

WOLF CREEK

NUCLEAR OPERATING CORPORATION

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MAR 10 2004

RA 04-0035

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: Docket No. 50-482: Changes to a Wolf Creek Generating Station Radiological Emergency Response Plan Implementing Procedure.

Gentlemen:

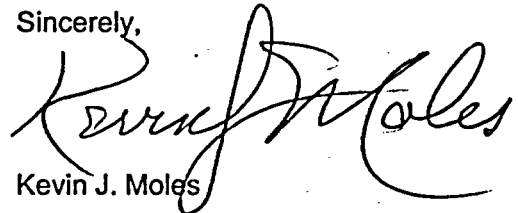
In accordance with 10 CFR 50, Appendix E, enclosed is a revision to a Wolf Creek Generating Station Radiological Emergency Response Plan implementing procedure. The following procedure is enclosed.

PROCEDURE

Effective February 11, 2004
EPP 06-012, Revision 6 (corrected copy)

If you have any questions concerning this submittal, please contact me at (620) 364-4126 or Ms. Jennifer Yunk at (620) 364-4272.

Sincerely,



Kevin J. Moles

KJM/rlg

Enclosures

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A045

CORRECTED COPY 02-11-2004



EPP 06-012

DOSE ASSESSMENT

Responsible Manager

Superintendent Emergency Planning

| | |
|-----------------------------------|-----------|
| Revision Number | 6 |
| Use Category | Reference |
| Administrative Controls Procedure | No |
| Infrequently Performed Procedure | No |
| Program Number | 06 |

DC2 02/18/03

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1.0 PURPOSE

1.1 This procedure provides guidance for determining release rates and for estimating off-site dose to the Whole Body and Thyroid.

2.0 SCOPE

2.1 The estimated release rate, total release values, off-site dose rates, and integrated doses to the Whole Body and Thyroid, are used in conjunction with EPP 06-006, PROTECTIVE ACTION RECOMMENDATIONS, as one basis for determining off-site protective actions to be recommended to State and County Officials.

3.0 REFERENCES AND COMMITMENTS

3.1 References

- 3.1.1 CHS AX-G01, SAMPLING OF UNIT AND RADWASTE VENTS FOR RADIOACTIVE GAS AND TRITIUM
- 3.1.2 EPP 06-006, PROTECTIVE ACTION RECOMMENDATIONS
- 3.1.3 EPP 06-009, DRILLS AND EXERCISE REQUIREMENTS
- 3.1.4 EPP 06-011, EMERGENCY TEAM FORMATION AND CONTROL
- 3.1.5 EPP 06-013, EXPOSURE CONTROL AND PERSONNEL PROTECTION
- 3.1.6 Radiological Emergency Response Plan (RERP)
- 3.1.7 Regulatory Guide 1.109, Calculation Of Annual Doses To Man From Routine Release Of Reactor Effluents For The Purpose Of Evaluating Compliance With 10CFR50, Appendix I, (Rev. 1, October, 1977)
- 3.1.8 Regulatory Guide 1.111, Methods For Estimating Atmospheric Transport And Dispersion Of Gaseous Effluents In Routine Releases From Light Water Cooled Reactors, (Rev. 1, July 1977)
- 3.1.9 Regulatory Guide 1.145, Atmospheric Dispersion Models For Potential Accident Consequence Assessments At Nuclear Power Plants, (August, 1979)
- 3.1.10 Regulatory Guide 1.23, Meteorological Programs In Support Of Nuclear Power Plants, (September, 1980)
- 3.1.11 Regulatory Guide 1.4, Assumptions Used For Evaluating The Potential Radiological Consequences Of A Loss Of Coolant Accident For Pressurized Water Reactors, (Rev. 2, June 1974)

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3.2 Commitments

3.2.1 ITIP 00101 (SOER 83-02, Recommendation R12), Ensure Estimates Of Dose Can Be Made For Two-Phase Or Liquid Releases Though S/G Safety And Relief Valves.

4.0 DEFINITIONS

4.1 Emergency Planning Zone (EPZ)

4.1.1 The area around WCGS in which emergency preparedness planning is conducted. The plume exposure EPZ has a radius of approximately 10 miles. The ingestion exposure pathway EPZ has a radius of about 50 miles.

4.2 Exclusion Area

4.2.1 That area within a 1200-meter radius surrounding WCGS in which WCNOG has the authority to determine all activities including exclusion or removal of persons and property from the area.

4.3 Integrated Dose

4.3.1 The amount of ionizing radiation that has been received during a given period of time by a population or group.

4.4 Pasquill Atmospheric Stability Classifications

4.4.1 Are measures of the stability or instability of an air mass based upon the vertical temperature differential between two points.

4.5 Projected Dose

4.5.1 The amount of ionizing radiation that is likely to be received by a population or group if no protective action measures are implemented.

4.6 Projected Integrated Dose

4.6.1 The summation of the Integrated Dose (previous) and the Projected Dose (future).

4.7 Protective Actions

4.7.1 Those emergency measures taken to minimize or prevent radiological exposures to personnel.

4.8 Release Rate

4.8.1 The quantity of radioactive material released to the environment expressed in curies per second (Ci/sec).

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4.9 Source Term

4.9.1 The calculated quantity of radioactive material available for or being released to the environment.

4.10 X/Q

4.10.1 A factor based on meteorological dispersion characteristics which relates atmospheric radionuclide release rates to offsite air concentrations.

4.11 Nuclear Plant Instrument System (NPIS)

4.11.1 A plant monitoring tool designed to view critical systems and components during normal and accident conditions.

4.12 Dose Assessment Program

4.12.1 A computer program developed at Wolf Creek designed to use site-specific source terms in the performance of Dose Assessment during an accident condition.

5.0 RESPONSIBILITIES

5.1 Shift Manager

5.1.1 Prior to activation of the Emergency Operations Facility (EOF), assures the Shift Chemist implements this procedure.

5.2 Radiological Coordinator

5.2.1 IF vent monitor(s) are inoperable, THEN consider dispatching Plant Team(s) to collect appropriate samples.

5.3 Shift Chemist

5.3.1 At the declaration of an ALERT or higher emergency classification reports to the Control Room to perform emergency dose calculations in accordance with this procedure.

5.4 Dose Assessment Coordinator

5.4.2 Recommends that Offsite Monitoring Teams be dispatched to determine offsite dose rates in accordance with EPP 06-011, EMERGENCY TEAM FORMATION AND CONTROL.

5.4.3 Informs the appropriate TSC or EOF management of the dose rate and projected integrated TEDE and Thyroid doses.

5.5 Dose Assessment Technician

5.5.1 Performs emergency dose calculations in accordance with this procedure.

6.0 PRECAUTIONS/LIMITATIONS

6.1 To confirm that the correct version of the Dose Assessment Program is in use, open the Dose Assessment Program, then click on 'Help' and 'Help About'. The correct version currently in use is Rev. 3.6. If the correct version is not loaded on your computer, it should be removed from your hard drive.

6.2 Offsite dose projection calculations should be performed at least once per hour during the first eight hours after the accident unless it is determined that releases of airborne radioactivity from the plant have been terminated.

NOTE

Use 15 minute MET data averages to determine if changes are in progress.

6.3 Offsite dose projection calculations may be updated anytime it is deemed necessary. Offsite dose projection calculations should be updated if any of the following conditions occur:

6.3.1 Release rate increases by more than 25 percent.

6.3.2 Wind direction changes by more than 22.5°.

6.3.3 Atmospheric stability classification changes.

6.3.4 Wind speed changes by more than 50 percent.

6.3.5 Prior to any planned releases.

6.4 IF a radiological release is already in progress before a dose assessment calculation is performed, THEN be sure to look at historical release data / trend on the NPIS to determine the maximum release rate, monitor readings, and meteorological conditions.

6.4.1 IF this is not done THEN an under estimation of an emergency dose projection can occur.

7.0 PROCEDURE

7.1 Program Description

NOTES

- o Tab and Shift Tab key manipulations may be used to move through a Model Screen.
- o Commonly practiced window manipulations may also be used to move through the program.

7.1.1 The following models may be selected by selecting the appropriate tab in the upper right hand corner of the program window.

1. Release Rate Model
2. Design Basis Accident (DBA)
3. SG Tube Rupture
4. Radiation Monitoring System
5. Field Team Data

7.1.2 Information

1. Selection of the INFORMATION heading on the tool bar allows access to the following screens:
 - a. Dose Projection Report/Dose by Subzone
 - b. Source Term
2. The Dose Projection Report/Dose by Subzone and Model Screen are two separate program windows and can both be visible at the same time, subject to limitations of screen resolution, and size.
 - a. The Model Screen includes:
 - 1) MET data section
 - 2) Release data section
 - 3) Performed/Verified signature section
 - 4) Release start time
 - 5) Calculation result section:

- a) Particulate, Noble Gas and Iodine release rates.
 - b) Projected Centerline Dose Segment - the results of the data entered above but not summed.
- 6) PAR section which is based on the Projected Dose Segment as well as the summed doses.
- a) Only evacuation recommended subzones are listed.
- b. Dose Projection Report/Dose by Subzone Screen includes:
- 1) Dose Rate to the Whole Body and Thyroid for Exclusion Area Boundary (EAB), 2, 5, and 10 miles in Roentgen per hour (R/hr).
 - 2) Plume arrival time in minutes for EAB, 2, 5, and 10 miles based on wind speed.
 - 3) Estimated hours until evacuation necessary for EAB, 1 REM TEDE or 1 REM thyroid.
 - 4) A list of both TEDE and Thyroid Dose for each subzone.
3. The source term option allows manipulation of DCF information.
- a. The source term enables the user to alter the distribution from the USAR Gap and default activities.
 - 1) Selection of the Activity heading on the source term screen tool bar allows the user to zero all activities for manual entry or to return to USAR Gap activities.
 - 2) Selection of the File heading on the source term screen tool bar allows for data file manipulation.

NOTE

If the containment spray is selected, the program will inquire whether the spray has been on for 30 minutes or more. If the spray has been on for 30 minutes or more, the filtration factor will be utilized; if not, the filtration factor will not be applied.

- b. Two additional nuclide distribution factors are available on the source term screen, HEPA filters and Containment Spray.
 - 1) A "Y" entry in the HEPA Filter Box reduces the Iodine Activity 90%. That is, 10% of the Iodine activity is released to the public.
 - 2) A "Y" entry in the Containment Spray Box reduces the Iodine Activity available for release by 75%. That is, 25% of the Iodine activity is released to the public.
 - 3) If both HEPA Filter and Containment Spray are answered "Yes", the Iodine Activity used in the offsite dose projections is reduced to 2.5% of its original activity level.
 - 4) Prior to performing real time calculations, the user must remember to check the source term screen values to ensure projection source term values are appropriate.
4. PARs selection from the Information Menu Bar provides information for review of Protective Action Recommendations.

NOTE

The notification form can only be printed if THE DOSE ASSESSMENT PROGRAM is running from the LAN.

5. The File Menu bar provides options to print the Notification form and calculation worksheet.

7.1.3 Data

1. Selection of Data from the Menu Bar allows selection of the following actions:
 - a. Sort Dose by Subzone

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- b. Sum Dose
- c. Perform Calculations
- d. Long Range Calculations

2. The Sort Dose by Subzone and Sum Dose actions are self-explanatory.

7.1.4 Calculations

1. The offsite doses will be calculated using the data displayed on the Model Screen.

7.1.5 Long Range Calculations

1. The offsite doses, and farthest evacuation distance will be calculated using the data displayed on the Model Screen.

7.2 Program Use

7.2.1 The Dose Assessment Program will normally be operated from an Icon on the desktop. The program is also available at K:\EDCP\EDCP.EXE.

7.2.2 Select a Release Model from the tabs in the upper right hand corner of the program screen.

7.2.3 Dose calculations may now be performed. Menu items necessary for operation of the program are selected from the Menu Bar.

NOTE

On a total loss of offsite power, certain radiation monitors are still available. See ATTACHMENT B for more information.

7.2.4 Obtain the following information:

- 1. Plant Status
- 2. MET data
- 3. Process Monitor data
- 4. Effluent Flow rate data

-OR-

5. If no data is available perform a DESIGN BASIS RCS LOCA using:

- a. DBA Release Rate
- b. Unfiltered Release Pathway
- c. Stability Class D for daytime or Stability Class F for night time

-OR-

- d. If the accident is deemed to be outside of Design Basis and is rapidly escalating, recommend to the Emergency Manager to use EPP 06-006, PROTECTIVE ACTION RECOMMENDATIONS.

7.2.5 Dose Assessment Program MET Information

1. Wind speed can be input as mph, kph, or mps by double-clicking within the box surrounding the input description until the appropriate description is displayed.
2. Projected release duration and time since reactor trip can both be input as hrs., mins., or days by double-clicking within the box surrounding the input description until the appropriate description is displayed.
3. A Stability Class-Wind Speed/Weather Conditions Help Screen is available by double-clicking within the stability class input field.
 - a. The user may generate a stability class by selecting the appropriate weather condition and inputting the proper wind speed.
 - b. The generated stability class is returned to the Model Screen by selecting FILE EXIT.

7.2.6 Dose Assessment Program Model Operations

1. Steps 7.2.7 through 7.2.11 contain information regarding data entry specific to each model

7.2.7 Option One, Release Rate Model

1. This model allows the user to input Gaseous and Iodine release rates in Ci/sec.
2. The following instructions may be useful in operating the Release Rate Model:
 - a. Gaseous Release Rate may be changed to Total Release Rate by double-clicking within the box surrounding the Gaseous Release Rate.

- 1) Likewise, the display can be changed to Gaseous Release Rate from Total Release Rate by double-clicking within the box surrounding the total release rate.
- b. Iodine Release Rate may be changed to a ratio by double-clicking within the box surrounding the Iodine Release Rate.
- 1) IF the ratio is known, THEN the value can be entered.
 - 2) If the ratio is unknown, a Help Screen may be displayed by double-clicking within the input field for the iodine ratio.
 - 3) Once the user selects the appropriate ratio from the list, FILE EXIT is used to return to the Model Page of the report.
 - 4) The display may be changed back to Iodine Release Rate by double-clicking within the box surrounding Iodine/Noble Gas Ratio.
- c. IF a leak rate (gal/min) and activity ($\mu\text{Ci/cc}$) is known or can be estimated, THEN the following calculation could be used to determine a release rate:

$$\left(\frac{\mu\text{Ci}}{\text{cc}}\right)\left(\frac{\text{gal}}{\text{min}}\right)\left(\frac{\text{min}}{60\text{s}}\right)\left(\frac{3.785\text{L}}{\text{gal}}\right)\left(\frac{1000\text{cc}}{\text{L}}\right)\left(\frac{\text{Ci}}{1\text{E}6\mu\text{Ci}}\right) = \frac{\text{Ci}}{\text{s}}$$

7.2.8 Option Two, Design Basis Accident (DBA) Model

1. This model allows the user to perform dose calculations based on USAR release rate data for various design accidents.
2. If this option is selected, the user may select from a list of nine DBAs:
 - a. Loss of Coolant
 - b. Main Steam Line Break
 - c. Loss of Offsite AC
 - d. Locked RCP Rotor
 - e. Waste Gas Decay Tank Rupture
 - f. CVCS Break

- g. SG Tube Rupture
- h. Fuel Handling Accident
- i. Control Rod Ejection

NOTE

Use field team data whenever available to provide the most accurate dose estimations.

7.2.9 Option Three, Steam Generator Tube Rupture

1. The SG Tube Rupture Model allows the user to perform dose calculations based on a steam generator tube rupture utilizing steam flow and shine monitor readings.
2. The following instructions may be helpful when performing SG Tube Rupture calculations:
 - a. Steam generator monitor readings may be input in mR/hr for either a steaming steam generator or a full steam generator.
 - 1) The input description is changed by double-clicking within the box surrounding the input description.
 - 2) Steam generator flow may be input in lbm/hr, thousands of lbm/hr, gph or as a pressure entered by the user.
 - a) Gallons per hour (gph) should be selected if the steam generator is full of water. This option represents a two-phase or liquid release from the steam generator. [Commitment Step 3.2.1]
 - b) The input description is changed by double-clicking within the box surrounding the input description.
 - 3) A Steam Generator PORV/Auxiliary Feed Exhaust Help Screen is available by double-clicking either the steam generator monitoring readings or steam generator flow input field.
 - a) Once the Help Screen is completed, the user can return the averaged flow and monitor readings to the Main Screen by selecting FILE EXIT.

7.2.10 Option Four, Radiation Monitoring System (RMS)

1. The RMS Model allows the user to input data from the unit and/or radwaste vent monitor as well as the vent flow rates to perform offsite dose calculations.
2. The following instructions may be helpful when performing RMS calculations:
 - a. Gaseous Activity - May be changed to Total Activity by double-clicking within the box surrounding Gaseous Activity.
 - 1) Likewise, if Total Activity is displayed it may be toggled back to Gaseous Activity by using the same technique.
 - b. Iodine Activity - May be changed to a ratio if necessary by entering the ratio value followed by double-clicking within the box surrounding the Iodine Activity. This is a toggle type of function and may be returned to an activity using the same technique.
 - 1) If the ratio is unknown, the value may be entered.
 - 2) If the ratio is unknown, once the display has been changed to a ratio input, double-clicking on the associated data field will access a Help Screen.
 - 3) Once the user selects the appropriate DBA ratio, FILE EXIT may be used to return the value to the Model Screen.
 - c. Vent Flow -- may be entered.
 - 1) A Help Screen is available by double-clicking the Vent Flow data box.
 - 2) Enter the fan status for each fan by entering the status and then pressing Enter.
 - 3) Select Vent Totals from the tool bar and total the flows required.
 - 4) Select FILE EXIT from the tool bar to forward the value to the Model Screen.

7.2.11 Option Five, Field Team Data Model

1. This model allows the user to input field team dose rates, iodine concentration, particulate concentration and distance information to back calculate the plant release rate and then ultimately the down field doses.
2. The following instructions may be helpful when performing the Field Team dose calculations:

NOTE

The Particulate/Iodine ratio used throughout the Dose Assessment Program is 0.112. If the Particulate/Iodine ratio is selected, unless an entry is made, the value of 0.0 will be used. This option only pertains to the field team model.

- a. Field Team Iodine Concentration may be changed to Iodine/Noble Gas Ratio by double-clicking in the box surrounding Field Team Iodine Concentration. This is a toggle-type function and may be changed back to concentration input using the same technique. By selecting Iodine/Noble Gas Ratio the particulate field will change to Particulate/Iodine Ratio.
 - 1) If the ratio is known, the value may be entered.
 - 2) If the ratio is unknown, once the display has been changed to a ratio input, double-clicking on the associated data field will access a Help Screen.
 - 3) Once the user selects the appropriate ratio, FILE EXIT may be used to return the value to the Model Screen.
- b. Field Team Distance may be toggled between units of miles and kilometers by double-clicking in the box surrounding the Field Team Distance.

7.3 Printer Use

- 7.3.1 Selection of FILE and PRINT from the tool bar will allow the user to print to a Network printer.

NOTE

There may be error messages received when printing the notification form. In most cases these are due to the PC configuration and not the Dose Assessment Program program. If the program does not abort, then you should get printed output.

7.3.2 The notification form will only print if the PC is connected to the LAN and the user is logged into a server.

8.0 INITIAL ACTIONS

8.1 None.

9.0 SUBSEQUENT ACTIONS

9.1 None.

10.0 RECORDS

10.1 Printouts associated with this procedure are considered records.

10.2 Records generated by this procedure during an actual emergency are considered lifetime QA records and shall be forwarded to Emergency Planning at the termination of the emergency.

10.3 Records generated by this procedure during a drill or exercise are considered non-QA records and shall be forwarded to Emergency Planning at the termination of the drill or exercise.

11.0 FORMS

11.1 None

- END -

ATTACHMENT A
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NPIS SCREEN DISPLAYS

Group Menu - Touch Screen for E-Plan Menu

E-Plan Menu - Touch Screen for one of the following

- | | |
|---|---|
| <p>I STATUS BOARD</p> <p>1. RCS</p> <p>2. Steam Generators</p> <p> a) Levels</p> <p> b) Pressures</p> <p>3. ECCS</p> <p>4. Containment</p> <p> a) Pressure</p> <p> b) Temperature</p> <p> c) H₂ concentration</p> <p> d) CHARM R/hr</p> <p>Press F3 Key</p> <p>5. Critical Parameters</p> <p>6. To exit press Group Key</p> | <p>II AREA RAD</p> <p>1. Radiological Status</p> <p> a.) <u>MET</u> Data</p> <p> b) Radmonitors µCi/cc</p> <p>Press F2 Key</p> <p>2. Area Radmonitors mR/hr</p> <p> and CHARM R/hr</p> <p>3. To exit press F6 Key</p> |
| <p>III MET TOWER DATA</p> <p>1. Stability Class</p> <p>2. Wind Speed</p> <p>3. Wind Direction</p> <p>4. Vert Temp Difference °F</p> <p>NOTE: To change to °C type</p> <p> GD MET and press</p> <p> Enter Key</p> <p>5. To exit press Group Key</p> | <p>IV GROUP DISPLAY</p> <p>1. SGCHEM 1</p> <p>2. SGCHEM 2</p> <p>3. SGCHEM 3</p> <p>4. PORVMSIV, etc.</p> <p>NOTE: a) To trend press F4</p> <p> Key</p> <p> b) For the New Group</p> <p> Display press F5 Key</p> <p>5. To exit press Group Key</p> |

NOTE: Screen Display Color Code

RED - Alarm
YELLOW - Alert
GREEN - Normal
BLUE - Invalid Reading

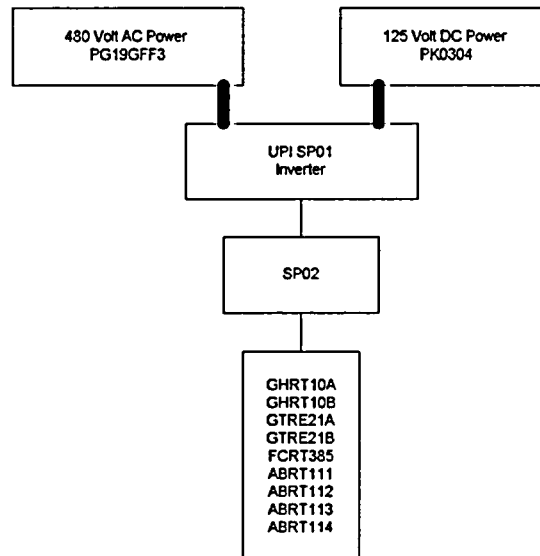
- END -

ATTACHMENT B
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RADIATION MONITOR INFORMATION

On a total loss of off-site power the following radiation monitors remain operable:

GHRT 10A Radwaste Building Vent - Part & Iodine
 GHRT 10B Radwaste Building Cent - WRGM
 GTRE 21A Unit Vent - Part & Iodine
 GTRE 21B Unit Vent - WRGM
 FCRT 385 Aux. Feedwater Turbine Discharge Monitor
 ABRT 111 Steam Line "D" PORV Discharge Monitor
 ABRT 112 Steam Line "C" PORV Discharge Monitor
 ABRT 113 Steam Line "B" PORV Discharge Monitor
 ABRT 114 Steam Line "A" PORV Discharge Monitor

1. These monitors have as their normal AC power SP02 which is supplied by AC power supply PG19GFF3 (480 Volt AC). This feeds or goes from PG19GFF3 to SP01 Inverter [an UPI] to SP02 to monitors.



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2. The SP01 Inverter is also fed by a 125 volt DC power PK0304 [plant batteries]. In the event of a loss of offsite power occurs (PG19GFF3) then the inverter (UPI) SP01 still feeds the monitors via SP02.
3. If after a total loss of offsite power, the plant would regain one of the NB buses, then the radiation monitors that are fed from that bus would also be available if flow was restored to the monitor.

NOTE

The Chemistry Technicians may have to remind the Control Room to restore flow to these monitors.

4. If the RM-11 is not available the flow to these monitors will have to be done from their RM-23's. (The RM-11 is not powered by NB bus).

- END -