

4911

# CASE

(CITIZENS ASSN. FOR SOUND ENERGY)

1426 S. Polk  
Dallas, Texas

214/946-9446

75224  
DOCKETED  
USNRC

November 23, 1987

'87 NOV 25 P3:30

Administrative Judge Peter E. Bloch  
U. S. Nuclear Regulatory Commission  
Atomic Safety & Licensing Board  
Washington, D.C. 20555

Dr. Kenneth A. McCollom  
1107 West Knapp Street  
Stillwater, Oklahoma 74075

OFFICE OF SECRETARY  
DOCKETING & SERVICE  
BRANCH

Dr. Walter H. Jordan  
881 W. Outer Drive  
Oak Ridge, Tennessee 37830

Dear Administrative Judges:

Subject: In the Matter of  
Texas Utilities Electric Company, et al.  
Application for an Operating License  
Comanche Peak Steam Electric Station  
Units 1 and 2  
Docket Nos. 50-445 and 50-446 - DL

Potential 10 CFR 50.55(e) Items

To assist the Board in its desire to be kept up-to-date on matters of potential significance, CASE is attaching copies of the potential 50.55(e) items (SDAR's, or Significant Deficiency Analysis Reports) which we have received since we provided the Board with such items on October 19, 1987.

It had been our intention to periodically continue to provide these SDAR's unless the Board indicates otherwise. However, since Applicants are now listing such reports in their Progress Reports, and in light of our current work load, we ask that the Board advise if it still would like to receive the SDAR's in this format (copies of the SDAR letters themselves). If so, CASE will still provide the copies as we have in the past; however, it may be less frequently.

Respectfully submitted,

CASE (Citizens Association for Sound Energy)

*Juanita Ellis*  
(Mrs.) Juanita Ellis  
President

cc: Service List, with Attachments

8712010038 871123  
PDR ADDCK 05000445  
G PDR

DS03



Log # TXX-6991  
File # 10110  
909.2  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 19, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SERVICE WATER SYSTEM ELECTRICAL  
DESIGN VERIFICATION  
SDAR: CP-87-117 (INTERIM REPORT)

Gentlemen.

On October 23, 1987, we verbally notified your Mr. R. F. Warnick of an issue involving the design requirements of the 118 volt circuits for the Station Service Water (SSW) System. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

We are evaluating this issue to determine the impact on plant safety and expect to submit our next report by March 1, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *J. S. Marshall*

J. S. Marshall  
Supervisor,  
Generic Licensing

MCP/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*87H240064*



Log # TXX-6990  
File # 10110  
909.2  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 19, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
PURGE GAS OVERPRESSURIZATION OF THE  
CONDENSATE AND THE REACTOR MAKEUP WATER  
STORAGE TANKS  
SDAR: CP-87-116 (INTERIM REPORT)

Gentlemen:

On October 23, 1987, we verbally notified your Mr. R. F. Warnick of an issue involving nitrogen purge regulating valves, and failure of these valves could result in overpressurizing the Condensate and the Reactor Makeup Storage tanks. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

We are evaluating the impact of the issue on plant safety and expect to submit our next report by March 2, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

*J. S. Marshall*

By:

J. S. Marshall  
Supervisor,  
Generic Licensing

MCP/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*8711240084*



Log # TXX-6984  
File # 10110  
903.7  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 18, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SEISMIC ANALYSIS OF SERVICE WATER  
INTAKE STRUCTURE  
SDAR: CP-87-115 (INTERIM REPORT)

Gentlemen:

On October 21, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving abnormally high calculated natural frequencies of the soil-structure system. Specifically, the soil-structure mass was underestimated and the soil subgrade stiffness was overestimated in the original analysis. This is a potentially reportable item under the provisions of 10CFR50.55(e).

We are continuing our evaluation and anticipate submitting our next report by February 19, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *J. S. Marshall*  
J. S. Marshall  
Supervisor,  
Generic Licensing

CBC/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711230438~~



Log # TXX-6983  
File # 10110  
903.9  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 18, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
FLEXIBLE METAL TUBING MISALIGNMENT  
SDAR: CP-87-114 (INTERIM REPORT)

Gentlemen:

On October 21, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving instrument tubing installations in safety-related applications which may be unacceptable. Specifically, anti-torque marks on flexible metal tubing have been observed to be misaligned prior to installation. This misalignment would cause excessive stress to be introduced to the tubing. This is a potentially reportable item under the provisions of 10CFR50.55(e).

We are continuing our evaluation and anticipate submitting our next report no later than February 19, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *J. S. Marshall*  
\_\_\_\_\_  
J. S. Marshall  
Supervisor,  
Generic Licensing

HAM/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*87112302TD*



Log # TXX-6945  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 17, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
AMPOWER BUTT SPLICES  
SDAR: CP-87-113 (INTERIM REPORT)

Gentlemen:

On October 19, 1987, we verbally notified your Mr. L. Ellershaw of a deficiency involving the use of Ampower butt splices for the connection of field cables to penetration hardline conductors which is not in conformance with Ampower design specifications. This condition was identified by site engineering personnel and is documented by Nonconformance Report (NCR) CE-87-10802. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

We are continuing our evaluation of the impact of this issue upon the safety of plant operations. Our next report will be submitted no later than January 22, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *J. S. Marshall*

J. S. Marshall  
Supervisor, Generic  
Licensing

WJH/mgt

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~87H230227~~



Log # TXX-6941  
File # 10110  
910.3  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 11, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NO. 50-446  
HOT SHUTDOWN PANEL SEISMIC ANALYSIS  
SDAR: CP-87-112 (INTERIM REPORT)

Gentlemen:

On October 12, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency identified by third party inspections indicating the Unit 2 hot shutdown panel and main control board may not be seismically qualified. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

We are continuing our evaluation of this issue and will submit our next report no later than January 27, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*

D. R. Woodlan  
Supervisor,  
Docket Licensing

CBC/mbk

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6916  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

November 9, 1987

William G. Counsell  
Executive Vice President

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NO. 50-446  
MOTOR CONTROL CENTER LOOSE TERMINAL  
SDAR: CP-87-111 (FINAL REPORT)

Gentlemen:

On October 12, 1987, we verbally notified your Mr. L. Ellershaw of a potentially reportable deficiency involving a loose connection on one phase of a circuit breaker terminal in a Unit 2 Motor Control Center (MCC). We have completed our evaluation and concluded this item is not reportable under the provisions of 10CFR50.55(e).

As stated in the Non-conformance report (NCR CE-87-9151-X), a third party inspection of the MCC discovered a loose connection at a connection point. This connection point is one of the power connections supplying a motor operated valve (MOV-2-8351D) located outside containment. This valve is a containment isolation valve for seal injection water to the reactor coolant pumps (RCPs).

The loose terminal does not represent a condition adverse to the safety of plant operation as:

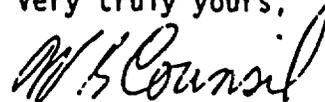
- a) MOV-2-8351D is a normally open (de-energized) valve which does not receive any automatic actuation signals. The failure mode is in the "as-is" position.
- b) The essential modes of operation for this valve are "open" for seal injection or "closed" for containment isolation. Flow into the containment via the charging pumps is essential to provide seal injection water to the RCPs, and the valve is required to be closed only when the charging pumps are not operating. The position of this valve is selected by the operator via remote manual operation and not by automatic actuation. Further, the valve is equipped with a local handwheel. Use of this handwheel is a recognized method of secondary actuation when desired for containment isolation.

TXX-6916  
November 9, 1987  
Page 2 of 2

Based upon these conditions, the potential for loss of remote manual control of this valve is not detrimental to safe operation of the plant.

Supporting documentation is available onsite for your Inspector's review.

Very truly yours,



W. G. Council

WJH/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6933  
File # 10110  
907.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 9, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
INOPERABLE SAFEGUARD SEQUENCER RELAY  
SDAR: CP-87-110 (INTERIM REPORT)

Gentlemen:

On October 12, 1987, we verbally notified your Mr. Lee Ellershaw of a deficiency involving Solid State Safeguard Sequencer (SSSS) relays which do not meet their contact interrupting rating requirements. Specifically, five of thirty relays manufactured by Allen Bradley have contacts with lower DC ratings than required for tripping and closing 480 volt load center breakers during the blackout SSSS operation. This issue was identified by third party review as a result of activities initiated under the Comanche Peak Response Team Program. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

Our evaluation of this issue to assess the potential impact of this deficiency is continuing. Our next report on this issue will be submitted no later than December 31, 1987.

Very truly yours,

W. G. Council

VIP/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

871130015



Log # TXX-6898  
File # 10110  
917.1  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 27, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
INAPPROPRIATE DEFICIENCY DOCUMENTATION  
SDAR: CP-87-109 (INTERIM REPORT)

Gentlemen:

On October 5, 1987, we verbally notified your Mr. H. S. Phillips, of a deficiency involving inappropriate deficiency documentation. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

Specifically, this deficiency involves documents that identify problems, request clarification, recommend changes, or request action for engineering resolution, which were inappropriately used as design change documents, or identified potential nonconforming or deficient conditions. These conditions may have resulted in the implementation of design changes which had not received appropriate design review and nonconformances or deficiencies not being resolved in accordance with established programs.

This issue is being evaluated concurrently with Corrective Action Report (CAR) 87-73. In the event discrepancies are identified, Deficiency Reports, Non-Conformance Reports or Design Change Authorizations (for non-safety related changes only) as appropriate, will be initiated. We will complete our evaluation for reportability upon completion of our documentation review. This review is currently scheduled to be completed by October 31, 1987.

Our next report on this issue will be submitted no later than January 20, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*

D. R. Woodlan

Supervisor, Docket Licenses

*87110207T2*

DAR/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6886  
File # 10110  
910.4  
Ref # 10CFR50.55(e)

William G. Counsil  
Executive Vice President

October 26, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSSES)  
DOCKET NOS. 50-445 AND 50-446  
AUXILIARY FEEDWATER PUMP LOW SUCTION TRIP  
SDAR: CP-87-108 (INTERIM REPORT)

Gentlemen:

On September 24, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving the low suction pressure trip function of the Auxiliary Feedwater (AFW) pumps which may render the AFW System inoperable. We have determined through engineering evaluation that if this condition remained uncorrected, the safe operation of the plant could have been adversely affected. Therefore, this issue is reportable under the provisions of 10CFR50.55(e) and the required information is as follows:

#### Description

The low suction pressure bi-stables 1PB-2475B and 1PB-2476B associated with the motor driven auxiliary feedwater pumps CP1-AFAPMD-01 and CP1-AFAPMD-02, respectively, and the pressure switches 1PS-2470A and 1PS-2470B, associated with the turbine driven auxiliary feedwater pump CP1-AFAPTD-01, can cause spurious signals. This could result in tripping the pumps due to pressure oscillations in the suction lines during pump startup. This is a significant deficiency in final design as approved and released for construction. This deficiency was discovered during the design validation process of the AFW System.

A parallel experience at the Millstone 3 facility was reported during the surveillance tests of the motor driven auxiliary feedwater pumps, as evidenced by NUSCO's Licensee Event Report (LER) 87-004-00 to the NRC.

#### Safety Implications

The auxiliary feedwater pumps are used under accident conditions when normal feedwater flow to the steam generators is not available to achieve and maintain safe plant shutdown. The described spurious trips could prevent achieving the design flows assumed in Chapter 15 (Accident Analysis) of the FSAR.

8711020490

TX-6886  
October 26, 1987  
Page 2 of 2

Corrective Action

We are currently evaluating the possibility of disconnecting the CPSES low pressure trip instrumentation.

Our next report for this issue will be submitted no later than February 23, 1988. At that time we will discuss the root cause and generic implications of this issue, the corrective action required, and the implementation schedule for the corrective action.

Very truly yours,

*W.G. Council*

W. G. Council

By: *D.R. Woodlan*

D. R. Woodlan  
Supervisor,  
Docket Licensing

MCP/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6887  
File # 10110  
910.4  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 27, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
AUXILIARY FEEDWATER DESIGN PRESSURE  
SDAR: CP-87-107 (FINAL REPORT)

Gentlemen:

On September 24, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving the calculated maximum Auxiliary Feedwater (AFW) System pressure which may exceed the design pressure of components within the AFW system. On October 27, 1987, we verbally notified your Mr. H. S. Phillips that our written report due to be submitted on October 26, 1987, would not be submitted until October 27, 1987, due to an administrative oversight. An engineering evaluation has determined that this issue is not reportable under the provisions of 10CFR50.55(e).

The design validation process of the auxiliary feedwater system and calculation 16345-ME(B)-022, Case SPDP #3, revealed that the maximum system pressure for motor driven auxiliary feedwater pumps CP1-AFAPMD-01 and CP1-AFAPMD-02 and for their bypass breakdown orifices CP1-AFORBO-01 and CP1-AFORBO-02 exceeds the design pressure of these components. Specifically, the component's design pressure is 1600 psig, while the maximum calculated system pressure, corresponding to the pump shut-off head at maximum water level in the condensate storage tank, amounts to 1613.1 psig. In case of an accident, when the motor driven pumps are required to operate, this could have led to overpressurization of the pumps and orifices.

The manufacturer of the subject pumps and orifices, Ingersoll-Rand (I-R), has confirmed that these components can sustain the calculated overpressure without damage to the components. Therefore, were this condition to remain uncorrected, it could not adversely affect the safety of operations of the plant and therefore is not reportable under the provisions of 10CFR50.55(e).

~~8711020435~~

TXX-6887  
October 27, 1987  
Page 2 of 2

No rerating of these components or other corrective action is required.

Records supporting this conclusion are available for review by your inspectors at the CPSES plant site.

Very truly yours,

W.G. Council

W. G. Council

By: D.R. Woodlan

D. R. Woodlan  
Supervisor, Docket Licensing

MCP/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6873  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 23, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
COMPONENT DESIGN SPECIFICATION DISCREPANCY  
SDAR: CP-87-106 (INTERIM REPORT)

Gentlemen:

On September 23, 1987, we verbally notified your Mr. R. F. Warnick of a deficiency involving the configuration of foundation bolting for safety-related mechanical equipment. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

Our evaluation of this issue is continuing. Our next report will be submitted no later than January 11, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By:

*D. R. Woodlan*

D. R. Woodlan  
Supervisor, Docket Licensing

CBC/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8710270389~~



Log # TXX-6985  
File # 10110  
908.3  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 18, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CABLE RACEWAY FILL CALCULATIONS:  
CABLE OUTSIDE DIAMETER  
SDAR: CP-87-98 (INTERIM REPORT)

Gentlemen:

On September 9, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving review of the Cable and Raceway Data System (CARDS) which indicates that the wrong cable diameter may have been used in calculations. Specifically, CARDS 2323-E1-1700 and 2323-E2-1700 used the nominal (versus maximum) cable diameter for calculating raceway fill. This issue was identified under the Corrective Action Program (CAP). This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report was logged TXX-6836 dated October 9, 1987.

We are continuing our evaluation and anticipate submitting our next report by February 19, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *J. S. Marshall*

\_\_\_\_\_  
J. S. Marshall  
Supervisor,  
Generic Licensing

WJH/grr

c - Mr. P. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711230171~~



Log # TXX-6883  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 27, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
118 VAC CIRCUIT BREAKER SIZING  
SDAR: CP-87-86 (FINAL REPORT)

Gentlemen:

On September 3, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving potentially undersized circuit breakers in Class 1E 118 V A.C. distribution panels. During the design validation program, a calculation has been completed which indicates three branch circuit breakers in the Class 1E 118 V A.C. instrument bus distribution panels may be undersized. Our last interim report was logged TXX-6802, dated October 5, 1987. After further evaluation we have determined that this item is not reportable under the provisions of 10CFR50.55(e). This is our final report on this issue.

Further investigation has revealed that these breakers (three branch circuit breakers located in Class 1E 118 V A.C. instrument distribution panels 1PC1, 1PC2 and 1PC4) are Class 1E breakers in series with a non-Class 1E breaker feeding a non-Class 1E load. Class 1E breaker sizing includes a factor to avoid nuisance tripping of vital loads, and was inadvertently omitted on these three breakers. However, since these breakers are feeding non-Class 1E loads, allowance to avoid nuisance tripping is not necessary.

Based on the above, the breakers, as presently sized, are adequate to assure that a fault or overload on the non-Class 1E system will not degrade the Class 1E system.

Supporting documentation is available for review by your inspectors at the CPSES plant site.

Very truly yours,

*W.G. Council*

W. G. Council

By: *D.R. Woodlan*

D. R. Woodlan

Supervisor, Docket Licensing

*2711020401*

WJH/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6942  
File # 10110  
903.8  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 13, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
EPOXY GROUT TEMPERATURES  
SDAR: CP-87-75 (INTERIM REPORT)

Gentlemen:

On August 24, 1987, we verbally notified your Mr. H. S. Phillips of contact surface temperatures that may not have been properly recorded prior to placement of epoxy type grouts. This deficiency was discovered during third party investigations and is addressed in Corrective Action Request (CAR) 80X. Our last interim report, logged TXX-6741, was submitted on September 23, 1987. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

We are continuing our evaluation of this issue and will submit our next report no later than February 2, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor,  
Docket Licensing

CBC/mbk

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*871170112*



Log # TXX-6884  
File # 10110  
903.8  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 28, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
UN SOUND CONCRETE MORTAR  
SDAR: CP-87-73 (INTERIM REPORT)

Gentlemen:

On August 24, 1987, we verbally notified your Mr. H. S. Phillips of a possible deficiency involving unsound mortar at construction joints. The conditions addressed in this issue are based upon a site initiated Corrective Action Request (CAR) 87-10 issued as a result of two Nonconformance Reports (NCRs) C-86-200154 and C-85-101580. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report on this issue was logged TXX-6739, dated September 23, 1987.

We are continuing our evaluation of this issue and will submit our next report no later than December 9, 1987.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor,  
Docket Licensing

CBC/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*8711020473*



Log # TXX-6971  
File # 10110  
903.8  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 18, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

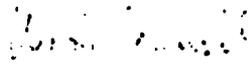
SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
HILTI BOLT INADEQUACIES  
SDAR: CP-87-68 (INTERIM REPORT)

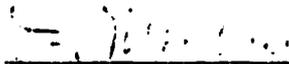
Gentlemen:

On August 24, 1987, we verbally notified your Mr. H. S. Phillips of several deficiencies involving concrete expansion anchors supplied by Hilti. The specific issues include "bottomed-out" nuts and unacceptable torque for bolts in general, improper application for bolts installed on rotating equipment, and lack of installation documentation for bolts that are inaccessible due to grout. These deficiencies were discovered during third party investigations and are addressed in Corrective Action Request, CAR 87-052. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest report was logged TXX-6735, dated October 2, 1987.

We are continuing our evaluation and anticipate submitting our next report no later than February 17, 1988.

Very truly yours,

  
W. G. Council

By:   
\_\_\_\_\_  
J. S. Marshall  
Supervisor,  
Generic Licensing

CBC/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

8711240010



Log # TAJ-6733  
File # 10110  
903.8  
Ref # 10CFR50.55(e)

September 25, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK TEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
STRUCTURAL BOLTING IN TENSION  
SDAR: CP-87-06 (INTERIM REPORT)

Gentlemen:

On August 24, 1987, we verbally notified your Mr. W. S. Phillips of a deficiency involving hand tightened bolts in structural steel connections with inadequately spoiled threads. The initial written report was due on September 23, 1987. On September 23, 1987, we verbally requested and received an extension from your Mr. P. F. Warnick until September 30, 1987. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

This deficiency was discovered during third-party investigations and addressed in Corrective Action Request (CAR-73).

For various sliding connections, structural design drawings specify hand tight bolts with jam nut or spoiled threads. The jam nut or spoiled threads are intended to restrain the nut from backing off. However, CAR-73 has identified five connections where hand tight nuts were installed as specified with spoiled threads but the bolt threads nearest the nut were left intact and the nuts could have backed off several turns before being restrained by the spoiled threads. The deficient connections could possibly transmit load through axial tension in the bolt. Should the nut back off, the connection could fail to perform as intended.

Our evaluation of this issue includes revision of Specification 2323-SS-16B, "Structural Steel/Miscellaneous Steel (Category I & II)," to include installation instructions and inspection criteria for hand tight bolts. A backfit validation will then be performed for all applicable connections to the revised installation instructions and inspection criteria. Any deficient connections that are identified will be documented on Nonconformance Reports and will be reworked to conform to the revised requirements of Specification 2323-SS-16B. The results of this backfit validation are required to determine the impact of this issue upon the safety of plant operations.

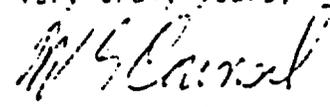
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TX-6733  
September 25, 1987  
Page 2 of 2

Currently, Design Change Authorization (DCA) 46624 has been issued to add installation instructions and inspection criteria to Specification 2003-S5-16B. To date, 285 connections involving hand tightened bolts which could be subjected to tensile loads have been identified in Unit 1 and DCA 49517 has been issued to perform backfit validation of these connections. Determination of the population of affected Unit 2 connections is currently being performed.

Our next report on this issue will be submitted no later than March 29, 1988.

Very truly yours,



W. G. Council

CBC:tgj

c- Mr. P. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6951  
File # 10110  
903.6  
Ref # 10CFR50.55(e)

November 16, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
COMPUTER MODELING ERROR: RELAP-3  
SDAR CP-87-58 (INTERIM REPORT)

Gentlemen:

On August 14, 1987, we verbally notified your Mr. H. S. Phillips of a potentially reportable item involving modeling errors in the computer code, RELAP-3. Our last interim report was logged TXX-6707 dated September 14, 1987. After further evaluation we have determined that this item is reportable under the provisions of 10CFR50.55(e) and the required information follows.

#### Description of Deficiency

The computer code RELAP-3 was used in calculating pipe rupture blowdown in the design of pipe whip restraints. A check of the RELAP-3 results for the main steam lines using RELAP-5 determined that the use of RELAP-3 resulted in miscalculation of loads applied to pipe whip restraints. A detailed review of the computer code identified a modeling error in RELAP-3. As a result, use of the RELAP-3 values may have resulted in non-conservative pipe whip restraint designs.

The cause of this deficiency is the modeling error in the RELAP-3 computer code. The deficiency extends to all pipe whip restraints designed using data provided by RELAP-3.

#### Safety Implications

Failure of pipe whip restraints could adversely affect the ability of safety-related systems to perform as required following post accident conditions, due to the effects from the increased pipe whip and jet impingement zones of influence.

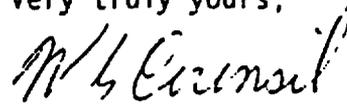
8711200079

TXX-6951  
November 16, 1987  
Page 2 of 2

Corrective Action

We are currently conducting a reanalysis of the affected pipe whip restraint designs using RELAP-5. Our corrective actions and schedule of completion for this deficiency will be submitted no later than February 3, 1988.

Very truly yours,



W. G. Council

HAM/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6929  
File # 10110  
909.4  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 16, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
COMPUTER MODELING ERROR: COMPARE MODIA  
SDAR: CP-87-57 (INTERIM REPORT)

Gentlemen:

On August 14, 1987, we verbally notified your Mr. H. S. Phillips of a potentially reportable item involving inaccurate modeling methods in the environmental analysis using the computer code COMPARE MODIA. After further evaluation, we have determined this item to be reportable under the provisions of 10CFR50.55(e), and the required information follows.

#### Description of Deficiency

Inaccurate modeling of the tornado dampers in the COMPARE MODIA computer code has been discovered which may have resulted in non-conservative room temperatures and/or pressures. The version of the code used, assumes that the damper opens at a set pressure, then relatches when the pressure falls below the set pressure. However, the tornado dampers installed in the field do not relatch when the pressure drops below the set pressure. In the unlatched condition the damper vanes allow flow in either direction in response to any applied differential pressure. This condition affects various rooms in the Auxiliary, Safeguards and Fuel Buildings in Units 1 and 2.

The cause of this deficiency was failure to adequately reflect plant design in the computer modeling. This deficiency affects only the tornado dampers; therefore, no other implications have been identified.

#### Safety Implications

In the event this deficiency had not been discovered, a preliminary evaluation has shown that post-accident pressures and/or temperatures for the affected rooms may have exceeded the design limits calculated by the computer runs. This, in turn, could have resulted in the inability of safety-related systems to perform as required following post-accident conditions.

871190272

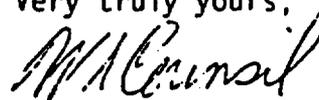
TXX-6929  
November 16, 1987  
Page 2 of 2

Corrective Action

After further evaluation we have decided to perform a 100% validation of all systems affected by this issue. This will be performed in lieu of an evaluation to determine the safety significance of this issue as a whole. This effort is not yet complete.

We will submit a follow-up report by March 1, 1988, to provide the results of our verification effort.

Very truly yours,



W. G. Council

MCP/mgt

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6965  
File # 10110  
903.6  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 13, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

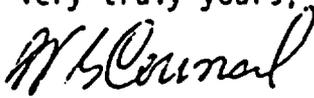
SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
DEFICIENT PIPE WHIP RESTRAINTS  
SDAR: CP-87-56 (INTERIM REPORT)

Gentlemen:

On August 7, 1987, we verbally notified your Mr. C. Hale of a deficiency involving construction problems observed in pipe whip restraints that may require extensive re-inspections and re-evaluation of the specifications. Our last interim report was logged TXX-6708, dated September 4, 1987. This is a interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

We are continuing our evaluation of the impact of this issue upon the safety of plant operations.

Our next report on this issue will be submitted no later than January 22, 1988.

Very truly yours,  
  
W. G. Council

HAM/mgt

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~871170130~~



Log # TXX-6925  
File # 10110  
908.3  
Ref: 10CFR50.55(e)

William G. Council  
Executive Vice President

November 6, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CLASS 1E MOV MOTOR STARTERS  
SDAR: CP-87-54 (INTERIM REPORT)

Gentlemen:

On July 31, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving electrical design drawings which do not apparently implement FSAR requirements for Class 1E motor-operated valve motor starters. The drawings do not provide the motor starter with thermal overloads connected only to alarms or thermal magnetic breakers to trip the starter for sustained locked motor conditions. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our most recent interim report was logged TXX-6684, dated August 31, 1987.

We are continuing our evaluation of the impact of this issue upon the safety of plant operations.

We anticipate submitting our next report by January 22, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *D. R. Woodlan*  
\_\_\_\_\_  
D. R. Woodlan  
Supervisor, Docket Licensing

WJH/tjj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~871160087~~



Log # TXX-6907  
File # 10110  
903.6  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 30, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
PIPE WHIP RESTRAINT DESIGN METHODOLOGY  
SDAR: CP-87-53 (INTERIM REPORT)

Gentlemen:

On July 29, 1987, we verbally notified your Mr. H. S. Phillips of deficiencies identified in calculations used to qualify pipe whip restraint designs. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report, logged TXX-6678, was submitted on August 28, 1987.

Our evaluation of the safety significance of this condition is continuing. We will provide our next report no later than January 29, 1988.

Very truly yours,

A handwritten signature in cursive script that reads 'W. G. Council'.

W. G. Council

BSD/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711040048~~



Log # TXX-6882  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Counsil  
Executive Vice President

October 30, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CLASS 1E BATTERY CAPABILITIES  
SDAR: CP-87-52 (INTERIM REPORT)

Gentlemen:

On July 29, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving an apparent inadequacy in vendor information used as design input which could result in unsatisfactory Class 1E battery capabilities. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report on this issue was logged TXX-6679, dated August 28, 1987.

We are continuing our evaluation of the impact of this issue upon the safety of plant operations.

Our next report on this issue will be submitted no later than January 13, 1988.

Very truly yours,

  
W. G. Counsil

WJH/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*87H050218*



Log # TXX-6954  
File # 10110  
910.4  
Ref # 10CFR50.55(e)

William G. Counsil  
Executive Vice President

November 13, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
TURBINE DRIVEN AUXILIARY FEEDWATER PUMP BEARING TEMPERATURE  
SDAR: CP-87-50 (FINAL REPORT)

Gentlemen:

On July 27, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving abnormal thrust bearing temperatures on the Turbine Driven Auxiliary Feedwater Pump. Our most recent interim report, logged TXX-6931 was submitted on November 6, 1987. After further evaluation we have determined that this is a reportable item under the provisions of 10CFR50.55(e), and the required information follows.

#### DESCRIPTION OF THE DEFICIENCY

During the preoperational test of the CPSES Unit 1 Turbine Driven Auxiliary Feedwater Pump CP1-AFAPTD-01, an excessively high thrust bearing temperature was recorded. Ingersoll-Rand (I-R), the pump manufacturer, was contacted to assist in determining the cause of the high temperatures. According to I-R, the problem of high thrust bearing temperatures is not uncommon to pump model HMTA, furnished for auxiliary feedwater service. The reason for the abnormal bearing temperature appears in part to be improper installation and centering of the balancing drum. Installation instructions in the pump manual are lacking in specific detail that would eliminate either misunderstanding of requirements or misinterpretation of design criteria.

#### SAFETY IMPLICATIONS

The Auxiliary Feedwater (AFW) Pumps function to provide emergency cooling water to the Steam Generators during or following certain events that cause the loss of main feedwater or create a need for additional makeup feedwater essential for safe plant shutdown. Although this problem was discovered during a test of the Turbine-Driven AFW Pump on Unit 1, the Motor-Driven AFW Pumps utilize the same pump model. Therefore, this condition creates the potential for the failure of all AFW pumps, possibly resulting in the complete loss of Auxiliary Feedwater. This condition affects both Units 1 and 2.

*STUTTS*

TXX-6954  
November 13, 1987  
Page 2 of 2

CORRECTIVE ACTION

The AFW pumps will be dismantled and the balancing drums installed in accordance with the recently issued I-R Technical Bulletin 60-87, which provides a more detailed description of balancing drum installation. This action is scheduled for completion by December 31, 1987, for all AFW pumps on Unit 1. Final system testing prior to fuel load will verify proper pump operation. This same corrective action will be performed on Unit 2 prior to Unit 2 fuel load.

Records supporting the implementation of corrective action on Unit 1 will be available for your inspectors to review at the CPSES site after January 31, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor, Docket Licensing

MCP/mgt

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6931  
File # 10110  
910.4  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 6, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
TURBINE DRIVEN AUXILIARY FEEDWATER PUMP  
BEARING TEMPERATURE  
SDAR: CP-87-50 (INTERIM REPORT)

Gentlemen:

On July 27, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving abnormal thrust bearing temperatures on the turbine driven auxiliary feedwater pump. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report on this issue was logged TXX-6681, dated August 31, 1987.

During startup testing of the Unit 1 turbine driven auxiliary feedwater pump (CP1-AFAPTD-01), an abnormally high temperature was identified for the thrust bearing. The pump manufacturer (Ingersoll-Rand), contacted to assist in determining the cause of the condition, indicated that the temperatures were not uncommon to the specific pump model (HMTA) and appear to be the result of improper installation and centering of the rotating element. Further, the vendor has indicated a technical bulletin is being prepared to provide more specific installation instructions in the pump manual that would eliminate misinterpretations of the installation requirements or design criteria.

We are continuing our evaluation of this issue and anticipate submitting our next report by January 15, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*

D. R. Woodlan  
Supervisor, Docket Licensing

*871130110*

MCP/tgj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6880  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 21, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SOLID STATE SAFEGUARDS SYSTEMS (SSSS) INTERNAL WIRING TERMINATIONS  
SDAR: CP-87-49 (INTERIM REPORT)

Gentlemen:

On July 22, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving a condition cited at another nuclear facility in which an open electrical connection on one crimp lug caused a failure in load sequencing equipment. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report, logged TXX-6674, was submitted August 21, 1987.

Due to unsatisfactory manufacturing techniques employed by the vendor (Vitro) prior to April, 1979, the possibility exists that internal wiring terminations in the CPSES SSSS may not meet the required quality standards necessary to assure complete reliability.

Our evaluation of this issue includes a review of the documentation for the SSSS to determine if any reinspection or rework has been performed to date. After this review an engineering validation of the sequencers' internal wiring terminations will be performed based on criteria supplied by the vendor.

Our next report on this issue will be submitted no later than January 11, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor, Docket Licensing

~~8710270237~~

VIP/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6950  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

November 13, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CONSUMABLE MATERIALS REPLACEMENT PROGRAM  
SDAR: CP-87-47 (INTERIM REPORT)

Gentlemen:

On July 17, 1987, we verbally notified your Mr. I. Barnes of an apparent inadequacy involving the program for technical approval of replacement consumables. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report was logged TXX-6645, dated August 14, 1987.

We are continuing our evaluation of this issue and will submit our next report no later than February 9, 1988.

Very truly yours,

*W.G. Council*

W. G. Council

By:   
D. R. Woodlan  
Supervisor,  
Docket Licensing

WJH/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711170275~~



Log # TXX-6869  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 16, 1987

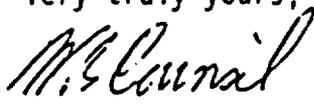
U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CONTAINMENT SPRAY PUMP MOTOR ROTOR/STATOR GAP  
SDAR: CP-87-46 (INTERIM REPORT)

Gentlemen:

On July 17, 1987, we verbally notified your Mr. Ian Barnes of a deficiency involving unsatisfactory internal clearances of the Containment Spray Pump Motors. Our last interim report, logged TXX-6641 dated August 13, 1987, erroneously identified the affected pump motors as "1" and "3". The correct motor numbers are "2" and "4". This is an interim report of this issue under the provisions of 10CFR50.55(e).

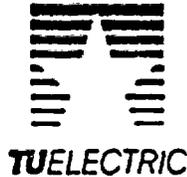
We are continuing our evaluation of this issue. Our next report on this issue will be submitted no later than December 8, 1987.

Very truly yours,  
  
W. G. Council

CBC/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8710210063~~



Log # TXX-6934  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

November 13, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
REACTOR COOLANT PUMP (RCP) MOTOR BACKUP CIRCUITRY  
SDAR: CP-87-45 (INTERIM REPORT)

Gentlemen:

On July 17, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving control logic and control power for the electrical breakers supplying the RCP motors. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

Our evaluation of this issue is continuing. We expect to submit our next report by January 25, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor, Docket Licensing

VIP/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711170186~~



Log # TXX-6878  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 23, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
UNISTRUT TUBING SUPPORT BOLTING  
SDAR: CP-87-44 (INTERIM REPORT)

Gentlemen:

On July 17, 1987, we verbally notified your Mr. Ian Barnes of a deficiency involving Specification CPES-1-1018 (Installation of Piping/Tubing and Instrumentation) which allows the use of either ASTM A-307, Grade A or B or SAE J-429, Grade 1 or 2 bolting in tubing supports. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report was logged TXX-6644, dated August 17, 1987.

The majority of safety-related tubing supports are fabricated utilizing typical Unistrut members and joint configurations. Unistrut Catalog 10R (dated 1986) requires the use of SAE J-429 Grade 2 bolts. The torquing requirements specified by Unistrut for SAE J-429, Grade 2 bolts results in a tensile stress which could cause "permanent set" if ASTM A-307, Grade A or B bolts are used instead of SAE J-429, Grade 2. These conditions could result in unpredictable bolting clamping forces and unknown load limits.

Our evaluation of this condition is continuing. Our next report on this issue will be submitted no later than January 7, 1988.

Very truly yours,

A handwritten signature in cursive script, appearing to read "W. G. Council", is written over the typed name.  
W. G. Council

DAR/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8210270215~~



Log # TXX-6862  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Counsil  
Executive Vice President

October 16, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SOLENOID VALVES INSULATION  
SDAR: CP-87-43 (INTERIM REPORT)

Gentlemen:

On July 15, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving cracked insulation coatings associated with the solenoid valves on the feed water isolation valves. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report, logged TXX-6647, was submitted on August 13, 1987.

We are continuing our evaluation of the impact of this issue upon the safety of plant operations.

Our next report on this issue will be submitted no later than December 8, 1987.

Very truly yours,

W. G. Counsil

HJH/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8710220017~~



Log # TXX-6863  
File # 10110  
909.5  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 16, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
FIRE DETECTION SYSTEM PC BOARDS  
SDAR: CP-87-38 (INTERIM REPORT)

Gentlemen:

On July 15, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving possible excessive current damage to the annunciator circuitry of the printed circuit boards (PC boards) for the Fire Detection System. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report was logged TXX-6651, dated August 14, 1987.

We are continuing our evaluation of the impact of this issue upon the safety of plant operations.

Our next report on this issue will be submitted no later than January 15, 1988.

Very truly yours,

A handwritten signature in cursive script, appearing to read 'W. G. Council'.

W. G. Council

JDS/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8710200619~~



Log = TXX-6904  
File = 10110  
908.3  
Ref = 10CFR50.55(e)

William G. Council  
Executive Vice President

October 30, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
RAYCHEM MOTOR CONNECTION KITS  
SDAR: CP-87-35 (INTERIM REPORT)

Gentlemen:

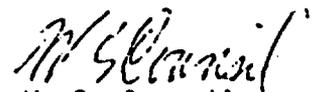
On July 15, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving Raychem Motor Connection Kits (Model No. MMCK-2V-35-00) which were provided to CPSES with shims which may not be compatible with the use range requirements of the supplied breakout components. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report was logged TXX-6648, dated August 13, 1987.

Motor connection kits are designed to insulate and environmentally seal Class 1E cable connections to low voltage motors (rated 1000 volts or less). The kit in question is used when the two conductors are side by side then bolted. To seal this type of assembly, a breakout, which looks like a pair of trousers, seals its legs around each conductor, then a cap is placed over the assembly. Since the motor conductor is typically much smaller than the field cable, a shim is used on the motor lead to increase the diameter to meet the breakout use range.

The Raychem Motor Connection Kits MMCK-2V-35-00, Lot Number K0770 and K1245 were supplied with the wrong size shim. The shim supplied was too large and would result in the breakout leg being held out at a diameter outside the qualified use range. Our evaluation of the acceptability of these kits is continuing.

Our next report for this issue will be submitted no later than January 12, 1988.

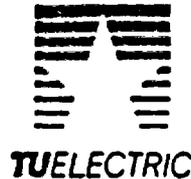
Very truly yours,

  
W. G. Council

~~8111030505~~

HJH/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6919  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 2, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
EQUIPMENT NOZZLE LOAD INTERFACES  
SDAR: CP-87-34 (FINAL REPORT)

Gentlemen:

On July 10, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving equipment nozzle load interfaces for Units 1 and 2. Our last interim report was logged TXX-6906 dated October 30, 1987. After further evaluation we have determined that this item is not reportable under the provisions of 10CFR50.55(e). This is our final report on this issue.

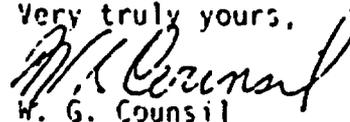
During evaluation of the loads generated at equipment nozzles in the pipe stress analysis, the proper interface was not consistently identified between the pipe stress analysis organization (SWEC-PSAS) and the equipment supplier (Westinghouse).

This deficiency is the result of inconsistent definition of the point of the application of the loads. The analyzing organization assumed all allowable loads were applicable at the pipe nozzle interface. This assumption may not always yield conservative results for the equipment where piping loads are compared at the nozzle to head junction.

One hundred thirty one (131) equipment nozzles were identified as affected by this condition. Engineering evaluation has shown that for all cases the new loads resulting from the correct interface were within acceptable limits. Therefore, in the event this condition had remained uncorrected, no adverse affect on the safety of the plant could have resulted.

Records supporting this conclusion are available for review by your inspectors at the CPSES plant site.

Very truly yours,



W. G. Council

~~8711060018~~

RSB/mgt

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # IXX-6906  
File # 10110  
903.2  
Ref: 10CFR50.55(e)

William G. Council  
Executive Vice President

October 30, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
EQUIPMENT NOZZLE LOAD INTERFACES  
SDAR: CP-87-34 (INTERIM REPORT)

Gentlemen:

On July 10, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving equipment nozzle load interfaces for Units 1 and 2. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report, logged TXX-6630, was submitted August 10, 1987.

During evaluation of the loads generated at equipment nozzles in the pipe stress analysis, the proper interface may not have been consistently identified between the pipe stress analysis organization (SWEC-PSAS) and the equipment supplier (Westinghouse).

Further evaluation is required to determine the impact of these conditions. We will submit the next report by November 20, 1987.

Very truly yours,

  
W. G. Council

RSB/mlh

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~87-11090-558~~



Log # TXX-6847  
File # 10110  
917.1  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 19, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
QUALITY CONTROL INSPECTOR CERTIFICATION  
SDAR: CP-87-32 (INTERIM REPORT)

Gentlemen:

On July 8, 1987, we verbally notified your Mr. R. F. Warnick of a deficiency involving apparent weaknesses in our Quality Control Inspector Certification Program. Specifically, procedures for establishing QC Inspector certification criteria did not address such matters as removal of unqualified personnel, reevaluation of inactive personnel, certification data, physical characteristics examinations, and Level III capabilities. Our last interim report was logged TXX-6624, dated August 7, 1987. On September 17, 1987, we verbally notified your Mr. H. S. Phillips that this issue was reportable under the provisions of 10CFR50.55(e). We also requested and received an extension to October 19, 1987, for submittal of this written report. After further evaluation we have determined that our initial conclusion of reportability was premature. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e).

We are continuing our evaluation of the impact of this issue on the safety of plant operations and will submit our next report no later than January 28, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*

D. R. Woodlan  
Supervisor,  
Docket Licensing

~~8710270304~~

DAR/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6871  
File # 10110  
903.5  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 16, 1987

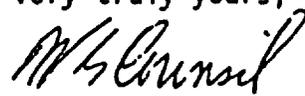
U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
ROLL-AWAY MISSILE SHIELDS (RAMS)  
SDAR: CP-87-30 (INTERIM REPORT)

Gentlemen:

On July 1, 1987, we verbally notified your Mr. Shannon Phillips of a deficiency involving errors in the seismic qualification report supplied by the vendor of the Roll-Away Missile Shields (RAMS). A preliminary review of these errors indicated that the RAMS were overstressed for the designed seismic conditions detailed in the applicable specifications. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last report was logged TXX-6601, dated July 28, 1987.

We are continuing our evaluation of this issue and will submit our next report by January 5, 1988.

Very truly yours,  
  
W. G. Council

CBC/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8710210071~~



Log # TXX-6890  
File # 10110  
909.5  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 26, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
DIESEL GENERATOR FUEL OIL TRANSFER PUMP SUCTION LIFT  
SDAR: CP-87-26 (INTERIM REPORT)

Gentlemen:

On June 29, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving the diesel generator fuel oil transfer pump. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last report on this issue was logged TXX-6695, dated August 27, 1987.

We are continuing our evaluation of the impact of this issue upon the safety of plant operations. Our next report on this issue will be submitted no later than December 14, 1987.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor,  
Docket Licensing

MCP/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

8711020229



Log # TXX-6908  
File # 10110  
903.8  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 27, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445  
LINER ATTACHMENT MATERIAL REQUIREMENTS: IMPACT TESTING  
SDAR: CP-87-22 (FINAL REPORT)

Gentlemen:

On June 17, 1987, we verbally notified your Mr. Shannon Phillips of a deficiency wherein FSAR (Section 3.8.1) and Specification (2323-SS-14) requirements regarding impact testing of materials attached to the containment liner have not been implemented. Our last interim report, logged TXX-6694, was submitted on August 31, 1987.

An engineering analysis has determined that this condition is not reportable under the provisions of 10CFR50.55(e). Specifically, a fatigue/fracture analysis was performed to determine if a postulated crack in the containment liner attachments would impair the integrity of the containment liner. The analysis of postulated cracks in the heat affected zone of the fillet welds between the liner and attachment plate shows that the liner integrity would not be compromised even if the attachment plate had a lower toughness than the liner. Thus, the intended safety function of the liner is maintained even though no impact tests on the attachments were performed.

The cause of this condition was determined to be a failure to sufficiently clarify the requirements of ASME-ACI 359 and the FSAR commitment in the original specification for attachments to the liner. Specification 2323/SS-14, "Containment Steel Liner," was modified (via Design Change Authorization (DCA) 49089) to clarify the requirements of attachments to the reactor containment liner and to stipulate impact testing of liner attachment materials greater than 5/8 inch in thickness.

~~8711020475~~

TXX-6908  
October 27, 1987  
Page 2 of 2

Records supporting these conclusions are available for review by your inspectors at the CPSES plant site.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor, Docket Licensing

BSD/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6891  
File # 10110  
903.8  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 26, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NO. 50-445  
LINER ATTACHMENT MATERIAL REQUIREMENTS: IMPACT TESTING  
SDAR: CP-87-22 (INTERIM REPORT)

Gentlemen:

On June 17, 1987, we verbally notified your Mr. Shannon Phillips of a deficiency wherein FSAR (Section 3.8.1) and Specification (2323-SS-14) requirements regarding impact testing of materials attached to the containment liner have not been implemented. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report, logged TXX-6694, was submitted on August 31, 1987.

Our evaluation of this issue is continuing. We will provide our next report no later than December 18, 1987.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor, Docket Licensing

BSD/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*8711020460*



Log # TXX-6892  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Counsil  
Executive Vice President

October 28, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
EFFECT OF THERMOLAG ON DERATING FACTORS  
SDAR: CP-87-21 (INTERIM REPORT)

Gentlemen:

On June 17, 1987, we verbally notified your Mr. H. S. Phillips of a deficiency involving recent evaluations which have established derating factors of 31% for single trays and 20% for single conduits enclosed in thermolag. These derating factors are larger than the derating factor (10%) used in the initial cable sizing calculations. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report, logged TXX-6650, was submitted August 17, 1987.

We are continuing our evaluation of this issue and will submit our next report no later than January 13, 1988.

Very truly yours,

*W.G. Counsil*  
W. G. Counsil

By: *D.R. Woodlan*  
D. R. Woodlan  
Supervisor, Docket Licensing

JDS/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711020337~~



Log # TXX-6927  
File # 10110  
909.2  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 6, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
COMPONENT COOLING WATER (CCW) HEAT EXCHANGER  
SEISMIC QUALIFICATION  
SDAR: CP-87-18 (FINAL REPORT)

Gentlemen:

On June 8, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving errors identified during a review of vendor qualification documentation for the CCW heat exchangers that indicated the seismic and nozzle loading conditions exceed specification requirements and could be overstressed. Our most recent report was logged TXX-6716, dated September 11, 1987.

The reanalysis of the Unit 1 & 2 heat exchanger utilizing the as-built piping nozzle loads have been completed. Calculations IMT-CA-EQ-0119-MS-49 for Unit 1 and IMT-CA-EQ-154-MS-49 for Unit 2 demonstrates that the heat exchangers are acceptable under the as-built loading without modification.

In addition, we have also completed an evaluation of the identified vendor qualification deficiencies. These deficiencies, which involve the vendor's methodology used in establishing the seismic qualification of the component, could affect other nuclear facilities. Calculation IMT-CA-EQ-0155-MS-49 demonstrated that qualification deficiencies discovered in vendor report No. 74-06-32467, "Seismic Qualification of CCW Heat Exchanger," resulted in the effected equipment not meeting the design requirements specified in Mechanical Specification 2323-MS-049, "CCW Heat Exchanger." Had the Comanche Peak as-built piping loads on this heat exchanger not been less than the purchase specification values, the heat exchanger would have been overstressed. Loss of this heat exchanger following a seismic event would represent a substantial safety hazard. This report completes our notification of a possible deficiency in the vendor design processes. This item would be reported under the provisions of 10CFR21 but the NRC has been adequately informed of the deficiency by previous TU Electric letters concerning this SDAR.

*871160287*

TXX-6927  
November 6, 1987  
Page 2 of 2

We have concluded in the event this issue had remained undetected, no condition adverse to safety would have existed. This issue is not reportable under the provisions of 10CFR50.55(e). Records supporting this conclusion are available for your inspectors to review at the CPSES Project Site.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor, Docket Licensing

MCP/mgt

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6962  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 13, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
AIR ACCUMULATORS FOR CONTROL VALVES  
SDAR: CP-87-15 (INTERIM REPORT)

Gentlemen:

On May 20, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving the air accumulators for air-operated control valves. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report was logged as TXX-6662, dated August 20, 1987.

Our evaluation of this issue is continuing. We expect to submit our next report by January 22, 1988.

Very truly yours,

A handwritten signature in dark ink, appearing to read 'W. G. Council', written in a cursive style.

W. G. Council

MCP/mgt

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*871170142*



Log # TXX-6866  
File # 10110  
917  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 21, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CONTROL OF DESIGN MODIFICATIONS  
SDAR: CP-87-10 (INTERIM REPORT)

Gentlemen:

On May 11, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving design change document control measures which may not have been adequately established prior to the implementation of the current design modification program. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report, logged TXX-6688, was submitted on August 28, 1987.

At this time, design documents noted in this deficiency (safety related Field Change Requests & Engineering Change Notices) are being reviewed to establish the technical adequacy of these documents and the possible effects of these changes on the safety and maintainability of plant operations.

This review is being conducted using the validated design established through the respective Corrective Action Programs. Our next report will be submitted upon completion of this review and is scheduled for January 12, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By:

*D. R. Woodlan*

D. R. Woodlan

Supervisor, Docket Licensing

DAR/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8110270354~~



Log # TXX-6870  
File # 10110  
907  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 21, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CALIBRATION ACCURACY OF PRESSURE STANDARD  
(DEAD WEIGHT TESTER)  
SDAR: CP-87-08 (INTERIM REPORT)

Gentlemen:

On April 20, 1987 we verbally notified your Mr. Cliff Hale of a deficiency regarding the calibration accuracy for a Dead Weight Tester used as a standard in calibrating pressure instrumentation. This tester has been used in the Operations (Instrumentation & Controls) test/maintenance program. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest report was logged TXX-6574, dated July 21, 1987.

As discussed in our last report, a total of seven calibration standards have been identified that did not meet the required calibration accuracy. We have completed reviews to identify post maintenance tests, post design modification tests, hot functional tests and preoperational tests associated with safety systems that utilized test instruments affected by the seven calibration standards. We are currently evaluating the impact of the inaccuracies on the affected tests.

~~8710270201~~

TXX-6870  
October 21, 1987  
Page 2 of 2

Our evaluation of this issue is being conducted in conjunction with Corrective Action Report (CAR) 87-053. We will submit our next report on this issue by January 6, 1988.

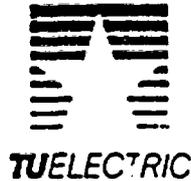
Very truly yours.

*W. G. Council*  
W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor,  
Docket Licensing

BSD/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6881  
File # 10110  
903.10  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 21, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SEISMIC QUALIFICATION OF AS-BUILT 480 VOLT  
POWER CENTERS  
SDAR: CP-87-06 (FINAL REPORT)

Gentlemen:

On April 6, 1987, we verbally notified your Mr. C. Hale of a deficiency involving the 480 volt, 2000 KVA dry type transformers. Our last interim report was logged TXX-6706, dated September 4, 1987. After further evaluation we have determined that this item is not reportable under the provisions of 10CFR50.55(e). This is our final report on this issue.

The transformers were originally seismically qualified by the vendor (Westinghouse) with the primary feed high voltage cables routed through the bottom of the transformer enclosure. In the CPSES installed configuration, the feed cables were routed through the top of the enclosure.

Effects of the top entry were evaluated using finite element analysis (Ref: Impell calculation IMT-CA-EQ-0214-ES06, Rev. 0) which included the additional weight due to the top entry. Stresses were evaluated locally (near the entry) and globally (in the cabinet), and were found to comply with the AISC Manual of Steel Construction limits. Therefore, top entry has no impact on the transformer qualification and no modifications are necessary.

~~8710270384~~

TXX-6881  
October 21, 1987  
Page 2 of 2

Documents supporting this conclusion are available for review by your inspectors at the CPSES plant site.

Very truly yours,

*W.G. Council*  
W. G. Council

By: *D.R. Woodlan*  
D. R. Woodlan  
Supervisor,  
Docket Licensing

WJH/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6988  
File # 10110  
903.11  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 18, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
ENVIRONMENTAL QUALIFICATION OF CABLE TO  
POST-LOCA HIGH RANGE RADIATION MONITOR  
SDAR: CP-87-05 (INTERIM REPORT)

Gentlemen:

On April 6, 1987, we verbally notified you, Mr. C. Hale of a deficiency involving the failure of High Range Radiation Monitor (HRRM) coaxial cables to meet the minimum cable environmental qualification for dielectric insulation requirements. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our most recent interim report was logged TXX-6578, dated July 21, 1987.

We are continuing our evaluation and anticipate submitting our next report on this issue no later than February 19, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *J. S. Marshall*  
\_\_\_\_\_  
J. S. Marshall  
Supervisor,  
Generic Licensing

WJH/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711230459~~



Log # TXX-6963  
File # 10110  
909.2  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 13, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
STATIC COMPUTER MODEL FOR THE SAFEGUARDS BUILDING  
SDAR: CP-87-01 (FINAL REPORT)

Gentlemen:

On January 14, 1987, we verbally notified your Mr. I. Barnes of a deficiency involving an error in the finite element computer model for the Safeguards Building. Our most recent report, logged TXX-6914, was submitted November 3, 1987.

In order to assess these conditions, a series of calculations have been prepared for the portions of the Safeguards Building which could have been affected by the error. This analysis has concluded the design of these areas is acceptable. Our review of other analyses has indicated no other building analysis contained similar errors.

We have concluded in the event this issue had remained uncorrected, no condition adverse to safety would have existed. This issue is not reportable under the provisions of 10CFR50.55(e). Records supporting this conclusion are available for your inspectors review at the CPSES Project Site.

Very truly yours,

A handwritten signature in cursive script, appearing to read "W. G. Council".

W. G. Council

MCP/mgt

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6914  
File # 10110  
909.2  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 11, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
STATIC COMPUTER MODEL FOR THE SAFEGUARDS BUILDING  
SDAR: CP-87-01 (INTERIM REPORT)

Gentlemen:

On January 14, 1987 we verbally notified your Mr. I. Barnes of a deficiency involving an error in the finite element computer model for the Safeguards Building. The error involves a fixed support point at the roof elevation where no such support exists. This condition produces an incorrect load distribution in the computer model, which will require a reevaluation of the structural members in the building. Our latest interim report, logged TXX-6701, was submitted on September 3, 1987.

We are continuing our evaluation of the impact of this issue upon the safety of plant operations. Our next report on this issue will be submitted no later than January 18, 1988.

Very truly yours,

A handwritten signature in cursive script, appearing to read "W. G. Council", is written over the typed name.

W. G. Council

MCP/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711100487~~



Log # TXX-6918  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

November 6, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION  
DOCKET NOS. 50-445 AND 50-446  
SEPARATION BARRIER MATERIAL ON POWER CABLES AND  
POWER RACEWAYS  
SDAR: CP-86-83 (INTERIM REPORT)

Gentlemen:

On June 5, 1987, we notified you by our letter logged TXX-6840 of a deficiency involving the use of Separation Barrier Material (SBM) on power cables and power raceways which we deemed reportable under the provisions of 10CFR50.55(e). This is an interim report submitted to status corrective actions implemented to date. Our latest interim report was logged TXX-6690, dated August 28, 1987.

Our effort to define the construction schedule for this issue is continuing. We will submit the next report no later than January 18, 1988.

Very truly yours,

*W.G. Council*

W. G. Council

By: *D.R. Woodlan*

D. R. Woodlan  
Supervisor, Docket Licensing

WJH:tgj

c-Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711130945~~



Log # TXX-6982  
File # 10110  
907.6  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 18, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
BOP SAFETY-RELATED INSTRUMENT SETPOINTS  
SDAR: CP-86-81 (INTERIM REPORT)

Gentlemen:

On November 7, 1986, we verbally notified your Mr. T. Westerman of a potentially reportable item involving the calculations for BOP safety-related instrument setpoints, indicating that omissions of required data or use of incorrect information occurred in performing the calculations required per Regulatory Guide 1.105, Rev. 2. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report was logged TXX-6758, dated September 18, 1987.

We are continuing our evaluation of this issue and will submit our next report no later than February 18, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *J. S. Marshall*

J. S. Marshall  
Supervisor,  
Generic Licensing

RDD/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711240026~~



Log # TXX-6910  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 6, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CABLE PULLING TENSION  
SOAR: CP-86-71 (INTERIM REPORT)

Gentlemen:

On October 2, 1986, we verbally notified your Mr. I. Barnes of a deficiency regarding electrical cable pulling. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our most recent interim report was logged TXX-6666, dated August 21, 1987.

Our review of cable pull calculations, described in our interim report logged TXX-6530, has been completed and the resulting revised cable pulling charts and cable installation criteria have been incorporated into Specification 2323-ES-100, "Electrical Installation Specification." We are continuing our evaluation of "worst case pull-bys" (cable pulls in conduits 20 feet or longer containing 10 or more cables and filled to 20 percent or more capacity) for raceways in harsh environments and the highest ranked (per above criteria) in mild environments. Cable pull calculations will be performed for these pull-bys. Our evaluation will include a historical search and will concentrate on determining if the cables were installed in a controlled manner within acceptable tension limits.

Our evaluation for reportability will be performed following the completion of the above effort. We expect to submit our next report on this issue by January 11, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*

D. R. Woodlan

Supervisor, Docket Licensing

WJH/mgt

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*87-1130245*



Log # TXX-6867  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 16, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
WEATHER PROTECTION FOR CLASS 1E COMPONENTS  
SDAR: CP-86-68 (INTERIM REPORT)

Gentlemen:

On October 2, 1986, we verbally notified your Mr. I. Barnes of a deficiency regarding provisions for weather protection of Class 1E components used in outdoor installations. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our most recent interim report was logged TXX-6698, dated August 28, 1987.

We are continuing our evaluation and will submit our next report by January 12, 1988.

Very truly yours,

A handwritten signature in dark ink, appearing to read 'W. G. Council', written in a cursive style.

W. G. Council

WJH/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6874  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 21, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
PRE-OPERATIONAL VIBRATIONAL TEST CRITERIA  
SDAR: CP-86-67 (INTERIM REPORT)

Gentlemen:

On March 19, 1987, we notified you by our letter logged TXX-6327 of a reportable deficiency regarding the design calculations and tests used to qualify the Pre-Operational Piping Vibration Program. This is an interim report of a reportable item under the provisions of 10CFR50.55(e).

Specifically, a review of the CPSES criteria document "Pre-Operational Vibration Test Program," Issue 1, dated June 1980, indicated that the mathematical formulas used to determine stress endurance limits, allowable deflections, and flexibility characteristics of certain piping systems may not have been accurate.

We are continuing our evaluation of this issue. Our next report will be submitted no later than January 22, 1988.

Very truly yours,

*W. G. Council*

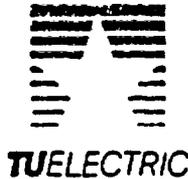
W. G. Council

By: *D. R. Woodlan*  
\_\_\_\_\_  
D. R. Woodlan  
Supervisor, Docket Licensing

BSD/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

7710280148



Log # TXA-6935  
File # 10110  
903.7  
Ref # 10CFR50.55(e)

November 11, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SEISMIC AIR GAP DESIGN ADEQUACY  
SDAR: CP-86-55 (INTERIM REPORT)

Gentlemen:

On August 14, 1986, we verbally notified your Mr. I. Barnes of a deficiency involving the seismic air gap design adequacy between the Auxiliary Building and Units 1 and 2 Containment Buildings. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report was logged TXA-6720, dated September 11, 1987.

We are continuing the development of the alternate calculations and these calculations are currently in the final stages of review. After the completion of these alternate calculations, a conclusion will be made regarding the safety of plant operations had this condition gone uncorrected.

Our next report on this issue will be submitted no later than January 8, 1988.

Very truly yours,

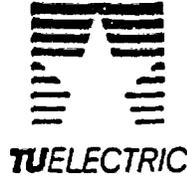
W. G. Council

By:   
D. R. Woodlan  
Supervisor, Docket Licensing

JCH/mgt

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~871170190~~



Log # TXX-6966  
File # 10110  
903.10  
Ref # 10CFR50.55(e)

November 13, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SEISMIC DESIGN OF CONDUIT  
SDAR: CP-86-53 (FINAL REPORT)

Gentlemen:

On October 21, 1986, we notified you by our letter, logged Txx-6054, of a deficiency involving the seismic design of conduit, which we deemed reportable under the provisions of 10CFR50.55(e). Our latest report on this issue was: TXX-6913, dated November 4, 1987.

The additional testing performed to evaluate the ability of the fiber glass cloth/resin material to resist torsional moments has been completed. The test results indicate that the fiberglass cloth/resin material will resist the torsional moments and is an acceptable solution for use outside containment. As an alternative, conduit supports may be substituted as required on a case by case basis for the fiberglass cloth/resin material to resist the torsional moments.

Installation of the fiberglass cloth/resin material or optional conduit supports is currently scheduled for completion by April 20, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *P. P. Hurdell*

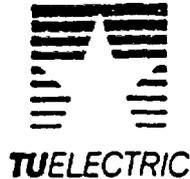
P. P. Hurdell

Supervisor, Docket Licensing

RSB:tgj

c-Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*871170177*



Log # TXX-6913  
File # 10110  
903.10  
Ref # 10CFR50.55(e)

William G. Counsil  
Executive Vice President

November 4, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SEISMIC DESIGN OF CONDUIT  
SDAR: CP-86-53 (INTERIM REPORT)

Gentlemen:

On October 21, 1986, we notified you by our letter, logged TXX-6054, of a deficiency involving the seismic design of conduit, which we deemed reportable under the provisions of 10CFR50.55(e). Our last report, logged TXX-6727 was submitted on September 17, 1987. This is an interim report submitted to status corrective action implemented to date.

In our last report (TXX-6727) we noted that the additional testing performed to evaluate the ability of the fiber glass cloth/resin material to resist torsional moments had been completed. Currently, we are continuing our analysis of the test results to determine the required corrective actions for the affected conduits.

Our next report on this issue will be submitted no later than December 4, 1987.

Very truly yours,

*W. G. Counsil*  
W. G. Counsil

By: *D. R. Woodlan*  
\_\_\_\_\_  
D. R. Woodlan  
Supervisor, Docket Licensing

RSB/grr

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*874410155*



Log # TXX-6699  
File # 10110  
903.11  
Ref: 10CFR50.55(e)

August 28, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CABLE TRAY SPLICES/CONNECTIONS  
SDAR: CP-86-52 (INTERIM REPORT)

Gentlemen:

On April 29, 1986, we notified you by our letter, logged TXX-6415, that the deficiency involving splice/connections used for cable trays in Units 1 and 2 was reportable under the provisions of 10CFR50.55(e). This is an interim report to status the corrective actions to date.

#### ENGINEERING EVALUATION

Walkdowns have been performed to gather data concerning Unit 1 and 2 splice connections. Engineering evaluation of walkdown data resulted in the identification of one unacceptable splice type (Z-type) and several splice types which required testing.

Trays in Unit 1 are manufactured by T. J. Cope; Unit 2 trays are a mixture of Burndy-Husky and T. J. Cope. Testing of T. J. Cope splice connections has been completed, and the test results have been evaluated. No notable difference in the T. J. Cope tray has been identified between Units 1 and 2. Results of testing and evaluation are therefore applicable to both Units 1 and 2.

Test results and evaluation indicate that some splice configurations under worst case loading conditions have not exhibited expected capacities. Field installations for these splice types will be evaluated based on actual load conditions. Any installations which cannot be qualified through evaluation will be modified as required in accordance with approved site procedures.

Further tests are currently being scheduled to provide for evaluation of Burndy-Husky splice connections.

~~8709010321~~

TXX-6699  
August 28, 1987  
Page 2

CORRECTIVE ACTION

The following measures have been implemented to prevent the misuse of splice plates in Units 1 and 2.

- 1) Design Change Authorizations (DCAs) have been issued to provide criteria per specifications ES-100, "Electrical Erection," via DCA 31431 and ES-19, "Cable Tray Specifications," via DCA 34029. Previously generated splice plate DCAs have been voided to prevent use.
- 2) Revisions to electrical construction procedure ECP-10, "Cable Tray Hanger Installation, Unit 1," and ECP-10A, "Cable Tray and Hanger Installation, Unit 2," have been prepared. These revisions, however, are on hold pending revision to Specifications ES-100 and ES-19/ES-19A.

NCRs have been generated to identify and initiate corrective action for installed "Z" type splices.

Upon completion of testing, we will perform evaluation of test results and inform you of any additional corrective action required. We will submit the next report on this issue no later than October 23, 1987.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*

D. R. Woodlan  
Supervisor, Docket Licensing

RSB/mlh

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6893  
File # 10110  
903.9  
903.11  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 21, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CABLE TRAY SPLICES/CONNECTIONS  
AND FIELD DRILLED CABLE TRAY HOLES  
SDAR: CP-86-52 (INTERIM REPORT)

Gentlemen:

On April 29, 1987, by our letter logged TXX-6415, we notified you that the deficiency involving splice/connections used for cable trays in Units 1 and 2 (SDAR CP-86-52, "Cable Tray Splices/Connections") was reportable under the provisions of 10CFR50.55(e). On September 25, 1987, by our letter logged TXX-6763, we notified you that the deficiency involving field drilled cable tray holes (SDAR CP-87-76, "Field Drilled Cable Tray Holes") was reportable under the provisions of 10CFR50.55(e). Our last report on SDAR CP-86-52 was logged TXX-6699, dated August 28, 1987. Our last report on SDAR CP-87-76 was logged TXX-6763, dated September 25, 1987. All subsequent reports for SDAR CP-86-52 will address the implementation of corrective actions for SDAR CP-87-76. This is an interim report to status the implementation of corrective actions.

Our evaluation of this issue including prescribed corrective actions is continuing. We will submit our next report on this issue no later than January 11, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*

D. R. Woodlan

Supervisor, Docket Licensing

CBC/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*8710280042*



Log # TXX-6868  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 20, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
ANCHOR BOLTS SUPPLIED BY HILTI  
SDAR: CP-86-51 (INTERIM REPORT)

Gentlemen:

On July 17, 1986 we verbally notified your Mr. I. Barnes of a deficiency involving a condition cited at another nuclear facility in which anchor bolts, supplied by Hilti do not meet average ultimate tensile loads in certain sizes as published by the supplier. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report, logged TXX-6552, was submitted on July 8, 1987.

A test procedure is being developed for testing of the 1/2" diameter anchor bolts in CPSES concrete to determine the ultimate tensile capacity. The procedure is scheduled for release in late October. Upon completion of testing, the results will be assessed to determine the impact of this issue upon the safety of plant operations.

Our next report on this issue will be submitted no later than January 12, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

By: *D. R. Woodlan*

D. R. Woodlan  
Supervisor,  
Docket Licensing

GLB/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8410210317~~



Log # TXX-6947  
File # 10110  
10110.1  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 17, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
ADEQUACY OF NONCONFORMANCE DISPOSITIONS  
SDAR: CP-86-48 (INTERIM REPORT)

Gentlemen:

On June 16, 1986, we verbally notified your Mr. C. Hale of a deficiency involving the adequacy of nonconformance dispositions. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report on this issue, logged TXX-6560, was submitted on July 15, 1987.

We are continuing our evaluation of the impact of this issue upon the safety of plant operations. Our next report on this issue will be submitted no later than February 17, 1988.

Very truly yours,

*W. G. Council*

W. G. Council

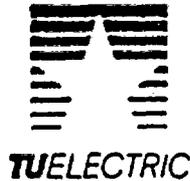
By:

*J. S. Marshall*  
\_\_\_\_\_  
J. S. Marshall  
Supervisor, Generic Licensing

GLB/mgt

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711240156~~



Log # TXX-6812  
File # 10110  
903.7  
Ref # 10CFR50.55(e)

William G. Counsell  
Executive Vice President

October 16, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SEISMIC CATEGORY II SYSTEMS AND COMPONENTS  
SDAR: CP-86-45 (INTERIM REPORT)

Gentlemen:

On June 6, 1986, we verbally notified your Mr. T. F. Westerman of a possible deficiency involving the interaction of Seismic Category II systems and components with Seismic Category I installations. We have concluded that this issue is reportable under the provisions of 10CFR50.55(e) and the required information follows.

DESCRIPTION

The FSAR (section 3.2) defines Seismic Category II installations as systems or components whose continued function is not required but whose inability to withstand seismic loads could reduce the function of any Seismic Category I system or component.

FSAR (section 3.7B.2.8) states that "Non-Category I equipment and components located in seismic Category I buildings are investigated by analysis or testing, or both, to ensure that under the prescribed earthquake loading, structural integrity is maintained, or the non-Category I equipment and components do not adversely affect the integrity or the operability, or both, of any designated seismic Category I structure, equipment, or component to the extent that these seismic Category I items cannot perform their required functions."

Contrary to the above FSAR requirements, during interdisciplinary review of a proposed Design Change Authorization (DCA) involving Seismic Category II handrails (DCA 21458), the Systems Interaction Program (SIP) observed that only the anchoring details were qualified. The integrity of the handrail structure was not considered.

A review of Seismic Category II equipment at CPSES has concluded that the Seismic Category II systems, structures and components can be classified in two groups. One group consists of seismically adequate equipment with seismically adequate anchors, and the other consists of equipment with seismically adequate anchors only.

~~8710200448~~

### SAFETY IMPLICATIONS

The condition represents a significant deficiency in final design as approved and released for construction. Had the condition remained uncorrected, it could have adversely affected the safety of plant operations.

Failure of Seismic Category II systems and components to perform as required during a seismic event could result in the failure of Seismic Category I systems and components to perform as required during a seismic event.

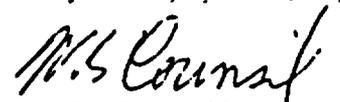
### CORRECTIVE ACTION

Our corrective action for this issue consists of the following:

- 1) Engineering validation to determine the extent of Seismic Category II installations which could interact with Seismic Category I installations is being performed in accordance with ECE 2.24, "Systems Interaction Program." Architectural features (includes handrails, gypsum board walls, doors and security barriers, ladders, grating, checkered plate, and stairs) were identified as having possible adverse interactions with seismic Category I installations. The safety significance evaluation of these items is being performed in SDAR CP-85-29, "Design of Architectural Features". SDAR CP-85-29 defines a program of evaluation, and in some cases, seismic qualification of these features.
- 2) Installations identified with potentially unacceptable interactions will be evaluated for acceptability per ECE 2.24 and DBD-ME-005, "Seismic/Non-seismic Systems Interaction Program."
- 3) Any remaining unresolved interactions will be evaluated for acceptability using an established earthquake experience data base and/or existing component seismic qualification records and anchorage design calculations.
- 4) Any interactions not resolved using the above methods will be evaluated further and DCAs will be issued for modifications when required.

A definitive schedule for completion of our corrective actions will be provided in our next report. Our next report on this issue will be submitted no later than January 8, 1988.

Very truly yours,

  
W. G. Council

JDS/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6986  
File # 10110  
903.11  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 13, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
APPLICATION OF NON-QUALIFIED AGASTAT RELAYS  
SDAR: CP-86-40 (INTERIM REPORT)

Gentlemen:

On October 10, 1986, we notified you by our letter logged TXX-6017 of a deficiency involving the procurement and use of non-qualified Agastat relays in several Class 1E applications which we deemed reportable under the provisions of 10CFR50.55(e). This is an interim report to status the implementation of corrective actions. Our latest report was logged TXX-6762, dated September 18, 1987.

Implementation of the rework associated with the corrective actions is still in progress. We will submit our next report by February 19, 1988.

Very truly yours,

*William G. Council*  
W. G. Council

By: *J. S. Marshall*  
\_\_\_\_\_  
J. S. Marshall  
Supervisor,  
Generic Licensing

WJH/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*8711230317*



Log # TXX-6924  
File # 10110  
908.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 6, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CLASS 1E BATTERY CHARGER COMPONENTS  
SDAR: CP-86-37 (INTERIM REPORT)

Gentlemen:

On May 14, 1986, we verbally notified your Mr. I. Barnes of a deficiency involving firing board assemblies and amplifier boards used in Class 1E battery chargers that do not conform to vendor assembly drawings. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our most recent interim report was logged IXX-6700, dated September 2, 1987.

We are continuing our evaluation and anticipate submitting our next report by January 22, 1988.

Very truly yours,

*W.G. Council*

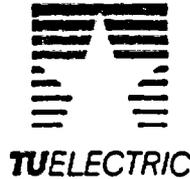
W. G. Council

By: *D.R. Woodlan*  
D. R. Woodlan  
Supervisor, Docket Licensing

WJH/tgj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~87H130308~~



Log # TXX-6900  
File # 10110  
903.11  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 30, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50 445 AND 50-446  
THRU WALL EMBEDDED CONDUIT SLEEVES  
SDAR: CP-86-32 (INTERIM REPORT)

Gentlemen:

On April 21, 1986, we verbally notified your Mr. T. F. Westerman of a deficiency involving thru wall embedded conduit sleeves which have not been identified by number and may be overfilled. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our latest interim report, logged TXX-6404, was submitted April 24, 1987.

Collection of the Unit 1 as-built data is complete. To date, 39 Nonconformance reports (NCRs) have been initiated for 133 overfilled sleeves.

We are continuing our evaluation of this issue with regards to cable derating, cable pulling tension, fire seal integrity, divisional train separation, system interaction and installation verification criteria.

Our next report on this issue will be submitted no later than January 11, 1988.

Very truly yours,

A handwritten signature in cursive script that reads "W. G. Council".  
W. G. Council

RSB/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6865  
File # 10110  
909.4  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 21, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SAFEGUARDS AREA FAN COOLERS  
SDAR: CP-86-22 (INTERIM REPORT)

Gentlemen:

On July 8, 1987, by our letter logged TXX-6551, we notified you of a deficiency involving the capacity of the present fan cooler design in the safeguards area to adequately handle the heat load in several rooms, which we deemed reportable under the provisions of 10CFR50.55(e). This is an interim report to status the corrective actions implemented to date.

The Unit 1 chilled water rebalancing is scheduled for completion by February 28, 1988.

The project has completed evaluation of Start-Up Test Procedures and has prepared a review matrix identifying reviewing disciplines/contractors who are required to review these procedures for various systems. Accordingly Ebasco will review Start-Up Test Procedures for all systems within its responsibility, including Safety Chilled Water System for Safeguards Area Fan Coolers. This will assure that the acceptance criteria specified in the procedure properly reflects the design basis calculation requirements.

Unit 2 calculations are scheduled for completion by March 31, 1988. The necessary revisions to the Unit 2 design documents will be completed by June 30, 1988. Any corrective action required for Unit 2 will be identified upon completion of the calculations.

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TXX-6865  
October 21, 1987  
Page 2 of 2

Our next report for this issue will be submitted no later than April 22, 1988.

Very truly yours,

*W.G. Council*

W. G. Council

By:

*D.R. Woodlan*

---

D. R. Woodlan  
Supervisor,  
Docket Licensing

GLB/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6875  
File # 10110  
903.6  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 16, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
JET IMPINGEMENT LOAD REVIEW  
SDAR: CP-86-13 (INTERIM REPORT)

Gentlemen:

On March 5, 1986, we verbally notified your Mr. T. F. Westerman of a deficiency involving a computer entry error which could invalidate portions of the jet impingement load review for high energy line breaks. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our last interim report, logged TXX-6670, was submitted on August 19, 1987.

We are continuing our evaluation of this issue and anticipate submitting our next report no later than February 15, 1988.

Very truly yours,

W. G. Council

CBC/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8710200624~~



Log # TXX-6960  
File # 10110  
909.2  
Ref # 10CFR50.55(e)

November 17, 1987

William G. Council  
Executive Vice President

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
COMPONENT COOLING WATER PUMP IMPELLER LINEAR INDICATIONS  
SDAR: CP-86-1! (FINAL REPORT)

Gentlemen:

On June 23, 1986, we notified you by our letter logged TXX-4871 of a deficiency involving linear indication of impellers for the Component Cooling Water (CCW) pumps which we deemed reportable under the provisions of 10CFR50.55(e). This is a final report submitted to status corrective actions implemented to date. Our last report was logged TXX-6682, dated September 4, 1987.

All CCW pump impellers for units 1 and 2 have been inspected and corrective action to repair the linear indications has been completed. This completes our corrective action on this issue.

Supporting documentation is available at the CPSES site for your Inspectors' review.

Very truly yours,

*W. G. Council*

W. G. Council

By: *J. S. Marshall*  
\_\_\_\_\_  
J. S. Marshall  
Supervisor, Generic Licensing

GLB/mgt

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

*8711230183*



Log # TXX-6889  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 19, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
INSTALLATION OF SUPER HILTI KWIK BOLT  
SDAR: CP-86-08 (INTERIM REPORT)

Gentlemen:

On February 5, 1987, we verbally notified your Mr. T. F. Westerman of a deficiency involving the control of stamps used to identify Super Hilti Kwik Bolts (HKB). This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our previous report was logged TXX-6591 dated July 22, 1987.

We are continuing our evaluation of this issue and will submit our next report by December 9, 1987.

Very truly yours,

*W. G. Council*

W. G. Council

By: *J. S. Marshall*

J. S. Marshall  
Supervisor,  
Generic Licensing

CBC/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8710260217~~



Log # TXX-6920  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 11, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NO. 50-445  
THERMOLAG ON CONDUIT SUPPORT  
SDAR: CP-85-42 (INTERIM REPORT)

Gentlemen:

On October 21, 1986, we notified you by our letter, logged TXX-6049, of a deficiency involving the possible adverse effect of the substitution of rectangular and oversized pre-formed sections of thermolag on conduit installations which we deemed to be reportable under the provisions of 10CFR50.55(e). Our last interim report logged TXX-6703, was submitted on September 9, 1987. This report is submitted to status the implementation of corrective actions.

We are continuing the implementation of our corrective actions and will submit our next report no later than February 3, 1988.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *D. R. Woodlan*  
D. R. Woodlan  
Supervisor,  
Docket Licensing

DAR/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~871170096~~



Log # TXX-6940  
File # 10110  
903.10  
Ref: 10CFR50.55(e)

William G. Council  
Executive Vice President

November 9, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
EQUIPMENT CONDUIT INTERFACE  
SDAR: CP-85-39 (INTERIM REPORT)

Gentlemen:

On September 18, 1986, we verbally notified your Mr. T. F. Westerman of a deficiency involving conduit which was not installed in accordance with design documents. This is an interim report of a potentially reportable item under the provisions of 10CFR50.55(e). Our most recent interim report was logged TXX-6726, dated September 15, 1987.

Our evaluation of this issue is continuing. We will submit the next report no later than January 25, 1988.

Very truly yours,

A handwritten signature in cursive script, appearing to read "W. G. Council".

W. G. Council

RSB/mlh

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6928  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 5, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
TRAIN "C" CONDUIT SUPPORTS  
SDAR: CP-85-36 (INTERIM REPORT)

Gentlemen:

On September 6, 1985, we verbally notified your Mr. C. Gould of a deficiency involving the installation of non-safety related conduit (two-inch diameter and smaller) on non-safety related dead weight supports in Category I areas. Our last interim report, logged TXX-6556, was submitted on July 7, 1987. After further evaluation we have determined that this item is reportable under the provisions of 10CFR50.55(e) and the required information follows.

Description of Deficiency

During the construction review conducted by the NRC Technical Review Team (TRT), the TRT examined the installation of non-safety related conduit supports in selected Category I areas of the plant. The support installation for non-safety related conduits less than or equal to 2 inches was inconsistent with seismic requirements. Further, no evidence was observed which substantiated the adequacy of the installation for non-safety related conduit of any size. According to Regulatory Guide 1.29 and the CPSES FSAR Section 3.7B.2.8, the Seismic Category II and non-seismic items should be designed in such a way that their failure would not adversely affect the function of safety-related components or cause injury to plant personnel. TU Electric was directed to propose a program that would assure the adequacy of the seismic support system installation for non-safety related conduit in all seismic Category I areas of the plant. The following was recommended:

- Provide the results of seismic analysis which demonstrates that all non-safety related conduits and their support systems, satisfy the provisions of Regulatory Guide 1.29 and FSAR Section 3.7B.2.8.
- Verify that non-safety related conduit less than or equal to 2 inches in diameter, not installed in accordance with the requirements of Regulatory Guide 1.29, satisfy applicable design requirements.

871100372

### Safety Implications

A Train C Corrective Action Program (CAP) was initiated for the Unit 1 and Common Areas in order to comply with the NRC direction. Under this program, to date, approximately 77,000 (73% of total) conduit supports have been evaluated and approximately 600 modifications have been made to comply with FSAR and Regulatory Guide 1.29 requirements. Since our evaluation has determined that hardware modifications were necessary, this item was determined to be reportable under the provisions of 10CFR50.55(e).

### Corrective Action

Currently, we are implementing the Train C Corrective Action Program (CAP) for the Unit 1 and Common Areas. The design validation process has been completed with the following results:

1. All Validation Procedures are in place for the resolution of all related CPRT issues (Issue Specific Action Plan I.c). These procedures have been reviewed and concurred with by the Third Party and are final (except for minor editorial and efficiency enhancement revisions which are ongoing as the program continues).
2. All DIR's (51) which resulted from the Third Party's review of Impell's Train C Validation Procedures have been resolved.
3. All complex (unique) supports have been evaluated and modifications identified and validated.
4. To date, approximately 77,000 conduit supports have been evaluated in the Unit 1 and Common Area. This represents about 73% of all supports in the Unit 1 and Common Area. This effort has resulted in approximately 600 modifications. The Post Construction Hardware Validation Program (PCHVP) for these conduits and supports was implemented concurrently with the design validation process.
5. The remaining 28,000 supports are single (generic) supports for which the design has been validated. These supports are currently being physically validated under the PCHVP Program.
6. A long term maintenance program has been established to provide for a review of all new construction activities which would affect previously validated areas. The review will be performed in accordance with the criteria established in the Train C Corrective Action Program.

TXX-6928  
November 5, 1987  
Page 3 of 3

The scheduled completion date for the remaining PCHVP activities and associated design modifications of the Unit 1 and Common Area is December 31, 1987. Our next report for the Unit 2 activities will be submitted no later than March 1, 1988.

Very truly yours,

*W.G. Council*  
W. G. Council

By: *D.R. Woodlan*  
\_\_\_\_\_  
D. R. Woodlan  
Supervisor, Docket Licensing

DAR/grr

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6968  
File # 10110  
903.9  
Ref. # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 16, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
CABLE HANGER DESIGN  
SDAR: CP-85-35 (INTERIM REPORT)

Gentlemen:

On October 21, 1986, we notified you by our letter, logged TXX-6048, of a deficiency involving the design and construction activities of the cable tray support program which we deemed reportable under the provisions of 10CFR50.55(e). Our latest interim report, logged TXX-6561, was submitted on July 7, 1987.

Our implementation of Corrective Actions are continuing. We anticipate submitting our next report on this issue no later than January 25, 1988.

Very truly yours,

A handwritten signature in black ink, appearing to read 'W. G. Council', is written over the typed name.  
W. G. Council

DAR/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8711190088~~



Log # TXX-6864  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

October 16, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NO. 50-445  
ELECTRICAL RACEWAY SUPPORT SYSTEM  
SDAR: CP-85-31 (INTERIM REPORT)

Gentlemen:

On December 5, 1986, by our letter logged TXX-6138, we notified you of a deficiency involving the Unit 1 Class 1E Electrical Raceway Support System which we deemed reportable under the provisions of 10CFR50.55(e). This is an interim report submitted to status corrective actions implemented to date. Our latest report, logged TXX-6689, was submitted August 28, 1987.

This deficiency involves the installation of Separation Barrier Material (SBM) and Radiant Energy Shield (RES) material on Unit 1 Class 1E Electrical Conduit Raceways without engineering interdisciplinary design review of the proposed support system change.

All SBM will be removed from power cables/raceways except where designated for RES purposes in the containment buildings (via NCR CE-87-4577).

The modification and addition of supports affected by the installation of RES is continuing.

Based on our current projected schedule, our next report for this issue will be submitted no later than January 6, 1988.

Very truly yours,

A handwritten signature in cursive script that reads 'W. G. Council'.

W. G. Council

DAR/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

~~8110200384~~



Log # TXX-6831  
File # 10110  
903.9  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 2, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, DC 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NO. 50-446  
UNAUTHORIZED SUPPORT REPAIRS  
SDAR: CP-85-14 (SUPPLEMENTAL REPORT)

Gentlemen:

On April 19, 1985, in our report logged TXX-4465, we notified you of a deficiency regarding an uninvolved employee that identified unauthorized repairs made to a hanger. We determined this issue to be reportable under the provisions of 10CFR50.55(e). This item was subsequently identified by our letter logged TXX-6435, dated May 13, 1987, as an item for which a final report had been issued. However, the originally scheduled completion date for the implementation of corrective actions has elapsed but the implementation of those actions was still in progress. This report is submitted to status the implementation of those corrective actions.

Rework for pipe support FW-2-096-432-C62R, originally scheduled for May 1, 1985, was completed January 13, 1986. Rework for cable tray supports CTH-11549 and CTH-9760, originally scheduled for May 1, 1985, was completed March 20, 1986. The delay in completion of the rework for these supports can be attributed to deferred engineering and construction activities for Unit 2 in support of Unit 1.

This report concludes our evaluation and corrective action for this issue. Supporting documentation is available for your review at the CPSES project site.

Very truly yours,

A handwritten signature in cursive script that reads "W. G. Council".  
W. G. Council

CBC/lp

c - R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6828  
File # 10110  
907.3  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 10, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
UNDETECTABLE FAILURE IN ENGINEERED SAFETY  
FEATURES ACTUATION SYSTEM  
SDAR CP-85-13 (SUPPLEMENTAL REPORT)

Gentlemen:

On April 15, 1985, in our report logged TXX-4457, we notified you of a deficiency regarding failures existing in our Engineered Safety Features Actuation Systems (ESFAS) which we deemed reportable under the provisions of 10CFR50.55(e). This item was subsequently identified by our last report logged TXX-6435, dated May 13, 1987, as an item for which a final report had been issued but the corrective actions required revision. This report is submitted to status the implementation of those corrective actions.

In our report TXX-4457, we indicated our corrective actions would involve a revision to our test procedures to incorporate the CPSES specific testing requirements recommended by Westinghouse. Since that report, this deficiency has been resolved by Westinghouse's submittal of hardware modifications (Field Changes TBM-10639 and TCXM-10601) which have been implemented by Design Modification DM-86-013 and Design Change Authorization 23881 for Units 1 and 2 respectively. The modification consists of a new switch and meter arrangement that provides correct results under all test conditions. The installation of the new circuit has been completed.

Due to the implementation of these hardware modifications, revision of the test procedures as described in TXX-4457 is no longer required.

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TXX-6828  
November 10, 1987  
Page 2 of 2

Supporting documentation is available at the CPSES site for your inspector's review.

Very truly yours,

*W. G. Council*  
W. G. Council

By: *J. S. Marshall*  
J. S. Marshall  
Supervisor, Generic Licensing

CBC/grr

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TXX-6922  
File # 10110  
907.2  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

November 10, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D. C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
SOLID STATE DETECTION SYSTEM UNDETECTABLE FAILURE  
SDAR: CP-82-09 (SUPPLEMENTAL REPORT)

Gentlemen:

On November 30, 1982, in our report logged TXX-3595, we notified you of a deficiency involving Solid State Protection System (SSPS) undetectable failure which we deemed reportable under the provisions of 10CFR50.55(e). This issue was subsequently identified by our report logged TXX-6435, dated May 13, 1987, as an item for which a final report had been issued but, the corrective actions required revision. This report is submitted to status the implementation of those corrective actions.

In our report TXX-3595, we indicated our corrective actions would involve a revision to our test procedures to incorporate the CPSES specific testing requirements recommended by Westinghouse. Since that report, Westinghouse has submitted instructions to modify the SSPS master relay test panel to provide positive detection of re-establishment of the slave relay coil circuits after testing. These instructions were transmitted via Field Change Notices TBXM-10605A and TCXM-10578 and were implemented via Design Change Authorizations 18348 (Unit 1) and 19853 (Unit 2). Rework for Unit 1 and Unit 2 was completed on August 16, 1983, and June 26, 1987 respectively.

Due to the implementation of these hardware modifications, revision of our test procedures as described in TXX-3595 is no longer required.

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TXX-6922  
November 10, 1987  
Page 2 of 2

This concludes our evaluation and corrective action for this issue.  
Supporting documentation is available for your inspectors to review at the  
CPSES site.

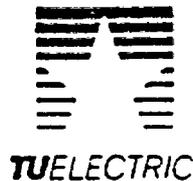
Very truly yours,

*W. G. Council*  
W. G. Council

By: *J. S. Marshall*  
J. S. Marshall  
Supervisor,  
Generic Licensing

CBC/gj

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)



Log # TX-6560  
File # 10110  
909.5  
Ref # 10CFR50.55(e)

William G. Council  
Executive Vice President

September 25, 1987

U. S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NO. 50-446  
DIESEL GENERATOR AUXILIARY SKID  
SDAP: CP-82-06 (SUPPLEMENTAL REPORT)

Gentlemen:

On January 10, 1984, in our report logged TX-4095, we notified you of a deficiency regarding linear indications identified in structural steel members of the Unit 2 emergency diesel generator auxiliary skid which we deemed reportable under the provisions of 10CFR50.55(e). This item was subsequently identified by our letter logged TX-6435, dated May 13, 1987 as an item for which a final report had been issued. However, the originally scheduled completion date for the implementation of corrective actions has elapsed and the implementation of those actions is still in progress. This report is submitted to status the implementation of those corrective actions.

In describing our corrective actions (TX-4095), we advised that the existing skids be scrapped and replacement skids fabricated on site "no later than September, 1984." Currently, corrective actions have been implemented on the train "B" diesel generator skid although a number of associated nonconformance reports are pending closure. These issues are currently scheduled for resolution by November 18, 1987. No further action is required for this item.

The auxiliary skid replacement for the train "A" diesel generator is complete. Documentation for the train "A" diesel generator is available for your inspectors review at the CPSES site. No additional reports are anticipated.

Very truly yours,

A handwritten signature in cursive script that reads 'W. G. Council'.

W. G. Council

CBC/mgt

c - Mr. P. D. Martin, Region IV  
Resident Inspectors, CPSES (3)

8709300021



Log # TXX-6711  
File # 10110  
203.9  
Ref: 10CFR50.55(e)

William G. Council  
Executive Vice President

September 3, 1987

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES)  
DOCKET NOS. 50-445 AND 50-446  
DIESEL GENERATOR PIPE SUPPORTS  
SDAR: CP-80-09 (SUPPLEMENTAL REPORT)

Gentlemen:

On July 29, 1981, in our report logged TXX-3374, we notified you of a deficiency involving the component supports associated with the auxiliary piping systems for the CPSES Diesel Generators which we deemed reportable under the provisions of 10CFR50.55(e). This item was subsequently identified by our last report logged TXX-6435, dated May 13, 1987, as an item for which a final report had been issued but the original completion date for the implementation of corrective actions has elapsed but the implementation of those actions is still in progress. This report is submitted to status the implementation of corrective actions.

In our report TXX-3415 dated October 2, 1981, we advised you that all corrective actions would be completed prior to fuel delivery to site.

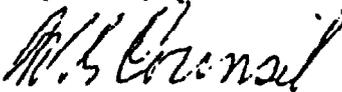
Corrective action implementation for the Unit 1 "A" and "B" train diesel generator pipe supports was completed on June 30, 1983. The Unit 2 "A" train diesel generator pipe supports were completed on January 15, 1986. Rework of the Unit 2 "B" train diesel generator pipe supports has been completed, although Nonconformance Report (NCR) M85-201621 related to these supports is scheduled for closure by November 18, 1987.

Delay in the completion of the rework for the Unit 2 diesel generator pipe supports can be attributed to deferred engineering and construction activities in Unit 2 in support of Unit 1.

TXX-6711  
September 3, 1987  
Page 2

This report concludes our evaluation and corrective actions for this issue. Supporting documentation is available for your inspectors to review at the CPSES project site.

Very truly yours,

  
W. G. Council

CBC/mlh

c - Mr. R. D. Martin, Region IV  
Resident Inspectors, CPSES (3)