	uires comments)
5/11		(OTVB/11 Teq	dires comments)
STEP	ELEMENT		STANDARD
7.	Remove Train 'B' FBEVAS module	from	Only the 'Bypass' light is lit.
/•	bypass by performing all the following		Only the Dypuss light is it.
	oypuss by performing an are follows		If requested cue: Concurrent Verification has
	1. Check that all lights except the		been performed.
	'BYPASS' light are clear (not lit) for	or	performed.
	module to be removed from Bypass.		
	module to be femoved from Bypass.	•	
SAT	UNSAT	(UNSAT regi	uires comments)
		` 1	,
STEP	ELEMENT		STANDARD
8. *	2. Turn Bypass key for the desired	module	Turn Bypass key for Train 'B' FBEVAS
	counterclockwise approximately 1/4 t		counterclockwise approximately ¼ turn.
			11 3
SAT	UNSAT	(UNSAT requ	uires comments)
COMMEN	NTS:		
		-	



STEP	ELEMENT		STANDARD
9.	3. Remove the key.		Key is removed.
	4. Check 'BYPASS' light is clear ((not	
	lit).		Key may be used to bypass Train 'A' FBEVAS
	5. Independently verify the module been removed from bypass.	e has	without returning it to the key locker.
	6. Return bypass key to key storage	e	If requested cue: Independent verification has
	location.		been performed.
SAT	UNSAT	(UNSAT requi	ires comments)
STEP	ELEMENT		STANDARD
10.	Examinee returns to section 4.6 and		Lamp test performed.
	performs a lamp test on BOP ESFAS	S train	
	'A'.		No intentional burnt out bulbs.
SAT	UNSAT	(UNSAT requi	ires comments)
STEP	ELEMENT		STANDARD
11.	Check That the redundant module		Addressed previously.
11.	(FBEVAS Train 'B') is not in bypass	s.	Traditional provisions,
SAT	UNSAT	(UNSAT requi	ires comments)
		(01.0111.014.01	
STEP	ELEMENT		STANDARD
12.	Place a check mark in the column pro	ovided	Examinee places check mark on RU-31/
	for the module(s) to be placed in By		FBEVAS 'A' row.
CAT			
SAT	UNSAT	(UNSAT requi	ires comments)
COMMENT	vC.		
COMMENT	5:		



y clockwise approximately 1/4 I the bypass light comes on. SAT (UNSAT I	If requested cue: Independent verification is complete.	

NORMAL TERMINATION POINT

COMMENTS:			



RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	04/01/03	6	New JPM, TES.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number) Procedure upgrade
- 3. Procedure upgrade.
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

The following plant conditions exist:

- The CRS has directed you to place BOP ESFAS FBEVAS 'A' in bypass in accordance with 40OP-9SA01 SECTION 4.6, due to RU-31 power supply degradation.
- The CRS and STA have reviewed applicable LCO's and ODCM requirements.
- Prerequisites have **NOT** been preformed.

SAFETY CONSIDERATIONS:



NRC EXAM 2003

JPM BASIS INFORMATION

TASK: 0150030901 Per TASK STANDARD: E K/A: P-S01S-004-020-A K/A: P-S01S-004-020-A APPLICABLE POSITION REFERENCES: 400P-9C SUGGESTED TESTING	Dilute the RCS. Stop Dil 4-06 4-01 N(S): RO H01 CVCS Normal Ope	ution when System A K/A RATING: K/A RATING: VALIDAT erations	Auto Stop Fails. RO: 3.6 RO: 3.8 TION TIME:	SRO: SRO: 10 min	
	A	PPROVAL			
DEVELOPER: T S REVISION DATE: 4/2		TECH REVIE APPROVAL:	W:		
	TEST	ING METHO	D		
ACTUAL TESTING ENV	VIRONMENT: SIN	MULATOR		PLANT	
TESTING METHOD:	SIMULATE	PERFO	ORM		
	EV	ALUATION			
EXAMINEE NAME:					
EVALUATOR NAME:		(p:	rint)		
		(p	rint)		
SATISFACTORY	UNSATI	SFACTORY		_	
Time Start	Time Stop				
REMEDIAL TRAINING (SEE OTG-04		ES	NO		



NRC EXAM 2003

1. SIMULATOR SETUP:

- A. IC#: Reset to any at power steady state IC.
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	After dilution is started:	This override causes the dilution to continue after 40
	IOR ZDCHFQIS210X RESET	gallons have been added.
	To activate:	The operator can stop the make-up by going to auto on
	 Go to page CV10 	the mode select switch, or closing CHN-UV210X (in
	• Click on FQIS 210X in the upper	either manual or auto on the flow controller.).
	right of the screen.	
	 Select 'Override Switches'. 	
	 Select 'Insert Override' 	
	 Select 'Reset' 	

C. SPECIAL INSTRUCTIONS:

- Reset both Reactor Makeup Water Flow Totalizers to 0.
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

• none



NRC EXAM 2003

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- You are directed to dilute the RCS by adding 40 gallons of Reactor Makeup Water, at a rate of 40 gpm, to the charging pump suction (Using 40OP-9CH01 and with CHN-FIC-210X in automatic).
- All Prerequisites are complete.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Supervisor of in-plant JPM performance.

SAFETY CONSIDERATIONS:

None



NRC EXAM 2003

STEP 1.	ELEMENT Obtain procedure	STANDARD 40OP-9CH01 obtained.
1.	40OP-9CH01.	
SAT	UNSAT (UN	SAT requires comments)
STEP	ELEMENT	STANDARD
2.	If a small change in boron concentration is desired (less than 1 ppm) Then determine	
	the gallons of Reactor Makeup Water	-
	required.	Note: steps 7.3.2 through 7.3.5 are N/A due to the small volume of Reactor Makeup Water being added.
SAT	UNSAT (UN	SAT requires comments)
STEP	ELEMENT	STANDARD
3.	Set RMW controller CHN-FIC-210X to flow rate as determined in step 7.3.1.	Examinee adjusts Setpoint to 40 gpm
SAT	UNSAT (UN	SAT requires comments)
STEP	ELEMENT	STANDARD
4. *	Set RMW flow totalizer FQIS-210X to desired volume (40 gal).	Totalizer set to 40 gal.
	ν, ο, ,	Note: step 7.3.8 is N/A
SAT	UNSAT (UN	SAT requires comments)
COMMI	FNTS.	
COMM	EN13.	



NRC EXAM 2003

STEP	ELEMENT	STANDARD	
5. *	Start the dilution as follows:		
	1. Place CHN-HS-210 in the DILUTE	Dilute selected.	
	position.		
	2. Depress the "Reset" pushbutton – the	Controller reset.	
	left pushbutton.		
	3. Depress the "Start" pushbutton – The	Start pushbutton depressed.	
	left pushbutton on the		
	Totalizer/counter module (Micro-Motion)		
	wouldn')		
SAT	UNSAT (UNSAT	requires comments)	
	(1	
STEP	ELEMENT	STANDARD	
6.	Check for both of the following:	Examinee checks:	
	 Verify an RMW pump running. 	RMW pump running.	
	CHN-FIC-210Y indicates no flow,	No boration flow.	
	(CHN-FV-210Y closed)		
SAT	UNSAT (UNSAT	requires comments)	
STEP	ELEMENT	STANDARD	
7.	Verify CHN-UV-527, Makeup to Charging	CHN-UV-527 is open in 'auto'.	
	Pumps, is open.		
SAT	UNSAT (UNSAT	requires comments)	
SAT	UNSAT (UNSAT	requires comments)	
STEP	ELEMENT	STANDARD	
8.	On the Foxboro Module check that	GPM flow indicated 40 ± 5 .	
•	'Process Flow' increases towards the Auto	-	
	setpoint, over shoots the Auto setpoint and		
	the stabilizes at the Auto setpoint.		
SAT	UNSAT (UNSAT	requires comments)	
COMMEN	NTS:		



NRC EXAM 2003

STEP 9.	ELEMENT Determines that Auto Dilution did not stop when 40 gallons was injected.	STANDARD Recognizes Auto dilution fails.
SAT	UNSAT (UNSAT I	requires comments)
STEP 10. *	ELEMENT Stop the dilution by performing any one of the following: a) Decrease auto setpoint on CHN-FIC-210X to 0%. b) Place controller in manual and decrease output to 0%. c) Select "AUTO" on the Make-up Mode Selector Switch.	STANDARD Dilution stopped prior to 75 gallons being added. After dilution is stopped give the following Cue. IMFORM CUE: Another RO will restore Make-up to normal after the totalizer failure is investigated.
SAT	UNSAT (UNSAT I	requires comments) NORMAL TERMINATION POINT
COMME	NTS:	
		_



NRC EXAM 2003

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
1	4/16/03	3	Revision 28 of 40OP-9CH01

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- You are directed to dilute the RCS by adding 40 gallons of Reactor Makeup Water, at a rate of 40 gpm, to the charging pump suction (Using 40OP-9CH01 and with CHN-FIC-210X in automatic).
- All Prerequisites are complete.

SAFETY CONSIDERATIONS:



JPM BASIS INFORMATION

JEWI DASIS INFORMATION			
TASK: 1250030401, Perform Actions for Loss of NC TASK STANDARD: RCPs are tripped, and Seal Bleedoff Isolated K/A: 3.4-003-A2.02 K/A RATING: RO: 3.7 SRO: 3.7 K/A: K/A RATING: RO: SRO: APPLICABLE POSITION(S): RO VALIDATION TIME: 3 min REFERENCES: 40AO-9ZZ03, Loss of Cooling Water SUGGESTED TESTING ENVIRONMENT: SIMULATOR X PLANT FNOTE: Alternate path JPM FNOTE: Time Critical JPM			
APPROVAL			
DEVELOPER: T Stahler TECH REVIEW: REVISION DATE: 04/25/03 APPROVAL:			
TESTING METHOD			
ACTUAL TESTING ENVIRONMENT: SIMULATOR PLANT			
TESTING METHOD: SIMULATE PERFORM			
EVALUATION			
EXAMINEE NAME:			
(print) EVALUATOR NAME:			
(print)	_		
SATISFACTORY UNSATISFACTORY			
Time Start Time Stop			
REMEDIAL TRAINING REQUIRED? YES NO SEE OTG-04)			



1. SIMULATOR SETUP:

A. IC#: 20

B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	IMF MV03:NCBUV403	Spurious closure of NCW supply upstream containment isolation
		valve. (in containment.)
2.	When NCBUV403 is closed	Mechanically fails NCB-UV403 in the closed position.
	Then IMF MV06:NCBUV403	
3.	Acknowledge alarms.	

C. SPECIAL INSTRUCTIONS:

- Go to Run
- Enter Malfunctions
- Ack any alarms
- Provide Initiating Cue.
- Go to Run on Simulator
- D. REQUIRED CONDITIONS:
- NCB-UV403 closed.

2. SPECIAL TOOLS/EQUIPMENT:



JS8 PVNGS JOB PERFORMANCE MEASURE 2003 NRC EXAM TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE: SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- The plant is operating at 100% power.
- A Loss of Nuclear Cooling Water to the RCPs has occurred.
- The CRS directs you to perform Section 4 of 40AO-9ZZ03, Loss of Cooling Water.
- This is a Time Critical JPM.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Supervisor of in-plant JPM performance.

SAFETY CONSIDERATIONS:



STEP	ELEMENT		STANDARD
1.	If seal injection is in service and co	ooling	Examinee will note time.
	water is NOT restored to any oper	rating	
	RCP within 10 minutes of the initi	ial loss,	
	THEN perform ALL of the follow	ving:	
			START TIME:
	Ensure that the Reactor is tripped.		
	Stop all of the RCPs		
	<u>Isolate</u> controlled bleedoff.		
SAT	UNSAT	(UNSAT requ	ires comments)
STEP	ELEMENT		STANDARD
2.	IF no Nuclear Cooling Water pur		Examinee will determine that a NCW Pump is
	running, AND at least one is available.		operating.
	THEN perform ALL of the follow	ving to	
	start a NC Pump.		
SAT	UNSAT	(LINSAT requ	ires comments)
JA1	UNSAT	_ (ONSAT Tequ	ines comments)
STEP	ELEMENT		STANDARD
3.	IF at least one Nuclear Cooling W	/ater	Examinee will determine that operating NCW
.	Pump is running, AND "NCWS P		Pump is operating normally and no low
	DSCH HDR PRESS HI-LO" (7A0		discharge pressure alarm exists.
	alarm due to low pressure, THEN		Step is marked N/A
	ANY of the following:		•
	_		
SAT	UNSAT	(UNSAT requ	ires comments)
COMMEN	TC		
COMMEN	18:		



STEP 4.	ELEMENT IF ANY of the NC Containment Isolation Valves have failed closed, AND there is NOT a valid CSAS signal present, THEN perform Both of the following:	STANDARD Examinee will recognize that NCB-UV-403 has failed closed.
	Open ANY closed isolation valves.	Examinee will attempt to open NCB-UV-403. It will not open.
		If Examinee recommends locally operating NCB-UV-403 THEN:
		Inform Cue: The CRS has determined a containment entry can not be performed.
SAT	UNSAT (UNSAT	requires comments)
STEP 5.	ELEMENT IF ANY of the NC Containment Isolation Valves will NOT open, THEN perform ALL of the following:	STANDARD
	1) Close all NC CTMT isolation valves.	Examinee closes NC Containment Valves NCB-UV-401 and NCA-UV-402.
SAT	UNSAT (UNSAT	requires comments)
STEP 6. *	ELEMENT 2) Ensure that the Reactor is tripped.	STANDARD Examinee trips the Reactor. If requested cue: another RO has verified Reactivity Control Safety function is met.
SAT	UNSAT (UNSAT	requires comments)
COMME	NTS:	



STEP	ELEMENT	STANDARD
7. *	3) Stop all of the RCPs.	Examinee stops all of the RCPs.
SAT	UNSAT	(UNSAT requires comments)
STEP	ELEMENT	STANDARD
8. *	<u>Isolate</u> seal bleedoff.	Isolate seal bleedoff from all RCPs
		Inform CUE: The CRS will evaluate TECH Spec 3.6.3 Other ROs will perform SPTA's.
		STOP TIME:
		NOTE: JPM must be completed within 10 minuts.
SAT	UNSAT	(UNSAT requires comments)
COMME	NTS:	NORMAL TERMINATION POINT



RECORD OF REVISIONS

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



JS8 PVNGS JOB PERFORMANCE MEASURE 2003 NRC EXAM INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- Operate the simulator as you would the unit.
- The examiner will provide all responses and indications required from outside the control room.
- Vocalize what indications you are checking and why.
- You may use any source of information normally available

INITIATING CUE:

- The plant is operating at 100% power.
- A Loss of Nuclear Cooling Water to the RCPs has occurred.
- The CRS directs you to perform Section 4 of 40AO-9ZZ03, Loss of Cooling Water.
- This is a Time Critical JPM.

SAFETY CONSIDERATIONS:



ADMIN TASK BASIS INFORMATION

TASK: 1270057402 Direct Power Ascension Above 20%. TASK STANDARD: Complete Appendix O of 40OP-9ZZ05 for planned power ascension from 40% to 60% at 4 EFPD. Calculated dilution 1900 ± 100 gallons. K/A: 2.1.20 K/A RATING: RO: 4.3 SRO: 4.2 APPLICABLE POSITION(S): RO/SRO VALIDATION TIME: 30 Minutes REFERENCES: 40OP-9ZZ05, Power Operations SUGGESTED TESTING ENVIRONMENT: SIMULATOR X PLANT					
A	APPROVAL				
DEVELOPER: Joe W. Allison REVISION DATE: 6/13/01	TECH REVIEW: APPROVAL:				
TEST	TING METHOD				
ACTUAL TESTING ENVIRONMENT: SI	IMULATOR PLANT				
TESTING METHOD: SIMULATE	PERFORM				
EV	ALUATION				
EXAMINEE NAME:					
(print) EVALUATOR NAME:					
	(print)				
SATISFACTORY UNSAT	TISFACTORY				
Time Start Time Stop					
REMEDIAL TRAINING REQUIRED?	YES NO				



1. SIMULATOR SETUP:

- A. IC#: N/A
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	N/A	

- C. SPECIAL INSTRUCTIONS:
- None
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

- Core Data Book Unit S Cycle 7.
- Simulator computers to LAN and open Boron OAP.
- Clean, current copy of 40OP-9ZZ05, Appendix O.



RA1-1 PVNGS JOB PERFORMANCE MEASURE NRC 2003 EXAM TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

- Use Core Data Book S cycle 7 (simulator core data book)
- You may use any source of information normally available.

INITIATING CUE:

Given the following conditions:

- The reactor is critical at 40% power, BOL, 4 EFPD following a Refueling Outage.
- Power ascension to 60% is planned over the next 12 hours.
- RCS temperature for the power ascension will remain "On Program".
- RCS Boron Concentration is 1200 ppm.
- Reactor Engineering has provided the following information:

Parameter	Initial	Final
Reg CEA Position	150	150
PLCEA Position	150	150
[Xenon]	72%	70%
[Iodine]	40%	49%
Reactivity (Xe)	-1839	-1786

• The Power Change Worksheet program is not available.

You have been directed to calculate a dilution using 40OP-9ZZ05 Power Operations, Appendix O, Power Change Worksheet manual version.



INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW NUREG-1021.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Manager of in-plant JPM performance.

SAFETY CONSIDERATIONS:



NRC 2003 EXAM			
STEP 1.	*	ELEMENT Enter the requested information on the Appendix O, Manual Power Change Worksheet.	Note: Examinee refers to values in the Core Data Book to find the corresponding reactivity value/worth. Examinee enters initial and final reactivity values for the following:
COM	MEN	TS:	
_			



STEP 2. *	ELEMENT Determines net pcm (total) and delta rho boron (pcm)	STANDARD Determines net pcm of -235 pcm and delta rho boron of +235 pcm
SAT	UNSAT (UN	ISAT requires comments)
STEP 3. *	ELEMENT Determines delta ppm boron	STANDARD -30ppm Delta ppm boron calculated
SAT	UNSAT (UN	SAT requires comments)
STEP 3. *	ELEMENT Examinee determines delta gallons dilution amount using the BORON OAP	STANDARD Examinee calculates a required dilution amount to be 1900 ± 100 gals.
SAT	UNSAT (UN	SAT requires comments)
		NORMAL TERMINATION POINT
COMME	NTS:	



RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	3/10/99	6	New Admin Task JPM
1	8/12/99	6	Modified JPM steps to enhance CUE's, more clearly identify critical steps, and enhance required band of required dilution.
2	8/29/99	6	Modified Iodine numbers to represent actual numbers.
3	6/13/01	6	Modified JPM for 2001 Audit to require a Manual Calculation.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



RA1-1 PVNGS JOB PERFORMANCE MEASURE NRC 2003 EXAM INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

- Use Core Data Book S cycle 7 (simulator core data book)
- You may use any source of information normally available.

INITIATING CUE:

Given the following conditions:

- The reactor is critical at 40% power, BOL, 4 EFPD following a Refueling Outage.
- Power ascension to 60% is planned over the next 12 hours.
- RCS temperature for the power ascension will remain "On Program".
- RCS Boron Concentration is 1200 ppm.
- Reactor Engineering has provided the following information:

Parameter	Initial	Final
Reg CEA Position	150	150
PLCEA Position	150	150
[Xenon]	72%	70%
[Iodine]	40%	49%
Reactivity (Xe)	-1839	-1786

• The Power Change Worksheet program is not available.

You have been directed to calculate a dilution using 40OP-9ZZ05 Power Operations, Appendix O, Power Change Worksheet manual version.



JPM BASIS INFORMATION

TASK: 1280010801, Perform Surveillance Test TASK STANDARD: Completes Steps 8.1-8.8 in ≤ 15 minutes and notifies CRS of criteria not met. K/A: 2.1.19 K/A RATING: RO: 3.0 SRO: APPLICABLE POSITION(S): RO VALIDATION TIME: 12 minutes REFERENCES: 72ST-9RX03 (COLSS out of service) SUGGESTED TESTING ENVIRONMENT: SIMULATOR X PLANT						
APPROVAL						
DEVELOPER: T Stahler TECH REVIEW: REVISION DATE: 4/29/03 APPROVAL:						
TESTING METHOD						
ACTUAL TESTING ENVIRONMENT: SIMULATOR PLANT						
TESTING METHOD: SIMULATE PERFORM						
EVALUATION						
EXAMINEE NAME:						
(print) EVALUATOR NAME:						
(print)						
SATISFACTORY UNSATISFACTORY						
Time Start Time Stop						
REMEDIAL TRAINING REQUIRED? YESNO						



1. SIMULATOR SETUP:

- A. Classroom setting
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	IMF RJ01	COLSS Failure
2.		
3.		
4.		

- C. SPECIAL INSTRUCTIONS:
- none
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

- Clean, Current copy of 72ST-9RX03. with data from cue entered in .
- Clean Current copy of Unit 1 COLR rev. 9.



S/RA1-2 PVNGS JOB PERFORMANCE MEASURE 2003 NRC EXAM TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

• You may use any source of information normally available.

INITIATING CUE:

The following plant conditions exist:

- The CRS has just declared COLSS Inoperable/Out of Service.
- Reactor Power is 100%.
- All (4) four CPC Channels are operable and <u>not</u> tripped.
- CEAC's are Operable
- Section 7.0 Prerequisites of 72ST-9RX03 are completed.
- The following data was obtained from CPCs for use in step 8.2

Parameter	CPC 'A'	CPC 'B'	CPC 'C'	CPC 'D'
DNBR (PID 406)	+1.923	+1.926	+1.920	+1.919
ASI (PID 187)	-0.0231	-0.0229	-0.0221	-0.0233
LHR (PID 179)	+12.50	+12.48	+12.47	+12.52

The CRS directs you to perform steps 8.1 through 8.8 of 72ST-9RX03 and immediately inform him of any acceptance criteria not met.

This is a time critical JPM.

INFORMATION FOR EVALUATOR'S USE:

• Steps 8.1 through 8.8 of 72ST-9RX03 are time critical and must be completed <u>within 15minutes</u> from the time COLSS was declared inoperable. This should be measured from the time the CUE is completed and understood by the candidate.

SAFETY CONSIDERATIONS:

• None



S/RA1-2 PVNGS JOB PERFORMANCE MEASURE 2003 NRC EXAM

STEP	ELEMENT	STANDARD
1.	Record the date and time that COLSS was	Examinee enters the data.
	declared out of service in Appendix A and	
	page 3 of 3 of Appendix B.	Inform CUE: The date of the loss is today's date and the time is 1 minute ago.
		Record actual time Initiating Cue was given:
SAT	UNSAT (UNSAT requ	uires comments)
STEP	ELEMENT	STANDARD
2.	Examinee refers to data on Appendix A	Note: Data has been entered in procedure.
SAT	UNSAT (UNSAT requ	ires comments)
STEP	ELEMENT	STANDARD
3. *	If any DNBR value is outside the region of	Examinee determines that DNBR valves are
	acceptable operation specified in the COLR	outside the region of acceptable operation, and
	for Tech. Spec. 3.2.4c or Tech. Spec. 3.2.4.d. <u>Circle</u> the value in Appendix A.	circles the values as appropriate.
SAT	UNSAT (UNSAT requ	ires comments)
STEP	ELEMENT	STANDARD
4. *	Determine success or failure of the DNBR surveillance using Appendix A criteria (NOTE 1)	Examinee applies criteria from Appendix A and determine that the DNBR surveillance is a failure. Examinee denotes "NO" in Appendix "A" as appropriate.
SAT	UNSAT (UNSAT requ	ires comments)
COMMEN	ΓS:	



S/RA1-2 PVNGS JOB PERFORMANCE MEASURE 2003 NRC EXAM

STEP ELEMENT

5. * If DNBR LCO is not met in step 8.4, then commence monitoring DNBR at intervals of \leq 15 minutes per Appendix C to satisfy Tech. Spec. 3.2.4 action B.1

STANDARD

Examinee informs CRS that the DNBR LCO is not met and that monitoring is required at intervals of \leq 15 minutes.

Inform Cue: Another RO will monitor DNBR at 15 minute intervals.

SAT	UNSAT (UN	SAT requires comments)
STEP 6.	ELEMENT If any LHR valve exceeds the maximum value specified in Tech. Spec. 3.2.1, circle the value Appendix A.	
SAT	UNSAT (UN	SAT requires comments)
STEP 7.	ELEMENT Determine success or failure of the LHR Surveillance using Appendix A criteria (NOTE 2).	STANDARD Examinee determines that the LHR surveillance succeeded. Examinee denotes "YES" in Appendix "A" as appropriate.
SAT	UNSAT (UN	SAT requires comments)
STEP 8.	ELEMENT If the LHR LCO is not met in step 8.7, the commence monitor LHR at intervals of ≤ 15 minutes per Appendix D. To satisfy Tech. Spec. 3.2.1 Action B.1	Examinee determines that the LHR LCO is met and informs the CRS, prior to exceeding 15 minutes time limit. Record actual time step 8.8 complete: *Total Time:(< 15 mins. Critical element)-
SAT	UNSAT (UN	SAT requires comments) NORMAL TERMINATION POINT
COMM	ENTS:	



S/RA1-2 PVNGS JOB PERFORMANCE MEASURE 2003 NRC EXAM

RECORD OF REVISIONS

REVISION DATE	REASON REVISED	COMMENTS
09/03/98	6	New JPM, R.L.
06/14/01	6	Added COLSS failure malfunction to simulator setup to provide more realism.
05/08/03	6	Added values to cue to allow performance in classroom setting.
	09/03/98 06/14/01	09/03/98 6 06/14/01 6

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



S/RA1-2 PVNGS JOB PERFORMANCE MEASURE 2003 NRC EXAM INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

INITIATING CUE:

The following plant conditions exist:

- The CRS has just declared COLSS Inoperable/Out of Service.
- Reactor Power is 100%.
- All (4) four CPC Channels are operable and <u>not</u> tripped.
- CEAC's are Operable
- Section 7.0 Prerequisites of 72ST-9RX03 are completed.
- The following data was obtained from CPCs for use in step 8.2

Parameter	CPC 'A'	CPC 'B'	CPC 'C'	CPC 'D'
DNBR (PID 406)	+1.923	+1.926	+1.920	+1.919
ASI (PID 187)	-0.0231	-0.0229	-0.0221	-0.0233
LHR (PID 179)	+12.50	+12.48	+12.47	+12.52

The CRS directs you to perform steps 8.1 through 8.8 of 72ST-9RX03 and immediately inform him of any acceptance criteria not met.

This is a time critical JPM.

SAFETY CONSIDERATIONS:

• None



NRC EXAM 2003

JPM BASIS INFORMATION

TASK: 1290310301 Perform a Tech Revie TASK STANDARD: Tech Review a Permi K/A: 2.2.13 APPLICABLE POSITION(S): RO REFERENCES: 40DP-90P29, Permit and Tag Drawings 01-E-DWB-01 & 01-M-DWP-02 SUGGESTED TESTING ENVIRONMENT:	t and determine three errors K/A RATING: RO: 3.6 SRO: 3.8 VALIDATION TIME: 20 minutes gging Process
A	PPROVAL
DEVELOPER: T Stahler REVISION DATE: 5/03/03	TECH REVIEW: APPROVAL:
TEST	ING METHOD
ACTUAL TESTING ENVIRONMENT: SI	MULATOR PLANT
TESTING METHOD: SIMULATE	PERFORM
EV	ALUATION
EXAMINEE NAME:	
EVALUATOR NAME:	(print)
	(print)
SATISFACTORY UNSAT	ISFACTORY
Time Start Time Stop	
REMEDIAL TRAINING REQUIRED? Y	YES NO



NRC EXAM 2003

1. SIMULATOR SETUP:

- A. IC#: Any at power IC (20 preferred)
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.		
2.		
3.		
4.		

- C. SPECIAL INSTRUCTIONS:
- None
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

• Copy of Test Permit 1-050603-1 Permit Details and Tag Assignment Sheet.



NRC EXAM 2003

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

• You may use any source of information normally available.

INITIATING CUE:

Unit 1 is 100% power

SWMS is down.

The CRS has directed you to perform Tech Review of Permit 1-050603-1.

- Identify three (3) errors (Non-clerical not typos).
- Determine any required action(s) that need to be done as a result of these 3 errors.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW NUREG-1021.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Manager of in-plant JPM performance.

SAFETY CONSIDERATIONS:

None



NRC EXAM 2003

STEP 1.		ELEMENT Reviews Permit or Work Order to determine scope of work to be perform	med.	STANDARD Examinee reviews Permit or Work Order and determines work scope to be replacing gasket on 1P-DWN-V050.
SAT		UNSAT	(UNSAT requi	ires comments)
STEP 2.	*	ELEMENT Reviews Tag Assignment Sheet and P to verify Permit adequacy for job scop		 Examinee determines the following inaccuracies/inadequacies. Tag 2 is the wrong circuit breaker (breaker is for the "A" pump) Tag 5 has wrong position (OPEN) for the discharge valve. Tag 7 has right valve but wrong system DS.
SAT		UNSAT	(UNSAT requi	ires comments)
STEP 3.	*	ELEMENT Examinee returns Permit for correction the Preparer	on by	Examinee returns Permit for correction (i.e. deficiencies noted shall not be corrected by the Tech Reviewer).
SAT _		UNSAT	(UNSAT requi	ires comments)
				NORMAL TERMINATION POINT
COM	MEN	ГS:		



NRC EXAM 2003

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	05/10 /01	6	New JPM.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



NRC EXAM 2003

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

• You may use any source of information normally available.

INITIATING CUE:

Unit 1 is 100% power

SWMS is down.

The CRS has directed you to perform Tech Review of Permit 1-050603-1.

- Identify three (3) errors (Non-clerical not typos).
- Determine any required action(s) that need to be done as a result of these 3 errors.

SAFETY CONSIDERATIONS:

None



2003 NRC EXAM

ADMIN TASK BASIS INFORMATION

TASK: 1290020301 Conduct On Shift Operations IAW Cotansk: 1290010301 Implement verification of plant activity (TASK STANDARD: Determine proper REP task, determine K/A 2.3.10 K/A RATIN K/A: 2.3.4 K/A RATIN APPLICABLE POSITION(S): RO VALUE (REFERENCES: NGW01, Initial Radiation Worker Practices. SUGGESTED TESTING ENVIRONMENT: SIMULATOR (SUGGESTED TESTING ENVIRONMENT: SIMULATOR)	ties. ne RCA entry requirements. G: RO: 2.9 SRO: 3.3 G: RO: 2.5 SRO: 3.1 DATION TIME: 20 minutes	
APPROVAL	ı	
DEVELOPER: T Stahler TECH RE REVISION DATE: 05/03/03 APPROV		
TESTING METI	HOD	
ACTUAL TESTING ENVIRONMENT: SIMULATOR _	PLANT X	
TESTING METHOD: SIMULATE PI	ERFORM X	
EVALUATIO	N	
EXAMINEE NAME:		
EVALUATOR NAME:	(print)	
	(print)	
SATISFACTORY UNSATISFACTORY		
Time Start Time Stop	_	
REMEDIAL TRAINING REQUIRED? YES	NO	



2003 NRC EXAM

SIMULATOR SETUP:

IC#: N/A

MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	N/A	

SPECIAL INSTRUCTIONS:

• None

REQUIRED CONDITIONS:

• None

SPECIAL TOOLS/EQUIPMENT:

- A copy of the Unit 3 outage REP.
- A copy of the Pressurizer Spray Valve galleries RP survey maps.
- A copy of 40TD-9RC01 section 2.



2003 NRC EXAM

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

• You may use any source of information normally available.

A copy of the following is attached:

- Unit 3 outage REP.
- Pressurizer Spray Valve galleries RP survey maps.
- 40TD-9RC01 section 3.

INITIATING CUE:

Given the following initial conditions:

- Unit 3 is in a refueling outage.
- Pressurizer Spray Valve RCE-PV-0100F has been isolated.
- You have been directed to drain and depressurize the Loop 1B Pressurizer Spray Valve RCE-PV-100F using 40TD-9RC01 Reactor Coolant System Step 3.2.6.

Your tasks are to:

- 1. Determine proper task for this evolution.
- 2. Is a RP Pre-Job Brief required prior to entering Pressurizer spray valve RCE-PV-100F valve gallery?
- 3. Determine RP coverage during job performance.
- 4. Determine dress-out requirements.
- 5. Determine required EPD settings.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set, then ensure the examinee has been briefed IAW NUREG 1021.

Step sequence is not critical unless noted or will prevent achieving the task standard.

SAFETY CONSIDERATIONS:

none



2003 NRC EXAM

STEP 1. *	ELEMENT Examinee reviews REP and survey a determines task he can enter on.	STANDARD Examinee determines entry on task 2 is required.
SAT	UNSAT	(UNSAT requires comments)
STEP 2. *	ELEMENT Examinee determines if RP pre-job le required for entry into Pressurizer Spalve RCE-PV-100F valve gallery.	
SAT	UNSAT	(UNSAT requires comments)
STEP 3. *	ELEMENT Examinee determines RP coverage requirements during job performance	STANDARD RP coverage is continuous for connect and disconnect of vent/drain equipment. Otherwise intermittent coverage is required.
SAT	UNSAT	(UNSAT requires comments)
STEP 4. *	ELEMENT Examinee determines dress-out requirements	STANDARD Clothing/protection requirements are Double set. Full set for containment entry. Second set for valve gallery. (RP may determine and authorize modified Double Set).
SAT	UNSAT	(UNSAT requires comments)
COMMEN	TTS:	



	20	003 NRC EXAM
STEP 5. *	ELEMENT Determine the REP Dosemetry requirements.	STANDARD Examinee determines EPD is required with settings of 25 mRem dose and 500 mREM/hr Dose Rate (as stated on the REP)
SAT	UNSAT	(UNSAT requires comments)
		NORMAL TERMINATION POINT
COMME	ENTS:	



2003 NRC EXAM

1.	Determine pr	oper task	for	this	evolution.
----	---------------------	-----------	-----	------	------------

Proper task is task 2.

2. Is a RP Pre-Job Brief required prior to entering Pressurizer spray valve RCE-PV-100F valve gallery?

Yes.

3. Determine RP coverage during job performance.

RP coverage is continuous for connect and disconnect of vent/drain equipment. Otherwise intermittent coverage is required.

4. Determine dress-out requirements.

Clothing/protection requirements are Double set. Full set for containment entry.

Second set for valve gallery. (RP may determine and authorize modified Double Set).

5. Determine required EPD settings.

Examinee determines EPD is required with settings of 25 mRem dose and 500 mREM/hr Dose Rate (as stated on the REP)

COMMENTS:					



2003 NRC EXAM

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	05/03/03	6	New Admin Task JPM

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



2003 NRC EXAM

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

• You may use any source of information normally available.

A copy of the following is attached:

- Unit 3 outage REP.
- Pressurizer Spray Valve galleries RP survey maps.
- 40TD-9RC01 section 3.

INITIATING CUE:

Given the following initial conditions:

- Unit 3 is in a refueling outage.
- Pressurizer Spray Valve RCE-PV-0100F has been isolated.
- You have been directed to drain and depressurize the Loop 1B Pressurizer Spray Valve RCE-PV-100F using 40TD-9RC01 Reactor Coolant System Step 3.2.6.

Your tasks are to:

- 1. Determine proper task for this evolution.
- 2. Is a RP Pre-Job Brief required prior to entering Pressurizer spray valve RCE-PV-100F valve gallery?
- 3. Determine RP coverage during job performance.
- 4. Determine dress-out requirements.
- 5. Determine required EPD settings.

SAFETY CONSIDERATIONS:

• None



2003 NRC EXAM

1. Determine proper task for this evolution.
2. Is a RP Pre-Job Brief required prior to entering Pressurizer spray valve RCE-PV-100F valve gallery?
3. Determine RP coverage during job performance.
4. Determine dress-out requirements.
5. Determine required EPD settings.

Appendix C						
SAFETY FUNCTION DETERMINATION TRACKING SHEET						
SFD#:	IN	OPERABLE SUPPORT I	FEATURE NAME / EQ ID#			
3-99-SFD-XXXX		EC / 3M				
LCO / Reqd. action		e & time LCO Entered	TSCCR#			
3.7.10 /ACTION		9:00	3-99-XXXX			
Restrictions and additional **Required TRM Act		ours.				
** Required Cascade If there is a need to exit Determination Worksheet' **OD's used or at	ed Tech Spec Action 3.0.6, then the required in the Support / Support flected – NONE	ons red cascaded actions are fo orted Feature Table.	and in Appendix B "Safety Function			
**Special instruction This SFDP suppo performed on the	Additional information (references): **Special instructions: This SFDP supports train B outage work to be performed in Unit 3 starting Today. Work is to be performed on the "b" train EC. When EC becomes inoperable:					
2) Invoke I	, , ,					
**Expected condition Mode 1 – condition	ns while SFDP is it ons as described above					
**Clarification for action of condition – NONE **Changes / Revisions with approvals – NONE						
PREPARED BY:		Samey Thomas NT & SIGN NAME	<u>today 07:00</u> DATE / TIME			
APPROVED BY:	APPROVED BY					
SHIFT MANAGER or CRS PRINT & SIGN NAME DATE / TIME (Preparer can NOT sign Approval)						
Closeout comments:						
C1 1 D17						
Closed BY:	SHIFT MANAGER o	r CRS PRINT & SIGN NAM	E DATE / TIME			

Appendix B Safety Function Determination Worksheet SFD 3-03-XXXX

INOPERABLE SUPPORT FEATURE

NAME/EQUIPMENT	Redundant	LCO/	LCO	Inoperable	Operable
	feature Operable? Y/N	required action	Completion Time	Date/Time	Date/Time
3MECBP01	Y	3.7.10.A.1	72 hrs	TODAY 0900	

SUPPORT/SUPPORTED FEATURE

Name/Equip ID	Redundant	LCO	LCO	MOST	Inoperable	Operable
	Feature	required	Completion			
	operable?	action #	time		Date/Time	Date/Time
	Y/N					
ECCS operating LPSI	Y	3.5.3.A.1	168 hrs	240 hrs	Today	
					0900	
ECCS operating HPSI	Y	3.5.3.B.1	72 hrs	144 hrs	Today	
					0900	
3MSIBP03 (CS)	Y	3.6.6.A.1	72 hrs	144 hrs	Today	
					0900	
3MEWBP01 (EW)	Y	3.7.7.A.1	72 hrs	144 hrs	Today	
					0900	
3MAFBP01 (AF)	Y	3.7.5.B.1	240 hrs	312 hrs	Today	
					0900	
3MHJBF04 (CREFS)	Y	3.7.11.A.1	168 hrs	240 hrs	Today	
					0900	



ADMIN IA	SK BASIS INFORMATION
TASK: 1290190102 Ensure Compliance TASK STANDARD: Review the compliance K/A: 2.1.12 APPLICABLE POSITION(S): SRO REFERENCES: 40DP-90P37, Safety Further Suggested Testing Environment	oleted SFDP forms, annotate three significant errors. K/A RATING: RO: 2.9 SRO: 4.0 VALIDATION TIME: 30 Minutes unction Determination Procedure
	APPROVAL
DEVELOPER: W. Drey REVISION DATE: 7/31/00	TECH REVIEW: APPROVAL:
TF	ESTING METHOD
ACTUAL TESTING ENVIRONMENT:	SIMULATOR PLANT
TESTING METHOD: SIMULATE	PERFORM
	EVALUATION
EXAMINEE NAME:	
EVALUATOR NAME:	(print)
	(print)
SATISFACTORY UN	SATISFACTORY
Time Start Time Stop	<u> </u>
REMEDIAL TRAINING REQUIRED? (SEE 15TD-0TR03)	YES NO



1. SIMULATOR SETUP:

- A. IC#: N/A
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	N/A	

- C. SPECIAL INSTRUCTIONS:
- None
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

- A marked up copy of 40DP-9OP37, Safety Function Determination Procedure, Appendix B, Safety Function Determination Worksheet and Appendix C, Safety Function Determination Tracking Sheet.
- Technical Specifications.



TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

• You may use any source of information normally available.

INITIATING CUE:

- Unit 3 is in Mode 1 at 100% power.
- The B Essential Chiller (ECB-E01) is on the schedule to be removed from service for routine maintenance at 0900.
- LCO 3.0.6 will be implemented.
- The STA has performed all required sections of 40DP-9OP37, Safety Function Determination Procedure, including Appendix B, Safety Function Determination Worksheet and Appendix C, Safety Function Determination Tracking Sheet.
- You, as the CRS, are to review and approve the Safety Function Determination.
- Identify at least three errors (non-clerical, non-typos) with corrections noted to the evaluator.
- The current time for purposes of this task is 0700 on today's date.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW NUREG-1021.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Manager of in-plant JPM performance.

SAFETY CONSIDERATIONS:

None



STEP 1.	ELEMENT Examinee obtains 40DP-9OP37, Safety	STANDARD Procedure is obtained.			
	Function Determination Program.	EVALUATOR NOTE: Steps 2, 3, and 4 may be performed in any order.			
		NOTE: If the candidate starts to perform 40DP-9OP37, Safety Function Determination Program procedure, then provide the following CUE:			
		Inform CUE: This portion of the procedure has already been completed by the STA.			
SAT	UNSAT (UNSA	T requires comments)			
STEP 2.	ELEMENT Review Appendix C, Safety Function Determination Tracking Sheet.	STANDARD NOTE: An incorrect reference is made to 40ST-9ZZ02, Inoperable Power Sources Action Statement on the Appendix C.			
		Error - incorrect Surveillance test is noted. (This is the critical portion of this step.)			
		The correct surveillance under the special instructions on Appendix C is 40ST-9EC03, Essential Chill Water & Ventilation System Inoperable Action Surveillance.			
SAT	UNSAT (UNSA	T requires comments)			
COMM	ENTS:				



STEP 3.	*	ELEMENT Review Appendix B, Safety Function Determination Worksheet.	STANDARD Error - incorrect LCO Completion Time of 240 hrs. and/or Maximum Out of Service Time (MOST) of 312 hrs for AFB-P01 are noted. (This is the critical portion of this step.)
			The correct LCO Completion Time, and Maximum Out of Service Time (MOST) are as follows:
			• LCO Completion Time – 72 hours.
			 Maximum Out of Service Time (MOST) 144 hours. 72 hours from Auxiliary Feedwater plus 72 hours from Essential Chill Water System LCO 3.7.10 Action A.1)
			NOTE: The incorrect LCO Completion Time entry of 240 hrs. has driven the MOST time error. If examinee counts this error as two errors, inform the examinee that this entry counts as one error.
SAT		UNSAT	(UNSAT requires comments)
COM	MEN	TS:	



STEP 4.	*	ELEMENT Refer to Appendix D, Support and Supported LCO Matrix.		STANDARD NOTE: An incorrect reference is made to LCO 3.7.11.A.1 Control Room Essential Filtration System (CREFS).
				Error - incorrect Supported LCO is noted. (This is the critical portion of this step.)
				The correct Supported LCO on the Appendix C is as follows:
				 Supported LCO - 3.7.12 CREATCS. (From Appendix D) The LCO completion time should be 720 hours and MOST time should be 792 hours on Appendix B.
				NOTE : The incorrect LCO Entry has resulted in incorrect LCO completion time and MOST times for LCO 3.7.12. The entry into the wrong LCO and wrong LCO completion time and wrong MOST time calculations count as one error. If examinee counts these errors as three errors, inform the examinee that this entry counts as one error.
SAT		UNSAT	(UNSAT req	uires comments)
STEP 5.		ELEMENT Examinee reports three errors and capprove Appendix C.	loes not	STANDARD Examinee reports three errors and does not approve Appendix C without corrections.
SAT		UNSAT	UNSAT req	uires comments)
				NORMAL TERMINATION POINT
COM	MEN'	TS:		



Appendix C		
SAFETY FUNC	TION DETERMINATION T	RACKING SHEET
SFD#: 3-99-SFD-XXXX	INOPERABLE SUPPORT I EC / 3M	FEATURE NAME / EQ ID# 3CBE01
LCO / Reqd. action(s) 3.7.10 /ACTION A.1	Date & time LCO Entered Today 9:00	TSCCR# 3-99-XXXX
**Required TRM Actions TRM T3.5.201.A.1 – res		
Determination Worksheet" in the **OD's used or affecte	then the required cascaded actions are for Support / Supported Feature Table.	,
Additional information (reference **Special instructions: This SFDP supports train performed on the "b" train	B outage work to be performed in Unit 3 sta	arting Today. Work is to be
When EC becomes inope	rable:	
	0.A.1 (EC)and 3.3.11.B.1 (RSDP), and T3 6 for LCOs listed in Appendix B (attached)	
3) Perform 40ST-9 Should be Perfo	ZZ02.	
**Expected conditions white Mode 1 – conditions as described as a condition of the conditions with the conditions white conditions white conditions are conditions as described by the condition of the conditions are conditions as described by the condition of the conditions are conditions as described by the condition of the condition o	escribed above. of condition – NONE	
PREPARED BY: Same	ey Thomas Samey Thomas PRINT & SIGN NAME	today 07:00 DATE / TIME
	MANAGER or CRS PRINT & SIGN NAM (Preparer can NOT sign	
Closeout comments:		
Closed BY:SHIFT	MANAGER or CRS PRINT & SIGN NAM	E DATE / TIME



Appendix B Safety Function Determination Worksheet SFD 3-03-XXXX

INOPERABLE SUPPORT FEATURE

NAME/EQUIPMENT	Redundant	LCO/	LCO	Inoperable	Operable
	feature Operable? Y/N	required action	Completion Time	Date/Time	Date/Time
3MECBP01	Y	3.7.10.A.1	72 hrs	TODAY 0900	

SUPPORT/SUPPORTED FEATURE

Name/Equip ID	Redundant	LCO	LCO	MOST	Inoperable	Operable
	Feature	required	Completion			
	operable?	action #	time		Date/Time	Date/Time
	Y/N					
ECCS operating LPSI	Y	3.5.3.A.1	168 hrs	240 hrs	Today	
					0900	
ECCS operating HPSI	Y	3.5.3.B.1	72 hrs	144 hrs	Today	
					0900	
3MSIBP03 (CS)	Y	3.6.6.A.1	72 hrs	144 hrs	Today	
					0900	
3MEWBP01 (EW)	Y	3.7.7.A.1	72 hrs	144 hrs	Today	
					0900	
3MAFBP01 (AF)	Y	3.7.5.B.1	240 hrs	312 hrs	Today	
			Should be	Should be	0900	
			72 hrs	144 hrs		
3MHJBF04 (CREFS)	Y	3.7.11.A.1	168 hrs	240 hrs	Today	
Should be			Should be	Should be	0900	
(CREATCS)			720 hrs	192 hrs		



RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	3 /24/99	6	New Admin Task JPM
1	8/12/99	6	Clarified initiating CUE.
2	8/29/99	6	Modified initiating CUE and CUEs in JPM to facilitate examinee performance.
3	7/31/00	6	Clarified CUEs and corrected typos in sample form.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

• You may use any source of information normally available.

INITIATING CUE:

- Unit 3 is in Mode 1 at 100% power.
- The B Essential Chiller (ECB-E01) is on the schedule to be removed from service for routine maintenance at 0900.
- LCO 3.0.6 will be implemented.
- The STA has performed all required sections of 40DP-9OP37, Safety Function Determination Procedure, including Appendix B, Safety Function Determination Worksheet and Appendix C, Safety Function Determination Tracking Sheet.
- You, as the CRS, are to review and approve the Safety Function Determination.
- Identify at least three errors (non-clerical, non-typos) with corrections noted to the evaluator.
- The current time for purposes of this task is 0700.

SAFETY CONSIDERATIONS:

• None



Appendix C SAFETY FUNCTION DETERMINATION TRACKING SHEET					
SFD#: 3-99-SFD-XXXX	INOPERABLE SUPPORT I EC / 3M	FEATURE NAME / EQ ID#			
LCO / Regd. action(s)	Date & time LCO Entered	TSCCR#			
3.7.10 /ACTION A.1	Today 9:00	3-99-XXXX			
Restrictions and additional actions: **Required TRM Actions TRM T3.5.201.A.1 – resto					
Determination Worksheet" in the St	the required cascaded actions are fou upport / Supported Feature Table.	and in Appendix B "Safety Function			
**OD's used or affected		ODE CIEIED			
**Support Features req	uired Position – NONE S	PECIFIED			
**Special instructions: This SFDP supports train I performed on the "b" train When EC becomes inopera 4) Enter LCO 3.7.10 5) Invoke LCO 3.0.6 6) Perform 40ST-9Z **Expected conditions while Mode 1 – conditions as de **Clarification for action of **Changes / Revisions with	Additional information (references): **Special instructions: This SFDP supports train B outage work to be performed in Unit 3 starting Today. Work is to be performed on the "b" train EC. When EC becomes inoperable: 4) Enter LCO 3.7.10.A.1 (EC)and 3.3.11.B.1 (RSDP), and T3.5.201.a.1 (SDC). 5) Invoke LCO 3.0.6 for LCOs listed in Appendix B (attached).				
PREPARED BY: Same	y Thomas Samey Thom				
	PRINT & SIGN NAME	DATE / TIME			
APPROVED BY:					
SHIFT N	MANAGER or CRS PRINT & SIGN (Preparer can NO)				
Closeout comments:	\ A	~ ^^ /			



Appendix B Safety Function Determination Worksheet SFD 3-03-XXXX

INOPERABLE SUPPORT FEATURE

NAME/EQUIPMENT		LCO/	LCO	Inoperable	Operable
	feature Operable? Y/N	required action	Completion Time	Date/Time	Date/Time
3MECBP01	Y	3.7.10.A.1	72 hrs	TODAY 0900	

SUPPORT/SUPPORTED FEATURE

Name/Equip ID	Redundant	LCO	LCO	MOST	Inoperable	Operable
	Feature	required	Completion			
	operable?	action #	time		Date/Time	Date/Time
	Y/N					
ECCS operating LPSI	Y	3.5.3.A.1	168 hrs	240 hrs	Today	
					0900	
ECCS operating HPSI	Y	3.5.3.B.1	72 hrs	144 hrs	Today	
					0900	
3MSIBP03 (CS)	Y	3.6.6.A.1	72 hrs	144 hrs	Today	
					0900	
3MEWBP01 (EW)	Y	3.7.7.A.1	72 hrs	144 hrs	Today	
					0900	
3MAFBP01 (AF)	Y	3.7.5.B.1	240 hrs	312 hrs	Today	
					0900	
3MHJBF04 (CREFS)	Y	3.7.11.A.1	168 hrs	240 hrs	Today	
					0900	

OTG-02 Rev. 1



2003 NRC EXAM

JPM BASIS INFORMATION

TASK: 12B0010202, Review TASK STANDARD: Identify f K/A: 2.2.12 APPLICABLE POSITION(S): REFERENCES: SUGGESTED TESTING ENVIR	our (4) errors and Direct Boration K/A RATING SRO VALID 72ST-9RX14, Shutdown Man	E: RO: SRO: 3.4 ATION TIME: 15 min. rgin Modes 3,4,and 5
	APPROVAL	
DEVELOPER: T. Stahler REVISION DATE: 05/01/03	TECH REV APPROVA	
	TESTING METH	OD
ACTUAL TESTING ENVIRON	MENT: SIMULATOR	PLANT
TESTING METHOD: SIMU	LATE PER	RFORM
	EVALUATION	J
EXAMINEE NAME:		
EVALUATOR NAME:		(print)
		(print)
SATISFACTORY	UNSATISFACTORY	
Time Start T	ime Stop	
REMEDIAL TRAINING REQU	RED? YES	NO



2003 NRC EXAM

1. SIMULATOR SETUP:

- A. IC#: No Simulator setup required. This JPM may be run in-plant or in the Simulator.
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	N/A	
2.		
3.		
4.		

- C. SPECIAL INSTRUCTIONS:
- None
- D. REQUIRED CONDITIONS:
- None

2. SPECIAL TOOLS/EQUIPMENT:

• Completed copy of 72ST-9RX14 with 4 errors.



2003 NRC EXAM

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- You may use any source of information normally available.
- Use Unit S Cycle 7 Core Data Book

INITIATING CUE:

- Unit 2 is in mode 4 in a short notice outage.
- The Core is at 130 EFPD
- RCS Boron is 890 ppm per sample (today @ 06:00)
- RCS Temperature is 250°F
- All CEAS are fully inserted, RTSG breakers are open
- The Reactor has been shutdown for 120 hours
- A heat-up is planed for this shift with the RCS expected to be at 450°F.

The required Shutdown Margin Modes 3,4,and 5 Surveillance Test (72ST-9RX14), for an expected RCS temperature of 450°F is complete.

Your task is to:

- Perform the Team Leader review of the Surveillance as the CRS.
- Identify 4 Errors (Non-clerical, not typos)
- Determine any required action(s) that need to be done as a result of these 4 errors to achieve desired plant conditions.
- Markup procedure as needed to assist in place keeping in 72ST-9RX14.

INFORMATION FOR EVALUATOR'S USE:

* Denotes Critical Step

SAFETY CONSIDERATIONS:

None



STEP 1.	ELEMENT Verify the completion of the prerequisites section.	STANDARD Examinee verified that Section 7.0 is initialed as complete.
SAT	UNSAT (UNSAT r	equires comments)
STEP 2.	ELEMENT Verify that the applicable Step(s) of Section 7 were performed	STANDARD Examinee determines that Section 8.1 is required when all full length CEAs fully inserted.
SAT	UNSAT (UNSAT r	equires comments)
STEP 3. *	ELEMENT Verify the data in Step 8.1.2 is correct for this performance.	Examinee notes that the RCS Boron is incorrectly listed as 980 ppm Should be 890 ppm, from initiating cue. Additionally a cognitive error in the Tcold picked (350 °F) is the one requiring the least amount of Boron. Step 6.5 is referenced, it directs selection the Tcold that requires the most amount of Boron. Two errors are addressed in this step.
SAT	UNSAT (UNSAT r	equires comments)
COMME	NTS:	



STEP 4.			it S	Examinee verifies correct Boron. The value of 825 ppm is correct for the Temperature selected (350 °F) in the previous step. However the required Boron is 1000 ppm at 450 °F. Note incorrect Boron value 3 rd error.
				Two medicer bolon value 3 citor.
SAT		UNSAT	(UNSAT requ	uires comments)
STEP 5.	*	ELEMENT Verify the response to Step 8.1.4 is a based on actual values.	correct,	STANDARD Examinee determines that Step 8.1.4 directs continuing to step 8.1.5. 'Current Boron' is less than required.
SAT		UNSAT	(UNSAT requ	uires comments)
STEP 6.		ELEMENT Evaluate Step 8.1.5 for inclusion of negative reactivity from Xenon		STANDARD Examinee determines that (at 85 hours past shutdown) the Core is essentially Xenon free. Cannot use Xenon adjustment.
				Inform CUE: Reactor Engineering has determined the Xenon Reactivity to be 'zero'.
SAT		UNSAT	(UNSAT requ	uires comments)
COM	MEN	TS:		



STEP 7.	*	ELEMENT ACCEPTANCE CRITERIA (Step 8. Current RCS is greater than Required Boron (Step 8.1.3) Acceptance Criteria Satisfied No.	l RCS o.	STANDARD The examinee determines that the ACCEPTANCE CRITERIA is NOT met. This is the 4 th error.
SAT		UNSAT	(UNSAT requi	ires comments)
STEP 8.	*	ELEMENT Address Step 8.1.11. If Acceptance Criteria is not satisfied perform All of the following: 1) Notify the CRS. 2) Borate the RCS to restore Shutdown Margin within lin Notify Reactor Engeneering.I		Examinee determines Boration is required to achieve shutdown margin within limit. (1000 ppm for 450 °F.) Required action.
SAT		UNSAT	(UNSAT requi	ires comments)
				NORMAL TERMINATION POINT
СОМ	MENT	ΓS:		



2003 NRC EXAM

Four Errors

- 1. RCS Boron from latest Chemistry sample (Step 8.1.2) is transposed:
 - a. should be 890 ppm
 - b. recorded as 980 ppm
- 2. Most conservative Toold (Step 8.1.2): a. Should be 450° F.

 - b. Recorded as 350 °F.
- 3. Required Boron (Step 8.1.3):
 - a. Should be 1000 ppm
 - b. Recorded as 825 ppm
- 4. ACCEPTANCE ERITERIA (Step 8.1.9)
 - a. Should be **NOT** Satisfied.
 - b. Recorded as Satisfied

COMMENTS:		



2003 NRC EXAM

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	09/08/98	6	New JPM, R.L.
1	05/01/03	6	Modified to make Tc selected one of the errors.

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



SRO Admin Task A3 (Previously SRO JPM A.2) PVNGS JOB PERFORMANCE MEASURE

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS

- You may use any source of information normally available.
- Use Unit S Cycle 7 Core Data Book

INITIATING CUE:

- Unit 2 is in mode 4 in a short notice outage.
- The Core is at 130 EFPD
- RCS Boron is 890 ppm per sample (today @ 06:00)
- RCS Temperature is 250°F
- All CEAS are fully inserted, RTSG breakers are open
- The Reactor has been shutdown for 120 hours
- A heat-up is planed for this shift with the RCS expected to be at 450°F.

The required Shutdown Margin Modes 3,4,and 5 Surveillance Test (72ST-9RX14), for an expected RCS temperature of 450°F is complete.

Your task is to:

- Perform the Team Leader review of the Surveillance as the CRS.
- Identify 4 Errors (Non-clerical, not typos)
- Determine any required action(s) that need to be done as a result of these 4 errors to achieve desired plant conditions.
- Markup procedure as needed to assist in place keeping in 72ST-9RX14.



ADMIN TASK BASIS INFORMATION

ADMIN TASK DASIS INFORMATION
TASK: 1290270202 Authorize Gaseous Radioactive Release. TASK STANDARD: Identify three (3) errors on the existing Release Permit K/A: 2.3.9 K/A RATING: RO: 2.5 SRO: 3.4 K/A: K/A RATING: RO: SRO: SRO: APPLICABLE POSITION(S): SRO VALIDATION TIME: 15 minutes REFERENCES: 40OP-9GR01 Gaseous Radioactive Waste Procedure 74RM-9EF20, Gaseous Radioactive Release Permits and Offsite Dose Assessment Rev 7. SUGGESTED TESTING ENVIRONMENT: SIMULATOR X PLANT
APPROVAL
DEVELOPER: Tom Stahler TECH REVIEW: REVISION DATE: 05/1/03 APPROVAL:
TESTING METHOD
ACTUAL TESTING ENVIRONMENT: SIMULATOR PLANT
TESTING METHOD: SIMULATE PERFORM
EVALUATION
EXAMINEE NAME:
EVALUATOR NAME: (print)
EVALUATOR NAME: (print)
SATISFACTORY UNSATISFACTORY
Time Start Time Stop
REMEDIAL TRAINING REQUIRED? YESNO



2003 NRC EXAM

1. SIMULATOR SETUP:

- A. IC#: N/A
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	None	

- C. SPECIAL INSTRUCTIONS:
 - None
- D. REQUIRED CONDITIONS:
 - None

2. SPECIAL TOOLS/EQUIPMENT:

- Effluent Release Permit
- Copy of 40OP-9GR01, Operation of the Gaseous Radwaste System Section 5.0
- Copy of 74RM-9EF20, Gaseous Radioactive Release Permits and Offsite Dose Assessment.



2003 NRC EXAM

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

• You may use any source of information normally available.

INITIATING CUE:

- A Unit 1 Waste Gas Decay Tank release permit has been generated to account for a planned radioactive release.
- The effluent tech brings you the release permit for review per 40OP-9GR01, Operation of the Gaseous Radwaste System and 74DP-9EF20, GASEOUS RADIOACTIVE RELEASE PERMITS AND OFFSITE DOSE ASSESSMENT.
- You are to review the release permit.
- Identify three errors (non-clerical, non-typos) to the evaluator.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW OTG-01.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Manager of in-plant JPM performance.
- This JPM is based on Step 5.1.2 of 40OP-9CP01. Examinee may refer to 74RM-9EF20.

SAFETY CONSIDERATIONS:

• None



STEF 1.	ELEMENT Obtains 74RM-9EF20 Section 8.0.	STANDARD Examinee obtains 74RM-9EF20 Section 8.0.
SAT	UNSAT	(UNSAT requires comments)
-		
STEF 2.	* Verifies permit number correct, per s 8.1.1.1.	STANDARD The decimation has a discrepancy, the Release Permit Request and the Release Log lists 'WGDT' 'B', the Release Permit lists WGDT 'A'.
SAT _	UNSAT	(UNSAT requires comments)
STEF 3.	ELEMENT Verifies the Start date/time and the expiration date/time are correct, per 8.1.1.2.	Note: the duration time (5.72 hrs) is a calculated
		value based on the tank pressure and flow rate. The permit expiration should be longer than this time.
SAT	UNSAT	(UNSAT requires comments)
COM	IMENTS:	



STEP	*	ELEMENT	1 -1 1	STANDARD
4.	4. * Verifies the sample data present and signed for, collected and analyzed and required Lower Limits of Detection for all release samples have been met.		iired	Examinee determines sample data present. If Requested Cue: all results entered as 'Zero' were less than Lower Limits of Detection. Sample results have been reviewed. Note: The examinee should identify that the sample is much greater than the LCO. The XE-133 value was inadvertently listed as 7.67E+3. Actual value was 7.67E-3.
				Critical nature of this step is to determine that release limits are exceeded.
SAT		UNSAT	(UNSAT requi	ires comments)
STEP 5.		ELEMENT Verifies all Technical Specification ODCM Surveillances have been per		STANDARD Examinee identifies proper signature For 74ST- 9SQ06 on Release Permit. If the examinee want to review 74ST-9SQ06 then give the following cue. If Requested Cue: All required RU and flow instruments are available with correct setpoints as verified by 74ST-9SQ06.
SAT		UNSAT	(UNSAT requi	ires comments)
COM	MENT	S:		



STEP		ELEMENT		STANDARD
6.	*	Verifies release is approved by appro	priate	Examinee determines that the permit does not
0.		level per step 8.2.2	Pilate	have RP approval
		level per step 0.2.2		nave to approvan.
				Note: After 3 errors are found give the
				following Cue:
				lone wing eac.
			Ī	Inform Cue: The RP supervisor will review
				the release and correct the noted errors.
			_	the release and correct the noted errors.
SAT		IINICAT	(INCAT seems	
SAI		UNSAT	(UNSAT requi	ires comments)
				NORMAL TERMINATION POINT
COMN	MENT	'S:		



2003 NRC EXAM

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	05/05/03	6	New

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



SA3 PVNGS JOB PERFORMANCE MEASURE 2003 NRC EXAM

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

• You may use any source of information normally available.

INITIATING CUE:

- A Unit 1 Waste Gas Decay Tank release permit has been generated to account for a planned radioactive release.
- The effluent tech brings you the release permit for review per 40OP-9GR01, Operation of the Gaseous Radwaste System and 74DP-9EF20, GASEOUS RADIOACTIVE RELEASE PERMITS AND OFFSITE DOSE ASSESSMENT.
- You are to review the release permit.
- Identify three errors (non-clerical, non-typos) to the evaluator.

SAFETY CONSIDERATIONS:

• None



ADMIN TASK BASIS INFORMATION TASK: 1240100202 Classify events requiring emergency plan implementation 1240100302 Direct an emergency response as the emergency coordinator (EC) 1240100402 Determine protective action recommendations (PAR) An Alert is declared within 15 minutes; form EP-0541 is filled out, Notification directed within 15 minutes of classification K/A: 2.4.38 K/A RATING: SRO: 4.0 APPLICABLE POSITION(S): SRO VALIDATION TIME: 15 minutes REFERENCES: EPIP-01, Satellite Technical Support Center Actions Rev. 13 SUGGESTED TESTING ENVIRONMENT: SIMULATOR X PLANT	
TIME CRITICAL	
APPROVAL DEVELOPER: T. Stahler TECH REVIEW: REVISION DATE: 05/01/03 APPROVAL:	
TESTING METHOD ACTUAL TESTING ENVIRONMENT: SIMULATOR PLANT TESTING METHOD: SIMULATE PERFORM	
EVALUATION EXAMINEE NAME: (print)	
EVALUATOR NAME:	
(print)	
SATISFACTORY UNSATISFACTORY	
Time Start Time Stop	
REMEDIAL TRAINING REQUIRED? YESNO	



NRC EXAM 2003

1. SIMULATOR SETUP:

- A. IC#: 18, At the end of NRC scenario #1.
- B. MALFUNCTIONS, OVERRIDES & REMOTE FUNCTIONS:

EVENT	COMMAND	DESCRIPTION
1.	NRC LOIT Scenario #1	ESD / ATWAS

C. SPECIAL INSTRUCTIONS:

• Ensure the following:

All alarms are acknowledged on RMS.

The simulator is in FREEZE following completion of Scenario #1

D. REQUIRED CONDITIONS:

none

2. SPECIAL TOOLS/EQUIPMENT:

- Form EP-0541 available, blocks 1, 4 and 6 complete.
- A watch or clock to time event classification and notifications.



NRC EXAM 2003

TASK CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

• You may use any source of information normally available.

INITIATING CUE:

The following plant conditions apply:

- You are in Unit 1.
- A plant event resulting an ATWAS, with PPS setpoints exceeded, occurred. The Reactor Trip was completed by de-energizing CEDMCS from the Control Room. Following the Reactor Trip an ESD outside containment occurred.

Based on this event and current conditions, perform all On-shift Emergency Coordinator duties for this event until properly relieved.

• This is a time critical JPM.

INFORMATION FOR EVALUATOR'S USE:

- * Denotes Critical Step
- This JPM has two (2) Time Critical elements. The first is to classify the event within 15 minutes from the time the initiating cue is given, the second is to initiate/direct the NAN notification by the STSC Communicator within 15 minutes from the time the classification is made.
- At the discretion of the Examiner/Evaluator, this JPM may be terminated when the Task Standard is met or adequate time has been allowed to complete the JPM. It shall be terminated when the Examinee has verbalized completion of the JPM.
- Any step marked UNSAT requires comments.
- If this is the first JPM of the set then ensure the examinee has been briefed IAW NUREG-1021.
- Step sequence is not critical unless noted or will prevent achieving the task standard.
- Notify unit Shift Manager of in-plant JPM performance.

SAFETY CONSIDERATIONS:

• None



STEP 1.	ELEMENT Obtain Procedure EPIP-01		STANDARD EPIP-01 obtained.
			Note to evaluator : The critical time to classify the event is 15 minutes from the time the examinee has received the initiating cue.
			START TIME:
SAT	UNSAT	(UNSAT re	quires comments)
STEP 2. *	ELEMENT Determines EAL Level currently or exceeded.(5.4)	y being met	Uses Appendix A and determines EAL as Failure of Reactor Protection System Instrumentation to Complete or Initiate a Automatic Reactor Scram Once a Reactor Protection Setpoint has Been Exceeded and Manual Scram Was Successful. Note to evaluator: This step may be performed at Step 4. If requested CUE: An RO reports that all four channels of DNBR and LPD were in 'TRIP' prior to de- energizing CEDMCS.
SAT	UNSAT	(UNSAT rea	quires comments)
COMME	ENTS:		



STEP	ELEMENT	STANDARD
3.	Directs the Onshift STA or another EC qualified individual to independently vEAL determination. (4.1)	Directs the On-shift STA or another EC
	EAL determination: (4.1)	If requested CUE: The STA concurs with your determination
SAT	UNSAT (UNSAT requires comments)
STEP 4. *	ELEMENT Classify the event. (4.1)	STANDARD Classifies event as an ALERT within 15 minutes of step 1 START TIME.
		Record CLASSIFICATION TIME:
		*Total time to classify (Critical ≤15 min)
		Record NOTIFICATION START TIME:
		above)
		If requested CUE: The STA concurs with your determination
SAT Steps 5 and	UNSAT (d 6 may be performed in any order	UNSAT requires comments)
COMME	NTS:	



STEP		ELEMENT	STANDARD	_
5.	*	Completes form EP-0541, Palo Verde	Examinee completes steps 3 and 5 of Form EP-	
		NAN Emergency Message Form. (4.2)	0541 as follows:	
			• Step 3 ALERT , UNIT 1	
			Status Code 5-4	
			• Step 5 NO Radioactive release is in	
			progress. NO Protective Actions are	
			required (see step 7 below)	
			After STSC Communicator completes steps 1,4	
			and 6	
			 Reviews form for accuracy and signs step 	
			6.	
			If requested CUE: The STSC Communicator	
			has arrived in the Unit 1 Control Room.	
			STSC Communicator(Examiner) provides	
			information for examinee to fill out steps 1,4 and 6 of Form EP-0541.	
			INFORM CUE: Provide the examinee with	
			the EP-0541 form with blocks 1,4, and 6 filled out:	
			Reviews form for accuracy and signs step 6.	
SAT		UNSAT (UNSAT	requires comments)	
STEP		ELEMENT	STANDARD	_
6.	*	Contact Security (CAS) (5.1)	Using the telephone or radio contacts CAS and	
			directs the CAS operator to notify the Security	
			Operations Section Leader to complete	
		Note: In Section 5.1, Procedure actions	supplemental notifications and activate the	
		are in Bold text. Non-bolded text is	auto dialer.	
		generally for information only.		
			If requested CUE:CAS has been notified.	
COM	MEN'	TS:		
				_
				_
				_
				_



SAT	UNSAT (UNSAT I	requires comments)
STEP 7.	ELEMENT Determine appropriate Protective Action Recommendations.(5.1)	STANDARD Consults Appendix B, Protective Action Recommendations. Protective Actions are NONE Recommended Note to evaluator: Examinee may have previously completed this action since it is
SAT		information to be included on form EP-0541. requires comments)
STEP 8. *	ELEMENT Direct the STSC Communicator to complete and transmit the Palo Verde NAN Emergency Message form.(5.1)	STANDARD Directs the STSC Communicator to transmit NAN form within 15 minutes of event CLASSIFICATION TIME in step 4.above.
	Note: This step may have been performed at step 5 above.	*Total time since NOTIFICATION START TIME (Critical <15 Minutes.) If requested CUE: The STSC Communicator
SAT	UNSAT (UNSAT 1	has initiated the NAN Message form requires comments)
COMME	ENTS:	
_		



STEP 9.	ELEMENT Notify Site Manager. (5.1) .	STANDARD Site Manager notified of the Emergency Situation and directed to come to the UNIT 1 Control room to assume the role of On-shift Emergency Coordinator.
		If requested CUE: Site Manager has been informed to report to the Unit 1 Control Room.
SAT	UNSAT	(UNSAT requires comments)
STEP 10.	ELEMENT Assemble the Onshift Emergency Response Organization (ERO) staff f	STANDARD Assembles ERO staff for briefing. or an
	initial briefing in the STSC general a (5.1)	Inform CUE: The Radiation Protection Monitor and Shift Technical Advisor have not yet arrived in the STSC. The briefing can be performed when the ERO staff has assembled in the STSC.
SAT	UNSAT	(UNSAT requires comments)
COMMEN	NTS:	



STEP		ELEMENT	STANDARD
11.	*	Conduct onsite notification using Step 5.1	As a minimum, step 5.1.5 "Standard
		Flowchart	Notification" message for ALERT is
			transmitted over the Unit Evacuation System
			Note: examinee may direct the following
			(Recommended unless the EC is fairly certain
			plant conditions will not deteriorate.):
			• "Assembly Notification" (step 5.1.20) and,
			· · · · · · · · · · · · · · · · · · ·
			• "Accountability Request"(step 5.1.3)
			If requested CUE: Notifications are
			complete.
			Inform CUE: the Site Manger has relieved you as the Emergency Coordinator.
SAT		UNSAT (UNSAT	requires comments)
			NORMAL TERMINATION POINT
COM	MEN	TS:	



NRC EXAM 2003

RECORD OF REVISIONS

REVISION NUMBER	REVISION DATE	REASON REVISED	COMMENTS
0	05/01/03	6	New Admin Task JPM for NRC EXAM 2003 scenario #1

REASON REVISED

Enter the numbers corresponding to the reason revised in the Reason Revised column and brief description of changes in Comments Column. Comments are to be numbered consecutively in each revision.

- 1. Vendor reference document upgrade
- 2. Plant modification (include number)
- 3. Procedure upgrade
- 4. Internal or External Agency Commitment (indicate item number)
- 5. Technical Specification Change (indicate amendment number)
- 6. Other (explain in comments)



NRC EXAM 2003

INITIAL CONDITIONS

INFORMATION PRESENTED TO EXAMINEE:

SPECIAL CONSIDERATIONS:

• You may use any source of information normally available.

INITIATING CUE:

The following plant conditions apply:

- You are in Unit 1
- A plant event occurred resulting an ATWAS, with PPS setpoints exceeded. The Reactor Trip was completed by de-energizing CEDMCS from the Control Room. Following the Reactor Trip an ESD outside containment occurred.

Based on this event and current conditions, perform all On-shift Emergency Coordinator duties for this event until properly relieved.

• This is a time critical JPM.

SAFETY CONSIDERATIONS:

• None

Facility: PVNGS	Scenario No.:	1	Op-Test No: 2003
Examiners:		Operators:	
-		-	
-	_	-	

Initial Conditions: IC #16, 50% power, MOC.

Turnover: The following equipment is out of service: HPSI pump "B" (6 hours); PW pump "B" (20 hours); DG "B" (2 hours). MFP "B" has been started and is ready to be placed in service to support increasing plant power to 100%.

Event No.	Malf. No.	Event Type*	Event Description
1		N (CO)	Place 'B MFP in service
			(CRS to direct and CO to perform)
2		R (ALL)	Power increase
			(CRS to direct and RO/CO to coordinate and perform)
3	TR01:MTNP	I (CO)	TLI 1 instrument fails high
	T11A 839	AOP	(CO to diagnose and perform actions and CRS to direct actions and refer to Tech Specs).
4	CV03A 0	C (RO)	CHN-UV-110P Flow control valve fails closed causing a loss of letdown
		AOP	(RO to diagnose and perform actions and CRS to direct recovery)
5	RD02E 100	C (ALL)	CEA#86 drops into core
	RD02F 100	AOP	Five minutes later a second CEA#89 drops into core
			(RO to diagnose and CRS to direct reactor trip)
6	ATWS	C (ALL)	Reactor Protection system failure to open Reactor Trip Switchgear breakers (PRA Significant)
			(Crew to diagnose and take action and CRS to Direct response)
			(Critical Task to trip reactor by opening L03 and L10 prior to leaving SPTAs)
7	MS03C 40 TD 7 min	M (ALL)	A main steam line on #2 SG breaks outside containment upstream of the MSIV's (after reactor trip EOP is entered)
TD / lilli			(Crew to diagnose and take actions and CRS to diagnose ESD and direct stabilization).
			(Critical task to stop feeding and steaming #2 SG)
End			Crew stabilizes heat removal on #1 SG
point			(Critical Task to control RCS parameters to prevent lifting Pzr Safeties)

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor NUREG-1021, Draft Revision 9

Appendix D Scenario Outline Form ES-D-1

Scenario Overview

- 1. The crew will complete the startup of 'B' MFP and then commence a power increase.
- 2. The crew will then experience a failure of a TLI instrument. This will require the crew to respond to the failure and take actions per RRS Malfunctions AOP. The crew will select the unaffected instrument per the AOP.
- 3. The crew will then experience a failure of the in service letdown flow control valve causing a loss of letdown. This will require the crew to stabilize CVCS and refer to Technical Specifications.
- 4. The crew will then experience a dropped CEA. The crew will respond to the dropped CEA and take actions per the CEA Malfunction AOP. Then a second CEA will drop requiring a manual reactor trip.
- 5. The crew will experience an ATWS condition. The crew is expected to open supply breakers for L03 and L10 in response to the ATWS.
- 6. When the crew has entered the Reactor Trip EOP a steam line on #2 SG will break inside containment resulting in an ESD.
- 7. The crew will transition from Reactor Trip to ESD and stabilize the plant following SG dryout.

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor NUREG-1021, Draft Revision 9

Supplemental Turnover

Plant conditions:

The unit is at 50% power, steady state conditions and core life is 225 EFPD. (Boron concentration is 831 PPM per chemistry sample.)

Equipment out of service:

Emergency Diesel Generator "B" is out of service for emergent work following discovery of a broken air connection on the safety system shutdown. T.S. 3.8.1 was entered 2 hours ago. Maintenance is expected to be completed in 4 hours.

High Pressure Safety Injection pump "B" is out of service for emergent work to replace a pump bearing that failed during its scheduled Surveillance Test 6 hours ago. Maintenance is expected to be completed in 14 hours. T.S. 3.5.3 condition B was entered 6 hours ago.

Plant Cooling Water pump "B" was removed from service 20 hours ago for scheduled maintenance on the pump motor. Maintenance is expected to be completed in 4 hours.

Planned shift activities:

The plant has been started up after an electrical grid disturbance caused a reactor/plant trip four (4) days ago. After startup, power was held at 50% for DFWCS testing and adjustment of nuclear instrumentation (NI). All tests were satisfactory.

Extended warm weather and the outages of several non-nuclear stations have caused a power shortage situation. Station management directs the crew to **immediately** begin a power ascension following turnover. ECC has been notified.

Procedure 40OP-9ZZ05 has been performed up to step 4.3.38.2.

Procedure 40OP-9FT02, Feedwater Pump Turbine, has been completed up to step 4.3.38 with MFP "B" currently at ~1000 rpm. Continue the MFP startup with step 4.3.38.1.

Following completion of the 'B' main feed pump startup, the crew is to recommence the power ascension to 100% power over the next 6 hours.

The required dilution has been calculated and verified by an STA to be 6511 gallons of Reactor Makeup Water (see power change worksheet). A dilution rate of 18 gpm will support the 8% per hour ascension rate allowed by 40OP-9ZZ05 fuel preconditioning guidelines.

The normal, shiftly surveillance's are complete.

Note:

The crew will walk down the control boards and assume the shift and then perform a reactivity brief prior to the commencement of the evaluation.

Facility: PVNC	<u>is</u>	Scenario No.:_		2	Op-Test No: 2003
Evaninan				Omenatona	
Examiners:			•	Operators:	

Initial Conditions: IC #20, 100% power, MOC.

Turnover: The following equipment is out of service: HPSI pump "B" (6 hours); PW pump "B" (20 hours); DG "B" has just completed a surveillance test run and is to be shutdown and placed in standby.

	nas jast complete	a a sar vern	and test rain and is to be shallown and placed in standay.
Event No.	Malf. No.	Event Type*	Event Description
1		N (CO)	Remove DG 'B' from service
			(CRS to direct and CO to perform)
2	DG06	C (CO)	DG 'B' trips when control switch is taken to stop
			(CO to diagnose and perform actions and CRS to refer to Tech Specs)
3	MC01A 3	R (ALL)	Condenser vacuum degrades requiring downpower
		AOP	(CO to diagnose and perform actions and CRS to direct stabilization)
4	CN01:CHNFI	I (RO)	Seal Injection flow controller fails valve closed
	C 241 100	AOP	(RO to diagnose and perform actions and CRS to direct recovery)
5	ED02	С	Grid disturbance results in a Loss of Offsite power
		(ALL)	(Crew to diagnose and perform actions and CRS to direct actions)
6	RD03G	C (RO)	Two CEAs fail to fully insert
	RD03L		(RO to diagnose and perform actions and CRS to direct boration)
			(Critical Task to establish boration to meet safety function requirements prior to completion of the SPTA's)
7	FW21B	С	Loss of Feedwater
		(CO)	(CO to diagnose and perform actions and CRS to direct recovery)
8	FW22	M	Loss of All Feedwater
		(ALL)	(Crew/CRS to diagnose and CRS to direct transition to FRP)
9			Crew transitions to FRP and cross ties PBA-S03 to PBB-S04
			(CO to perform actions and CRS to diagnose and direct actions)
			(Critical Task to establish feedwater to a SG prior to lifting primary safeties) (PRA Significant)
End point			Crew stabilizes plant with AFB feeding at least one SG.

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor NUREG-1021, Draft Revision 9

Scenario Overview

- 1. While securing the "B" Emergency Diesel Generator "B" from a test run it will trip/fail to properly stop. The CRS will declare it inoperable and enter T.S. 3.8.1.b action B.
- 2. Condenser vacuum will then degrade requiring the CRS to take the actions outlined in 40O-9ZZ07, Loss of Vacuum (a power reduction will be required). When an operator is sent to investigate, a leaking dogbone seal will be discovered. The water seal will be reestablished and vacuum will recover.
- 3. The crew will then experience a failure of the seal injection flow controller (CHNFIC241) resulting in a loss of seal injection flow to RCP 1A. The CRS will refer to 40AO-9ZZ04, Reactor Coolant Pump Emergencies for further information.
- 4. Grid instabilities in California will then result in a loss of the grid. This will result in a reactor trip due to a loss of offsite power.
- 5. Two CEA's will stick on the trip requiring the RO to initiate emergency boration. Following SPTA's the CRS will enter 40EP-9EO07, LOOP/LOFC (PBB-S04 is deenergized).
- 6. The crew will then experience a loss of feedwater (AFA-P01 will overspeed and AFN will trip on electrical fault) requiring the CRS to enter 40EP-9EO09, Functional Recovery (the CRS may enter 40EP-9EO06, Loss of All Feedwater first but due to a lack of MVAC, he will be redirected to the functional recovery procedure).
- 7. The CRS will jeopardize MVAC-2 and cross tie PBA-S03 powered by Diesel Generator "A" to PBB-S04 in order to use AFB-P01.
- 8. The scenario will terminate when feedwater is restored to at least one steam generator.

Supplemental Turnover Sheet

Plant Conditions:

The unit is at 100% power, steady state conditions and core life is 225 EFPD. (Boron concentration is 675 ppm per chemistry sample.)

Equipment out of service:

Emergency Diesel Generator "B" is running unloaded in test mode following a surveillance test.

High Pressure Safety Injection pump 'B' is out of service for emergent work to replace a pump bearing that failed during its scheduled Surveillance Test 6 hours ago. Maintenance workers expect to finish pump repairs in 14 hours from now. T.S. 3.5.3 condition B was entered 6 hours ago.

Plant Cooling Water pump 'B' was removed from service 20 hours ago for scheduled maintenance on the pump motor. Maintenance workers expect to finish pump repairs in 24 hours total time.

Planned shift activities:

The crew is to shutdown the diesel generator per 40OP-9DG02, Emergency Diesel Generator "B", continuing with step 7.3.74.

No other evolution's are planned at this time

The normal, shiftly surveillance's are complete.

Note:

The crew will walk down the control boards and assume the shift and then perform a reactivity brief prior to the commencement of the evaluation.

Appendix	D	Scenario Outline	Form ES-D-1

Facility: PVNGS S		Scei	nario No.:	3	Op-Test No:	2003
Examiners:			Operators:			
Initial Co	nditions: IC #20,	100% pow	er, MOC.			
hours); D		A leak in '	'A" low pressu	e: HPSI pump "B" (6 re feedwater heater s		
Event No.	Malf. No.	Event Type*	Event Description			
1		R (All)	Downpower to 80% power. (CRS to direct and RO/CO to coordinate and perform)			
2	RP06H1	I (RO) AOP	Inadvertent CSAS actuation (RO to diagnose and perform actions and CRS to direct and addresses Technical Specifications)			
3	TC13	C (All) AOP	Turbine Trip/ Load Reject/Reactor Power Cutback (Crew to diagnose and perform actions and CRS to direct stabilization)			
4	RD11B	C (All)	CEAs continue to insert/Manual Reactor Trip (Crew to diagnose and perform actions and CRS to direct reactor trip)			
5	TR01: SGNPT1024 (980)	I (CO)	(CO to diagnostabilization)	control system instructions and perform action to control RCS parameters)	ons and CRS to di	
6	ED02	M (All)	Loss of Off-Site Power on Reactor Trip (Crew to diagnose and CRS to direct actions) (Critical Task to establish feed to S/G's for level control)			
7	EG06A	C (CO)	"A" DG Fails	(PRA Significant) ose and CRS to direc		· ·
End point			CRS directs p	owering PBA-S03 w	rith gas turbine ge	enerator

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor NUREG-1021, Draft Revision 9

Scenario Overview

- 1. The crew will commence a downpower using boration in preparation for removing low pressure feedwater heater string "A" from service for emergent work.
- 2. An inadvertent CSAS will occur requiring the crew to stop the running containment spray pumps and override and close containment spray valves. The crew will address affected containment HVAC equipment and restore as necessary. CRS may address technical specifications 3.3.5 and 3.3.6.
- 3. A turbine trip/large load reject/reactor power cutback will then occur. The crew will respond and attempt to stabilize the plant.
- 4. Following the reactor power cutback, CEAs will continue to drive into the core. The crew will attempt to stop the rod motion but will be unsuccessful requiring the CRS to direct a manual Rx. trip.
- 5. Upon the reactor trip, SBCS will fail due to an instrument failure requiring the CO to manually control SG pressure and Tc.
- 6. The crew will perform SPTAs and may enter the Reactor Trip EOP. Once SG pressure is under control a loss of offsite power will occur requiring entry into the LOOP/LOFC EOP.
- 7. When the CRS enters into the LOOP/LOFC EOP, the 'A' diesel generator will trip requiring entry into the Blackout EOP.
- 8. The scenario will end when PBA-S03 has been reenergized using at least one GTG.

Appendix D	Scenario Outline	Form ES-D-1

Facility: PVNGS		Scei	nario No.:	3	Op-Test No:	2003
Examiners:				Operators:		
Initial Co	nditions: IC #20,	100% pow	er, MOC.			
hours); D	•	A leak in '	'A" low pressi	e: HPSI pump "B" ure feedwater heater etails.		-
Event No.	Malf. No.	Event Type*	Event Description			
1		R (All)	Downpower to 80% power. (CRS to direct and RO/CO to coordinate and perform)			
2	RP06H1	I (RO) AOP	Inadvertent CSAS actuation (RO to diagnose and perform actions and CRS to direct and addresses Technical Specifications)			
3	TC13	C (All) AOP	Turbine Trip/ Load Reject/Reactor Power Cutback (Crew to diagnose and perform actions and CRS to direct stabilization)			
4	RD11B	C (All)	CEAs continue to insert/Manual Reactor Trip (Crew to diagnose and perform actions and CRS to direct reactor trip)			
5	TR01: SGNPT1024 (980)	I (CO)	Steam bypass control system instrument failure (CO to diagnose and perform actions and CRS to direct stabilization) (Critical Task to control RCS parameters to prevent lifting Pressurizer Safeties)			
6	ED02	M (All)	Loss of Off-Site Power on Reactor Trip (Crew to diagnose and CRS to direct actions) (Critical Task to establish feed to S/G's for level control)			
7	EG06A	C (CO)	"A" DG Fails (PRA Significant) (CO to diagnose and CRS to direct transition to Blackout)			
End			CRS directs powering PBA-S03 with gas turbine generator			

point

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor NUREG-1021, Draft Revision 9

Scenario Overview

- 1. The crew will commence a downpower using boration in preparation for removing low pressure feedwater heater string "A" from service for emergent work.
- 2. An inadvertent CSAS will occur requiring the crew to stop the running containment spray pumps and override and close containment spray valves. The crew will address affected containment HVAC equipment and restore as necessary. CRS may address technical specifications 3.3.5 and 3.3.6.
- 3. A turbine trip/large load reject/reactor power cutback will then occur. The crew will respond and attempt to stabilize the plant.
- 4. Following the reactor power cutback, CEAs will continue to drive into the core. The crew will attempt to stop the rod motion but will be unsuccessful requiring the CRS to direct a manual Rx. trip.
- 5. Upon the reactor trip, SBCS will fail due to an instrument failure requiring the CO to manually control SG pressure and Tc.
- 6. The crew will perform SPTAs and may enter the Reactor Trip EOP. Once SG pressure is under control a loss of offsite power will occur requiring entry into the LOOP/LOFC EOP.
- 7. When the CRS enters into the LOOP/LOFC EOP, the 'A' diesel generator will trip requiring entry into the Blackout EOP.
- 8. The scenario will end when PBA-S03 has been reenergized using at least one GTG.

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor NUREG-1021. Draft Revision 9

Supplemental Turnover

Plant conditions:

The unit is at 100% power, steady state conditions at 225 EFPD. Boron concentration is 665 PPM per chemistry sample.

Equipment out of service:

Emergency Diesel Generator "B" is out of service for emergent work following discovery of a broken air connection on the safety system shutdown. T.S. 3.3.1 was entered 2 hours ago. Maintenance is expected to be completed in 4 hours.

Plant Cooling Water pump "B" was removed from service 10 hours ago for scheduled maintenance on the pump motor. Maintenance workers expect to finish pump repairs in 24 hours total time.

A leak in "A" train low pressure feedwater heater has been discovered and the "A" low pressure feedwater heater string is to be removed from service.

Planned shift activities:

The plant is at 100% power and has been at steady state conditions for the past 32 days. Low pressure feedwater heater string "A" is to be removed from service due to a leak and plant management has directed the crew to reduce power to 1109 MW over the next two hours using section 6.0 of 40OP-9ZZ05 in preparation for isolating low pressure feedwater heater string 'A' for maintenance.

Procedure 40OP-9ZZ05 has been performed though step 6.3.3.

The required boration has been calculated and verified by an STA to be 671 gallons of boron at a rate of 5.6 gpm.

The normal, shiftly surveillance's are complete.

Note:

The crew will walk down the control boards and assume the shift and then perform a reactivity brief prior to the commencement of the evaluation.

Appendix	D	 perator Actions	Form ES-D-2

Op-Test No: 2003 Scenario N	o.: 1	Event No: 1	Page 1	of 9
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Event Description: Place "B" Main Feed Pump in service

Time	Position	Applicant's Actions or Behavior	
T=0	CRS	Refers to 40OP-9ZZ05, Power Operations, Section 4.3 for power increase	
		Directs crew activities	
		Briefs crew appropriately for coordination of "B" Main Feedwater Pump start and power increase	
	CO	Refers to 40OP-9FT02, FW Pump Turbine "B", step 4.3.38	
		• Close FTN-HV10	
		Adjust FTN-HS54 to match pump discharge pressures	
		• Ensure discharge valve is open (FWN-HV-32)	
		Adjust FWPT manual/auto speed setpoints to same value	
		Adjust bias on SGN-FIC-1108 to obtain zero deviation	
		Place FWPT "B" speed control in AUTO (FTN-HS-100)	
		Balance FWPT A & B performance using bias's on speed controllers on "A" & "B" FWPT's	

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>1</u> Event No: <u>2</u> Page <u>2</u> of <u>9</u>

Event Description: Power increase. Crew to initiate power increase as directed in turnover. CRS to use 40OP-9ZZ05 to control power change.

Time	Position	Applicant's Actions or Behavior	
T=0	CRS	Refers to 40OP-9ZZ05, Power Operations, Section 4.3 for power increase	
		Directs crew activities	
		Directs RO to initiate dilution per turnover	
	RO	Initiates dilution of RCS at rate determined from turnover/power change worksheet	
		• Uses 40OP-9CH01, Section 9.0	
		Observes plant for indication of dilution	
		Reactor power	
		RCS temperature	
	CO/RO	Adjusts turbine load to maintain RCS temperature in program band (manipulation may be performed by RO).	

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: 2003 Scenario No.: 1 Event No: 3 Page 3 of 9

Event Description: TLI 1 instrument fails high. Crew to determine that it is an instrument failure and take actions as directed by RRS Malfunctions AOP (40AO-9ZZ16).

Time	Position	Applicant's Actions or Behavior	
T=25	CO	Recognizes TLI 1 failed high.	
		Responds to B04 alarms	
		AMI (Auto Motion Inhibit (Window 4A 10B)	
		RRS Input CH Deviation (RK alarm)	
		• Tave/Tref HiLo (Window 4A 8B)	
		Reports condition to RO and CRS	
		Verifies conditions do not support instrument reading and plant response	
	CRS	Assesses plant condition and directs crew activities as directed by 40AO-9ZZ16, RRS Malfunctions	
		 Verifies conditions do not support instrument reading and plant response (Tref prompt jump high) 	
		Ensures CEDMCS NOT in Auto Sequential	
		May ensure SBCS NOT in Remote Automatic	
		Determines impact of failed instrument	
		• Tref >Tave	
		CEA withdrawal light illuminated	
		Directs removal of failed instrument from service	
	RO	Ensures CEDMCS Not in Auto Sequential	
	CO	Ensures SBCS NOT in Remote Automatic	
		Validates TLI instrument failure	
		Selects the unaffected TLI instrument at the RRS Test Panel	
	RO	Monitors plant parameters to ensure perturbations are minimized	
		Monitor Tavg/Tref mismatch is 3°F or less	
		Returns CEDMCS to Auto Sequential per CRS direction	
	CO	Returns SBCS to Remote Automatic operation per CRS direction	

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: 2003 Scenario No.: 1 Event No: 4 Page 4 of 9

Event Description: Letdown flow control valve (CHN-UV0110P) fails closed resulting in a loss of Letdown. Crew should take appropriate actions to cope with loss of letdown. It is not expected that letdown will be recovered.

letdown wi	ii de lecoveled.	
Time	Position	Applicant's Actions or Behavior
T=35	CRS	Directs response to B03 alarms
		LD HDR SYS TRBL (Window 3A 10A)
		LD Process Mon TRBL (Window 3A 10B)
		Assess condition and operator inputs
		Letdown flow and backpressure trending down
		Letdown control valve 110P indicates closed
		Enters 40AO-9ZZ05, Loss of Letdown
	RO	Observes B03 alarms and refers to Alarm Response
		Diagnose letdown control valve 110P indicates closed/loss of letdown
		Secures one charging pump
	CO	Monitors secondary systems and continues power increase
	CRS	May direct recovery per step 11 of loss of letdown
		Addresses TS (LCO 3.4.9) for Pressurizer level
		May brief crew on impact of event and associated Tec Specs.
	RO	May review restoration of letdown using Appendix A if time permits.

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: 2003	Scenario No.: 1	Event No: 5	Page 5 of 9
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Event Description: One CEA will drop to the bottom of the core. While the crew is responding to this Malfunction per 40AO-9ZZ11, CEA Malfunctions, a second CEA will drop into the core requiring a manual trip from B01.

Position	Applicant's Actions or Behavior	
RO	Determines a CEA has dropped into the core by observing Core mimic/B04 alarms/CEDMCs control panel	
	Alarms include:	
	CEA Tech Spec Violation (Window 4A 4B)	
	CEA Position Out of Limits (Window 4A 6B)	
	• CWP (Window 4A 9B)	
	AMI (Window 4A 10B)	
	CPC/CEAC TRBL (Window 5A 13B)	
CRS	Implements 40AO-9ZZ11	
	Directs CEDMCS to "STANDBY"	
	Directs stopping dilution	
	Monitors CEA's	
	May direct completion of 73ST-RX03 COLSS Operability	
RO	Places CEDMCS in "STANDBY"	
	Stops dilution	
CO	Continues to adjust turbine load to match Tave/Tref	
RO/CO	Notices second dropped CEA and informs CRS	
CRS	Verifies second dropped CEA	
	Directs manual reactor trip per 40AO-9ZZ11, CEA Malfunctions	
RO	Attempts to trip reactor using manual reactor trip pushbuttons on BO5	
	RO CRS RO CO RO/CO CRS	

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>1</u> Event No: <u>6</u> Page <u>6</u> of <u>9</u>

Event Description: Reactor protection system fails to open reactor trip switchgear breakers (PRA significant) when RPS setpoints are exceeded.

Time	Position	Applicant's Actions or Behavior
T=47	CRS	Recognizes Reactor has not tripped using manual push buttons.
		Directs opening L03 and L10 supply breakers.
	RO	Opens L03 and L10 supply breakers. (Critical Task to trip reactor by opening L03 and L10 prior to completion of SPTAs)
		May reclose after 5 seconds.
	RO/CO	Verifies reactor tripped.
	CRS	Implements SPTA's
	RO	Determines Reactivity safety function
		Reactor power decreasing
		Negative SUR
		All CEA's inserted
	CO	Determines Maintenance of Vital Auxiliaries safety function
		Main turbine tripped
		Main generator breakers open
		Station loads transfer to offsite power
		Vital/non-vital busses energized
T=47	RO	Determines RCS Inventory Control safety function
		Pressurizer level control
		• RCS subcooling ≥ 24°F
		All RCP's have BOTH
		Seal injection
		Nuclear Cooling water
		Determines RCS Pressure control safety function
		Pressurizer pressure 1837-2285 psia
		 Pressurizer pressure trending to 2225-2275
		Determines Core Heat Removal safety function
		At least one RCP operating
		• Loop delta T < 10°F
		• RCS subcooling ≥ 24°F

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: 2003 Scenario No.: 1 Event No: 6 (Continued) Page 7 of 9

Event Description: Reactor protection system fails to open reactor trip switchgear breakers (PRA significant) when RPS setpoints are exceeded. (Continued)

Time	Position	Applicant's Actions or Behavior	
T=47	СО	Determines RCS Heat Removal safety function	
Continued		At least one SG with BOTH:	
		• Level ≥ 35% WR	
		 Feedwater restoring or maintaining level 45-60% NR 	
		• Tc 560 - 570°F	
		• SG pressure 1140 – 1200 psia	
		Determines Containment Isolation safety function	
		• Containment pressure < 2.5 psig	
		Check radiation monitors:	
		no valid containment area alarms	
		no valid steam plant activity alarms	
		Determines CTPC safety function	
		• Containment temperature <117°F	
		• Containment pressure <2.5 psig	
	CRS	Completes SPTA Diagnostic and proceeds to 40EP-9EO01, Reactor Trip	

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>1</u> Event No: <u>7</u> Page <u>8</u> of <u>9</u>

Event Description: Steam line break outside containment upstream of MSIV's when CRS implements 40EP-9EO02, Reactor Trip.

Time	Position	Applicant's Actions or Behavior
T=55	Crew	Recognizes plant response to steam leak
		Possible MSIS/SIAS/CIAS
	CRS	Diagnoses steam leak in containment and transitions to 40EP-9EO04, ESD
		Confirms diagnosis and directs crew activities
		Ensures SG sample valves open
		Directs Chemistry to perform 74DP-9ZZ05, Abnormal Occurrence checklist
		Ensures event is being classified
		Opens placekeeper and enters EOP entry time
		IF RCS pressure drops to SIAS setpoint, verify SIAS actuates
	RO	When RCS pressure drops below SIAS setpoint, verify SIAS actuation.
		Verify HPSI pumps start
		Verify LPSI pumps start
		Verify adequate SI flow
	CO	Ensures MSIS actuated
	RO	IF pressurizer pressure remains below SIAS setpoint:
		Ensure one RCP is stopped in each loop
	CRS	Determines #2 SG is most affected and directs isolation of #2 SG
	СО	Isolates #2 SG
		Stops feeding and steaming #2 SG (Critical task to stop feeding and steaming #2 SG)
		Secures all auxiliary feed valves to #2 SG
		Secures steam from #2 SG to AFA-P01
	RO/CO	Monitor RCS parameters for SG blowdown/RCS rebound
		Select target temperature/pressure
		RCS T-cold stable/increasing
		RCS pressure increasing
	СО	Upon rebound indication, uses SG #1 ADV to control RCS temperature and pressure (Critical Task to control RCS parameters to prevent lifting Pressurizer Safeties)

Op-Test No: 2003 Scenario No.: 1 Event No: 7 (Continued) Page 9 of 9.

Event Description: Steam line break outside containment upstream of MSIV's when CRS implements 40EP-9EO02, Reactor Trip.

Time	Position	Applicant's Actions or Behavior	
T=55	RO	Throttle SI flow as required. (Critical Task to control RCS parameters to	
Continued		prevent lifting Pressurizer Safeties)	
	RO/CO	Perform Safety Function Status Check for ESD every 15 minutes	
T≈65	END	Scenario will end when RCS has been stabilized and the unisolated Steam Generator Level is being maintained 40–60% NR	
		OR	
		Actions have been taken for plant stabilization and the unisolated Stem Generator level is trending towards 40-60% NR	

Appendix	D	Operator Actions	Form ES-D-2

 Op-Test No: <u>2003</u>
 Scenario No.: <u>2</u>
 Event No: <u>1</u>
 Page <u>1</u> of <u>10</u>

Event Description: Remove DG 'B' from service following completion of a surveillance test.

Time	Position	Applicant's Actions or Behavior	
T=0	CRS	Directs CO to remove DG 'B' from service using 40OP-9DG02, Emergency Diesel Generator "B".	
	СО	Refers to 40OP-9DG01, Emergency Diesel Generator "B", step 7.3.74 • Stops DG 'B' by placing DGB-HS-1 to the "STOP" position	

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>2</u> Event No: <u>2</u> Page <u>2</u> of <u>10</u>

Event Description: Diesel Generator 'B' trips when control switch is taken to stop and remains Out of service

Time	Position	Applicant's Actions or Behavior
T=5	СО	Notices that DG 'B' immediately stops (Does NOT go through expected cooldown cycle)
		Reports malfunction to CRS
	CRS	Directs CO to investigate
		Directs implementation of 41ST-1ZZ02, Inoperable Power Sources
		May call I&C for troubleshooting
		Refers to Technical Specification 3.8.1.b
		• Enters actions B.1 AND B.2

Op-Test No: 2003 Scenario No.: 2 Event No: 3 Page 3 of 10

Event Description: Condenser vacuum degrades requiring downpower. Vacuum is restored after A reduction in power when the area operator restores vacuum breaker seal water.

Time	Position	Applicant's Actions or Behavior	
T=10	СО	Notices degrading vacuum in 'A' condenser shell and reports event to CRS	
	CRS	Assesses plant conditions directs crew activities per 40AO-9ZZ07, Loss of Condenser Vacuum	
		Directs AO to investigate conditions in field	
		Addresses diagnostic flowchart	
	СО	Ensures that 'D' Air Removal Pump is running and is aligned to 'A' condenser shell.	
	CRS	Determines power reduction of at least 5% is required	
		Calculated boration required to achieve power reduction	
		Directs downpower	
	RO	Initiates a downpower using boration and/or CEA insertion	
	RO/CO	Adjusts turbine load to maintain Tave/Tref ± 5°F	
		Monitors condenser vacuum and reports improvement when AO has restored vacuum breaker water seal	

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>2</u> Event No: <u>4</u> Page <u>4</u> of <u>10</u>

Event Description: Seal injection flow controller fails closed. Crew addresses loss of seal injection to an operating RCP.

Time	Position	Applicant's Actions or Behavior				
T=25	RO	Responds to B03 alarms indicating loss of seal injection to RCP 1A				
		Verifies condition and informs CRS				
	CRS	Assesses plant condition and directs RO to perform Alarm Response actions				
		Enters 40AO-9ZZ04, Reactor Coolant Pump Emergencies				
	RO	Takes actions per alarm response, RCP SEAL INJ FLOW HI-HI OR LO (Window 3A11B)				
		Determines that Seal injection to RCP 1A is low				
		Attempts manual control of RCP 1A flow controller (FIC241)				
		Monitors seal injection parameters per 40AO-9ZZ04, Appendix B				
	CRS	May stop boration				

Appendix D	Operator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>2</u> Event No: <u>5</u> Page <u>5</u> of <u>10</u>

Event Description: Loss of Offsite power.

	T				
Time	Position	Applicant's Actions or Behavior			
T=30	CRS	Observes Loss of Offsite Power			
		Implements SPTA's			
	RO	Determines Reactivity safety function			
		Reactor power decreasing			
		Negative SUR			
		Notes that two CEAs have failed to fully insert by observing Core mimic/CEDMCS control panel			

Op-Test No: 2003 Scenario No.: 2 Event No: 6 Page 6 of 10

Event Description: Two CEAs fail to fully insert, crew initiates emergency boration.

	T		
Time	Position	Applicant's Actions or Behavior	
T=30	CRS	Implements Standard Post Trip Actions	
		Directs RO to emergency borate	
	RO	Initiates emergency boration	
		 Ensures at least one charging pump is running 	
		• Opens CHUV536	
		Closes CHUV501	
		 Reports to CRS that boration is initiated (critical task to establish boration to meet safety function requirements prior to completion of the SPTA's) 	
	CO Determines Maintenance of Vital Auxiliaries safety function		
		Main turbine tripped	
		Main generator breakers open	
		 Loss of offsite power 	
		PBA-S03 energized by Emergency Diesel Generator 'A'	
		Vital/non-vital DC busses energized	
	RO	Determines RCS Inventory Control safety function	
		Pressurizer level control	
		• RCS subcooling ≥ 24°F	
		All RCP's tripped	
		Seal bleedoff isolated	
		Determines RCS Pressure control safety function	
		Pressurizer pressure 1837-2285 psia	
		 Pressurizer pressure trending to 2225-2275 	
		Determines Core Heat Removal safety function	
		Natural Circulation being established	

Op-Test No: 2003 Scenario No.: 2 Event No: 6 (continued) Page 7 of 10

Event Description: Two CEAs fail to fully insert, crew initiates emergency boration.

Time	Position	Applicant's Actions or Behavior		
T=35	Establishes Feedwater to SG's using AFA-P01 or AFN-P01			
		Establishes heat removal from both SG's using ADV's		
	CRS	Completes SPTA diagnostic and transitions to 40EP-9EO04, Loss of Offsite Power/Loss of Forced Circulation		
	RO	Establishes RCS pressure control using auxiliary sprays		

Appendix	D	Ot	perator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>2</u> Event No: <u>7</u> Page <u>8</u> of <u>10</u>

Event Description: Loss of Feedwater (AFA-P01 or AFN-P01 depending on which pump was Started on LOOP)

Time	Position	Applicant's Actions or Behavior			
T=40	CO	Recognizes loss of feedwater and reports to CRS			
	CRS	Directs starting the non-running auxiliary feedwater pump			
		• AFA-P01			
		• AFN-P01			
	CO	Starts non-running auxiliary feedwater pump			
		• AFA-P01			
		 Opens steam supply valve to start turbine 			
		 Opens aux. feedwater isolation valves to at least one SG 			
		• AFN-P01			
		 Opens BOTH supply isolation valves 			
		Starts AFN-P01			
		 Aligns pump discharge to at least one SG 			
		Establishes feedwater to at least one SG to restore level(s)			
· · · · · · · · · · · · · · · · · · ·					

Op-Test No: <u>2003</u> Scenario No.: <u>2</u> Event No: <u>8</u> Page <u>9</u> of <u>10</u>

Event Description: Loss of All Feedwater requiring transition to Functional Recovery Procedure

Time	Position	Applicant's Actions or Behavior				
T=50	СО	Recognizes loss of running remaining auxiliary feedwater pump				
		AFN-P01 or				
		• AFA-P01				
		Reports loss to CRS				
	CRS	Verifies Loss of All Feedwater				
		Transitions to 40EP-9EO6, Loss of All Feedwater (may transition directly to FRP)				
		Confirm diagnosis				
		Checks Safety Function Status Check Criteria				
		Direct Chemistry to perform Abnormal Occurrence Checklist				
		Verify event is classified				
		Open Placekeeper				
		Direct conserving SG inventory				
		• Determines transition to FRP is required due to loss of MVAC (identified in step 6.1)				
	CO	Closes Blowdown Containment Isolation Valves				
		Close SG Sample Valves (this may be delayed due to being opened in the FRP).				

Op-Test No: <u>2003</u> Scenario No.: <u>2</u> Event No: <u>9</u> Page <u>10</u> of <u>10</u>

Event Description: Transition to FRP to cross tie PBA-S03 (energized by a DG) to PBB-S04.

Time	Dogition	Applicant's Actions on Debouien			
Time	Position	Applicant's Actions or Behavior			
T=50	Crew	Transitions to 40EP-9EO09, Functional Recovery Procedure			
		 Ensures event is classified 			
		Records EOP entry time			
		Directs RO/CO initial actions			
	CO	Opens SG sample valves			
	RO	Places Hydrogen Analyzer in service			
	CRS	Identifies MVAC-2 as success path for LOAF			
		Completes Safety Function Status Check for MVAC			
		Directs energizing PBB-S04 from PBA-S03 using Appendix 59			
	CO	Directs AO to complete attachment 59-A, Disable PBB-S04 Breakers			
		Ensures all the following breakers are open:			
		• NAN-S03A			
		• PBA-S03K			
		• PBA-S03L			
		• NAN-S04A			
		• PBB-S04L			
		• PBB-S04K			
		Ensures PBB-S04B, DG B Breaker is open			
		Places all the following in "Pull TO Lock"			
		Train B containment Normal ACUs			
		Train B CEDM ACUs			
		Closes PBA-S03L Normal Supply Breaker			
	CO	When attachment 59-A is complete then:			
		Parallel and close PBB-S04L, PBB-S04 Alternate Supply Breaker			
	СО	Start AFB-P01 and establish feed to at least one SG (Critical Task to establish feedwater [feed > steam to at least one SG prior to losing RCS pressure control)			
T≈60	END	Scenario will end when feedwater has been established to at least one SG			
		OR			
		As deemed appropriate by the examination team.			

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: 2003	Scenario No.:	3	Event No: 1	Page 1	of 7
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Event Description: Downpower to 1109 MW

Time	Position	Applicant's Actions or Behavior		
T=0	CRS	Directs RO and CO commence a downpower to 1109 MW using boration and reducing turbine load as necessary to maintain Tave/Tref close.		
		When boration is started, the CRS verifies boration		
	RO	Refers to 40OP-9CH01, Charging and Letdown, section 6.3, Instructions for Makeup-Borate Mode		
		Commences a boration of 300 gallons		
		Sets flow rate on CHN-FIC-210Y		
		Sets "Target" volume on CHN-FQIS-210Y		
		Places CHN-HS-210 in BORATE		
		Depress Reset on totalizer/counter module (micro-motion)		
		Depress Start on totalizer/counter module (micro-motion)		
		• Ensures		
		One boric acid pump starts		
		CHN-FIC-21X indicates no RMW flow		
		• CHN-UV-527 is open		
		 Checks for proper flow indicated on CHN-FIC-210Y 		
	CO	Adjusts turbine load to maintain Tave/Tref within limits as described by CRS		
	Crew	Monitors the effects of boration on CEA motion, RCS temperature, and neutron level		

Op-Test No: <u>2003</u> Scenario No.: <u>3</u> Event No: <u>2</u> Page <u>2</u> of <u>7</u>

Event Description: Inadvertent CSAS actuation

Time	Position	Applicant's Actions or Behavior		
T=15	Crew	Notices CSAS actuation		
	RO	Investigates BO04 and determines "A" Containment Spray pump is running and there is no spray flow into containment (valves remain closed).		
		May report to CRS that containment spray pump is running with no spray flow and that containment pressure is below CSAS setpoint		
	CRS	Enters 40AA-9ZZ17, Inadvertent PPS-ESFAS Actuation's Section 5.0, CSAS and directs crew to respond to inadvertent CSAS:		
		May tailboard starting CS pump if a SIAS has not actuated:		
		Direct an AO to cycle control power to the CS pump breaker (breaker will close when control power is restored)		
	RO	Stops the running Containment Spray pump		
		Evaluates NC flow to running RCP's		
		RO/CO to open NC containment isolation valves as necessary		
		NCA-UV-402, NCW Cont. Downstream Return Isolation VIv		
		NCB-UV-403, NCW Cont. Upstream Return Isolation Vlv		
		NCB-UV-401, NCW Cont. Upstream Supply Isolation Vlv		
		Evaluates Letdown		
		Overrides and stops running Control Room Essential AHUs		
		Restores RCP Seal Bleedoff to VCT		
	CO	CO/RO to open NC containment isolation valves as necessary		
		NCA-UV-402, NCW Cont. Downstream Return Isolation Vlv		
		NCB-UV-403, NCW Cont. Upstream Return Isolation Vlv		
		NCB-UV-401, NCW Cont. Upstream Supply Isolation Vlv		
		Evaluates IA to containment		
		Evaluates SG Blowdown isolation valves		
	СО	Performs Appendix C, PPS-ESFAS Check to determine status of ESFAS equipment		
	CRS	May address technical specifications 3.3.5 and 3.3.6.		
<u></u>	CRS	May evaluate reportability of event.		
	1			

Appendix	D O	Operator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>3</u> Event No: <u>3</u> Page <u>3</u> of <u>7</u>

Event Description: Turbine Trip/Load Rejection/Reactor Power Cutback

Time	Position	Applicant's Actions or Behavior		
T=25	Crew	Identifies turbine trip and reactor power cutback		
	CRS	Assesses plant conditions directs crew activities per 40AO-9ZZ08, Load Rejection		
		STA to perform Appendix F, Status Check Load Rejection		
		May direct stopping boration		
	RO	Verify Reactor Power Cutback and CEA subgroups 4, 5, and 22 have inserted		
		Verify that CEAs are inserting to reduce reactor power		
	CO	Verify generator output breakers are open		
		Ensure generator excitation is "OFF"		
	RO	Monitor CEAs for deviations/reactor trip criteria		
	CO	Restore and maintain SG levels 45-60% NR		

Appendix	D	Operator Actions	Form ES-D-2

Op-Test No: <u>2003</u> Scenario No.: <u>3</u> Event No: <u>4</u> Page <u>4</u> of <u>7</u>

Event Description: CEAs continue to insert (uncontrolled) following RPCB.

TD:	D :::				
Time	Position	Applicant's Actions or Behavior			
T=25	RO	Identifies CEA insertion when not required (or after AMI)			
	CRS	Directs CEA controls to any position other than auto sequential (AS)			
	RO	Places CEA controls in directed position and verifies that CEAs continue to insert			
	CRS	Directs reactor trip and SPTA's			
	RO	Initiates manual reactor trip verifies reactivity safety function:			
		Reactor Power decreasing			
		Negative startup rate			
		All full length CEAs fully inserted			
	CO	Determines Maintenance of Vital Auxiliaries safety function			
		Main turbine tripped			
		Main generator breakers open			
		Station loads transfer to offsite power			
		Vital/non-vital busses energized			
	RO	Determines RCS Inventory Control safety function			
		Pressurizer level control			
		• RCS subcooling ≥ 24°F			
		All RCP's have BOTH			
		Seal injection			
		Nuclear Cooling water			
		Determines RCS Pressure control safety function			
		Pressurizer pressure 1837-2285 psia			
		 Pressurizer pressure trending to 2225-2275 			
		Determines Core Heat Removal safety function			
		At least one RCP operating			
		• Loop delta T < 10°F			
		• RCS subcooling ≥ 24°F			
	CO	Establishes Feedwater to SG's			
		Establishes heat removal from both SG's			

Appendix	D	Operator Actions	Form ES-D-2

 Op-Test No: 2003
 Scenario No.: 3
 Event No: 5
 Page 5 of 7

Event Description: Loss of SBCS pressure control.

Time	Position	Applicant's Actions or Behavior		
	1 05111011	Tippineant streams of Benavior		
T=30	CRS	Notes failure of SBCS to modulate and control Tc		
		Directs CO to take manual control of SBCS		
	NOTE: Use of ADV's is not optimal but is acceptable			
CO Determines failure mode of SBCS Controller				
	Places Controller in Local Auto			
		Or establishes heat removal using ADV's		

Op-Test No: 2003 Scenario No.: 3 Event No: 6 Page 6 of 7

Event Description: Loss of Offsite Power.

Time	Position	Applicant's Actions or Behavior		
T=35	Crew	Identifies loss of Offsite Power		
	CRS	Transitions to LOOP/LOFC EOP		
		Directs crew actions per EOP		
		Confirms diagnosis		
		Directs Safety Function Status Check		
		Directs Chemistry to perform Abnormal Occurrence Checklist		
		Ensures the event is being classified		
	RO	Ensures charging pump is running and seal injection		
	CO	Actuates MSIS		
		Verifies Tc maintained less than 570°F using ADV's		
		Ensures at least one SG is being maintained or restored to 45-60% NR		
	RO	Ensures pressurizer level is 10-65% AND trending to 33-53%		
		Verifies natural circulation		

Op-Test No: 2003 Scenario No.: 3 Event No: 7 Page 7 of 7

Event Description: Loss of only running diesel generator (Blackout).

Time	Position	Applicant's Actions or Behavior		
T=45	Crew	Identifies and acknowledges Blackout.		
	CRS	Transitions to Blackout EOP		
		Directs crews actions per Blackout EOP		
	CO	Controls Tc using ADV's		
		Establish feedwater to at least one SG using AFA-P01		
	RO	Performs Standard Appendix 53, "Align Deenergized Buses"		
		Performs Standard Appendix 80 to align GTG to PBA-S03		
T=60	END	Scenario will end when PBA-S03 has been energized using at least one GTG		
		OR		
		As deemed appropriate by the examination team.		

Supplemental Information

SIMULATOR SETUP

Cert Load

• IC # 16

Malfunctions

None

Remote Functions

MRF B201:PWNP01B OPEN

• MRF B202:PWNP01B RACK_OUT

• MRF B201:SIBP02 OPEN

MRF B202:SIBP02 RACK_OUT

'B' PW Pump control power off

'B' PW Pump removed from service

'B' HPSI Pump control power off

'B' HPSI Pump removed from service

Overrides

None

Triggers

None

CAE's

• CAE! atws prevents RTSG from opening

• CAE! oos/edgb removes 'B' diesel generator from service

MATERIALS REQUIRED

- Start 'A' PW pump, then place 'B' PW pump in PTL
- Place yellow tags on the following handswitches:
 - 'B' PW pump
 - 'B' HPSI Pump
 - 'B' Emergency Diesel Generator handswitch
 - PBB-S04B, Diesel Generator Output Breaker
- Standard Simulator documentation

EVENT	TIME	SYNTAX	DESCRIPTION	MISC.
1	T=0		Crew takes shift and conducts reactivity briefing	If directed as AO to observer MFP 'B' report no abnormal indications, no rubs, no vibrations
2	T = 0			Power increase requires no outside action.
3	T= 25	IMF TR01:MTNPT11A 839	TLI instrument fails high	If called as I&C tell control room that personnel will be sent to help as soon as possible.
4	T=35	IMF CV03A 0	CHN-UV-110P Fails closed	Results in a loss of letdown
				If directed as AO, report to control room for instructions on placing 110Q valve in service
5	T=45	IMF RD02E 100	Dropped/slipped CEA #86	If called as I&C tell control room that personnel will be sent to help as soon as possible
				If called as AO to investigate dropped CEA at CEDMCS panels, respond that you are on the way.
	T=47	IMF RD2F 100	Dropped/slipped CEA #89	Upon dropping of second CEA crew will trip reactor.
				Report as AO, that CEDMCS indicated several trouble alarms.
6	T=47	Cae! Atws	Reactor fails to trip	No actions required.
		In setup		
7	T=55	IMF MS03C 40	Steam line break outside containment upstream of the MSIV's on #2 SG	If called as AO to look for steam leaks, report large amounts of steam from MSSS.

Supplemental Information

SIMULATOR SETUP

Cert Load

IC # 20

Malfunctions

IMF RD03G Stuck CEA #43IMF RD03L Stuck CEA #38

Remote Functions

See materials required to shift running PW Pumps before performing the following:

MRF B201:PWNP01B OPEN
 'B' PW Pump control power off

MRF B202:PWNP01B RACK_OUT
 'B' PW Pump removed from service

• MRF B201:SIBP02 OPEN 'B' HPSI Pump control power off

• MRF B202:SIBP02 RACK_OUT 'B' HPSI Pump removed from service

Overrides

None

Triggers

- Assign ZOPENEIG02.LT.0.5 (this monitors the voltage of DG 'B') (to do this, go to page EG3, click on the popup window "EI PEN-EI-G02", select event trigger popup window, select "LT", hit enter to return line to event trigger number, assign an available trigger #, then select "accept". You now have the trigger file setup). Now link the following command to the trigger:
- IMF EG06B

CAE's

None

MATERIALS REQUIRED

- Start Spray Pond Pump 'B'
- Start DG 'B'
- Start 'A' PW pump, then stop 'B' PW pump and place HS in PTL
- Place yellow tags on the following handswitches:
 - 'B' PW pump
 - 'B' HPSI Pump
- Standard Simulator documentation

EVENT	TIME	SYNTAX	DESCRIPTION	MISC.
1	T=0		Crew takes shift and conducts reactivity briefing	If directed as AO to verify DG 'A' Essential AHU Filter DP, report DP is 0.6".
2	T = 5	IMF EG06B On trigger	DG 'B' Generator Differential Trip Make sure this malfunction occurs when the operator stops the DG	If directed as AO to investigate DG trip, report generator differential trip and DG is stopped. If requested, no problem with PBB-S04 bus and no sign of fire.
3	T= 10	IMF MC01A 5	Condenser Air Inleakage The objective is to downpower 3-5%	When sent as AO wait 5 minutes and report dogbone seal has lost water seal. Attempting to refill seal at this time.
	T= 20	DMF MC01A Wait for downpower to start.	Removes condenser air inleakage malfunction	Report as AO that water seal has been reestablished to dogbone seal.
4	T= 25	IMF CN01:CHNFIC 241 100	Seal Injection Flow Controller malfunction	
5	T= 35	IMF ED02	Loss of Grid Voltage	If called as ECC report grid down due to disturbance in California. Trying to recover at this time, estimated time to reenergize is 4 hours. PVNGS is a priority.
6		IMF RD03G IMF RD03L In setup	Two stuck CEA's	
7	T= 50	IMF FW21A Or IMF FW22	Loss of feedwater pump in use When CRS enters LOOP/LOFC EOP	Insert the malfunction for the feedwater pump that is being used to supply feedwater. *If called as AO report AFN breaker has 86 lockout but no other flags.
8	T=55	IMF FW22 Or IMF FW21A	Loss of All Feedwater	Insert the malfunction for the feedwater pump that is being used to supply feedwater. *If called as AO report AFA-P01 has a broken shaft.
	T= When directed	Cae! EOP/attachment 59A	Performs steps of Appendix 59a Attachment	Report as AO when attachment is complete.

SIMULATOR SETUP

Cert Load

• IC # 20

Malfunctions

IMF RD03G Stuck CEA #43IMF RD03L Stuck CEA #38

Remote Functions

See materials required to shift running PW Pumps before performing the following:

MRF B201:PWNP01B OPEN
 'B' PW Pump control power off

• MRF B202:PWNP01B RACK_OUT 'B' PW Pump removed from service

MRF B201:SIBP02 OPEN
 B' HPSI Pump control power off

• MRF B202:SIBP02 RACK_OUT

'B' HPSI Pump removed from service

Overrides

None

Triggers

- Assign ZOPENEIG02.LT.0.5 (this monitors the voltage of DG 'B') (to do this, go to page EG3, click on the popup window EI "PEN-EI-G02", select event trigger popup window, select "LT", hit enter to return line to event trigger number, assign available trigger #, then select "accept". You now have the trigger file setup). Now link the following command to the trigger:
- IMF EG06

CAE's

None

MATERIALS REQUIRED

- Start Spray Pond Pump 'B'
- Start DG 'B'
- Start 'A' PW pump, then stop 'B' PW pump and place HS in PTL
- Place yellow tags on the following handswitches:
 - 'B' PW pump
 - 'B' HPSI Pump
- Standard Simulator documentation

EVENT	TIME	SYNTAX	DESCRIPTION	MISC.
1	T=0		Crew takes shift and conducts reactivity briefing	If directed as AO to verify DG 'A' Essential AHU Filter DP, report DP is 0.6".
2	T = 5	IMF EG06B On trigger	DG 'B' Generator Differential Trip Make sure this malfunction occurs when the operator stops the DG	If directed as AO to investigate DG trip, report generator differential trip and DG is stopped. If requested, no problem with PBB-S04 bus and no sign of fire.
3	T= 10	IMF MC01A 3	Condenser Air Inleakage The objective is to downpower 3-5%	When sent as AO to determine source of air inleakage, wait 5 minutes and report dogbone seal has lost water seal. Attempting to refill seal at this time.
	T= 20	DMF MC01A	Removes condenser air inleakage malfunction	Report as AO that water seal has been reestablished to dogbone seal.
4	T= 25	IMF CN02:CHNFIC 241 100	Seal Injection Flow Controller malfunction	
5	T= 35	IMF ED02	Loss of Grid Voltage	If called as ECC report grid down due to disturbance in California. Estimated time to recover is 4 hours. PVNGS is a priority.
6		IMF RD03G IMF RD03L In setup	Two stuck CEA's	
7	T= When CRS enters LOOP/LO FC EOP T= 50	IMF FW21B Or IMF FW22	Loss of feedwater pump in use	Insert the malfunction for the feedwater pump that is being used to supply feedwater.
8	T=50	IMF FW22 Or IMF FW21B	Loss of All Feedwater	Insert the malfunction for the feedwater pump that is being used to supply feedwater.
	T= When directed	Cae! EOP/attachment 59A	Performs steps of Appendix 59a Attachment	Report as AO when attachment is complete.

Supplemental Information

SIMULATOR SETUP

Cert Load

• IC # 20

Malfunctions

Remote Functions

See materials required to shift running PW Pumps before performing the following.

• MRF B201:PWNP01B OPEN 'B' PW Pump control power off

• MRF B202:PWNP01B RACK OUT 'B' PW Pump removed from service

• MRF B201:SIBP02 OPEN 'B' HPSI Pump control power off

MRF B202:SIBP02 RACK_OUT
 'B' HPSI Pump removed from service

Overrides

IOR for alarms on B06: A train heater high levels OR take manual control of a normal heater level controller and fail it closed.

Triggers

Assign RPSCHC to a trigger file and link the following command:

- IMF TR01:SGNPT1024 980 (or asis)
- IMF ED02 with a time delay of 10 minutes

Assign the following two malfunctions to an unassigned trigger file:

• IMF TC13 Turbine Trip

• IMF RD11B 30 Uncontrolled CEA Insertion with a 30 second time delay after TC13

CAE's

• CAE! oos/edgb removes 'B' diesel generator from service

MATERIALS REQUIRED

- Start 'A' PW pump, then place 'B' PW pump in PTL
- Place yellow tags on the following handswitches:
 - 'B' PW pump
 - 'B' HPSI Pump
 - 'B' Emergency Diesel Generator handswitch
 - PBB-S04B, Diesel Generator Output Breaker
- Standard Simulator documentation

EVENT	TIME	SYNTAX	DESCRIPTION	MISC.
1	T=0		Crew takes shift and conducts reactivity briefing for downpower	Power decrease requires no outside action.
2	T= 15	IMF RP06H1	Inadvertent CSAS actuation	If called as I&C tell control room that personnel will be sent to help as soon as possible.
3	T=25 Initiate using trigger command	IMF TC13	Turbine trip/large load rejection	
4	T=25 Linked to event 3	IMF RD11B 30	Uncontrolled CEA Insertion with a 30 second time delay	If called as I&C tell control room that personnel will be sent to help as soon as possible If called as AO to investigate dropped CEA at CEDMCS panels, respond that you are on the way.
5	T=35 In setup	IMF TR01:SGNPT1024 980	Failure of SBCS to control Tc Linked to reactor trip	
6	T=35 In setup	IMF ED02	Loss of Offsite Power 10 minutes after reactor trip	If called as ECC inform control room that grid is lost due to line overloads to California. Estimated time of return is unknown but will be several hours at the minimum.
7	T=45	IMF EG06A	Loss of only running diesel generator (Blackout)	If called as AO to investigate PBA-S03/EDG "A", report generator differential trip on EDG "A" but breaker is open and bus is deenergized with no damage and only undervoltage relays.
	T= when directed	CAE! EOP/Attachment 80a	Performs Standard Appendix 80a	If called as AO to perform SA 80a, run CAE and report completion to control room as required.