

February 17, 2004

Our File: 108US-01321-021-001  
Your File: Project No. 722

U.S. Nuclear Regulatory Commission,  
Document Control Desk,  
Washington, D.C. 20555

Dear Ms. Sosa,

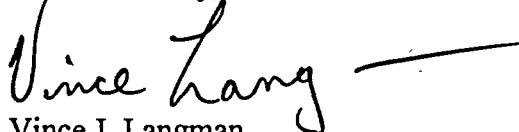
Re: **Additional Information in Support of the 3<sup>rd</sup> PIRT TH Subpanel Meeting on ACR-700**

In support of the 3<sup>rd</sup> PIRT Thermal Hydraulics (TH) Subpanel meeting on ACR-700 of February 18-20, 2004, and in support of the NRC's pre-application review of the ACR-700, please find enclosed the following additional information and documentation:

- CD containing "CATHENA Simulation of RD-14M Critical Break LOCA Experiment B9401", AECL report 108US-03532-225-001, Revision 0, February 2004;
- 7 paper copies of "CATHENA: A thermalhydraulic code for CANDU analysis", by B.N. Hanna, published in Nuclear Engineering and Design 180 (1998) 113-131;
- Paper copies of the following drawings and posters:
  - "ACR Fuel Channel"
  - "Reactor Coolant System Conditions T = 0 s"
  - "Broken Pass" Schematic
  - "RD-14M Loop Schematic"
  - RD-14M "Inlet Header 8"
  - RD-14M "Outlet Header 5"
  - RD-14M "Heated Sections - Fuel Element Simulators (FES)"
  - RD-14M "Test Section"

If you have any questions on this letter and/or the enclosed material please contact the undersigned at (905) 823-9060 extension 6543.

Yours sincerely,



Vince J. Langman  
ACR Licensing Manager

**/Enclosures:**

1. CD titled "CATHENA Simulation of RD-14M Critical Break LOCA Experiment B9401" containing the electronic copy of AECL report as provided in this letter.
2. Paper copies of the material listed in this letter have been sent to Belkys Sosa, ACR-700 project manager, under a separate cover.