



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

January 23, 2004

Mr. M. R. Blevins
Senior Vice President
& Principal Nuclear Officer
TXU Energy
ATTN: Regulatory Affairs
P. O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES), UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: INCREASING FLEXIBILITY IN MODE
RESTRAINTS (TAC NOS. MC0007 AND MC0008)

Dear Mr. Blevins:

The Commission has issued the enclosed Amendment No. 109 to Facility Operating License No. NPF-87 and Amendment No. 109 to Facility Operating License No. NPF-89 for CPSES, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated July 18, 2003.

The amendments revise TS requirements to adopt the provisions of Industry/Technical Specification Task Force (TSTF) change TSTF-359, "Increase Flexibility in Mode Restraints." The availability of a model application relating to the modification of requirements regarding TS mode change limitations was published in the *Federal Register* on April 4, 2003 (68 FR 16579).

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in cursive script, reading "Mohan C. Thadani".

Mohan C. Thadani, Senior Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. 109 to NPF-87
2. Amendment No. 109 to NPF-89
3. Safety Evaluation

cc w/encs: See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 109

TO FACILITY OPERATING LICENSE NO. NPF-87

AND AMENDMENT NO. 109

TO FACILITY OPERATING LICENSE NO. NPF-89

DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
3.0-2	3.0-2
3.0-5	3.0-5
3.3-35	3.3-35
3.3-40	3.3-40
3.4-23	3.4-23
3.4-28	3.4-28
3.4-40	3.4-40
3.4-44	3.4-44
3.5-8	3.5-8
3.6-22	3.6-22
3.7-10	3.7-10
3.7-12	3.7-12
3.8-2	3.8-2

3.0 LCO APPLICABILITY (continued)

- LCO 3.0.4** When an LCO is not met, entry into a MODE or other specified condition in the Applicability shall only be made:
- a. When the associated ACTIONS to be entered permit continued operation in the MODE or other specified condition in the Applicability for an unlimited period of time;
 - b. After performance of a risk assessment addressing inoperable systems and components, consideration of the results, determination of the acceptability of entering the MODE or other specified condition in the Applicability, and establishment of risk management actions, if appropriate; exceptions to this Specification are stated in the individual Specifications, or
 - c. When an allowance is stated in the individual value, parameter, or other Specification.

This Specification shall not prevent changes in MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

- LCO 3.0.5** Equipment removed from service or declared inoperable to comply with ACTIONS may be returned to service under administrative control solely to perform testing required to demonstrate its OPERABILITY or the OPERABILITY of other equipment. This is an exception to LCO 3.0.2 for the system returned to service under administrative control to perform the testing required to demonstrate OPERABILITY.
-

- LCO 3.0.6** When a supported system LCO is not met solely due to a support system LCO not being met, the Conditions and Required Actions associated with this supported system are not required to be entered. Only the support system LCO ACTIONS are required to be entered. This is an exception to LCO 3.0.2 for the supported system. In this event, an evaluation shall be performed in accordance with Specification 5.5.15, "Safety Function Determination Program (SFDP)." If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered.

When a support system's Required Action directs a supported system to be declared inoperable or directs entry into Conditions and Required Actions for a supported system, the applicable Conditions and Required Actions shall be entered in accordance with LCO 3.0.2.

(continued)

3.0 SR APPLICABILITY (continued)

SR 3.0.3

If it is discovered that a Surveillance was not performed within its specified Frequency, then compliance with the requirement to declare the LCO not met may be delayed, from the time of discovery, up to 24 hours or up to the limit of the specified Frequency, whichever is greater. This delay period is permitted to allow performance of the Surveillance.

If the Surveillance is not performed within the delay period, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered. A risk evaluation shall be performed for any Surveillance delayed greater than 24 hours and the risk impact shall be managed.

When the Surveillance is performed within the delay period and the Surveillance is not met, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.

SR 3.0.4

Entry into a MODE or other specified condition in the Applicability of an LCO shall only be made when the LCO's Surveillances have been met within their specified Frequency, except as provided by SR 3.0.3. When an LCO is not met due to Surveillances not having been met, entry into a MODE or other specified condition in the Applicability shall only be made in accordance with LCO 3.0.4.

This provision shall not prevent entry into MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

3.3 INSTRUMENTATION

3.3.3 Post Accident Monitoring (PAM) Instrumentation

LCO 3.3.3 The PAM Instrumentation for each Function in Table 3.3.3-1 shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3

ACTIONS

-----NOTE-----
Separate Condition entry is allowed for each Function.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more Functions with one required channel inoperable.	A.1 Restore required channel to OPERABLE status.	30 days
B. Required Action and associated Completion Time of Condition A not met.	B.1 Initiate action in accordance with Specification 5.6.8.	Immediately

(continued)

3.3 INSTRUMENTATION

3.3.4 Remote Shutdown System

LCO 3.3.4 The Remote Shutdown System Functions in Table 3.3.4-1 and the required hot shutdown panel (HSP) controls shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3

ACTIONS

NOTE

Separate Condition entry is allowed for each Function and required HSP control.

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. One or more required Functions inoperable.</p> <p><u>OR</u></p> <p>One or more required HSP controls inoperable.</p>	<p>A.1 Restore required Function and required HSP controls to OPERABLE status.</p>	30 days
<p>B. Required Action and associated Completion Time not met.</p>	<p>B.1 Be in MODE 3.</p> <p>AND</p> <p>B.2 Be in MODE 4.</p>	<p>6 hours</p> <p>12 hours</p>

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.11 Pressurizer Power Operated Relief Valves (PORVs)

LCO 3.4.11 Each PORV and associated block valve shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3

ACTIONS

NOTE
Separate Condition entry is allowed for each PORV.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more PORVs inoperable and capable of being manually cycled.	A.1 Close and maintain power to associated block valve.	1 hour
B. One PORV inoperable and not capable of being manually cycled.	B.1 Close associated block valve.	1 hour
	<u>AND</u>	
	B.2 Remove power from associated block valve.	1 hour
	<u>AND</u>	
	B.3 Restore PORV to OPERABLE status.	72 hours

(continued)

ACTIONS

-----NOTE-----
 LCO 3.0.4.b is not applicable when entering MODE 4.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more safety injection pumps capable of injecting into the RCS.	A.1 Initiate action to verify a maximum of zero safety injection pumps are capable of injecting into the RCS.	Immediately
B. Three charging pumps capable of injecting into the RCS.	B.1 Initiate action to verify a maximum of two charging pumps are capable of injecting into the RCS.	Immediately
C. An accumulator not isolated when the accumulator pressure is greater than or equal to the maximum RCS pressure for existing cold leg temperature allowed in the PTLR.	C.1 Isolate affected accumulator.	1 hour

(continued)

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.15 RCS Leakage Detection Instrumentation

LCO 3.4.15 The following RCS leakage detection instrumentation shall be OPERABLE:

- a. One Containment Sump Level and Flow Monitoring System;
- b. One containment atmosphere particulate radioactivity monitor; and
- c. One containment air cooler condensate flow rate monitor or one containment atmosphere radioactivity monitor (gaseous).

APPLICABILITY: MODES 1, 2, 3, and 4

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Required Containment Sump Level and Flow Monitoring System Inoperable.	A.1 -----NOTE----- Not required until 12 hours after establishment of steady state operation. -----	
	Perform SR 3.4.13.1.	Once per 24 hours
	<u>AND</u> A.2 Restore Containment Sump Level and Flow Monitoring System to OPERABLE status.	30 days

(continued)

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.16 RCS Specific Activity

LCO 3.4.16 The specific activity of the reactor coolant shall be within limits.

APPLICABILITY: MODES 1 and 2,
 MODE 3 with RCS average temperature (T_{avg}) $\geq 500^{\circ}\text{F}$

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. DOSE EQUIVALENT I-131 > 0.45 $\mu\text{Ci/gm}$.	-----Note----- LCO 3.0.4.c is applicable. -----	
	A.1 Verify DOSE EQUIVALENT I-131 within the acceptable region of Figure 3.4.16-1.	Once per 4 hours
	<u>AND</u> A.2 Restore DOSE EQUIVALENT I-131 to within limit.	48 hours

(continued)

3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)

3.5.3 ECCS — Shutdown

LCO 3.5.3 One ECCS train shall be OPERABLE.

-----NOTE-----
An RHR train may be considered OPERABLE during alignment and operation for decay heat removal, if capable of being manually realigned to the ECCS mode of operation.

APPLICABILITY: MODE 4

ACTIONS

-----NOTE-----
LCO 3.0.4.b is not applicable to ECCS Centrifugal Pump subsystem.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Required ECCS residual heat removal (RHR) subsystem inoperable.	A.1 Initiate action to restore required ECCS RHR subsystem to OPERABLE status.	Immediately
B. Required ECCS Centrifugal Charging Pump subsystem inoperable.	B.1 Restore required ECCS Centrifugal Charging Pump subsystem to OPERABLE status.	1 hour
C. Required Action and associated Completion Time of Condition B not met.	C.1 Be in MODE 5.	24 hours

3.6 CONTAINMENT SYSTEMS

3.6.8 Hydrogen Recombiners

LCO 3.6.8 Two hydrogen recombiners shall be OPERABLE.

APPLICABILITY: MODES 1 and 2

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One hydrogen recombinder inoperable.	A.1 Restore hydrogen recombinder to OPERABLE status.	30 days
B. Two hydrogen recombiners inoperable.	B.1 Verify by administrative means that the hydrogen control function is maintained.	1 hour <u>AND</u> Once per 12 hours thereafter
	<u>AND</u> B.2 Restore one hydrogen recombinder to OPERABLE status.	7 days

(continued)

3.7 PLANT SYSTEMS**3.7.4 Steam Generator Atmospheric Relief Valves (ARVs)**

LCO 3.7.4 Four ARV lines shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required ARV line Inoperable.	A.1 Restore required ARV line to OPERABLE status.	7 days
B. Two required ARV lines Inoperable.	B.1 Restore at least one ARV line to OPERABLE status.	72 hours
C. Three or more required ARV lines Inoperable.	C.1 Restore at least two ARV lines to OPERABLE status.	24 hours

(continued)

3.7 PLANT SYSTEMS

3.7.5 Auxiliary Feedwater (AFW) System

LCO 3.7.5 Three AFW trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3

ACTIONS

-----NOTE-----
LCO 3.0.4.b is not applicable.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One steam supply to turbine driven AFW pump inoperable.	A.1 Restore steam supply to OPERABLE status.	7 days <u>AND</u> 10 days from discovery of failure to meet the LCO
B. One AFW train inoperable for reasons other than Condition A.	B.1 Restore AFW train to OPERABLE status.	72 hours <u>AND</u> 10 days from discovery of failure to meet the LCO

(continued)

ACTIONS

-----NOTE-----
LCO 3.0.4.b Is not applicable to DGs.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required offsite circuit Inoperable.	A.1 Perform SR 3.8.1.1 for required OPERABLE offsite circuit.	1 hour <u>AND</u> Once per 8 hours thereafter
	<u>AND</u> -----NOTE----- In MODES 1, 2 and 3, the TDAFW pump is considered a required redundant feature. -----	
	A.2 Declare required feature(s) with no offsite power available Inoperable when its redundant required feature(s) is Inoperable.	24 hours from discovery of no offsite power to one train concurrent with Inoperability of redundant required feature(s)
	<u>AND</u>	
	A.3 Restore required offsite circuit to OPERABLE status.	72 hours <u>AND</u> 6 days from discovery of failure to meet LCO <u>OR</u> 21 days for a one time preventive maintenance outage on Startup Transformer XST2 to be completed by February 28, 2002

(continued)



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 17, 2004

Mr. M. R. Blevins
Senior Vice President
& Principal Nuclear Officer
TXU Energy
ATTN: Regulatory Affairs
P. O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES), UNITS 1 AND 2 -
CORRECTION TO TECHNICAL SPECIFICATION PAGE 3.0-2 TO
AMENDMENT NO. 109, RE: INCREASING FLEXIBILITY IN MODE
RESTRAINTS (TAC NOS. MC0007 AND MC0008)

Dear Mr. Blevins:

On January 23, 2004, the Commission issued the Amendment No. 109 to Facility Operating License No. NPF-87 and Amendment No. 109 to Facility Operating License No. NPF-89 for CPSES, Units 1 and 2, respectively. The Amendments consisted of changes to the Technical Specifications (TSs) in response to your application dated July 18, 2003.

Subsequently, your staff informed the Commission staff that the correction bar to TS page 3.0-2 was partially missing. Upon investigation, we found that the correction bar had been inadvertently partially deleted from TS page 3.0-2 accompanying the subject Amendments.

We regret any inconvenience caused you by the error, and enclose the corrected TS page 3.0-2 for incorporation in the subject Amendments.

Sincerely,

A handwritten signature in cursive script, reading "Mohan C. Thadani", is positioned above the typed name.

Mohan C. Thadani, Senior Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosure: Technical Specification Page 3.0-2 to NPF-87 and NPF-89

cc w/encls: See next page

3.0 LCO APPLICABILITY (continued)

LCO 3.0.4

When an LCO is not met, entry into a MODE or other specified condition in the Applicability shall only be made:

- a. When the associated ACTIONS to be entered permit continued operation in the MODE or other specified condition in the Applicability for an unlimited period of time;
- b. After performance of a risk assessment addressing inoperable systems and components, consideration of the results, determination of the acceptability of entering the MODE or other specified condition in the Applicability, and establishment of risk management actions, if appropriate; exceptions to this Specification are stated in the individual Specifications, or
- c. When an allowance is stated in the Individual value, parameter, or other Specification.

This Specification shall not prevent changes in MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

LCO 3.0.5

Equipment removed from service or declared Inoperable to comply with ACTIONS may be returned to service under administrative control solely to perform testing required to demonstrate its OPERABILITY or the OPERABILITY of other equipment. This is an exception to LCO 3.0.2 for the system returned to service under administrative control to perform the testing required to demonstrate OPERABILITY.

LCO 3.0.6

When a supported system LCO is not met solely due to a support system LCO not being met, the Conditions and Required Actions associated with this supported system are not required to be entered. Only the support system LCO ACTIONS are required to be entered. This is an exception to LCO 3.0.2 for the supported system. In this event, an evaluation shall be performed in accordance with Specification 5.5.15, "Safety Function Determination Program (SFDP)." If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered.

When a support system's Required Action directs a supported system to be declared Inoperable or directs entry into Conditions and Required Actions for a supported system, the applicable Conditions and Required Actions shall be entered in accordance with LCO 3.0.2.

(continued)



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 29, 2003

Mr. C. Lance Terry
Senior Vice President
& Principal Nuclear Officer
TXU Energy
ATTN: Regulatory Affairs
P. O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2 - ISSUANCE
OF AMENDMENTS RE: REVISION OF OPERATING LICENSE, APPENDIX B,
ENVIRONMENTAL PROTECTION PLAN (TAC NOS. MB6794 AND MB6795)

Dear Mr. Terry:

The U. S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 104 to Facility Operating License No. NPF-87 and Amendment No. 104 to Facility Operating License No. NPF-89 for Comanche Peak Steam Electric Station, Units 1 and 2, respectively. The amendments consist of changes to Appendix B to the Facility Operating Licenses, Environmental Protection Plan (EPP), in response to your application dated November 19, 2002, as supplemented by letters dated February 5 and May 5, 2003.

The amendments revise the EPP to replace references to the U. S. Environmental Protection Agency's National Pollutant Discharge Elimination System expired permit. The amendments also contain minor changes to the EPP to be consistent with the provisions of the current Texas Pollutant Discharge Elimination System permit and the Final Environmental Statement - Operating License Stage and consolidate the Unit 1 and Unit 2 EPPs into a single document.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "D. H. Jaffe", is written over a circular stamp.

David H. Jaffe, Senior Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. 104 to NPF-87
2. Amendment No. 104 to NPF-89
3. Safety Evaluation

cc w/encls: See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 104

TO FACILITY OPERATING LICENSE NO. NPF-87

AND AMENDMENT NO. 104

TO FACILITY OPERATING LICENSE NO. NPF-89

DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Unit 1 and Unit 2 Appendix B Environmental Protection Plans with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Title Page
Appendix B Table of Contents
1-1
2-1
3-1
3-2
4-1
4-2
5-1
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Insert

Title Page
Appendix B Table of Contents
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APPENDIX B
TO FACILITY OPERATING LICENSE NOS. NPF-87 & NPF-89

TXU GENERATION COMPANY LP
COMANCHE PEAK STEAM ELECTRIC STATION UNITS 1 & 2
DOCKET NOS. 50-445 AND 50-446

ENVIRONMENTAL PROTECTION PLAN
(NON RADIOLOGICAL)

Amendment No. ~~68, 90,~~ 104

COMANCHE PEAK STEAM ELECTRIC STATION
UNITS 1 AND 2

ENVIRONMENTAL PROTECTION PLAN
(NON RADIOLOGICAL)

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1.0 Objectives of the Environmental Protection Plan

The purpose of the Environmental Protection Plan (EPP) is to provide for protection of non radiological environmental values during operation of the nuclear facility. The principal objectives of the EPP are as follows:

- (1) Verify that the facility is operated in an environmentally acceptable manner, as established by the Final Environmental Statement - Operating License Stage (FES-OL) and other NRC environmental impact assessments.
- (2) Coordinate NRC requirements and maintain consistency with other Federal, State, and local requirements for environmental protection.
- (3) Keep NRC informed of the environmental effects of facility construction and operation and of actions taken to control those effects.

Environmental concerns identified in the FES-OL which relate to water quality matters are regulated by way of the licensee's TPDES permit.

2.0 Environmental Protection Issues

In the FES-OL, dated September 1981, the staff considered the environmental impacts associated with the operation of the two-unit Comanche Peak Steam Electric Station (CPSES). Certain environmental issues were identified which required study or license conditions to resolve environmental concerns and to assure adequate protection of the environment.

2.1 Aquatic Issues

The aquatic issues identified by the State in the FES-OL were as follows:

- (1) Effects of the intake structure on aquatic biota during operation (FES-OL Section 5.5.2.3).
- (2) Effects of the circulating water chlorination system on aquatic biota during operation (FES-OL Sections 4.2.4.1, 5.3.4.1, and 5.11.3.1).

The second issue above, "Effects of the circulating water chlorination system on aquatic biota during operation (FES-OL Sections 4.2.4.1, 5.3.4.1, and 5.11.3.1)," no longer applies because the TPDES permit no longer requires that such a study be performed.

Aquatic matters are addressed by the effluent limitations and monitoring requirements contained in the effective TPDES permit issued by the Texas Commission on Environmental Quality. The NRC will rely on this agency for regulation of matters involving water quality and aquatic biota.

2.2 Terrestrial Issues

The terrestrial issue identified by the staff in the FES-OL was as follows:

- (1) Potential impacts resulting from the use of groundwater by the station during operation (FES-OL Section 5.3.1.2).

NRC requirements with regard to the terrestrial issue are specified in Subsection 4.2 of this EPP.

3.0 Consistency Requirements

3.1 Plant Design and Operation

The licensee may make changes in station design or operation or perform tests or experiments affecting the environment provided such activities do not involve an unreviewed environmental question and do not involve a change in the EPP*. Changes in station design or operation or performance of tests or experiments which do not affect the environment are not subject to the requirements of this EPP. Activities governed by Subsection 3.3 are not subject to the requirements of this Section.

Before engaging in additional construction or operational activities which may significantly affect the environment, the licensee shall prepare and record an environmental evaluation of such activity. Activities are excluded from this requirement if all measurable nonradiological environmental effects are confined to the onsite areas previously disturbed during site preparation and plant construction. When the evaluation indicates that such activity involves an unreviewed environmental question, the licensee shall provide a written evaluation of such activity and obtain prior NRC approval. When such activity involves a change in the EPP, such activity and change to the EPP may be implemented only in accordance with an appropriate license amendment as set forth in Subsection 5.3 of this EPP.

A proposed change, test, or experiment shall be deemed to involve an unreviewed environmental question if it concerns: (1) a matter which may result in a significant increase in any adverse environmental impact previously evaluated in the FES-OL, in environmental impact appraisals, or in any decisions of the Atomic Safety and Licensing Board; or (2) a significant change in effluents or power level; or (3) a matter, not previously reviewed and evaluated in the documents specified in (1) of this Subsection, which may have a significant adverse environmental impact.

The licensee shall maintain records of changes in facility design or operation and of tests and experiments carried out pursuant to this Subsection. These records shall include written evaluations which provide bases for the determination that the change, test, or experiment does not involve an unreviewed environmental question or constitute a decrease in the effectiveness of this EPP to meet the objectives specified in Section 1.0. The licensee shall include as part of the Annual Environmental Operating Report (per Subsection 5.4.1) brief descriptions, analyses, interpretations, and evaluations of such changes, tests, and experiments.

3.2 Reporting Related to the TPDES Permit

Changes to, or renewals of, the TPDES permit shall be reported to the NRC within 30 days following the date the change or renewal is approved. If a permit, in part or in its entirety, is appealed and stayed, the NRC shall be notified within 30 days following the date the stay is granted.

*This provision does not relieve the licensee of the requirements of 10 CFR 50.59.

The licensee shall notify the NRC of changes to the effective TPDES permit that are proposed by the licensee by providing NRC with a copy of the proposed change at the same time it is submitted to the permitting agency. The licensee shall provide the NRC with a copy of the application for renewal of the TPDES permit at the same time the application is submitted to the permitting agency.

3.3 Changes Required for Compliance with Other Environmental Regulations

Changes in plant design or operation and performance of tests or experiments which are required to achieve compliance with other Federal, State, and local environmental regulations are not subject to the requirements of Subsection 3.1.

4.0 Environmental Conditions

4.1 Unusual or Important Environmental Events

Any occurrence of an unusual or important event that indicates or could result in significant environmental impact causally related to plant operation shall be recorded and reported to the NRC within 24 hours, followed by a written report per Subsection 5.4.2. The following are examples of such events: excessive bird Impaction events, onsite plant or animal disease outbreaks, mortality or unusual occurrence of any species protected by the Endangered Species Act of 1973, fish kills, increase in nuisance organisms or conditions, and unanticipated or emergency discharge of waste water or chemical substances.

No routine monitoring programs are required to implement this condition.

4.2 Environmental Monitoring

4.2.1 Groundwater Levels and Station Water Use Monitoring

Groundwater levels in the onsite observation wells identified as OB-3 and OB-4 in the FES-OL (Figure 4-3) shall be monitored and recorded monthly when the groundwater pumpage rate by CPSES is less than or equal to 30 gallons per minute (gpm) and weekly when the CPSES average monthly rate exceeds 30 gpm for the previous month. Water levels shall be read and recorded on approximately the same day of the month when monitoring monthly and on the same day of the week when monitoring weekly (an aid in interpreting the results by minimizing the influence of cyclic water use patterns of the aquifer by others on the observed water levels).

A monthly record of the total number of gallons pumped from each of the onsite production wells shall be maintained, including an average monthly pumpage rate in gpm.

A monthly record showing the rate and total amount of surface water processed by the onsite water treatment facility shall be maintained by the licensee on a monthly basis. This record shall include the process rate in gallons per minute and the total amount in gallons.

The licensee shall include the results of this monitoring program as part of the Annual Operating Report (see Subsection 5.4.1).

4.2.2 Water Treatment Facility Outages Impact Assessment and Reporting

The following outage of the onsite water treatment facility shall be reported to the NRC if groundwater is used to supplement the supply of treated surface water during the outage:

- (1) Routine or unplanned outages that exceed 30 consecutive days.
- (2) Any outage of at least 24 hours duration, beginning with the third such outage in a calendar year, if these outages are accompanied by an increase in the monthly average groundwater pumpage to a rate exceeding 30 gpm. When it is determined that either

routine or unplanned outages will exceed 30 consecutive days and when the groundwater pumpage rate will be greater than 30 gpm when averaged over the outage period, the licensee will prepare and submit a report to the NRC within 15 days after a determination of the extended outage is made. This report shall include (1) a discussion of the reason for the extended outage, (2) the expected duration of the outage, (3) an estimate of the date or the time required to return the onsite water treatment facility to operation, (4) a determination of the potential for lowering the groundwater levels in offsite wells, (5) an assessment of the impact of the projected groundwater level decline, and (6) a proposed course of action to mitigate any adverse effects.

5.0 Administrative Procedures

5.1 Review and Audit

The licensee shall provide for review and audit of compliance with the EPP. The audits shall be conducted independently of the individual or groups responsible for performing the specific activity. A description of the organization structure utilized to achieve the independent review and audit function and the results of audit activities shall be maintained and made available for inspection.

5.2 Records Retention

Records and logs relative to the environmental aspects of station operation shall be made and retained in a manner convenient for review and inspection. These records and logs shall be made available to NRC on request.

Records of modifications to station structures, systems, and components determined to potentially affect the continued protection of the environment shall be retained for the life of the station. All other records, data and logs relating to this EPP shall be retained for 5 years or, where applicable, in accordance with the requirements of other agencies.

5.3 Changes in Environmental Protection Plan

Requests for changes in the EPP shall include an assessment of the environmental impact of the proposed change and a supporting justification. Implementation of such changes in the EPP shall not commence prior to NRC approval of the proposed changes in the form of a license amendment incorporating the appropriate revision to the EPP.

5.4 Plant Reporting Requirements

5.4.1 Routine Reports

An Annual Environmental Operating Report describing implementation of this EPP for the previous year shall be submitted to the NRC prior to May 1 of each year. The initial report shall be submitted prior to May 1 of the year following issuance of the operating license. The period of the first report shall begin with the date of issuance of the operating license.

The report shall include summaries and analyses of the results of the environmental protection activities required by Subsection 4.2 of this EPP for the report period, including a comparison with related preoperational studies, operational controls (as appropriate), and previous nonradiological environmental monitoring reports, and an assessment of the observed impacts of plant operation on the environment. If harmful effects or evidence of trends toward irreversible damage to the environment are observed, the licensee shall provide a detailed analysis of the data and a proposed course of mitigating action.

The Annual Environmental Operating Report shall also include:

- (1) A list of EPP noncompliances and the corrective actions taken to remedy them.
- (2) A list of all changes in station design or operation, tests, and experiments made in accordance with Subsection 3.1 which involved a potentially significant unreviewed environmental question.
- (3) A list of nonroutine reports submitted in accordance with Subsection 5.4.2.
- (4) A summary list of TPDES permit-related reports relative to matters identified in Subsection 2.1 which were sent to the Texas Commission on Environmental Quality during the report period.

In the event that some results are not available by the report due date, the report shall be submitted noting and explaining the missing results. The missing results shall be submitted as soon as possible in a supplementary report.

5.4.2 Nonroutine Reports

A written report shall be submitted to the NRC within 30 days of occurrence of a nonroutine event. The report shall (a) describe, analyze, and evaluate the event, including extent and magnitude of the impact and plant operating characteristics; (b) describe the probable cause of the event; (c) indicate the action taken to correct the reported event; (d) indicate the corrective action taken to preclude repetition of the event and to prevent similar occurrences involving similar components or systems; and (e) indicate the agencies notified and their preliminary responses.

Events reportable under this subsection which also require reports to other Federal, State or local agencies shall be reported in accordance with those reporting requirements in lieu of the requirements of this subsection. The NRC shall be provided with a copy of such a report at the same time it is submitted to the other agency.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 15, 2003

Mr. C. Lance Terry
Senior Vice President
& Principal Nuclear Officer
TXU Energy
ATTN: Regulatory Affairs
P. O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES), UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: DELETION OF UNNECESSARY LICENSE
CONDITIONS AND REPORTING REQUIREMENTS (TAC NOS. MB5770 AND
MB5771)

Dear Mr. Terry:

The U. S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 103 to Facility Operating License (FOL) No. NPF-87 and Amendment No. 103 to FOL No. NPF-89 for CPSES, Units 1 and 2, respectively. The amendments consist of changes to the FOLs and the Technical Specifications (TSs) in response to your application dated July 25, 2002, as supplemented by letters dated February 5 and February 11, 2003.

The amendments change the CPSES FOLs as follows: the license conditions related to Decommissioning Trusts, specified in Sections 2.C.(4)(a), 2.C.(4)(b), 2.C.(4)(d), 2.C.(4)(e), and 2.C.(6) are deleted, and Section 2.E, which requires reporting any violations of the requirements contained in Section 2.C of the licenses, is deleted. Additionally, TS Table 5.5-2 "Steam Generator Tube Inspection," TS Table 5.5-3, "Steam Generator Repaired Tube Inspection for Unit 1 Only," and TS 5.6.10c, "Steam Generator Tube Inspection Report," are revised to delete the requirement to notify the NRC pursuant to Section 50.72(b)(2), "Immediate notification requirements for operating nuclear power reactors," of Title 10 of the *Code of Federal Regulations* if the steam generator tube inspection results are in a Category C-3 classification.

C. Terry

-2-

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

David H. Jaffe, Senior Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. 103 to NPF-87
2. Amendment No. 103 to NPF-89
3. Safety Evaluation

cc w/encls: See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 103

TO FACILITY OPERATING LICENSE NO. NPF-87

AND AMENDMENT NO. 103

TO FACILITY OPERATING LICENSE NO. NPF-89

DOCKET NOS: 50-445 AND 50-446

Replace the following pages of licenses and the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

FACILITY OPERATING LICENSE NO. NPF-87

<u>Remove</u>	<u>Insert</u>
Page 4	Page 4
Page 5	Page 5
Page 6	Page 6

FACILITY OPERATING LICENSE NO. NPF-89

<u>Remove</u>	<u>Insert</u>
Page 4	Page 4
Page 5	Page 5
Page 6	Page 6

APPENDIX A TECHNICAL SPECIFICATIONS

<u>Remove</u>	<u>Insert</u>
5.0-19	5.0-19
5.0-19a	5.0-19a
5.0-36	5.0-36

(3) Antitrust Conditions

DELETED

(4) License Transfer

The TXU Generation Company LP Decommissioning Master Trust Agreement for the facility at the time the license transfers are effected and thereafter, is subject to the following:

(a) DELETED

(b) DELETED

(c) The appropriate section of the decommissioning trust agreement must state that investments made in trust by the trustee, investment advisor, or anyone else directing the investments made in the trusts shall adhere to investment guidelines established by the PUCT (e.g., 16 Texas Administration Code 25.301);

(d) DELETED

(e) DELETED

(5) License Transfer

TXU Generation Company LP shall provide decommissioning funding assurance, to be held in a decommissioning trust for the facility upon the direct transfer of the facility license to TXU Generation Company LP, in an amount equal to or greater than the balance in the facility decommissioning trusts immediately prior to the transfer. In addition, TXU Generation Company LP shall ensure that all contractual arrangements referred to in the application for approval of the transfer of the facility license to TXU Generation Company LP, to obtain necessary decommissioning funds for the facility through a non-bypassable charge are executed and will be maintained until the decommissioning trusts are fully funded, or shall ensure that other mechanisms that provide equivalent assurance of decommissioning funding in accordance with the Commission's regulations are maintained.

(6) License Transfer

DELETED

(7) License Transfer

TXU Generation Company LP and its subsidiaries agree to provide the Director, Office of Nuclear Reactor Regulation, a copy of any application, at the time it is filed, to transfer (excluding grants of security interests or liens) from TXU Generation Company LP or its subsidiaries to its proposed parent, or to any other affiliated company, facilities for the production of electric energy having a depreciated book value exceeding ten percent (10%) of such licensee's consolidated net utility plant, as recorded on TXU Generation Company LP's book of accounts.

D. The following exemptions are authorized by law and will not endanger life or property or the common defense and security. Certain special circumstances are present and these exemptions are otherwise in the public interest. Therefore, these exemptions are hereby granted pursuant to 10 CFR 50.12.

- (1) The facility requires a technical exemption from the requirements of 10 CFR Part 50, Appendix J, Section III.D.2(b)(ii). The justification for this exemption is contained in Section 6.2.5 of Supplement 22 to the Safety Evaluation Report dated January 1990. The staff's environmental assessment was published on November 14, 1989 (54 FR 47430).

Therefore, pursuant to 10 CFR 50.12(a)(1), and 10 CFR 50.12(a)(2)(ii) and (iii), the Comanche Peak Steam Electric Station, Unit 1 is hereby granted an exemption from the cited requirement and instead, is required to perform the overall air lock leak test at pressure P_a prior to establishing containment integrity if air lock maintenance has been performed that could affect the air lock sealing capability.

- (2) The facility was previously granted an exemption from the criticality monitoring requirements of 10 CFR 70.24 (see Materials License No. SNM-1912 dated December 1, 1988 and Section 9.1.1 of Supplement 22 to the Safety Evaluation Report dated January 1990). The staff's environmental assessment was published on November 14, 1989 (54 FR 47432). The Comanche Peak Steam Electric Station, Unit 1 is hereby exempted from the criticality monitoring provisions of 10 CFR 70.24 as applied to fuel assemblies held under this license.
- (3) The facility requires a temporary exemption from the schedular requirements of 10 CFR 50.33(k) and 10 CFR 50.75. The justification for this exemption is contained in Section 20.6 of Supplement 22 to the Safety Evaluation Report dated January 1990. The staff's environmental assessment was published on November 14, 1989 (54 FR 47431). Therefore, pursuant to 10 CFR 50.12(a)(1), 50.12(a)(2)(iii) and 50.12(a)(2)(v), the Comanche Peak Steam Electric Station, Unit 1 is hereby granted a temporary exemption from the schedular requirements of 10 CFR 50.33(k) and 10 CFR 50.75 and is required to submit a decommissioning funding report for Comanche Peak Steam Electric Station, Unit 1 on or before July 26, 1990.

E. DELETED

F In order to ensure that TXU Generation Company LP will exercise the authority as the surface landowner in a timely manner and that the requirements of 10 CFR Part 100.3 (a) are satisfied, this license is subject to the additional conditions specified below: (Section 2.1.1, SER)

- (1) For that portion of the exclusion area which is within 2250 ft of any seismic Category I building or within 2800 ft of either reactor containment building, TXU Generation Company LP must prohibit the exploration and/or exercise of subsurface mineral rights, and if the subsurface mineral rights owners attempt to exercise their rights within this area, TXU Generation Company LP must immediately institute immediately effective condemnation proceedings to obtain the mineral rights in this area.

(4) License Transfer

The TXU Generation Company LP Decommissioning Master Trust Agreement for the facility at the time the license transfers are effected and thereafter, is subject to the following:

(a) DELETED |

(b) DELETED |

(c) The appropriate section of the decommissioning trust agreement must state that investments made in trust by the trustee, investment advisor, or anyone else directing the investments made in the trusts shall adhere to investment guidelines established by the PUCT (e.g., 16 Texas Administration Code 25.301);

(d) DELETED |

(e) DELETED |

(5) License Transfer

TXU Generation Company LP shall provide decommissioning funding assurance, to be held in a decommissioning trust for the facility upon the direct transfer of the facility license to TXU Generation Company LP, in an amount equal to or greater than the balance in the facility decommissioning trusts immediately prior to the transfer. In addition, TXU Generation Company LP shall ensure that all contractual arrangements referred to in the application for approval of the transfer of the facility license to TXU Generation Company LP, to obtain necessary

decommissioning funds for the facility through a non-bypassable charge are executed and will be maintained until the decommissioning trusts are fully funded, or shall ensure that other mechanisms that provide equivalent assurance of decommissioning funding in accordance with the Commission's regulations are maintained.

(6) License Transfer

DELETED

(7) License Transfer

TXU Generation Company LP and its subsidiaries agree to provide the Director, Office of Nuclear Reactor Regulation, a copy of any application, at the time it is filed, to transfer (excluding grants of security interests or liens) from TXU Generation Company LP or its subsidiaries to its proposed parent, or to any other affiliated company, facilities for the production of electric energy having a depreciated book value exceeding ten percent (10%) of such licensee's consolidated net utility plant, as recorded on TXU Generation Company LP's book of accounts.

- D. The following exemptions are authorized by law and will not endanger life or property or the common defense and security. Certain special circumstances are present and these exemptions are otherwise in the public interest. Therefore, these exemptions are hereby granted:

- (1) The facility requires a technical exemption from the requirements of 10 CFR Part 50, Appendix J, Section III.D.2(b)(ii). The justification for this exemption is contained in Section 6.2.5.1 of Supplement 26 to the Safety Evaluation Report dated February 1993. The staff's environmental assessment was published on January 19, 1993 (58 FR 5036). Therefore, pursuant to 10 CFR 50.12(a)(1), 10 CFR 50.12(a)(2)(ii) and (iii), the Comanche Peak Steam Electric Station, Unit 2 is hereby granted an exemption from the cited requirement and instead, is required to perform the overall air lock leak test at pressure P_a prior to establishing containment integrity if air lock maintenance has been performed that could affect the air lock sealing capability.

The facility was previously granted exemption from the criticality Monitoring requirements of 10 CFR 70.24 (see Materials License No. SNM-1986 dated April 24, 1989 and Section 9.1.1 of SSER 26 dated February 1993.) The staff's environmental assessment was published on

January 19, 1993 (58 FR 5035). The Comanche Peak Steam Electric Station, Unit 2 is hereby exempted from the criticality monitoring provisions of 10 CFR 70.24 as applied to fuel assemblies held under this license.

- E. DELETED
- F. In order to ensure that TXU Generation Company LP will exercise the authority as the surface landowner in a timely manner and that the requirements of 10 CFR 100.3 (a) are satisfied, this license is subject to the additional conditions specified below: (Section 2.1, SER)
- (1) For that portion of the exclusion area which is within 2250 ft of any seismic Category I building or within 2800 ft of either reactor containment building, TXU Generation Company LP must prohibit the exploration and/or exercise of subsurface mineral rights, and if the subsurface mineral rights owners attempt to exercise their rights within this area, TXU Generation Company LP must immediately institute immediately effective condemnation proceedings to obtain the mineral rights in this area.
 - (2) For the unowned subsurface mineral rights within the exclusion area not covered in item (1), TXU Generation Company LP will prohibit the exploration and/or exercise of mineral rights until and unless the licensee and the owners of the mineral rights enter into an agreement which gives TXU Generation Company LP absolute authority to determine all activities - including times of arrival and locations of personnel and the authority to remove personnel and equipment - in event of emergency. If the mineral rights owners attempt to exercise their rights within this area without first entering into such an agreement, TXU Generation Company LP must immediately institute immediately effective condemnation proceedings to obtain the mineral rights in this area.
 - (3) TXU Generation Company LP shall promptly notify the NRC of any attempts by subsurface mineral rights owners to exercise mineral rights, including any legal proceeding initiated by mineral rights owners against TXU Generation Company LP.
- G. TXU Generation Company LP shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report through Amendment 87 and as approved in the SER (NUREG-0797) and its supplements through SSER 27, subject to the following provision:

5.5 Programs and Manuals

5.5.9 Steam Generator (SG) Tube Surveillance Program (continued)

TABLE 5.5-2
STEAM GENERATOR TUBE INSPECTION

Sample size	1 ST SAMPLE INSPECTION		2 ND SAMPLE INSPECTION		3 RD SAMPLE INSPECTION	
	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of S Tubes per S.G.	C-1	None	N.A.	N.A.	N.A.	N.A.
	C-2	Plug or repair* defective tubes and inspect additional 2S tubes in this S.G.	C-1	None	N.A.	N.A.
			C-2	Plug or repair* defective tubes and inspect additional 4S tubes in this S.G.	C-1	None
					C-2	Plug or repair* defective tubes
					C-3	Perform action for C-3 result of first sample
			C-3	Perform action for C-3 result of first sample	N.A.	N.A.
	C-3	Inspect all tubes in this S.G., plug or repair* defective tubes and inspect 2S tubes in each other S.G.	All other S.G.s are C-1	None	N.A.	N.A.
			Some S.G.s C-2 but no additional S.G. C-3	Perform action for C-2 result of second sample	N.A.	N.A.
			Additional S.G. is C-3	Inspect all tubes in each S.G. and plug or repair* defective tubes.	N.A.	N.A.

(continued)

S - 12/n% Where n is the number of steam generators inspected during an inspection

* for Unit 1 only

5.5 Programs and Manuals

5.5.9 Steam Generator (SG) Tube Surveillance Program (continued)

TABLE 5.5-3
STEAM GENERATOR REPAIRED TUBE INSPECTION FOR UNIT 1 ONLY

1 ST SAMPLE INSPECTION			2 ND SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required
A minimum of 20% of repaired tubes (1)	C-1	None	N.A.	N.A.
	C-2	Plug defective repaired tubes and inspect 100% of the repaired tubes in this S.G.	C-1	None
			C-2	Plug defective repaired tubes
			C-3	Perform action for C-3 result of first sample
	C-3	Inspect all repaired tubes in this S.G., plug defective tubes and inspect 20% of the repaired tubes in each other S.G.	All other S.G.s are C-1	None
			Same S.G.s C-2 but no additional S.G. are C-3	Perform action for C-2 result of first sample
			Additional S.G. is C-3	Inspect all repaired tubes in each S.G. and plug defective tubes.

(continued)

- (1) Each repair method is considered a separate population for determination of initial inservice inspection and scope expansion.

5.6 Reporting Requirements (continued)

5.6.7 Not used

5.6.8 PAM Report

When a report is required by the required actions of LCO 3.3.3, "Post Accident Monitoring (PAM) Instrumentation," a report shall be submitted within the following 14 days. The report shall outline the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the Function to OPERABLE status.

5.6.9 Not used

5.6.10 Steam Generator Tube Inspection Report

- a. Within 15 days following the completion of each inservice inspection of steam generator tubes, the number of tubes plugged, repaired or designated as an F* tube in each steam generator shall be reported to the Commission;
- b. The complete results of the steam generator tube inservice inspection shall be submitted to the Commission in a report within 12 months following the completion of the inspection. This report shall include:
 - 1) Number and extent of tubes and (for Unit 1 only) sleeves inspected.
 - 2) Location and percent of wall-thickness penetration for each indication of an imperfection, and
 - 3) Identification of tubes plugged or repaired.
- c. Results of steam generator tube inspections which fall into Category C-3 shall be reported to the Commission in a report within 30 days and prior to resumption of plant operation. This report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

(continued)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 20, 2002

Mr. C. Lance Terry
Senior Vice President
& Principal Nuclear Officer
TXU Energy
Attn.: Regulatory Affairs Department
P. O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES), UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: CHANGE TO TECHNICAL
SPECIFICATIONS FOR ALLOWED OUTAGE TIME FOR FEEDWATER
ISOLATION VALVES (TAC NOS. MB4640 AND MB4641)

Dear Mr. Terry:

The Commission has issued the enclosed Amendment No. ⁹⁷ to Facility Operating License No. NPF-87 and Amendment No. 97 to Facility Operating License No. NPF-89 for CPSES, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated March 25, 2002, as supplemented by the letter dated April 23, 2002.

The amendment revises TS 3.7.3, "Feedwater Isolation Valves (FIVs) and Associated Bypass Valves," to adopt the NUREG-1431, "Standard Technical Specifications for Westinghouse Plants," Revision 2 version of the specification. The revised TS 3.7.3 adds, among other things, operability and suitable surveillance requirements for Feedwater Control Valves and Associated Bypass Valves and allows for the extended out-of-service time for one or more FIVs. In addition, a footnote which allowed a one-time extension for Condition A Completion Time, is being deleted because it is no longer applicable.

C. Terry

-2-

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read 'D. H. Jaffe', with a stylized flourish extending to the right.

David H. Jaffe, Senior Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. ⁹⁷ to NPF-87
2. Amendment No. ⁹⁷ to NPF-89
3. Safety Evaluation

cc w/encls: See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 97

TO FACILITY OPERATING LICENSE NO. NPF-87

AND AMENDMENT NO. 97

TO FACILITY OPERATING LICENSE NO. NPF-89

DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove
3.7-8

Insert
3.7-8

3.7 PLANT SYSTEMS

3.7.3 Feedwater Isolation Valves (FIVs) and Feedwater Control Valves (FCVs) and Associated Bypass Valves

LCO 3.7.3 Four FIVs, four FCVs, and associated bypass valves shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3 except when FIV, FCV or associated bypass valve is either closed and de-activated or isolated by a closed manual valve

ACTIONS

-----NOTE-----
Separate Condition entry is allowed for each valve.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more FIVs inoperable.	A.1 Close or isolate FIV.	72 hours
	<u>AND</u> A.2 Verify FIV is closed or isolated.	Once per 7 days
B. One or more FCVs inoperable.	B.1 Close or isolate FCV.	72 hours
	<u>AND</u> B.2 Verify FCV is closed or isolated.	Once per 7 days

(continued)

October 12, 2001

Mr. C. Lance Terry
Senior Vice President
& Principal Nuclear Officer
TXU Electric
Attn: Regulatory Affairs Department
P. O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES), UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: INCREASE IN ALLOWABLE THERMAL
POWER TO 3458 MWT AND DELETION OF TEXAS MUNICIPAL POWER
AGENCY FROM THE OPERATING LICENSES (TAC NOS. MB1625 AND
MB1626)

Dear Mr. Terry:

The Commission has issued the enclosed Amendment No. 89 to Facility Operating License (FOL) No. NPF-87 and Amendment No. 89 to FOL No. NPF-89 for CPSES, Units 1 and 2, respectively. The amendments consist of changes to FOL Nos. NPF-87 and NPF-89 and the Technical Specifications in response to your application dated April 5, 2001, as supplemented by letters dated June 28, August 2, and September 10, 2001.

The amendments increase the maximum, licensed, thermal power of CPSES, Units 1 and 2, to 3458 MWt, which represents an increase of approximately 1.4 percent of the currently licensed thermal power for CPSES, Unit 1, and an increase of approximately 0.4 percent for CPSES, Unit 2. In addition, the amendments remove Texas Municipal Power Agency (TMPA) from both Unit 1 and Unit 2 licenses since transfer of partial ownership from TMPA to TXU Electric was completed.

A copy of our related Safety Evaluation, and Notice of Issuance, for publication in the *Federal Register*, are also enclosed.

Sincerely,

/RA/

David H. Jaffe, Senior Project Manager, Section 1
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. 89 to NPF-87
2. Amendment No. 89 to NPF-89
3. Safety Evaluation
4. Notice of Issuance

cc w/encls: See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 89
TO FACILITY OPERATING LICENSE NO. NPF-87
AND AMDNDMENT 89
TO FACILITY OPERATING LICENSE NO. NPF-89
DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
3.3-15	3.3-15
3.3-16	3.3-16

Table 3.3.1-1 (page 1 of 6)
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE(a)
1. Manual Reactor Trip	1,2	2	B	SR 3.3.1.14	NA
	3(b), 4(b), 5(b)	2	C	SR 3.3.1.14	NA
2. Power Range Neutron Flux					
a. High	1,2	4	D	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.7 SR 3.3.1.11 SR 3.3.1.16	≤ 110.8% RTP
b. Low	1(c), 2	4	E	SR 3.3.1.1 SR 3.3.1.8 SR 3.3.1.11 SR 3.3.1.16	≤ 27.7% RTP
3. Power Range Neutron Flux Rate High Positive Rate	1,2	4	E	SR 3.3.1.7 SR 3.3.1.11	≤ 6.3 % RTP with time constant ≥ 2 sec
4. Intermediate Range Neutron Flux	1(c), 2(d)	2	F,G	SR 3.3.1.1 SR 3.3.1.8 SR 3.3.1.11	≤ 31.5% RTP
5. Source Range Neutron Flux	2(e)	2	I,J	SR 3.3.1.1 SR 3.3.1.8 SR 3.3.1.11	≤ 1.4 E5 cps
	3(b), 4(b), 5(b)	2	J,K	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.11	≤ 1.4 E5 cps

(continued)

- (a) The Allowable Value defines the limiting safety system setting. See the Bases for the Trip Setpoints.
(b) With Rod Control System capable of rod withdrawal or one or more rods not fully inserted.
(c) Below the P-10 (Power Range Neutron Flux) Interlock.
(d) Above the P-6 (Intermediate Range Neutron Flux) Interlock.
(e) Below the P-6 (Intermediate Range Neutron Flux) Interlock.

Table 3.3.1-1 (page 2 of 6)
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE(a)
6. Overtemperature N-16	1,2	4	E	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.3 SR 3.3.1.6 SR 3.3.1.7 SR 3.3.1.10 SR 3.3.1.16	Refer to Note 1
7. Overpower N-16	1,2	4	E	SR 3.3.1.1 SR 3.3.1.2 SR 3.3.1.7 SR 3.3.1.10 SR 3.3.1.16	≤ 112.9% RTP
8. Pressurizer Pressure					
a. Low	1(g)	4	M	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.10 SR 3.3.1.16	≥ 1863.6 psig (Unit 1) ≥ 1865.2 psig (Unit 2)
b. High	1,2	4	E	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.10 SR 3.3.1.16	≤ 2400.8 psig (Unit 1) ≤ 2401.4 psig (Unit 2)
9. Pressurizer Water Level - High	1(g)	3	M	SR 3.3.1.1 SR 3.3.1.7 SR 3.3.1.10	≤ 93.9% of instrument span

(continued)

(a) The Allowable Value defines the limiting safety system setting. See the Bases for the Trip Setpoints.

(g) Above the P-7 (Low Power Reactor Trips Block) interlock.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 3, 1999

Mr. C. Lance Terry
Senior Vice President
& Principal Nuclear Officer
TU Electric
Attn: Regulatory Affairs Department
P. O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION, UNITS 1 AND 2 - ISSUANCE
OF AMENDMENTS RE: SURVEILLANCE REQUIREMENT CHANGES AND
EDITORIAL CORRECTIONS (TAC NOS. MA5418 AND MA5419)

Dear Mr. Terry:

The Commission has issued the enclosed Amendment No. 66 to Facility Operating License No. NPF-87 and Amendment No. 66 to Facility Operating License No. NPF-89 for the Comanche Peak Steam Electric Station, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TSs) in response to your application dated May 4, 1999, as supplemented by letter dated June 4, 1999.

The amendments correct a number of editorial errors in the TSs that occurred with the issuance of License Amendment No. 64 regarding the improved Technical Specifications conversion. In addition, a Surveillance Requirement (SR) 3.8.4.7 is changed to allow the substitution of a modified performance discharge test, for a service test, for the 125 VDC batteries and SRs 3.8.1.7, 3.8.1.12, 3.8.1.15, and 3.8.1.20 are revised to separate the voltage and frequency acceptance criteria for the diesel generator start surveillances into two sets of criteria; those criteria required to be met within 10 seconds, and those criteria required to be met following achievement of steady state conditions.

A copy of our related Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "D. H. Jaffe", is written over a horizontal line.

David H. Jaffe, Senior Project Manager, Section 1
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures: 1. Amendment No. 66 to NPF-87
2. Amendment No. 66 to NPF-89
3. Safety Evaluation

cc w/encs: See next page

ATTACHMENT TO LICENSE AMENDMENT NO. 66 AND 66
TO FACILITY OPERATING LICENSE NO. NPF-87AND NPF-89
DOCKET NOS. 50-445 AND 50-446

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
3.9-10	3.9-10

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. continued	A.3 Initiate action to satisfy RHR loop requirements.	Immediately
	<u>AND</u> A.4 Close all containment penetrations providing direct access from containment atmosphere to outside atmosphere.	4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.9.5.1 Verify one RHR loop is in operation and circulating reactor coolant at a flow rate of ≥ 3800 gpm.	12 hours