April 6, 2004

Mr. Michael R. Kansler, President Entergy Nuclear Operations, Inc. 440 Hamilton Avenue White Plains, NY 10601

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING STEAM

GENERATOR TUBE INSPECTION REPORTS FOR THE 2003 OUTAGE, INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 (TAC NO. MC1912)

Dear Mr. Kansler:

In letter dated April 25, August 19, and December 8, 2003, Entergy Nuclear Operations, Inc. (ENO) submitted reports summarizing the steam generator (SG) tube inspections performed at Indian Point Nuclear Generating Unit No. 3 (IP3) during refueling outage 12 in April 2003.

The Nuclear Regulatory Commission staff is reviewing the information provided in these submittals and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). During a telephone call on March 26, 2004, the ENO staff indicated that a response to the RAI would be provided within 60 days.

If you should have any questions, please do not hesitate to call me.

Sincerely,

/RA/

Patrick D. Milano, Sr. Project Manager, Section 1
Project Directorate 1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-286

Enclosure: RAI

cc w/encl: See next page

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Indian Point Nuclear Generating Unit No. 3

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Indian Point Nuclear Generating Unit No. 3

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Indian Point Nuclear Generating Unit No. 3

CC:

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REQUEST FOR ADDITIONAL INFORMATION

REGARDING STEAM GENERATOR TUBE INSPECTIONS

ENTERGY NUCLEAR OPERATIONS, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3 (IP3)

DOCKET NO. 50-286

In letters dated April 25 (ADAMS Accession No. ML031200250), August 19 (ML032330262), and December 8, 2003 (ML033450339), Entergy Nuclear Operations summarized the steam generator (SG) tube inservice inspection (ISI) performed at IP3 during refueling outage (RFO) 12. The Nuclear Regulatory Commission (NRC) staff has the following questions regarding the information provided in these reports:

- 1. The SGs at IP3 were replaced in 1989 with Westinghouse Model No. 44F SGs. In several locations, the reports reference tube support structures (e.g., 1C) and tube locations (e.g., Row 29, Column 17). In order for the NRC staff to better understand the location of the indications, provide a sketch of the IP3 SGs that depicts the tube and support naming conventions. In addition, provide the following general design information: (a) tube manufacturer and (b) tube support (including anti-vibration bar) thickness. Discuss reference datum for the measurements from a tube support such that it is clear whether the measurements are from the middle of the support or the edge of the support (e.g., does AV2 minus 0.6 inch specify an indication 0.6 inch from the bottom edge of the second anti vibration bar?).
- 2. Provide the completion date for RFO 11 and state whether any SG tube inspections were performed during that RFO. The staff notes that the August 19 report implies that inspections were performed (i.e., "...wear scars were not identified during Refueling Outage 11 because sludge lancing occurred after the ISI."). However, the December 8 report indicates that no eddy current inspections were performed.
- 3. Provide the completion date for your SG tube inspections in 1999.
- 4. Three tubes were reported as having restrictions such that a 0.700-inch bobbin probe could not pass through the U-bend region. Discuss: (a) the largest size probe that passed through these tubes during RFO 12, (b) the largest size probe that ever passed through these tubes, and (c) the source of these restrictions. Also, provide the bend radii of the first three rows of tubes.
- 5. All hot leg dents in the straight section of the tubes were inspected with a rotating probe if the dent voltage was greater than 5 volts. Discuss whether the calibration procedure (for measuring dent size) is consistent with the guidance described in Generic Letter (GL) 95-05, "Voltage-Based Repair Criteria for Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking," or is consistent with industry guidelines. Also, discuss whether the dents and dings found during the RFO 12

inspection were traceable back to the baseline inspection and discuss any changes in magnitude. If the dents or dings are not traceable to your baseline inspection and/or have changed in magnitude, discuss the reason for any change.

- 6. One tube was classified as having a "trackable anomaly." Define this classification.
- 7. Clarify whether the tubes with possible loose parts signals were visually inspected to confirm the nature of any potential loose parts. Provide the location of these indications. If visual inspections were not performed and/or the part was not removed, discuss the analyses performed to ensure these potential parts do not compromise tube integrity for the period of time between inspections.
- 8. In 1997, a free span bobbin indication was identified in one tube in SG 34 (Row 8, Column 21). During that outage, a rotating probe inspection was performed at this location which revealed the presence of a small ding. Discuss whether any indications were identified in this region of the tube during your 2003 outage, and discuss whether the bobbin signal has continued to change.
- 9. During the 1997 outage, possible indications of erosion-corrosion were identified in two J-tube joints (refer to page 11 of 31 of the December 19, 1997, report). The report further indicated that these J-tubes would be reinspected at a later outage to determine if the erosion-corrosion is progressing. Have these additional inspections been performed? If so, describe the results of these inspections.
- 10. The staff's review of the 2003 inspection results included referring back to the previous inspection results which were provided by letter dated December 17, 1999. In reviewing this document (ML003670350), it appears that portions of Attachment 2 were not incorporated into the NRC's document management system. Provide a copy of Attachment 2 to the December 17, 1999, letter.