April 23, 2004

Mr. Stephen A. Byrne
Senior Vice President - Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
P. O. Box 88
Jenkinsville, SC 29065

SUBJECT: ISSUANCE OF RENEWED FACILITY OPERATING LICENSE NO. NPF-12 FOR VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

Dear Mr. Byrne:

The U.S. Nuclear Regulatory Commission (NRC) has issued Renewed Facility Operating License No. NPF-12 (Enclosure 1) for the Virgil. C. Summer Nuclear Station, Unit No. 1 (V. C. Summer, Unit No. 1). The renewed facility operating license has been issued on the basis of our review of your application dated August 6, 2002, as supplemented by letters dated September 12, 2002, and March 19, May 21, June 12, July 31, September 2, September 24, and November 5, 2003. The Technical Specifications for V. C. Summer, Unit No. 1, were not amended as a result of our review.

Enclosure 1 contains Renewed Facility Operating License No. NPF-12 with three attachments: (1) Appendix A (Technical Specifications), (2) Appendix B (Environmental Protection Plan), and (3) Appendix C (Additional Conditions).

Renewed Facility Operating License No. NPF-12 expires at midnight, August 6, 2042.

The technical basis for issuing the renewed license is set forth in NUREG-1787, “Safety Evaluation Report Related to the License Renewal of the Virgil C. Summer Nuclear Station,” dated March 2004. The results of the environmental reviews related to the issuance of the renewed license are contained in NUREG-1437, "Generic Environmental Impact Statement for License Renewal of Nuclear Plants, Supplement 15, Regarding Virgil C. Summer Nuclear Station,” dated February 2004.

Enclosure 2 is a copy of the related *Federal Register* notice of issuance of the renewed license. The original has been sent to the Office of the *Federal Register* for publication.

Sincerely,

/RA/

Rajender Auluck, Sr. Project Manager
License Renewal Section A
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Docket No.: 50-395

Enclosures: 1. Renewed Facility Operating License No. NPF-12
2. *Federal Register* Notice

cc w/encls (w/o Attachments to Operating License): See next page

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cc w/encls (w/o Attachments to Operating License): See next page
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Accession No: ML041000374 - Enclosure 1 - Ltr. to S. A. Byrne w/Operating License NPF-12
Accession No: ML041000447 - Technical Specifications (Appendix A-C)
Accession No: ML041040087 - Enclosure 2 - Federal Register Notice

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Enclosure 1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, DC 20355

SOUTH CAROLINA ELECTRIC & GAS COMPANY
SOUTH CAROLINA PUBLIC SERVICE AUTHORITY
DOCKET NO. 50-395
VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1
RENEWED FACILITY OPERATING LICENSE NO. NPF-12

1. The U.S. Nuclear Regulatory Commission (the Commission or the NRC) having previously made the findings set forth in License No. NPF-12 issued August 6, 1982, has now found that:

   A. The application to renew License No. NPF-12 filed by South Carolina Electric & Gas Company acting for itself and South Carolina Public Service Authority (the licensees) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;

   B. Actions have been identified and have been or will be taken with respect to (1) managing the effects of aging during the period of extended operation on the functionality of structures and components that have been identified to require review under 10 CFR 54.21(a)(1), and (2) time-limited aging analyses that have been identified to require review under 10 CFR 54.21(c), such that there is reasonable assurance that the activities authorized by this renewed license will continue to be conducted in accordance with the current licensing basis, as defined in 10 CFR 54.3, for Virgil C. Summer Nuclear Station (V. C. Summer), Unit 1, and that any changes made to the plant's current licensing basis in order to comply with 10 CFR 54.29(a) are in accord with the Act and the Commission's regulations;

   C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   D. There is reasonable assurance: (i) that the activities authorized by this renewed operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;

   E. South Carolina Electric & Gas Company[1] is technically qualified to engage in the activities authorized by this renewed operating license in accordance with the rules and regulations of the Commission;

[1]South Carolina Electric & Gas company is authorized to act as agent for the South Carolina Public Service Authority and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

F. The licensees have satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements," of the Commission's regulations;

G. The issuance of this renewed license will not be inimical to the common defense and security or to the health and safety of the public;

H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the Commission concludes that the issuance of Renewed Facility Operating License NPF-12, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 51, of the Commission's regulations and all applicable requirements have been satisfied; and

I. The receipt, possession, and use of source, byproduct and special nuclear material as authorized by this renewed license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40, and 70.

2. On the basis of the foregoing findings regarding this facility, Facility Operating License No. NPF-12, issued August 6, 1982, is superseded by Renewed Facility Operating License No. NPF-12, which is hereby issued to the South Carolina Electric & Gas Company and the South Carolina Public Service Authority (the licensees) to read as follows:

A. This renewed license applies to the Virgil C. Summer Nuclear Station, Unit 1, a pressurized water reactor and associated equipment (the facility), owned by the South Carolina Electric & Gas Company and South Carolina Public Service Authority. The facility is located in Fairfield County, South Carolina, and is described in South Carolina Electric & Gas Company's Final Safety Analysis Report, as amended through No. 33, and the Environmental Report, as amended through Amendment No. 5.

B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses:

(1) South Carolina Electric & Gas Company (SCE&G), pursuant to Section 103 of the Act and 10 CFR Part 50, to possess, use, and operate the facility at the designated location in Fairfield County, South Carolina, in accordance with the procedures and limitations set forth in this renewed license;

(2) South Carolina Public Service Authority to possess the facility at the designated location in Fairfield County, South Carolina, in accordance with the procedures and limitations set forth in this renewed license;

(3) SCE&G, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as amended through Amendment No. 33;

(4) SCE&G, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed neutron sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(5) SCE&G, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) SCE&G, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain, and is subject to, the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

SCE&G is authorized to operate the facility at reactor core power levels not in excess of 2900 megawatts thermal in accordance with the conditions specified herein and in Attachment 1 to this renewed license. The preoperational tests, startup tests and other items identified in Attachment 1 to this renewed license shall be completed as specified. Attachment 1 is hereby incorporated into this renewed license.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment 167, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. South Carolina Electric & Gas Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Conduct of Work Activities During Fuel Load and Initial Startup

SCE&G shall review by committee all facility construction, preoperational testing, and system demonstration activities performed concurrently with facility initial fuel loading or with the facility startup test program to assure that the activity will not affect the safe performance of the facility fuel loading or the portion of the facility startup program being performed. The review shall address, as a minimum, system interaction, span of control, staffing, security, and health physics, with respect to the performance of the activity concurrently with the facility fuel loading or the portion of the facility startup program being performed. The committee for the review shall be composed of at least three members, knowledgeable in the above areas, and who meet the qualifications for professional-technical personnel specified by Section 4.4 of ANSI N18.7-1971. At least one of these three shall be a senior member of the Plant Manager's staff.

(4) Initial Test Program

SCE&G shall conduct the post-fuel-loading initial test program set forth in Chapter 14 of the Final Safety Analysis Report, as amended through No. 33, without making any major modifications to this program unless such modifications have been identified and have received prior NRC staff approval. Major modifications are defined as:

a. Elimination of any test other than those identified as non-essential in Chapter 14 of SCE&G's Final Safety Analysis Report, as amended through Amendment No. 33,

b. Modifications of test objectives, methods or acceptance criteria for any test other than those identified as non-essential in Chapter 14 of SCE&G's Final Safety Analysis Report, as amended through Amendment No. 33,

c. Performance of any test at a power level different from the power level indicated in the described program; and

d. Failure to complete any tests included in the described program (planned or scheduled for power levels up to the authorized power level).

For the performance of startup testing as described in Table 14.1-75 of the Final Safety Analysis Report, as amended through Amendment No. 33, compliance with items 3 and 4 of Table 3.3-1 of the Technical Specifications is not required.

(5) Stability of the West Embankment and its Effects on the Intake Structure (Section 2.5.4, SSER 3)

SCE&G shall conduct the monitoring program discussed in Section 2.5.4.10.6.2 of the Final Safety Analysis Report, as amended through Amendment No. 33, to specifically include the following:

a. At the vicinity of the pumphouse and intake structure, four settlement points capable of monitoring both horizontal and vertical movements shall be established to monitor the embankment movements.

b. The submerged slope profile of the west embankment over the intake structure shall be established and monitored to detect any unusual movements that may affect the intake structure.

c. The schedule and the reporting requirements of the above inspection shall be in accordance with the recommendations stated in Regulatory Guide 1.127.

d. The condition of the intake structure shall be monitored to detect new cracks and changes to the old grouted or ungrouted cracks. Observed changes (length or width) in existing cracks and any new cracks shall be reported by SCE&G to the NRC staff. The maximum inspection interval for this monitoring of the intake structure is five (5) years.

e. The condition of the intake structure shall also be monitored as specified in (d.) above following any earthquake during which the plant seismic instrumentation indicates that the operating basis earthquake has been exceeded.

(6) Design Verification Program (Section 3.7.4, SSER #5)

Prior to December 31, 1982 SCE&G shall provide a final report to the NRC staff delineating the final resolution of the actions taken to satisfy the recommendations of the independent design verification conducted by Stone & Webster Engineering Corporation.

(7) Thermal Sleeves (Section 3.9.3, SSER #5)

Prior to startup after the first refueling outage, SCE&G shall provide, for NRC staff review and approval, justification for continued operation with the thermal sleeves removed from selected nozzles in the reactor coolant system.

(8) Environmental Qualification of Mechanical and Electrical Equipment (Section 3.11, SSER 4)

a. SCE&G shall complete all actions related to environmental qualification of equipment on a schedule specified in Section 3.11 of Supplement 4 to the Safety Evaluation Report.

b. Complete and audible records shall be available and maintained at a central location by SCE&G. Such records shall describe the environmental qualification methods used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with NUREG-0588, “Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment,” Revision 1, dated July 1981. Such records shall be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified to document compliance with NUREG-0588.

c. Prior to startup after the first major shutdown or refueling outage after June 1983, SCE&G shall be in compliance with the provisions of NUREG-0588 for safety-related electrical equipment exposed to a harsh environment.

(9) Mechanical Performance (Section 4.2.3, SER)

Prior to startup after the first refueling outage, SCE&G shall examine fuel rods for baffle-jetting failure as specified in Section 4.2.3 of the Safety Evaluation Report. Should damage be observed at that time, a corrective action plan shall be submitted to the NRC staff for review and approval.

(10) Overpressurization Protection (Section 5.2.2, SSER 4)

Prior to startup after the first refueling outage, SCE&G shall install an NRC staff-approved low-temperature overpressurization protection system. The preliminary design shall be provided for NRC staff review not later than June 30, 1983.

(11) Inservice Inspection and Testing (Section 5.2.4, SSER 3)

SCE&G shall perform the following actions in conjunction with the first inservice examination:

a. Demonstrate the ability of the ultrasonic examination procedure to detect actual flaws and/or artificial reflectors in the volume subject to examination to the acceptance standards of Paragraph IWB-3500 in weldments representative of the design and materials of construction.

b. In the event that one-third thickness semi-circular reference flaws cannot be detected and discriminated from inherent anomalies, the entire volume of the weld shall be examined during the inservice inspection.

c. The reporting of the inservice inspection examination results shall be documented in a manner to define qualitatively whether, the weldment and the heat affected zone and adjacent base metal on both sides of the weld were examined by ultrasonic angle beam techniques.

(12) Design Description - Control (Section 4.3.2, SER)

SCE&G is prohibited from using part-length rods during power operation.

(13) Deleted

(14) Deleted

(15) Deleted

(16) Cable Tray Separation (Section 8.3.3, SSER 4)

Prior to startup after the first refueling outage, SCE&G shall implement the modifications to the cable trays discussed in Section 8.3.3 of Supplement No. 4 to the Safety Evaluation Report or demonstrate to the NRC staff that faults induced in non-class 1E cable trays will not result in failure of cable in the adjacent Class 1E cable trays.

(17) Alternate Shutdown System (Section 9.5.1, SSER 4)

Prior to startup after the first refueling outage, SCE&G shall install a source range neutron flux monitor independent of the control complex as part of the alternate shutdown system.

(18) Fire Protection System (Section 9.5.1, SSER 4)

Virgil C. Summer Nuclear Station shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility, and as approved in the Safety Evaluation Report (SER) dated February 1981 (and Supplements dated January 1982 and August 1982) and Safety Evaluations dated May 22, 1986, November 26, 1986, and July 27, 1987 subject to the following provisions:

The licensee may make changes to the approved fire protection program without prior approval of the

Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of fire.

(19) Instrument and Control Vibration Tests for Emergency Diesel Engine Auxiliary Support Systems (Section 9.5.4, SER)

Prior to startup after the first refueling outage, SCE&G shall either provide test results and results of analyses to the NRC staff for review and approval which validate that the skid-mounted control panels and mounted equipment have been developed, tested, and qualified for operation under severe vibrational stresses encountered during diesel engine operation, or SCE&G shall floor mount the control panels presently furnished with the diesel generators separate from the skid on a vibration-free floor area.

(20) Solid Radioactive Waste Treatment System (Section 11.2.3, SSER 4)

SCE&G shall not ship “wet” solid wastes from the facility until the NRC staff has reviewed and approved the process control program for the cement solidification system.

(21) Process and Effluent Radiological Monitoring and Sampling Systems (Section 11.3, SSER 4)

Prior to startup after the first refueling outage, SCE&G shall install and calibrate the condensate demineralizer backwash effluent monitor RM-L11.

(22) Core Reactivity Insertion Events (Section 15.2.4, SSER 4)

For operations above 90% of full power, SCE&G shall control the reactor manually or the rods shall be out greater than 215 steps until written approval is received from the NRC staff authorizing removal of this restriction.

(23) NUREG-0737 Conditions (Section 22)

SCE&G shall complete the following conditions to the satisfaction of the NRC staff. Each item references the related subpart of Section 22 of the SER and/or its supplements.

a. Procedures for Transients and Accidents (I.C.1, SSER 4)

Prior to startup after the first refueling outage, SCE&G shall implement emergency operating procedures based on guidelines approved by the NRC staff.

b. Special Low Power Testing and Training (I.G.1, SSER 4)

Within twelve months following completion of the startup test program, SCE&G shall provide a report describing the results of a comparison of actual plant data taken during the natural circulation test program to the simulator responses described in the SCE&G letter, T. C. Nichols, Jr. to H. R. Denton dated March 31, 1982.

c. Direct Indication of Safety Valve Position (II.D.3, SSER 4)

Prior to exceeding 5 percent of full power, the safety valve position indication system shall be seismically qualified by SCE&G consistent with the component or system to which it is attached, and documentation of this shall be provided to the NRC staff for review and approval.

d. Inadequate Core Cooling Instruments (II.F.2, SSER 4)

Prior to startup after the first major shutdown or refueling outage after June 30, 1983, SCE&G shall complete upgrading of the incore thermocouple wiring and qualification of isolators, reference junction boxes and connectors.

e. Plant-Specific Calculations for Compliance with 10 CFR Section 50.46 (II.K.3.31, SSER 1)

Within one year after model revisions are approved by the NRC staff, SCE&G shall provide a supplemental plant-specific analysis to verify compliance with 10 CFR 50.46, using the revised models developed under item II.K.3.30 of NUREG-0737.

f. Upgrade Emergency Support Facilities (III.A.1.2, SSER 4)

SCE&G shall complete its emergency response facilities as follows:

(i) Safety parameter display system - April 1, 1983
(ii) Emergency operations facility - April 1, 1983
(iii) Technical support center - April 1, 1983

(24) Deleted

(25) Confirmatory Seismic Analysis (ASLB Partial Initial Decision, July 20, 1982, Section VI.2)

During the first year of operation, SCE&G shall successfully complete the confirmatory program on plant equipment and components within the guidelines established in the findings contained in the ASLB Partial Initial Decision dated July 20, 1982, to demonstrate to the NRC staff's satisfaction that explicit safety margins exist for each component necessary for shutdown and continued heat removal in the event of the maximum potential shallow earthquake.

(26) Plume Exposure Emergency Planning Zone (ASLB Supplemental Partial Initial Decision, August 4, 1982, Section VIII.3)

During the first year of operation, SCE&G shall assure that the plume exposure emergency planning zone has been expanded to include the Kelly Miller, Greenbriar Headstart and Chapin Elementary schools and the emergency evacuation plans have been adjusted accordingly.

(27) Transportation Planning (ASLB Supplemental Partial Initial Decision, August 4, 1982, Section VIII.4)

During the first year of operation, SCE&G shall assure that the defects in transportation planning discussed in Finding 24 of the ASLB Supplemental Partial Initial Decision dated August 4, 1982 have been remedied.

(28) Food Pathway Contamination (ASLB Supplemental Partial Initial Decision, August 4, 1982, Section VIII.6)

During the first year of operation, SCE&G shall assure that plans to implement remedial and preventive measures for consumer protection against food pathway contamination have been formulated and communicated to the agricultural community.

(29) Siren Alerting System (ASLB Supplemental Partial Initial Decision, August 4, 1982, Section VIII.7)

Prior to exceeding 5% of full power, SCE&G shall complete installation and satisfactory testing of its siren alerting system.

(30) Emergency Facilities and Staffing (ASLB Supplemental Partial Initial Decision, August 4, 1982, Section VIII.8)

SCE&G shall complete the following three items related to emergency preparedness to the satisfaction of the NRC staff, consistent with the Supplement No. 2 to the Safety Evaluation Report, page A-13:

(i) Minimum shift manning requirements
(ii) Emergency response facilities
(iii) Meteorological and dose assessment capability

(31) Final NRC Approval of Emergency Preparedness (ASLB Supplemental Partial Initial Decision, August 4, 1982, Section VIII.9)

Prior to exceeding 5% of full power, final NRC staff approval of the state of emergency preparedness for the Virgil C. Summer Nuclear Station site shall be required.

(32) Deleted

(33) Emergency Preparedness Exercise (Section 13.3, SSER #5)

Prior to March 31, 1983, SCE&G shall conduct an emergency exercise similar to that conducted on May 5, 1982 but which includes full participation of the local governments and partial State participation.

D. An exemption to the requirements of Paragraph III.B.4 of Appendix G to 10 CFR Part 50 is described in Section 5.3.1 of Supplement No. 1 to the Office of Nuclear Reactor Regulation's Safety Evaluation Report. A limited exemption to the requirements of Section IV.F.1(b) of Appendix E to 10 CFR Part 50 is described in a letter from B.J. Youngblood, NRC to O.W. Dixon, Jr., dated November 2, 1982. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

E. SCE&G shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Virgil C. Summer Nuclear Station Physical Security Plan," with revisions submitted through December 15, 1987; "Virgil C. Summer Nuclear Station Guard Training and Qualification Plan," with revisions submitted through March 24, 1987; and "Virgil C. Summer Nuclear Station Safeguards Contingency Plan," with revisions submitted through March 24, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.

F. This renewed license is subject to the following additional condition for the protection of the environment:

Before engaging in activities that may result in a significant adverse environmental impact that was not evaluated or that is significantly greater than that evaluated in the Final Environmental Statement, SCE&G shall provide a written notification of such activities to the NRC Office of Nuclear Reactor Regulation and receive written approval from that office before proceeding with such activities.

G. Reporting to the Commission:

(1) SCE&G shall report any violations of the requirements contained in Section 2, Items C(1), C(3) through (33), E, F, K, and L of this renewed license within twenty-four (24) hours by telephone and confirm by telegram, mailgram, or facsimile transmission to the NRC Regional Administrator, Region II, or designee, not later than the first working day following the violation, with a written followup report within fourteen (14) working days.

(2) SCE&G shall notify the Commission, as soon as possible but not later than one hour, of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.

H. The licensees shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.

I. In accordance with the Commission's direction in its Statement of policy, Licensing and Regulatory Policy and Procedures for Environmental Protection: Uranium Fuel Cycle Impacts, October 29, 1982, this license is subject to the final resolution of the pending litigation involving Table S-3. See, Natural Resources Defense Council v. NRC, No. 74-1586 (April 27, 1982).

J. Additional License Conditions

The Additional Conditions contained in Appendix C, as revised through Amendment No. 137, are hereby incorporated into this renewed license. South Carolina Electric & Gas Company shall operate the facility in accordance with the Additional Conditions.

K. Updated Final Safety Analysis Report

The South Carolina Electric & Gas Company Updated Final Safety Analysis Report supplement, submitted pursuant to 10 CFR 54.21(d), describes certain future activities to be completed prior to the period of extended operation. The

South Carolina Electric & Gas Company shall complete these activities no later than August 6, 2022, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement, as revised, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4) following issuance of this renewed license. Until that update is complete, the South Carolina Electric & Gas Company may make changes to the programs and activities described in the supplement without prior Commission approval, provided that the South Carolina Electric & Gas Company evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

L. All capsules in the reactor vessel that are removed and tested must meet the test procedures and reporting requirements of ASTM E 185-82 to the extent practicable for the configuration of the specimens in the capsule. Any changes to the capsule withdrawal schedule, including spare capsules, must be approved by the NRC prior to implementation. All capsules placed in storage must be maintained for future insertion. Any changes to storage requirements must be approved by the NRC.

M. This renewed license is effective as of the date of issuance and shall expire at midnight, August 6, 2042.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

J. E. Dyer, Director
Office of Nuclear Reactor Regulation

Enclosures:

1. Appendix A (Technical Specifications)
2. Appendix B (Environmental Protection Plan)
3. Appendix C (Additional Conditions)

Date of Issuance: April 23, 2004

Attachment 1 to Renewed Operating License No. NPF-12

This attachment identifies certain preoperational tests, system demonstrations and other items which must be completed to the satisfaction of NRC Region II. SCE&G shall not proceed without written confirmation from NRC Region II that the following items have been completed in accordance with the conditions and schedules set forth below:

1. Prior to initial criticality, SCE&G shall complete to the satisfaction of NRC Region II the following 10 CFR Part 21 identified item:

   Correct the diesel generator slow start times attributed to fuel oil header drain down (81-29-01).

2. Prior to initial criticality, SCE&G shall complete to the satisfaction of NRC Region II the following open items:

   a. Complete the replacement of all damaged prefilters external to the containment building (82-07-03).

   b. Complete the repair and testing of diesel generator B before entering Mode 4.

3. Prior to exceeding 5% of full power, SCE&G shall complete to the satisfaction of NRC Region II the requirements of the following bulletins:

   a. Failure of gate-type valves to close against differential pressure (81-BU-02).

   b. Seismic analysis for as-built safety-related piping systems required to support operations above 5% of full power (79-BU-14).

4. Prior to exceeding 5% of full power, SCE&G shall complete to the satisfaction of NRC Region II the following 10 CFR 21 identified items:

   a. Westinghouse 3-inch gate valve closure failure (80-37-10).

   b. Westinghouse 4-inch gate valve closure failure (81-05-09).

5. Prior to exceeding 5% of full power, SCE&G shall complete to the satisfaction of NRC Region II the following open items:

   a. Correct the hydrogen recombiner high hydrogen alarm set at 6% versus proposed Technical Specification limit of 2% (80-06-07).

   b. Satisfactorily complete the capability test for Fe-55 analyses of liquid waste samples.

6. Prior to full power operation, SCE&G shall complete to the satisfaction of NRC Region II the requirements of the following bulletin:

   Audibility problems encountered during evacuation alarm in high noise area (79-BU-18).

7. Prior to full power operation, SCE&G shall complete to the satisfaction of NRC Region II the following open item:

   Resolve the problem of the reactor building temperature being greater than expected during hot functional testing (80-25-09).

cc:

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