

(2) Implement the upgraded EOP's

Prior to startup following the first refueling outage

(e) Emergency Response Facilities

(1) Technical Support Center fully functional with exception of Regulatory Guide 1.97 implementation

Prior to startup following the first refueling outage

(2) Operational Support Center fully functional with exception of Regulatory Guide 1.97 implementation

Prior to startup following the first refueling outage

(3) Emergency Operations Facility fully functional with exception of Regulatory Guide 1.97 implementation

Prior to startup following the first refueling outage



LICENSE AUTHORITY FILE COPY  
UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DO NOT REMOVE

November 1, 1984

Docket No. 50-416

Mr. J. B. Richard  
Senior Vice President, Nuclear  
Mississippi Power & Light Company  
P.O. Box 23054  
Jackson, Mississippi 39205

*Posted*  
License NPF-29  
100% Power  
Operating License  
(supersedes License  
NPF-15 in its entirety)

Dear Mr. Richard:

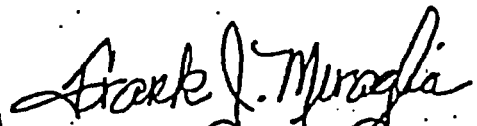
Subject: Issuance of Facility Operating License NPF-29 -  
Grand Gulf Nuclear Station, Unit 1

The U.S. Nuclear Regulatory Commission has issued the enclosed Facility Operating License NPF-29, together with Technical Specifications and Environmental Protection Plan for the Grand Gulf Nuclear Station, Unit 1. This action has been taken in accordance with the Commission's Order dated October 25, 1984. This license authorizes operation at 100% power (3833 megawatts thermal).

Also enclosed are copies of a related notice, the original of which has been forwarded to the Office of the Federal Register for publication, and of an evaluation of the effect of 40 years of license duration with respect to environmental matters.

Four signed copies of Amendment No. 2, Indemnity Agreement No. B-72 which covers the activities authorized under License No. NPF-29 are also enclosed. Please sign all copies and return one copy to this office.

Sincerely,

*for*   
Darrell G. Eisenhart, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosures:

1. Facility Operating License  
NPF-29
2. Federal Register Notice
3. 40-year Evaluation
4. Amendment No. 2 to Indemnity  
Agreement B-72

cc w/encl:  
See next page

## LICENSE AUTHORITY FILE COPY

DO NOT REMOVE

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-416GRAND GULF NUCLEAR STATION, UNIT 1NOTICE OF ISSUANCE OF FACILITY OPERATING LICENSE

On June 16, 1982, the U.S. Nuclear Regulatory Commission (the Commission) issued Facility Operating License No. NPF-13 to the Mississippi Power & Light Company, Middle South Energy, Inc., and South Mississippi Electric Power Association (the licensees) authorizing operation of the Grand Gulf Nuclear Station, Unit 1, (the facility) at reactor core power levels not in excess of 191 megawatts thermal (five percent of full power) in accordance with the provisions of the license, the Technical Specifications and the Environmental Protection Plan.

The Commission has now issued Facility Operating License No. NPF-29 which authorizes operation of the Grand Gulf Nuclear Station, Unit 1, at reactor core power levels not in excess of 3833 megawatts thermal in accordance with the provisions of the license, the Technical Specifications, and the Environmental Protection Plan. License No. NPF-29 supersedes NPF-13, as amended, pursuant to Commission Order CLI-84-19, dated October 25, 1984.

The Grand Gulf Nuclear Station, Unit 1, is a boiling water reactor located at the licensees' site in Claiborne County, Mississippi. The license is effective as of its date of issuance and shall expire at midnight on June 16, 2022.

The application for the license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter 1, which are set forth

in the license. Prior public notice of the overall action involving the proposed issuance of an operating license was published in the Federal Register on July 28, 1978 (43 FR 32903).

The Commission has determined that the issuance of this license will not result in any environmental impacts other than those evaluated in the Final Environmental Statement since the activity authorized by the license is encompassed by the overall action evaluated in the Final Environmental Statement.

For further details with respect to this action, see (1) Facility Operating License No. NPF-29 complete with Technical Specifications and the Environmental Protection Plan; (2) the Commission's Safety Evaluation Report, dated September 1981 (NUREG-0831), and Supplements 1 through 7; (3) the Final Safety Analysis Report and Amendments thereto; (4) the Final Environmental Statement, dated September 1981 (NUREG-0777); (5) the Evaluation of the Effect of License Duration on Matters Discussed in the Final Environmental Statement for the Grand Gulf Nuclear Station Units 1 and 2 (dated September 1981); (6) the Commission's Memorandum dated August 1, 1984; and (7) Commission Order CLI-84-19, dated October 25, 1984.

These items are available at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C. 20555, and at the George McLendon Library, Hinds Junior College, Raymond, Mississippi 39154. A copy of Facility Operating License NPF-29 may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing. Copies of the Safety Evaluation Report and its Supplements (NUREG-0831) and the Final Environmental Statement (NUREG-0777) may be purchased at current rates from the National Technical Information Service, Department of

Commerce, 5285 Port Royal Road, Springfield, Virginia 22161, and through the NRC GPO sales program by writing to the U.S. Nuclear Regulatory Commission, Attention: Sales Manager, Washington, D. C. 20555. GPO deposit account holders may call (301) 492-9530.

Dated at Bethesda, Maryland, this 1<sup>st</sup> day of November 1984.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas M. Novak, Assistant Director  
for Licensing  
Division of Licensing

EVALUATION OF THE EFFECT OF LICENSE DURATION ON MATTERS DISCUSSED  
IN THE FINAL ENVIRONMENTAL STATEMENT FOR THE GRAND GULF NUCLEAR STATION  
UNITS 1 AND 2 (Dated September 1981)

INTRODUCTION

The Final Environmental Statement (FES) for the operation of the Grand Gulf Nuclear Station Unit Nos. 1 and 2 was published in September 1981. At that time it was staff practice to issue operating licenses for a period of 40 years from the date of the construction permit. For Grand Gulf the CPs were issued in September 1974, thus, approximately 30 years of operating life would be available.

By letter dated June 10, 1982, Mississippi Power & Light Company requested that the operating license for Grand Gulf Nuclear Station, Unit 1 then under consideration by the staff, have a duration of 40 years from the date of issuance. On June 16, 1982, a license, conditioned to not exceed 5% power, effective for a 40-year period from issuance of the CP, was issued to MP&L for the operation of Grand Gulf Nuclear Station, Unit 1.

DISCUSSION

The staff has reviewed the Grand Gulf FES to determine which aspects considered in the FES are affected by the duration of the operating license. In general, the FES assesses various impacts associated with operation of the facility in terms of annual impacts and balances these against the anticipated annual energy production benefits. Thus, the overall assessment and conclusions would not be dependent on specific operating life. There are, however, three areas in which a specific operating life was assumed:

1. Project costs are based on a 30-year levelized cost.
2. Radiological assessments are based on a 15-year plant midlife.
3. Uranium fuel cycle impacts are based on one initial core load and annual refuelings.

These were assessed to determine whether the use of a 40-year operating period rather than a 30-year operating period would significantly affect our assessment concerning these areas.

EVALUATION:

The staff's appraisal of the significance of the use of 40 years of operation rather than 30 as it affects these three areas is presented in the following discussions:

1. Projected Costs - The projected costs of the facility which includes the cost of decommissioning are based on a 30-year operating life and are levelized over that period of time. The use of a 40-year operating period rather than a 30-year period would not significantly affect the operating and maintenance cost. If the facility's capital cost were spread over a 40-year period the overall resulting cost of facility operation would be lowered. Therefore, any extension in the operating life of the facility would result in savings in system production costs. The production of energy at reduced cost results in an incremental net benefit for the use of a 40-year operating life of the facility.
2. Radiological Assessments - The NRC staff calculates dose commitments to the human population residing around nuclear power reactors to assess the impact on people from radioactive material released from these reactors. The annual dose commitment is calculated to be the dose that would be received over a 50-year period following the intake of radioactivity for 1 year under the conditions that would exist 15 years after the plant began operation.

The 15 year period is chosen as representing the midpoint of plant operation and factors into the dose models by allowing for buildup of long life radionuclides in the soil. It affects the estimated doses only for radionuclides ingested by humans that have half-lives greater than a few years. For a plant licensed for 40 years, increasing the buildup period from 15 to 20 years would increase the dose from long life radionuclides via the ingestion pathways by 33% at most. It would have much less effect on dose from shorter life radionuclides. Tables D-4 and D-5 of Appendix D to the FES indicate that the estimated doses via the ingestion pathways are only a fraction of the regulatory design objectives. For example, the ingestion dose to the thyroid is 7.0 mrem/yr compared to an Appendix I design objective of 15 mrem/yr. Thus, for 7 mrem/yr, an increase of even as much as 33% in these pathways results in a dose within the Appendix I guidelines and would still not be significant.

3. Uranium Fuel Cycle Impacts - The impacts of the uranium fuel cycle are based on 30 years of operation of a model LWR. The fuel requirements for the model LWR were assumed to be one initial core load and 29 annual refuelings (approximately 1/3 core). The annual fuel requirement for the model LWR averaged out over a 40-year operating life (1 initial core and 39 refuelings of approximately 1/3 core) would be reduced slightly as compared to the annual fuel requirement averaged for a 30-year operating life.

The net result would be an approximately 1.5% reduction in the annual fuel requirement for the model LWR. This small reduction in fuel requirements would not lead to significant changes in the impacts of the uranium fuel cycle. The staff does not believe that there would be any changes to Grand Gulf FES Table 5.10 (S-3) that would be necessary in order to consider 40 years of operation. If anything, the values in Table 5.10 become more conservative when a 40-year period of operation is considered.

#### CONCLUSION

The staff has reviewed the Grand Gulf FES and determined that only three of the areas related to its NEPA analysis discussed in the statement were tied directly to a 30-year operating period. The staff has concluded, based on the reasons discussed in the sections above, that the impacts associated with a 40-year operating license duration are not significantly different from those associated with a 30-year operating license duration assessed in the Grand Gulf FES. Therefore, the staff considers that the Grand Gulf FES sufficiently addresses the environmental impacts associated with a 40-year operating period.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket No. 50-416

Amendment to Indemnity Agreement No. B-72  
Amendment No. 4

Effective \_\_\_\_\_, Indemnity Agreement No. B-72, between Mississippi Power and Light Company, System Energy Resources, Inc., and South Mississippi Electric Power Association and the Nuclear Regulatory Commission dated July 15, 1981, as amended, is hereby further amended as follows:

The following named licensee "Entergy Operations, Inc." is added to the indemnity agreement.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

*Eileen M. McKenna*

Eileen M. McKenna, Acting Chief  
Policy Development and Technical Support Branch  
Program Management, Policy Development  
and Analysis Staff  
Office of Nuclear Reactor Regulation

Accepted \_\_\_\_\_, 1989

Accepted \_\_\_\_\_, 1989

By \_\_\_\_\_  
Mississippi Power and Light  
Company

By \_\_\_\_\_  
System Energy Resources, Inc.

Accepted \_\_\_\_\_, 1989

Accepted \_\_\_\_\_, 1989

By \_\_\_\_\_  
South Mississippi Electric  
Power Association

By \_\_\_\_\_  
Entergy Operations, Inc.

### 3.0 SURVEILLANCE REQUIREMENT (SR) APPLICABILITY

---

SR 3.0.1           SRs shall be met during the MODES or other specified conditions in the Applicability for individual LCOs, unless otherwise stated in the SR. Failure to meet a Surveillance, whether such failure is experienced during the performance of the Surveillance or between performances of the Surveillance, shall be failure to meet the LCO. Failure to perform a Surveillance within the specified Frequency shall be failure to meet the LCO except as provided in SR 3.0.3. Surveillances do not have to be performed on inoperable equipment or variables outside specified limits.

---

SR 3.0.2           The specified Frequency for each SR is met if the Surveillance is performed within 1.25 times the interval specified in the Frequency, as measured from the previous performance or as measured from the time a specified condition of the Frequency is met.

For Frequencies specified as "once," the above interval extension does not apply.

If a Completion Time requires periodic performance on a "once per . . ." basis, the above Frequency extension applies to each performance after the initial performance.

Exceptions to this Specification are stated in the individual Specifications.

---

SR 3.0.3           If it is discovered that a Surveillance was not performed within its specified Frequency, then compliance with the requirement to declare the LCO not met may be delayed, from the time of discovery, up to 24 hours or up to the limit of the specified Frequency, whichever is greater. This delay period is permitted to allow performance of the Surveillance. A risk evaluation shall be performed for any surveillance delayed greater than 24 hours and the risk impact shall be managed.

If the Surveillance is not performed within the delay period, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

(continued)

---

3.0 SURVEILLANCE REQUIREMENT (SR) APPLICABILITY

---

SR 3.0.3  
(continued)

When the Surveillance is performed within the delay period and the Surveillance is not met, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

---

(continued)

Primary Containment and Drywell Isolation Instrumentation  
3.3.6.1

Table 3.3.6.1-1 (page 1 of 5)  
Primary Containment and Drywell Isolation Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION C.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
<b>1. Main Steam Line Isolation</b>					
a. Reactor Vessel Water Level - Low Low Low, Level 1	1,2,3	2	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≥ -152.5 inches
b. Main Steam Line Pressure - Low	1	2	E	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≥ 837 psig
c. Main Steam Line Flow - High	1,2,3	2 per MSL	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7 SR 3.3.6.1.8	≤ 176.5 psid
d. Condenser Vacuum - Low	1,2 <sup>(a)</sup> , 3 <sup>(a)</sup>	2	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≥ 8.7 inches Hg vacuum
e. Main Steam Tunnel Ambient Temperature - High	1,2,3	2	D	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.5 SR 3.3.6.1.7	≤ 191°F
f. Manual Initiation	1,2,3	2	G	SR 3.3.6.1.7	NA
<b>2. Primary Containment and Drywell Isolation</b>					
a. Reactor Vessel Water Level - Low Low, Level 2	1,2,3	2 <sup>(b)</sup>	H	SR 3.3.6.1.1 SR 3.3.6.1.2 SR 3.3.6.1.3 SR 3.3.6.1.6 SR 3.3.6.1.7	≥ -43.8 inches

(continued)

(a) With any turbine stop valve not closed.

(b) Also required to initiate the associated drywell isolation function.

Recirculation Loops Operating  
3.4 .1

Figure 3.4.1-1  
Deleted  
via  
Amendment 141  
No new Page was issued

3.9 REFUELING OPERATIONS

3.9.1 Refueling Equipment Interlocks

LCO 3.9.1 The refueling equipment interlocks shall be OPERABLE.

APPLICABILITY: During in-vessel fuel movement with equipment associated with the interlocks.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. One or more required refueling equipment interlocks inoperable.</p>	<p>A.1 Suspend in-vessel fuel movement with equipment associated with the inoperable interlock(s).</p>	<p>Immediately</p>
	<p><u>OR</u></p>	
	<p>A.2.1 Insert a control rod withdrawal block.</p> <p><u>AND</u></p> <p>A.2.2 Verify all control rods are fully inserted in core cells containing one or more fuel assemblies.</p>	<p>Immediately</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.9.1.1 Perform CHANNEL FUNCTIONAL TEST on each of the following required refueling equipment interlock inputs: <ul style="list-style-type: none"><li>a. All-rods-in,</li><li>b. Refuel platform position, and</li><li>c. Refuel platform main hoist, fuel loaded.</li></ul>	7 days

by the aerial surveys, and walking patrols will be directed to the problem areas to evaluate the extent of the problem to be corrected.

The Erosion Control Inspection Program shall begin upon commencement of normal transmission line inspection procedures. Semi-annual surveys shall continue until stabilization of soil and vegetation (i.e., ground cover establishment) is achieved.

A summary of the field inspection program and any procedures implemented to control abnormal erosion conditions associated with transmission line maintenance activities shall be reported in the Annual Environmental Operation Report in accordance with Subsection 5.4.1. Field logs indicating locations of erosion damage and measures taken to rectify erosion problem areas and estimation of the time to achieve effective stabilization will be maintained and available for inspection for a period of five years. Results reported shall contain information encompassing but not limited to inspection date, estimated size of erosion problem area, probable cause of erosion, type of stabilization program, and date of effective stabilization, as appropriate.

#### 4.2.2 Cooling Tower Drift Program

Seven sampling sites were utilized to measure cooling tower drift-deposition. At least two of the sampling sites had duplicate sampling devices. Six of the seven sites were located in areas where maximum salt deposition was predicted. These areas were extrapolated from the Bechtel Salt Deposition Model developed for the GGNS Final Environmental Report. The seventh sampling site was a control site located south of Raymond, Mississippi. An eighth offsite control site was added in 1985 in Port Gibson, Mississippi.

Fallout samples were collected on a quarterly basis and analyzed for ten specific constituents. The details of the sampling procedure and chemical



analysis were submitted to the NRC's Environmental Engineering Branch for review and approval prior to plant operation above 5% power. An evaluation of the results of the Cooling Tower Drift Program indicated that the operation of the GGNS cooling tower produced no statistically significant effect upon the salt deposition rate for those chemical species evaluated. The cooling tower drift program was therefore terminated.

ENCLOSURE 2  
CORRECTIONS TO LICENSEE'S COPY  
BASED ON NRC STAFF REVIEW OF THE LICENSEE'S RECORDS  
(CD-ROM DATED 12/16/2003)  
GRAND GULF NUCLEAR STATION, UNIT 1  
DOCKET NO. 50-416



LICENSE AUTHORITY FILE COPY  
UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DO NOT REMOVE

November 1, 1984

Docket No. 50-416

Mr. J. B. Richard  
Senior Vice President, Nuclear  
Mississippi Power & Light Company  
P.O. Box 23054  
Jackson, Mississippi 39205

*Posted*  
*License NPF-29*  
*100% Power*  
*Operating License*  
*(supersedes License)*  
*NPF-15 in its entirety*

Dear Mr. Richard:

Subject: Issuance of Facility Operating License NPF-29 -  
Grand Gulf Nuclear Station, Unit 1

The U.S. Nuclear Regulatory Commission has issued the enclosed Facility Operating License NPF-29, together with Technical Specifications and Environmental Protection Plan for the Grand Gulf Nuclear Station, Unit 1. This action has been taken in accordance with the Commission's Order dated October 25, 1984. This license authorizes operation at 100% power (3833 megawatts thermal).

Also enclosed are copies of a related notice, the original of which has been forwarded to the Office of the Federal Register for publication, and of an evaluation of the effect of 40 years of license duration with respect to environmental matters.

Four signed copies of Amendment No. 2, Indemnity Agreement No. B-72 which covers the activities authorized under License No. NPF-29 are also enclosed. Please sign all copies and return one copy to this office.

Sincerely,

*for* *Frank J. Muraglia*  
Darrell G. Eisenhart, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Enclosures:

1. Facility Operating License  
NPF-29
2. Federal Register Notice
3. 40-year Evaluation
4. Amendment No. 2 to Indemnity  
Agreement B-72

cc w/encl:  
See next page



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

December 14, 1989

Docket No. 50-416

Mr. W. T. Cottle  
Vice President, Nuclear Operations  
System Energy Resources, Inc.  
Post Office Box 469  
Port Gibson, Mississippi 39150

Dear Mr. Cottle:

SUBJECT: ISSUANCE OF AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE  
NO. NPF-29 - GRAND GULF NUCLEAR STATION, UNIT 1, REGARDING  
TRANSFER OF OPERATIONS TO ENTERGY OPERATIONS, INC.  
(TAC NO. 74351)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 65 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1 (the facility). This amendment consists of changes to the facility operating license and Technical Specifications (TS) in response to your application dated August 15, 1989, as supplemented by letters dated August 22, September 22, and September 27 (two letters), October 27, October 30, November 21, and November 30, 1989.

The amendment implements the authorization to transfer control and performance of licensed activities from System Energy Resources, Inc. (SERI) to Entergy Operations, Inc. (EOI). SERI remains 90% owner and leaseholder of the facility and South Mississippi Electric Power Association (SMEPA) continues to own the remaining 10% of the facility. SERI and SMEPA have designated EOI as their agent in licensing matters. Mississippi Power & Light Company (MP&L) remains as a licensee subject to the completion of an antitrust review which will address whether MP&L should be removed from the license as requested by a previous application dated September 2, 1986.

We have reviewed the operating agreement between EOI and SERI and have determined that it is not appropriate to require it by a license condition. SERI and EOI are joint licensees under the facility operating license conditions, each responsible for specific areas and jointly responsible for regulatory compliance and response. The operating agreement between SERI and EOI will not circumvent these responsibilities. Finally, since EOI is to be the operator of the plant, we intend to communicate on most matters with EOI. However, EOI is expected to communicate with SERI to provide data, information, and other support, as may be necessary, to enable SERI and SMEPA to satisfactorily discharge any responsibilities they may have as owners of the plant, with regard to maintaining regulatory compliance.

8912280170 891214  
PDR ADDCK 05000416  
P PNU

DF01  
11

CP-1

Mr. W. T. Cottle

-2-

A copy of the Safety Evaluation supporting the amendment and the transfer of control of licensed activities from SERI to EOI is also enclosed. The Notice of Issuance will be included in the Commission's bi-weekly Federal Register notice.

Five signed copies of Amendment No. 4 to Indemnity Agreement No. B-72 which covers the activities authorized under License No. NPF-29 are also enclosed. Please sign all copies and return one copy to this office.

Sincerely,

Original Signed By:

Dennis M. Crutchfield, Associate Director  
for Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 65 to NPF-29
- 2. Safety Evaluation
- 3. Amendment No. 4 to Indemnity Agreement No. B-72

cc w/enclosures:  
See next page

5	:LA:PD21:DRPR:PM:PD21:DRPR:OGC	:ADSP	D:PD21:DRPR	:	:	:
	: <i>[Signature]</i>	: <i>[Signature]</i>	: <i>[Signature]</i>	:	:	:
EJ	:PAnderson	:L:Kintner:sw:	V:Vinitz	:DCrutchfield:	E:Adensam	:
E	:11/28/89	:11/28/89	:12/13/89	:12/14/89	:12/14/89	:

August 26, 2002

Mr. William A. Eaton  
Vice President, Operations GGNS  
Entergy Operations, Inc.  
P. O. Box 756  
Port Gibson, MS 39150

SUBJECT: GRAND GULF NUCLEAR STATION, UNIT 1 - ISSUANCE OF AMENDMENT  
RE: MISSED SURVEILLANCES (TAC NO. MB5470)

Dear Mr. Eaton:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 152 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1. This amendment revises the Technical Specifications in response to your application dated June 12, 2002.

The amendment revises Surveillance Requirement (SR) 3.0.3 to extend the delay period, before entering a Limiting Condition for Operation, following a missed surveillance. The delay period is extended from the current limit of "... up to 24 hours or up to the limit of the specified Frequency, whichever is less" to "...up to 24 hours or up to the limit of the specified Frequency, whichever is greater." In addition, the following requirement is added to SR 3.0.3: "A risk evaluation shall be performed for any Surveillance delayed greater than 24 hours and the risk impact shall be managed."

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

*/RA/*

David H. Jaffe, Sr. Project Manager, Section 1  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosures: 1. Amendment No. 152 to NPF-29  
2. Safety Evaluation

cc w/encls: See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 152

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO. 50-416

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

3.0-4

-

Insert

3.0-4

3.0-4a

May 17. '996

Mr. C. Randy Hutchins  
Vice President, Operations GGNS  
Entergy Operations, Inc.  
P. O. Box 756  
Port Gibson, MS 39150

SUBJECT: CORRECTION TO AMENDMENT NO. 120 TO FACILITY OPERATING LICENSE  
NO. NPF-29 - GRAND GULF NUCLEAR STATION, UNIT 1 (TAC NO. M88101)

Dear Mr. Hutchinson:

On February 21, 1995, the Nuclear Regulatory Commission issued Amendment No. 120 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1. This amendment replaced entirely, the then current Technical Specifications (TSs) for Unit 1 with new TSs based on NUREG-1434, "Improved BWR-6 Technical Specifications," dated September 1992.

Your staff has identified a typographical error in the new TSs. The error is in the number of channels required to be operable for the main steam tunnel ambient temperature trip system, tripping on high temperature, in Table 3.3.6.1-1, Primary Containment and Drywell Isolation Instrumentation, Page 3.3-54. The correct number of channels per trip system is 2, as was stated in the TSs for this instrumentation before Amendment No. 120 was issued; however, the number stated in the new TSs is 8. The correct number, 2, is consistent with the discussion on this instrumentation in the Bases of the new TSs, on Page B 3.3-144.

In the safety evaluation enclosed with its February 21, 1995, letter, the NRC staff addressed the proposed changes to the then current TS requirements. The number of required channels was not addressed in Section 3.3, Instrumentation of the safety evaluation because the intent of Amendment No. 120 was not to change the required number of channels. Based on discussions with your staff, the typographical error occurred when the incorrect number, 8, was inadvertently taken from NUREG-1434 in the typing of the pages for the new TSs. The typographical error was not identified by the NRC staff before the new TSs were issued. A revised TS Page 3.3-54 is being issued with the correct number of required channels, 2, and is enclosed with this letter.

Sincerely,  
*Jack N. Donohew*  
Jack N. Donohew, Senior Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: TS Page 3.3-54

cc w/encl: See next page

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		LHurley, RIV	EAdensam(EGAI)

Document Name: GG88101.COR

SEE PREVIOUS CONCURRENCE\*

DFC	LA/PD4-1	PM/PD4-1	OGC	PD/PD4-1
NAME	PNoonan	JDonohew/vw	EHotter*	WBeckner
DATE	5/13/96	5/17/96	5/9/96	5/17/96
COPY	YES/NO	YES/NO	YES/NO	YES/NO

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20535-0001

January 19, 2000

Mr. William A. Eaton  
Vice President, Operations GGNS  
Entergy Operations, Inc.  
P. O. Box 756  
Port Gibson, MS 39150

**SUBJECT: GRAND GULF NUCLEAR STATION, UNIT 1 - ISSUANCE OF AMENDMENT  
RE: REACTOR CORE STABILITY ENHANCED OPTION 1-A  
(TAC NO. MA3406)**

Dear Mr. Eaton:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. <sup>141</sup> to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station (GGNS), Unit 1. This amendment revises the Technical Specifications (TSs) in response to your application dated July 20, 1998, as supplemented by letter dated June 29, 1999.

The amendment incorporates the TS changes necessary for implementation of the Boiling Water Reactor Owners' Group Reactor Stability Long-Term Solution, Enhanced Option 1-A.

You are requested to implement this amendment within 120 days of the amendment date, and to inform the staff when these operational changes, including associated component modifications, have been completed.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in cursive script that reads "S. Patrick Sekerak".

S. Patrick Sekerak, Project Manager, Section 1  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosures: 1. Amendment No. <sup>141</sup> to NPF-29  
2. Safety Evaluation

cc w/encls: See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 141

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO. 50-416

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
1.0-3	1.0-3
---	1.0-3a
---	3.2-4
---	3.2-5
---	3.2-6
3.3-4	3.3-4
---	3.3-4a
3.3-5	3.3-5
---	3.3-5a
3.3-6	3.3-6
---	3.3-13a
---	3.3-13b
3.4-1	3.4-1
3.4-2	3.4-2
3.4-3	3.4-3
3.4-4	3.4-4
3.4-5	---
5.0-18	5.0-18
5.0-20	5.0-20



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

June 18, 1999

Mr. William A. Eaton  
Vice President, Operations GGNS  
Entergy Operations, Inc.  
P. O. Box 756  
Port Gibson, MS 39150

SUBJECT: GRAND GULF NUCLEAR STATION, UNIT 1 - ISSUANCE OF CORRECTED  
PAGES OF TECHNICAL SPECIFICATIONS FOR AMENDMENT NO. 138  
RE: REFUELING EQUIPMENT INTERLOCKS (TAC NO. M95490)

Dear Mr. Eaton:

On May 7, 1999, the Nuclear Regulatory Commission issued Amendment No. 138 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station (GGNS), Unit 1. This amendment revised the Technical Specifications (TSs) in response to your application dated January 12, 1999, that superseded amendment request submittal dated May 31, 1996. The amendment adds an additional required action to the Limiting Condition for Operation (LCO) 3.9.1, "Refueling Equipment Interlocks," of the GGNS TSs.

The revised TS pages issued as an attachment to Amendment No. 138 were inadvertently typed with a different font from that used in the GGNS TSs. This resulted in a slight difference in page format between the revised TS pages and the original GGNS TSs. In addition, the final safety analyses report (FSAR) reference to the Refueling Interlocks System included on TS Bases page B 3.9-4, as reference 2, inappropriately listed a section number (7.7.1.5) from the River Bend FSAR, since River Bend was the lead facility on this issue. These discrepancies have been corrected in the enclosed TS pages.

The corrected TS pages and TS Bases pages enclosed with this letter are being issued as replacement pages for the TS pages previously issued with Amendment No. 138. The corrected pages affect only the format of the TS, and do not change the technical content or meaning of the TS pages associated with Amendment No. 138. The staff's evaluation and conclusions contained in its Safety Evaluation dated May 7, 1999, are not affected by these corrections.

Please replace the TS pages originally issued with Amendment No.138 with the TS pages enclosed with this letter.

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We apologize for any inconvenience this may have caused you.

Sincerely,

A handwritten signature in black ink that reads "S. Patrick Sekerak". The signature is written in a cursive style with a large initial "S".

S. Patrick Sekerak, Project Manager, Section 1  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-416

Enclosure: TS 3.9.1 replacement pages

cc w/encl: See next page



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

May 18, 1992

Docket No. 50-416

Mr. William T. Cottle  
Vice President, Operations GGNS  
Entergy Operations, Inc.  
Post Office Box 756  
Port Gibson, Mississippi 39150

Dear Mr. Cottle:

SUBJECT: ISSUANCE OF AMENDMENT NO. 96 TO FACILITY OPERATING LICENSE  
NO. NPF-29 - GRAND GULF NUCLEAR STATION, UNIT 1 (TAC NO. M82748)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 96 to Facility Operating License No. NPF-29 for the Grand Gulf Nuclear Station, Unit 1. This amendment revises the Environmental Protection Plan (EPP) in response to your application dated February 7, 1992.

The amendment requests the termination of the Cooling Tower Drift Program and changes references to the program to reflect the program's termination.

A copy of our related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Paul W. O'Connor".

Paul W. O'Connor, Senior Project Manager  
Project Directorate IV-1  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 96 to NPF-29
2. Safety Evaluation

cc w/enclosures:  
See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 96

FACILITY OPERATING LICENSE NO. NPF-29

DOCKET NO. 50-416

Replace the following pages of the Appendix B Environmental Protection Plan with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

REMOVE PAGES

2-1

4-2

4-3

INSERT PAGES

2-1

4-2

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