

March 31, 2004

LICENSEE: Constellation Energy Group (Constellation)

FACILITY: Nine Mile Point Nuclear Station, Units 1 and 2

SUBJECT: SUMMARY OF MEETING TO DISCUSS THE PLANNED LICENSE RENEWAL APPLICATION FOR NINE MILE POINT, UNITS 1 AND 2

On February 25, 2004, representatives of Constellation met with the Nuclear Regulatory Commission (NRC) staff to discuss the submittal of a license renewal application for Nine Mile Point (NMP), Units 1 and 2. The renewal application is scheduled to be submitted by May 31, 2004. Attendees at the meeting are listed in Enclosure 1 and the agenda is provided in Enclosure 2. Constellation's presentation materials used at the meeting are contained in Enclosure 3.

Constellation provided an overview of the intended format and content for each of the sections in the license renewal application. The license for NMP Unit 1 expires on August 22, 2009, and for Unit 2 on October 31, 2026. A May 31, 2004, submittal, will meet the five year timely renewal provision contained in 10 CFR 2.109.

The NMP renewal application will be the first application for a GE BWR 2/Mark I design (Unit 1) and a GE BWR 5/Mark II design (Unit 2). One document will be submitted addressing both units. Designators "A" and "B" will be used for subsections where significant differences exist between Units 1 and 2. The application will follow the "Class of 2003" standard license renewal application format and will address both draft and final Interim Staff Guidance positions.

There are a total of 37 aging management programs for NMP Units 1 and 2 consisting of 32 existing and 5 new programs. Approximately 75% of the programs are consistent with the Generic Aging Lessons Learned Report. Constellation had not decided yet whether to spend the resources to perform the review to identify where its programs are consistent with those previously approved (past precedent). They will monitor the experience of other applicants who performed the review and currently have applications under review before making a decision.

The NRC staff requested clarification on selected topics and provided comments but did not identify any significant problems with Constellation's proposed approach.

/RA/

Stephen T. Hoffman, Senior Project Manager
License Renewal Section B
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Dockets Nos.: 50-220 and 50-410

Enclosures: As stated

cc w/encls: See next page

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NAME	YEdmonds <i>YE</i>	SHoffman <i>SH</i>	SWest <i>SW</i>
DATE	3/30/04	3/31/04	3/31/04

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RLEP RF

Stephen Hoffman

E-MAIL:

PUBLIC

J. Craig

D. Matthews

F. Gillespie

C. Grimes

RidsNrrDe

E. Imbro

G. Bagchi

K. Manoly

W. Bateman

J. Calvo

R. Jenkins

P. Shemanski

J. Fair

S. Black

B. Boger

D. Thatcher

R. Pettis

G. Galletti

C. Li

M. Itzkowitz

R. Weisman

M. Mayfield

A. Murphy

S. Smith (srs3)

S. Duraiswamy

Y. L. (Renee) Li

RLEP Staff

C. Holden

R. Laufer

P. Tam

B. Fuller, RI

E. Knutson, RI

J. Trapp, RI

T. Mensah

Receptionist

OPA



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

March 31, 2004

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FACILITY: Nine Mile Point Nuclear Station, Units 1 and 2
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A handwritten signature in cursive script that reads "Stephen T. Hoffman".

Stephen T. Hoffman, Senior Project Manager
License Renewal Section B
License Renewal and Environmental Impacts Program
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Dockets Nos.: 50-220 and 50-410

Enclosures: As stated

cc w/encls: See next page

Nine Mile Point Nuclear Station, Unit Nos. 1 and 2

cc:

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 126
Lycoming, NY 13093

Supervisor
Town of Scriba
Route 8, Box 382
Oswego, NY 13126

Mr. James R. Evans
LIPA
P.O. Box 129
Lycoming, NY 10393

Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, NY 10271

Mr. Paul D. Eddy
Electric Division
NYS Department of Public Service
Agency Building 3
Empire State Plaza
Albany, NY 12223

C. Adrienne Rhodes
Chairman and Executive Director
State Consumer Protection Board
5 Empire State Plaza, Suite 2101
Albany, NY 12223-1556

Mark J. Wetterhahn, Esquire
Winston & Strawn
1400 L Street, NW
Washington, DC 20005-3502

Mr. Michael J. Wallace
President
Nine Mile Point Nuclear Station, LLC
c/o Constellation Energy Group, Inc.
750 East Pratt Street
Baltimore, MD 21202

Mr. James M. Petro, Jr., Esquire
Counsel
Constellation Energy Group
750 East Pratt Street, 5th Floor
Baltimore, MD 21202

Mr. Peter R. Smith, Acting President
New York State Energy, Research, and
Development Authority
17 Columbia Circle
Albany, NY 12203-6399

Mr. Fred Emerson
Nuclear Energy Institute
1776 I St., NW, Suite 400
Washington, DC 20006-3708

M. Steven Leonard, Acting
General Supervisor - Regulatory Matters
Nine Mile Point Nuclear Station, LLC
P.O. Box 63
Lycoming, NY 13093

Peter A. Mazzaferro
Site Project Manager - License Renewal
Nine Mile Point Nuclear Station, LLC
P.O. Box 63
Lycoming, NY 13093

Mr. Peter E. Katz
Vice President Nine Mile Point
Nine Mile Point Nuclear Station, LLC
P.O. Box 63
Lycoming, NY 13093

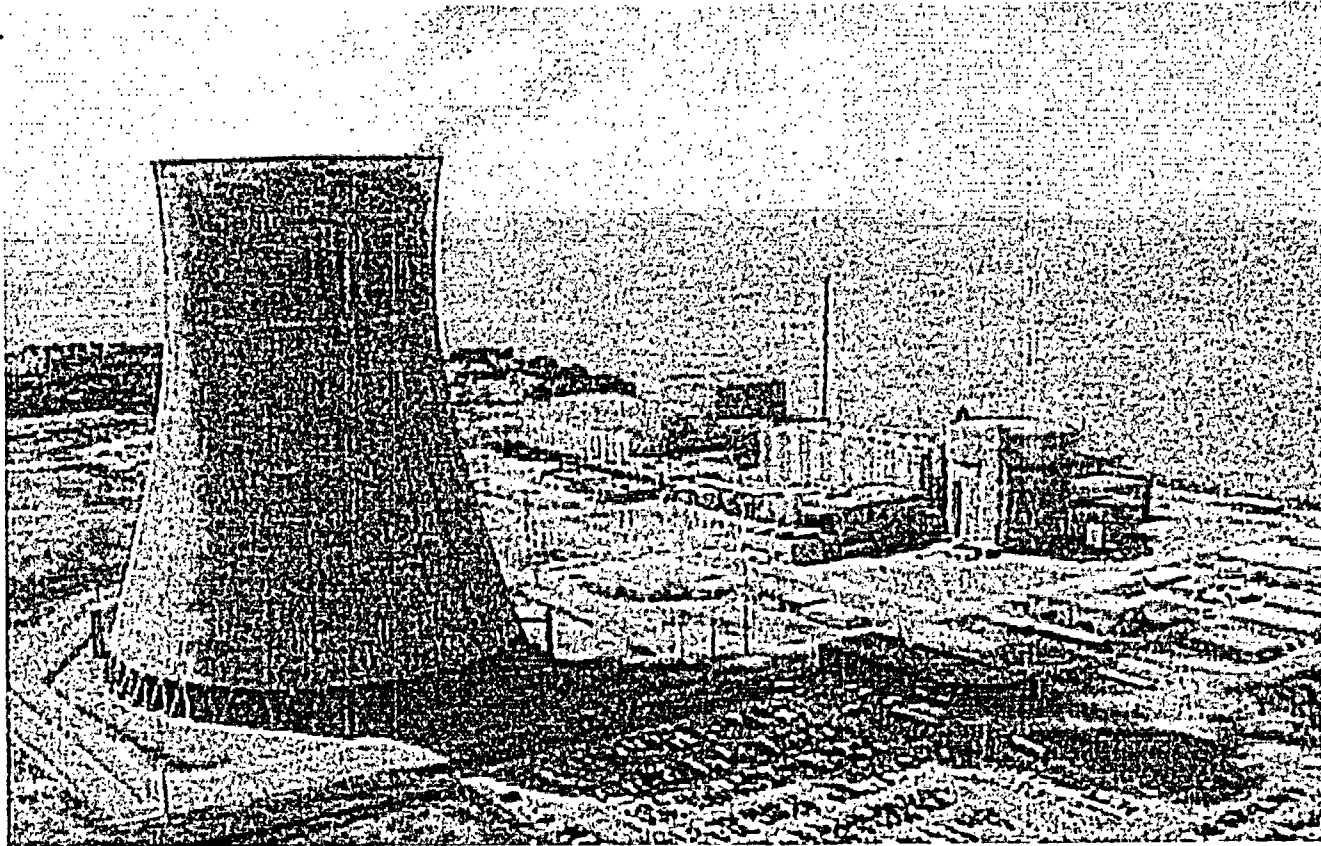
ATTENDANCE LIST
NRC STAFF MEETING WITH CONSTELLATION ENERGY GROUP
NINE MILE POINT, UNITS 1 AND 2
FEBRUARY 25, 2004

<u>NAME</u>	<u>ORGANIZATION</u>
Steve Hoffman	NRC/NRR/DRIP/RLEP
David Dellario	NMPNS/CNS
Ernie Taormina	NMPNS/CNS
Tony Greci	CNS
Russ Wells	CNS
Peter Mazzaferro	Constellation/NMP
TJ Kim	NRC/NRR/DRIP/RLEP
Dennis Vandeputte	NMPNS
Jenny Weil	McGraw-Hill
Greg Makar	NRC/NRR/DE/EMCB
Al Fulvio	Exelon
Peter Tam	NRC/NRR/DLPM
David C. Jeng	NRC/NRR/DE/EMEB
Barry Elliot	NRC/NRR/DE/EMCB
Angelo Stubbs	NRC/NRR/DSSA/SPLB
Duc Nguyen	NRC/NRR/DE/EEIB
Leslie Fields	NRC/NRR/DRIP/RLEP
Greg Cranston	NRC/NRR/DRIP/RLEP
Stewart Bailey	NRC/NRR/DE/EMEB
Sam Lee	NRC/NRR/DRIP/RLEP
Kimberley Corp	NRC/NRR/DRIP/RLEP
Raj Auluck	NRC/NRR/DRIP/RLEP
Mike Masnick	NRC/NRR/DRIP/RLEP
P.T. Kuo	NRC/NRR/DRIP/RLEP

AGENDA
NRC MEETING WITH CONSTELLATION ENERGY GROUP
CONCERNING THE LICENSE RENEWAL APPLICATION FOR
NINE MILE POINT (NMP), UNITS 1 AND 2
FEBRUARY 25, 2004
(Times are approximate)

<u>Topic</u>	<u>Constellation Presenter</u>	<u>Time</u>
1. Opening remarks and introduction	Bill Holston	9:00 - 9:10 a.m.
2. NMP License renewal organization	David Dellario	9:10 - 9:20 a.m.
3. Description and background of NMP1 & NMP2	Rus Wells	9:20 - 9:35 a.m.
4. Outline and format of NMP renewal application	Rus Wells	9:35 - 9:50 a.m.
5. Programs overview	Pete Mazzaferro	9:50 - 10:00 a.m.
6. Questions and answers		10:00 - 10:45 a.m.
7. Public participation		10:45 - 10:55 a.m.
8. Closing		10:55 - 11:00 a.m.

Nine Mile Point Units 1 and 2 License Renewal Application



2/25/2004

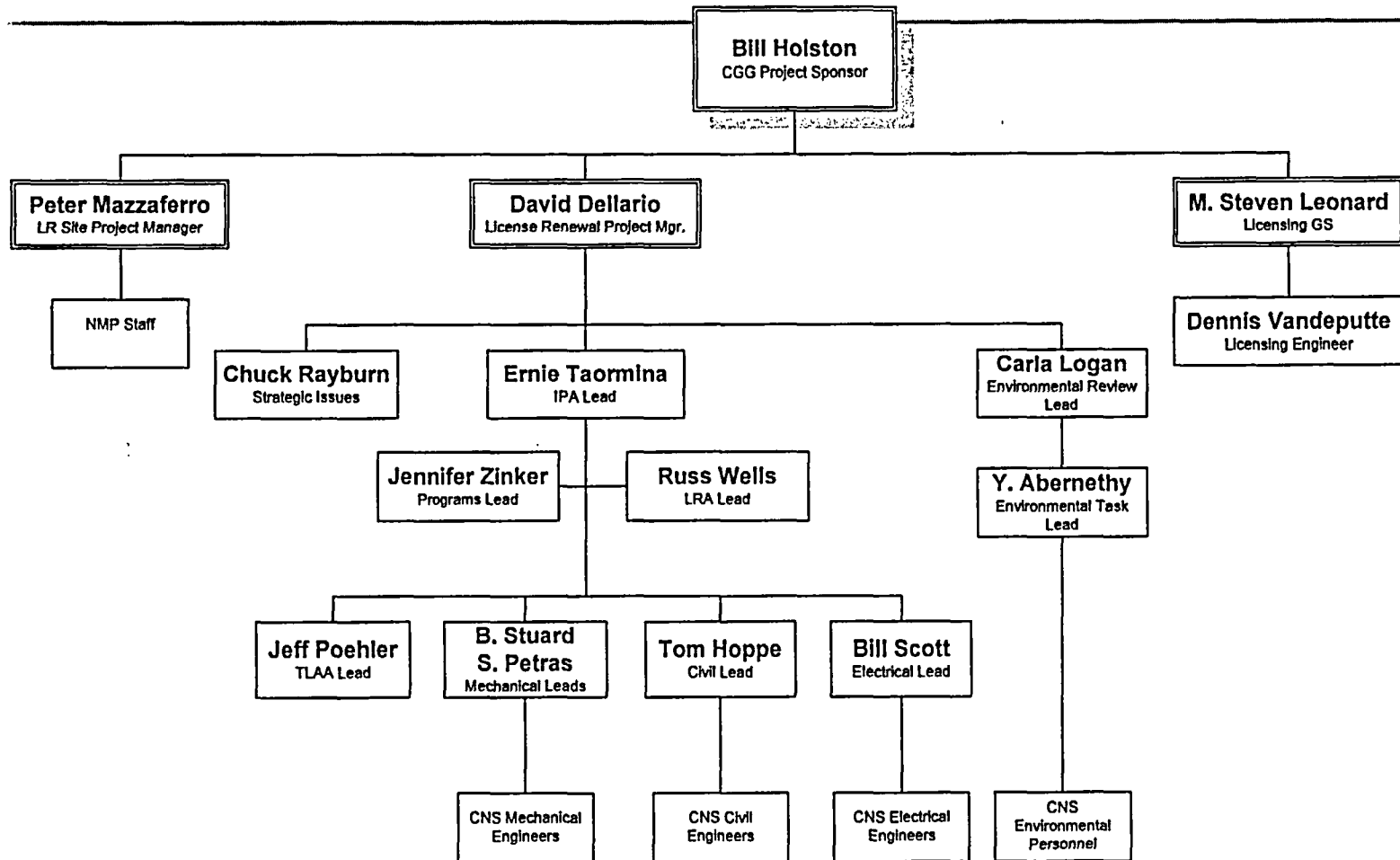
Constellation Attendees

<u>NAME</u>	<u>TITLE</u>
David Dellario	CNS LR Project Manager
Tony Grenci	CNS Director License Renewal
Pete Mazzaferro	NMP LR Project Manager
Ernie Taormina	CNS IPA Lead
Dennis Vandeputte	NMP Licensing Engineer
Russ Wells	CNS Lead Licensing Engineer

Agenda

- Opening Remarks/Introduction D. Dellario
- NMP License Renewal Organization D. Dellario
- Description and Background of NMP 1 & 2 P. Mazzaferro
- Outline and Format of NMP LRA R. Wells
- Programs Overview P. Mazzaferro
- Questions and Answers
- Public Participation

NMP License Renewal Organization



Description and Background of NMP 1 & 2

- The Nine Mile Point Nuclear Station (NMPNS) consists of two nuclear plants, on the shore of Lake Ontario, approximately 5 miles northeast of Oswego, NY.
- Constellation Energy Group purchased the NMPNS on November 7, 2001.
- Long Island Power Authority owns 18% of NMP2.

Description and Background of NMP 1 & 2

- NMP1 is a BWR/2 design (1850 MWt) with a Mark I containment.
- NMP2 is a BWR/5 design (3467 MWt) with a Mark II containment.

Description and Background of NMP 1 & 2

- NMP1 OL expires August 22, 2009.
- NMP2 OL expires October 31, 2026.
- LRA to be submitted by May 31, 2004.
- NMP2 received an exemption from 10 CFR 54.17(c) in order to submit prior to 20 years of operating experience.

Outline and Format of NMP LRA

- The Standard LRA (SLRA) has been incorporated into NEI 95-10, Rev. 4.
- The LRA for NMPNS follows the format and content of the SLRA.
- The LRA addresses the NRC Interim Staff Guidance (ISG) documents.

LRA TABLE OF CONTENTS

- Section 1 - Administrative Information
- Section 2 – Scoping and Screening Methodology for Identifying Structures and Components Subject to Aging Management Review, and Implementation Results
- Section 3 – Aging Management Review Results
- Section 4 – Time-Limited Aging Analyses
- Appendix A – Updated Final Safety Analysis Report Supplements
- Appendix B – Aging Management Programs
- Appendix C – Commodity Groups (Optional) (not used)
- Appendix D – Technical Specification Changes
- Appendix E – Environmental Report

LRA Section 1 Contents

Contains general information such as:

- Name and address of NMPNS, LLC
- Expiration dates of OLs
- Description of CEG and NMPNS, LLC
- Organization and Management of NMPNS, LLC
- Application Structure
- Description of NMP1 and NMP2

LRA Section 2 Contents

- 2.1, Scoping and Screening Methodology
 - 2.1.1, Introduction
 - 2.1.2, Plant Level Scoping
 - 2.1.3, System Realignment and Commodity Groupings
 - 2.1.4, Application of License Renewal Scoping Criterion
 - 2.1.5, Component Screening
 - 2.1.6, ISG Discussion
 - 2.1.7, References
- 2.2, Plant Level Scoping Results
 - Lists all of the plant systems and structures and identifies whether they are in scope or not.

LRA Section 2 Contents (cont'd)

- 2.3, Scoping and Screening Results: Mechanical Systems
 - Section 2.3.1.A, NMP1 Reactor Vessel, Internals, and Reactor Coolant Systems
 - Section 2.3.1.B, NMP2 Reactor Vessel, Internals, and Reactor Coolant Systems
 - Section 2.3.2.A, NMP1 Engineered Safety Features Systems
 - Section 2.3.2.B, NMP2 Engineered Safety Features Systems
 - Section 2.3.3.A, NMP1 Auxiliary Systems
 - Section 2.3.3.B, NMP2 Auxiliary Systems
 - Section 2.3.4.A, NMP1 Steam and Power Conversion Systems
 - Section 2.3.4.B, NMP2 Steam and Power Conversion Systems

LRA Section 2 Contents (cont'd)

- 2.4, Scoping and Screening Results: Structures
 - 2.4.A, NMP1 Structures
 - 2.4.B, NMP2 Structures
 - 2.4.C, NMPNS Structural Commodities
 - Section 2.4.C.1, Component Supports
 - Section 2.4.C.2, Fire Stops and Seals

LRA Section 2 Contents (cont'd)

- 2.5, Scoping and Screening Results: Electrical and Instrumentation and Controls
 - 2.5.A, NMP1 Electrical and I&C Systems
 - 2.5.B, NMP2 Electrical and I&C Systems
 - 2.5.C, NMPNS Electrical Commodities
 - Section 2.5.C.1, Cables and Connectors
 - Section 2.5.C.2, Bus Bars
 - Section 2.5.C.3, Containment Electrical Penetrations
 - Section 2.5.C.4, Switchyard Components

LRA Section 3 Contents

- Section 3.0, Aging Management Review Results
 - Table 3.0-1, Environments
 - Table 3.0-2, Aging Effects Requiring Management
- Section 3.1, Aging Management of Reactor Coolant Systems
 - Separate “Table 1s” for NMP1 and NMP2 (e.g., Table 3.1.1.A for NMP1 Reactor Coolant Systems and Table 3.1.1.B for NMP2 Reactor Coolant Systems)
 - Each system subject to an AMR has its own “Table 2” which is the SLRA nine-column format (e.g., Table 3.1.2.A-1 NMP1 Reactor Pressure Vessel and Table 3.1.2.B-1 NMP2 Reactor Pressure Vessel)

LRA Section 3 Contents (cont'd)

- Section 3.2, Aging Management of Engineered Safety Features Systems
- Section 3.3, Aging Management of Auxiliary Systems
- Section 3.4, Aging Management of Steam and Power Conversion Systems
- Section 3.5, Aging Management of Structures
- Section 3.6, Aging Management of Electrical and Instrumentation and Controls

LRA Section 4 Contents

- Section 4.1, Identification of Time Limited Aging Analyses
 - 4.1.1, Process Overview
 - 4.1.2, Identification Of Exemptions
- Section 4.2, Reactor Vessel Neutron Embrittlement
 - 4.2.1, Upper Shelf Energy
 - 4.2.2, Pressure-Temperature (P-T) Limits
 - 4.2.3, Elimination of Circumferential Weld Inspection
 - 4.2.4, Reactor Vessel Axial Weld Failure Probability

LRA Section 4 Contents (cont'd)

- Section 4.3, Metal Fatigue Analysis
 - 4.3.1, Reactor Vessel Fatigue Analysis
 - 4.3.2, ASME Section III Class I Piping and Components Fatigue Analysis (NMP2 only)
 - 4.3.3, Feedwater Nozzle Fatigue and Control Rod Drive Return Line (CRDL) Nozzle Cracking
 - 4.3.4, Non-ASME Section III Class 1 Piping and Components Fatigue Analysis
 - 4.3.5, Reactor Vessel Internals Fatigue Analysis
 - 4.3.6, Environmentally Assisted Fatigue Analysis
 - 4.3.7, Fatigue of the Emergency Condenser (NMP1 only)

LRA Section 4 Contents (cont'd)

- Section 4.4, Environmental Qualification (EQ) of Electrical Equipment
 - 4.4.1, Electrical Equipment EQ Analysis
 - 4.4.2, Mechanical Environmental Equipment Qualification (NMP2 only)
- Section 4.5, Loss of Prestress in Concrete Containment Tendons - N/A (the NMP1 and NMP2 containments do not have prestress tendons)

LRA Section 4 Contents (cont'd)

- Section 4.6, Containment Liner Plate, Metal Containment and Penetrations Fatigue Analysis
 - 4.6.1, NMP1 Torus Shell and Vent System Fatigue
 - 4.6.2, NMP1 Torus Attached Piping Analysis
 - 4.6.3, NMP1 Torus Thickness
 - 4.6.4, NMP2 Containment Liner Analysis
- Section 4.7, Other Plant Specific TLAAs
 - 4.7.1, NMP2 RPV Biological Shield Wall
 - 4.7.2, NMP2 Main Steam Isolation Valve Corrosion Allowance
 - 4.7.3, NMP2 Core Plate Holdown Bolts
 - 4.7.4, NMP1 Reactor Vessel and Reactor Vessel Closure Head Weld Flaw Evaluations

Appendix A - Updated (Final) Safety Analysis Report Supplement

- Appendix A1, NMP1 Updated Final Safety Analysis Report (UFSAR) Supplement
- Appendix A2, NMP2 Updated Safety Analysis Report (USAR) Supplement
- Both Appendices contain:
 - A summarized description of the programs and activities for managing the effects of aging
 - A summary of the TLAAs

Appendix B – Aging Management Programs

- Provides a list of Aging Management Programs (AMPs) alphabetically identifying whether the program is a new or existing program.
- Includes a table comparing the NMP AMPs to the GALL.
- The information in this appendix applies to both NMP1 and NMP2, unless otherwise specified.

Appendix D – Technical Specification Changes

- No new Technical Specifications (Tech. Specs.) are required.
- Tech. Spec. changes will be needed for the Pressure-Temperature (P-T) curves and NMP1 Tech. Spec. 5.3, Reactor Vessel (design lifetime of 40 years).
- These changes to the Tech. Specs. will be submitted under the normal license amendment process.

Appendix E – Environmental Report

- Purpose and need for action
- Description of site and environmental interfaces
- Proposed action
- Impact analyses for applicable Category 2 issues (including Severe Accident Mitigation Alternatives)
- New & significant information review process
 - Analysis of new and significant issues not in GEIS
 - Analysis of new and significant information relative to Category 1 issues
- Comparative impact analysis of alternatives
- Compliance status

Programs Overview

- Total of 37 programs
 - 32 existing
 - 2 plant specific programs
 - 5 new
 - 2 plant specific programs

- ~75% of the programs are consistent with NUREG-1801

NUREG-1801 Programs

- 10 CFR 50 Appendix J Program
- ASME Section XI Inservice Inspection (Subsection IWE) Program
- ASME Section XI Inservice Inspection (Subsection IWF) Program
- ASME Section XI Inservice Inspection (Subsection IWL) Program (NMP2 only)
- ASME Section XI Inservice Inspection (Subsections IWB, IWC, IWD) Program
- Boraflex Monitoring Program
- Buried Piping and Tanks Inspection Program (new)
- BWR Feedwater Nozzle Program
- BWR Penetrations Program
- BWR Reactor Water Cleanup System Program

NUREG-1801 Programs

- BWR Stress Corrosion Cracking Program
- BWR Vessel ID Attachment Welds Program
- BWR Vessel Internals Program
- Closed Cycle Cooling Water System Program
- Compressed Air Monitoring Program
- Environmental Qualification Program
- Fatigue Monitoring Program
- Fire Protection Program
- Fire Water System Program
- Flow Accelerated Corrosion Program
- Fuel Oil Chemistry Program
- Inspection of Overhead Heavy Load and Light Load Handling Systems Program

NUREG-1801 Programs

- Masonry Wall Program
- Non EQ Electrical Cables and Connections Program (new)
- Non EQ Electrical Cables Used in Instrumentation Circuits Program
- One Time Inspection Program (new)
- Open Cycle Cooling Water System Program
- Preventive Maintenance Program
- Reactor Head Closure Studs Program
- Reactor Vessel Surveillance Program
- Selective Leaching of Materials Program (NMP1 only)
- Structures Monitoring Program
- Water Chemistry Control Program

Plant Specific Programs

- Bus Inspection Program (new)
- Fuse Holder Program (new)
- Systems Walkdown Program
- Torus Corrosion Monitoring Program (NMP1 only)