



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
SAM NUNN ATLANTA FEDERAL CENTER  
61 FORSYTH STREET SW SUITE 23T85  
ATLANTA, GEORGIA 30303-8931**

April 6, 2004

Florida Power and Light Company  
ATTN: Mr. J. A. Stall, Senior Vice President  
Nuclear and Chief Nuclear Officer  
P. O. Box 14000  
Juno Beach, FL 33408-0420

SUBJECT: NOTIFICATION OF TURKEY POINT NUCLEAR PLANT - SAFETY SYSTEM  
DESIGN AND PERFORMANCE CAPABILITY INSPECTION - NRC  
INSPECTION REPORT 05000250/2004008 AND 05000250/2004008

Dear Mr. Stall:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a safety system design and performance capability inspection at your Turkey Point Nuclear Plant during the weeks of June 7 and June 21, 2004. A team of five inspectors will perform this inspection. The inspection team will be led by Mr. Jim Moorman, a Senior Reactor Inspector from the NRC Region II Office. This biennial inspection will be conducted in accordance with baseline inspection program Attachment 71111.21, "Safety System Design And Performance Capability."

The inspection will evaluate the capability of installed plant equipment to detect and respond to a steam generator tube rupture event. Procedures which direct the mitigating actions for this event will also be evaluated.

During a telephone conversation on April 5, 2004, Mr. Norman Merriweather, and Mr. Walt Parker of your staff, confirmed arrangements for an information gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information gathering visit: Week of May 16, 2004
- Onsite inspection weeks: June 7 and June 21, 2004

The purpose of the information gathering visit is to obtain information and documentation, outlined in the Enclosure, that is needed to support the inspection. Mr. Walt Rogers, a Region II Senior Reactor Analyst, may accompany Mr. Moorman during the information gathering visit to review probabilistic risk assessment data and identify risk significant components which will be examined during the inspection. Please contact Mr. Moorman prior to preparing copies of the materials listed in the Enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation. During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for site access; and the availability of knowledgeable plant engineering and licensing personnel to serve as points of contact during the inspection.

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact Mr. Moorman at (404) 562-4647 or me at (404) 562-4605.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its Enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publically Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

**/RA: Walter Rogers for:/**

Charles R. Ogle, Chief  
Engineering Branch 1  
Division of Reactor Safety

Docket Nos.: 50-250, 50-251  
License Nos.: DPR-31, DPR-41

Enclosure: Information Request for the Safety System Design and Performance Capability Inspection.

cc w/encl:  
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Michael O. Pearce  
Plant General Manager  
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(cc w/encl cont'd - See page 3)

(cc w/encl cont'd)

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Distribution w/encl: See page 4

FP&L

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NAME	MOORMAN	MUNDAY					
DATE	4/6 /2004	4/6 /2004					
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**INFORMATION REQUEST FOR THE SAFETY SYSTEM DESIGN AND PERFORMANCE  
CAPABILITY INSPECTION  
TURKEY POINT STEAM GENERATOR TUBE RUPTURE MITIGATION**

(Please provide the information electronically in searchable “.pdf” files on CDROM. The CDROM should be indexed and hyperlinked to facilitate ease of use. Information in “lists” should contain enough information to be easily understood by someone who has a knowledge of pressurized water reactor technology.)

1. Piping and instrumentation drawings for Safety Injection System (SI), Auxiliary Feedwater System (AFW), Main Steam (MS), Chemical and Volume Control System (CVCS), Residual Heat Removal System (RHR), Intake Cooling Water (ICW), Component Cooling Water, (CCW) Reactor Coolant System, including the pressurizer (RCS), and Instrument Air (IA) (Paper copies (2) are preferred for these)
2. The normal operating procedures for the systems listed above. Include the procedures which specify the normal valve alignment for these systems.
3. Surveillance procedures used to ensure the operability of equipment required by your Technical Specifications that is used during the mitigation of the SGTR event.
4. Procedures used for the operational testing of check valves in portions of the emergency core cooling systems used during mitigation of the SGTR event.
5. System description and operator training modules for a SGTR event.
6. Procedures used to sample the reactor coolant system and steam generators during an SGTR event.
7. Calibration and functional testing procedures for all radiation monitoring instrumentation used in the detection and mitigation phases of the SGTR event.
8. Calibration and functional test procedures for instruments used to monitor RCS pressure, pressurizer level and pressure, SG level and pressure, RCS hot and cold leg temperature, RCS subcooling, feedwater flow, steam flow, core exit temperature, high pressure injection flow, SI flow, refueling water storage tank level, pressurizer heater status, safety relief valve position indicator, AFW flow, condensate storage tank (CST) level, makeup flow, and letdown flow.
9. Test procedures for the primary and secondary system safety relief valves and pressurizer power operated relief valves.
10. Test procedures for the AFW starting logic.
11. Emergency Operating Procedures (EOPs) and supporting procedures, EOP basis documents, step deviation document, writers guide, and users guide. Abnormal Operating Procedures that would be used during a steam generator tube leak/rupture event.

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12. Calculations used to support the setpoints in Emergency Operating Procedures for a SGTR event.
13. A list of engineering calculations applicable to AFW, SI, CVCS, RCS, CC, and other systems used in the mitigation of the SGTR event.
14. A list of temporary modifications and operator work-arounds since 2001.
15. A list of operability determinations performed since 1999.
16. A list of corrective action program documents and non-routine work requests initiated since 2001 for the following: CVCS, AFW, SI, ICW, CCW, MS, Radiation Monitoring, intermediate voltage electrical, low voltage electrical, direct current systems as well as the Pressurizer PORVs and Steam Generator atmospheric dump valves.
17. System Health Reports and System Performance Trends for all systems going back 2 years.
18. A copy of the Maintenance Rule performance criteria for all plant systems. A list of plant systems currently monitored under 50.65(a)(1) and the performance goals. A list of Maintenance Rule functional failures since 2000.
19. Quality Assurance audits, self-assessments and third party assessments performed on AFW, SI, CVCS, RCS, IA systems and other related systems in the last 24 months.
20. Quality Assurance audits, self-assessments, and third party assessments performed on engineering and maintenance department activities performed in the last 24 months.
21. Operator training lesson plans, system descriptions, and job performance measures for the EOPs, EOP support procedures, abnormal, and normal operating procedures that would be used to mitigate an SGTR event.
22. Key electrical single line drawings of the intermediate and low voltage alternating current (AC) and DC power systems. (Paper copies (2) are preferred for these)
23. The electrical system load list(s).
24. A brief description of the mitigation strategy for handling the SGTR event, including operator actions and equipment used.
25. A list of Operating Experience Program evaluations of industry, vendor, or NRC generic issues for the past 3 years.
26. A list of equipment and operator actions with a Risk Achievement Worth (RAW) greater than 1.02.

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27. Probabilistic Risk Assessment (PRA) Event Tree for SGTR initiating event. A List of PRA system dependencies and success criteria for AFW, SI, CVCS, MS, CCW systems, and their support systems.
28. Design Basis Manual for major plant electrical, mechanical, and control systems.
29. Plant Technical Specifications, Bases, and Technical Requirements Manual
30. A current copy of the Updated Final Safety Analysis Report.
31. A list of all plant permanent plant changes, design changes, setpoint changes, procedure changes, equivalency evaluations, suitability analyses, calculations, and commercial grade dedications since 2001.
32. Plant procedures that provide the guidelines for the following: 1) Corrective Action Program, 2) Program for the evaluation of changes, tests, and experiments [50.59 Program] 3) Program for the incorporation of local and industry operating experience into Turkey Point programs and procedures 4) Program for modifications to safety-related plant systems 5) Program for implementation of the Maintenance Rule
33. A current copy of the Offsite Dose Calculation Manual