

February 19, 2004

NRC 2004-0020  
10 CFR 50.55a

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2  
Dockets 50-266 and 50-301  
License Nos. DPR-24 and DPR-27  
Filing of Owner's Inservice Inspection Summary Report for Point Beach Nuclear Plant  
Refueling Outage U2R26 and 33rd Year Tendon Surveillance and Concrete Examination

Nuclear Management Company, LLC, (NMC) is submitting Inservice Inspection Summary Reports for inspections conducted prior to and during the Point Beach Nuclear Plant (PBNP) refueling outage U2R26 that concluded on November 21, 2003. The enclosed Class 1 and 2 report is submitted pursuant to the requirements of subarticle IWA-6240 of the 1998 Edition with addenda through 2000 of Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code. The IWE Class MC and IWL Class CC reports are submitted pursuant to the requirements of subarticle IWA-6240 of the 1992 Edition with addenda through 1992 of ASME Section XI.

The enclosed summary report includes Form NIS-1, the Owner's Report for Inservice Inspections, and Form NIS-2, the Owner's Reports for Repair/Replacement Activities. These reports include the first outage of the first period of the fourth ten-year interval for Class 1 and 2 systems, the second outage of the second period of the first ten-year interval for Class MC at PBNP Unit 2, and the 33rd year concrete and tendon surveillances of Class CC components for both PBNP units.



Gary D. Van Middlesworth  
Site Vice-President, Point Beach Nuclear Plant  
Nuclear Management Company, LLC

Enclosures

A047

cc (with enclosures):

Administrator, Region III, USNRC  
Project Manager, Point Beach Nuclear Plant, USNRC  
Resident Inspector, Point Beach Nuclear Plant, USNRC  
PSCW  
Mike Verhagen, State of Wisconsin

NIS-1 Inservice Inspection  
Summary Report for  
U2R26

Nuclear Management Company, LLC  
700 First Street  
Hudson, Wisconsin 54016

Point Beach Nuclear Plant Unit 2  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

Commercial Service Date: October 1, 1972  
USNRC Docket Number: Unit 2: 50-301

Report Date: February 11, 2004

## FORM NIS-1 OWNER'S REPORT FOR INSERVICE INSPECTIONS

1. Owner: Nuclear Management Company LLC  
700 First Street  
Hudson, Wisconsin 54016
2. Plant: Point Beach Nuclear Plant, 6610 Nuclear Road, Two Rivers, Wisconsin 54241
3. Plant Unit: 2
4. Owner Certificate of Authorization: N/A
5. Commercial Service Date: October 1, 1972
6. National Board Number for Unit: N/A
7. Components Inspected

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Reactor Pressure Vessel	Combustion Engineering	69101	U009973	NB-21009
Regenerative Heat Exchanger	Sentry	3996A3-7	N/A	NB-0361 NB-0362 NB-0363
Steam Generator B	Westinghouse	SG2L-12173	U009975	NB-0060
Accumulator T-34A	Delta Southern Co.	41036-69-1	U009976	N/A
Accumulator T-34B	Delta Southern Co.	41036-69-2	U009977	N/A
Class 1 Piping	Westinghouse/ Bechtel	N/A	N/A	N/A
Class 2 Piping	Westinghouse/ Bechtel	N/A	N/A	N/A

8. Examination Dates: July 1, 2002 to November 21, 2003
9. Inspection Period Identification: First Period from July 1, 2002 to June 30, 2005
10. Inspection Interval Identification: Fourth Interval from July 1, 2002 to June 30, 2012
11. Applicable Editions of Section XI: 1998 Edition, 2000 Addenda
12. Date/Revision of Inspection Plan: May 29, 2003, revision 1

13. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for the Inspection Plan:

An Inservice Examination of selected Class 1 and 2 components and piping systems of Nuclear Management Company's (NMC) Point Beach Nuclear Plant (PBNP) Unit 2 was performed during the time period of July 1, 2002 to November 21, 2003, which included the Unit 2 Refueling Outage 26 (U2R26). This was the first outage of the first period of the fourth ten-year interval.

The components and piping systems examined were selected in accordance with the Fourth Ten Year Inservice Inspection Program, which was prepared in accordance with the requirements of the American Society of Mechanical Engineers (ASME) Section XI, 1998 Edition with addenda through 2000, and applicable relief requests. In addition, augmented examinations were performed on the Safety Injection Accumulators and Reactor Pressure Vessel in accordance with various commitments.

Snubber functional testing and visual examinations were conducted in accordance with the OM Code, 1987 Edition with OMa-1988. We Energies, NMC, and Lambert, MaGill, Thomas Inc. (LMT) personnel performed the examinations. See Attachment 2 for test results and the ISI Examination Tables for examination results of Section XI classified snubbers.

Manual ultrasonic, visual, radiographic, and liquid penetrant non-destructive techniques were used to examine components, piping, and their supports. We Energies and NMC personnel supported by examiners supplied by LMT, Inc. and Professional Welding Associates (PWA), performed the examinations. See Attachment 3 for the listing on Class 1 and 2 components examined.

System pressure testing was conducted by the plant to ASME Section XI and applicable plant procedures. We Energies, NMC, LMT, and PWA personnel performed the examinations. Minor leakage was noted at mechanical connections. See the attached tables for a summary of examinations.

Repairs and replacements were performed on Class 1 and 2 Section XI components since the previous outage. The NIS-2 reports are included with this report. NIS-2 reports are also filed at the plant in the permanent record system.

The following Code Cases were used during the reporting period and the U2R26 outage:

Code Case	Title
N-460	Alternate Examination Coverage for Class 1 and Class 2 Welds, Section XI, Division 1
N-498-4	Alternate Rules for 10 Year Hydrostatic Pressure Testing for Class 1, 2, and 3 Systems, Section XI, Division 1
N-522	Pressure Testing of Containment Penetration Piping, Section XI, Division 1
N-566-1	Corrective Action for Leakage Identified at Bolted Connections
N-623	Deferral of Inspections of Shell-to-Flange and Head to Flange Welds of a Reactor Vessel

#### 14. Abstract of Results of Examinations and Tests:

See Attachment 1 for numbers and percentages of examinations or tests completed.

Every indication noted was accepted as-is by evaluation. However, most indications were addressed by Work Orders to return the component to its design configuration where needed. For leakage at mechanical connections, Work Orders were initiated to correct the leakage. On valve stem leakage, operations or engineering personnel adjusted the packing so leakage was stopped or slowed down to an acceptable limit. Leakage collection systems were operational in each case.

#### Class 1 Systems

##### Reactor Pressure Vessel

Visual examinations were performed on various portions of the RPV, including the vessel, upper head, and closure head bolting. Remote ultrasonic examinations were performed on the CRDM tubes. Direct and remote bare metal visual examinations were performed on the upper and lower head penetrations to detect evidence of leakage. No reportable indications were noted.

The interior of the reactor vessel was examined using remote visual equipment on accessible areas to the extent practical. No relevant indications were noted.

##### Pressurizer

Visual examinations were performed on the manway bolting. Eight of the bolts exhibited evidence of minor corrosion, nicks, and gouges. All were replaced as a conservative measure.

##### Auxiliary Coolant

Volumetric examinations were performed on selected auxiliary coolant welds. No indications were noted.

##### Reactor Coolant

Volumetric examinations were performed on selected reactor coolant welds. No indications were noted.

Visual and surface examinations were performed on supports and welded attachments. One support was noted as being in contact with scaffold. When the scaffold was removed, the support was acceptable. Another spring support was found to contain debris and the load plates were missing. An evaluation showed the load to be correct and the debris was a housekeeping issue. It was determined the debris would not interfere with the function of the support.

##### Safety Injection

Volumetric examinations were performed on selected Safety Injection welds. No indications were noted.

Visual and surface examinations were performed on supports and welded attachments. No indications were noted.

Three welds were removed from service in order to replace a valve. Three new welds were installed and preservice examinations were performed with no indications noted.

#### Charging System

Visual examinations were performed on supports. No indications were noted.

#### Chemical Volume Control System

Visual examinations were performed on supports. No indications were noted.

Surface examinations were performed on select welds. No indications were noted.

#### Pressure Tests

The Reactor Coolant system leakage test and other Class 1 pressure tests were conducted. These tests met the requirements for periodic pressure testing. Evidence of leakage was noted at several mechanical joints. These were addressed through plant procedures. No through wall leakage was noted.

### Class 2

#### Steam Generator

One snubber was examined. It was noted that several bolts had inadequate thread engagement. This was accepted as-is by engineering evaluation.

#### Main Steam System

Visual examinations were performed on selected supports. No indications were noted.

#### Auxiliary Feedwater

A direct visual examination was performed on the inside surface of one weld. This examination revealed a linear indication. After removing the piping for additional access, minor surface conditioning was performed. The indication was found to be a normal part of the weld root, which is an acceptable condition. Baseline examinations were performed on the new weld.

#### Pressure Tests

Pressure tests were conducted on Class 2 systems. These tests met the requirements for periodic pressure testing. Evidence of leakage was noted at several mechanical joints. These were addressed through plant procedures. No through wall leakage was noted.

### 15. Abstract of Corrective Measures.

The auxiliary feedwater weld was investigated further upon detection of the linear indication. This involved removing the pipe next to the weld and performing a direct visual and liquid penetrant examination, which showed the indication to be a normal part of the weld root.

During the performance of pressure tests, evidence of leakage was noted at mechanical connections. These were corrected using normal maintenance work procedures.

Several component supports were noted as having a discrepancy with the design drawings. In each case, these were accepted as-is by engineering evaluation. The indications were either left in place as-is, work orders were written to restore them to their design configuration, or a drawing change was initiated to show the as-built configuration.

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI, and c) corrective measures taken conform to the rules of the ASME Code, Section XI.


Certificate of Authorization No. (if applicable): N/A Expiration Date: N/A

Date: 2/17/2004 Signed: Nuclear Management Co. by Russell E. Turner  
Owner

#### CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by Hartford Steam Boiler of Hartford, Connecticut, have inspected the components described in this Owner's Report during the period July 1, 2002 to November 21, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owner's Report in accordance with the Inspection Plan and as required by the ASME Code, Section XI.

By Signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

  
Inspector's Signature

Commissions

WI 247972

National Board, State, Province,  
and Endorsements

Date 2-17-2004



Nuclear Management Company LLC  
700 First Street  
Hudson, Wisconsin 54016  
National Board Number for Unit: N/A

Commercial Service Date: 10/1/72  
Docket 50-301  
Pt. Beach Unit2

## Attachment 1

Status of Examinations for the Period/Interval

Nuclear Management Company LLC  
700 First Street  
Hudson, Wisconsin 54016  
National Board Number for Unit: N/A

Commercial Service Date: 10/1/72  
Docket 50-301  
Pt. Beach Unit2

Examination Category	Total Examinations Required for the Interval	Total Examinations Credited for this Period	Total Examinations Credited (%) for the Period	Total Examinations Credited (%) to Date for the Interval	Remarks
B-A	6	0	0	0	Partial deferral permissible
B-B	7	0	0	0	
B-D	25	0	0	0	
B-F	See Remarks				Part of Risk-Informed ISI Program, see Category R-A
B-G-1	6	1	17	17	
B-G-2	8	3	38	38	
B-J	See Remarks				Part of Risk-Informed ISI Program, see Category R-A
B-K	4	2	50	50	
B-L-1	0	0	0	0	Deferred to the end of the interval
B-L-2	0	0	0	0	Examined only if disassembled for maintenance, repair, or volumetric examination
B-M-1	0	0	0	0	Scheduled for the third interval
B-M-2	0	0	0	0	Examinations will be performed when valves are opened
B-N-1	3	1	33	33	Accessible portions of the vessel internal surface
B-N-2	0	0	0	0	Deferred to the end of the interval
B-N-3	0	0	0	0	Deferred to the end of the interval
B-O	2	0	0	0	Deferred to the end of the interval
B-P	6	1	17	17	Pressure tests cross class boundaries
B-Q	0	0	0	0	Examined in accordance with plant Technical Specifications
C-A	6	0	0	0	
C-B	7	0	0	0	
C-C	12	0	0	0	
C-D	0	0	0	0	There are no class 2 bolts greater than 2" at PBNP
C-F-1	See Remarks				Part of Risk-Informed ISI Program, see Category R-A
C-F-2	See Remarks				Part of Risk-Informed ISI Program, see Category R-A
C-G	1	0	0	0	Scheduled for the third interval
C-H					Pressure tests cross class boundaries
F-A	125	10	8	8	Class 1 and 2 component supports
R-A	137	19	14	14	Risk-informed welds, includes class 1 and 2 piping welds

Note: all numbers rounded to the nearest 1%.

## Attachment 2

### Snubber Functional Testing

The following snubbers were functionally tested:

Snubber Number	System	Manufacturer	Code Class	Results-Date
HS-26	SI	Grinnell	1	SAT - 10/8/2003
HS-27	RC	Grinnell	1	SAT - 10/8/2003
HS-36	MS	Ankor-Holth	2	SAT - 10/5/2003

Nuclear Management Company LLC  
700 First Street  
Hudson, Wisconsin 54016  
National Board Number for Unit: N/A

Commercial Service Date: 10/1/72  
Docket 50-301  
Pt. Beach Unit2

## Attachment 3

### Class 1 and 2 Examination Tables

## Examination Results Guide

DATE: 2/16/04  
(A)

POINT BEACH NUCLEAR PLANT UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBR COMPONENTS  
(B)

PAGE: 00  
(C)

Reactor Pressure Vessel (D)

SUMMARY EXAMINATION AREA		CATGY ITEM NO	STATUS		DATA SHEET	N I O G N G T R S E H E I O E C G M R			Remarks
NUMBER	IDENTIFICATION		EXAM METHOD			WO Number			
System ID - R-1, Drawing No. ISI-1101 (E)									
001000	RPV-14-683 (G)	(J) B-A	C	UT 5	168-0001	-	-	-	X 9/19/2003 - UT complete.
(F)	Shell to Flange (H)	(K) B1.21	(L)	UT 11	168-0001	-	-	-	X Geometric Reflectors from
				UT 18	168-0001	-	-	-	X the flange radius.
				(M)	(N)				(P)
WO 0213402 (R)									

- (A) Date the report table was printed  
(B) Specific outage and report information (Note: Class "P" components are for pressure tests. Many of the systems cross class boundary's, requiring a multiple class designation.)  
(C) Page number  
(D) System description  
(E) System / ISI drawing number  
(F) Summary number of the record (unique, for computer tracking only)  
(G) Component or Component Support identification  
(H) Component or Component Support Description  
(J) ASME Code category  
(K) ASME Code item number  
(L) Shows status of examination  
P - Partial examination  
A - Augmented examination  
C - Code completed, greater than 90% coverage  
L - Limited code complete, less than or equal to 90% coverage  
E - Expanded (additional) sample examination  
B - Baseline (Preservice exam)  
R - Removed from service  
O - Other examination  
N - No examination  
If no Code, this is for information only  
(Other codes may be used as necessary)  
(M) NDE methods  
(N) Data sheet number  
(P) Types of indications found  
(R) Remarks and Work Order Number

DATE: 02/11/04

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

PAGE: 1

Reactor Pressure Vessel

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - N/A, Drawing No. ISI-2101										
003900	RPV Interior Surface Vessel Interior Surfaces	B-N-1 B13.10	L	VT-3	756-0001	X	-	-	-	10/16/2003 - VT-3 performed, no recordable indications. Examined upper internals and accessible areas of the core barrel. No examination of interior areas due to core barrel in place and flange guard installed.

DATE: 02/11/04

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

PAGE: 2

## Reactor Pressure Vessel Closure Head

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		EXAM METHOD	DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
System ID - R-1, Drawing No. ISI-2103											
004500	RPV Closure Head Studs Studs 1 thru 48	B-G-1 B6.30	P	UT 0	169-0001	X	-	-	-	10/3/2003 - UT performed, no recordable indications. Examined studs 1 through 16. WO 0212617	
-----											
004600	RPV Closure Head Nuts Nuts 1 thru 48	B-G-1 B6.10	P	VT-1	750-0002	X	-	-	X	10/12/2003 - VT-1 performed, no recordable indications. Examined nuts 1 through 16. WO 0212616	
-----											
004700	RPV Closure Head Washers Washers 1 thru 48	B-G-1 B6.50	P	VT-1	750-0003	X	-	-	-	10/12/2003 - VT-1 performed, no recordable indications. Examined washers 1 through 16. WO 0212618	

## Pressurizer

[illegible]



POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

## Safety Injection Loop B Injection

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - Pipe SI 2501R-01, Drawing No. ISI-2124										
036700	AC-10-SI-2001-12	R-A	C	THICK	109-0011	-	-	-	-	10/16/2003 - UT performed, no
	Pipe to Elbow	R1.20		UT 0L	109-0011	X	-	-	-	recordable indications.
				UT 45	173-0001	X	-	-	-	WO 0213389
-----										
036800	AC-10-SI-2001-13	R-A	C	THICK	109-0012	-	-	-	-	10/16/2003 - UT performed, no
	Elbow to Pipe	R1.20		UT 0L	109-0012	X	-	-	-	recordable indications.
				UT 45	173-0002	X	-	-	-	WO 0213389
-----										
037100	AC-10-SI-2001-14	R-A	C	THICK	109-0013	-	-	-	-	10/16/2003 - UT performed, no
	Pipe to Tee	R1.20		UT 0L	109-0013	X	-	-	-	recordable indications. Limited
				UT 45	173-0003	X	-	-	-	examination due to tee configuration limiting access. WO 0213389
-----										
037200	AC-10-SI-2001-15	R-A	C	THICK	109-0014	-	-	-	-	10/16/2003 - UT performed, no
	Tee to Pipe	R1.20		UT 0L	109-0014	X	-	-	-	recordable indications. Limited
				UT 45	173-0004	X	-	-	-	examination due to tee configuration limiting access. WO 0213389
-----										
037600	AC-10-SI-2001-17	R-A	L	THICK	109-0015	-	-	-	-	10/16/2003 - UT performed, no
	Elbow to Valve	R1.11		UT 0L	109-0015	X	-	-	-	recordable indications. Limited
	2SI-867B			UT 45	173-0005	X	-	-	-	examination due to valve configuration limiting access. WO 0213389

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

### Residual Heat Removal to Loop B

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS EXAM METHOD	DATA SHEET #	N O R E C	I N S I G M	O T H E R	REMARKS WO NUMBER
System ID - SI-2501R-02, Drawing No. ISI-2125								
040500	SIS-10-SI-2002-11	R-A	C THICK	109-0010	-	-	-	10/16/2003 - UT performed, no
	Pipe to Tee	R1.20	UT 0L	109-0010	X	-	-	recordable indications. Limited
			UT 45	173-0006	X	-	-	examination due to tee configuration.
								WO 0213389

DATE: 02/11/04

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

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## Reactor Vessel Injection

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N O R E C	I N S I G	G E O M	T H E R	REMARKS WO NUMBER
			EXAM METHOD							
System ID - SI-2501R-05, Drawing No. ISI-2128										
046300	SI-2501R-5-2SI-13 Rigid Support	F-A F1.10B	C	VT-3	754-0004	X	-	-	-	10/06/2003 - VT-3 performed, no recordable indications. WO 0213390
-----										
046400	SI-2501R-5-2SI-13-IWA Welded Attachment	B-K B10.20	C	PT	451-0017	X	-	-	-	10/16/2003 - PT performed, no recordable indications. WO 0213390

### Pressurizer Relief to PORV's

[illegible]

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

## Loop A RTD Bypass

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - Pipe RC 2501R-14, Drawing No. ISI-2138										
084700	RC-03-BP-2001-06 Pipe to Elbow	R-A R1.20	C	THICK	109-0006	-	-	-	-	10/10/2003 - UT performed, no
				UT 0L	109-0006	X	-	-	-	recordable indications.
				UT 45	173-0008	X	-	-	-	WO 0213391
				UT 70	173-0008	X	-	-	-	
084800	RC-03-BP-2001-07 Elbow to Pipe	R-A R1.20	C	THICK	109-0007	-	-	-	-	10/10/2003 - UT performed, no
				UT 0L	109-0007	X	-	-	-	recordable indications.
				UT 45	173-0009	X	-	-	-	WO 0213391
				UT 70	173-0009	X	-	-	-	
084900	RC-03-BP-2001-08 Pipe to Elbow	R-A R1.20	C	THICK	109-0008	-	-	-	-	10/10/2003 - UT performed, no
				UT 0L	109-0008	X	-	-	-	recordable indications.
				UT 45	173-0010	X	-	-	-	WO 0213391
				UT 70	173-0010	X	-	-	-	
085000	RC-03-BP-2001-09 Elbow to Pipe	R-A R1.20	C	THICK	109-0009	-	-	-	-	10/10/2003 - UT performed, no
				UT 0L	109-0009	X	-	-	-	recordable indications.
				UT 45	173-0011	X	-	-	-	WO 0213391
				UT 70	173-0011	X	-	-	-	
085200	RC-2501R-14-2H2 Dual Spring Hanger	F-A F1.10C	C	VT-3	754-0018	-	-	-	X	10/15/2003 - VT-3 performed, spring
				IDR	2003-0009	-	-	-	-	can indicator is in contact with 2"x2" tube steel which is used to support lead blankets; spring can has moderate corrosion inside can housing and light corrosion on spring and upper portion of rod. The tube is part of the scaffold and was removed. The corrosion was accepted as-is by evaluation. WO 0213392
085300	RC-2501R-14-2H2-IWA Welded Attachment	B-K B10.20	C	PT	451-0020	X	-	-	-	10/16/2003 - PT performed, no recordable indications. WO 0213393

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

## Loop B RTD Bypass

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O R S E E I O C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - Pipe RC 2501R-14, Drawing No. ISI-2139										
090700	RC-03-BP-2002-06 Pipe to Elbow	R-A R1.20	C	THICK	109-0001	-	-	-	-	10/06/2003 - UT performed, no
				UT 0L	109-0001	X	-	-	-	recordable indications.
				UT 45	173-0012	X	-	-	-	WO 0213393
				UT 60	173-0012	X	-	-	-	
-----										
090800	RC-03-BP-2002-07 Elbow to Pipe	R-A R1.20	C	THICK	109-0002	-	-	-	-	10/6/2003 - UT performed, no
				UT 0L	109-0002	X	-	-	-	recordable indications.
				UT 45	173-0013	X	-	-	-	WO 0213393
				UT 70	173-0013	X	-	-	-	
-----										
090900	RC-03-BP-2002-08 Pipe to Elbow	R-A R1.20	C	THICK	109-0003	-	-	-	-	10/6/2003 - UT performed, no
				UT 0L	109-0003	X	-	-	-	recordable indications.
				UT 45	173-0014	X	-	-	-	WO 0213393
				UT 70	173-0014	X	-	-	-	
-----										
091000	RC-03-BP-2002-09 Elbow to Pipe	R-A R1.20	C	THICK	109-0004	-	-	-	-	10/3/2003 - UT performed, no
				UT 0L	109-0004	X	-	-	-	recordable indications.
				UT 45	173-0015	X	-	-	-	WO 0213393
				UT 70	173-0015	X	-	-	-	
-----										
091200	RC-2501R-14-2H1 Dual Spring Hanger	F-A F1.10C	B	VT-3	754-0017	-	-	-	X	10/17/2003 - Baseline VT-3
				IDR	2003-0010	-	-	-	-	performed, debris noted inside spring can and no load plates on either spring can. Accepted as-is by evaluation. WO 0206026

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

Reactor Coolant Pump A Seal Injection

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER		
			EXAM METHOD									
System ID - Pipe CH 2501R-01, Drawing No. ISI-PRI-2146												
118000	24-PH Spring Hanger	F-A F1.10C	C	VT-3	754-0003	X	-	-	-	10/6/2003 - VT-3 performed, no recordable indications. WO 0213394		

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

## Reactor Coolant Pump B Seal Injection

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	NON G T O R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - Pipe CH 2501R-01, Drawing No. ISI-PRI-2148										
122800	CVC-02-PSI-2002-21-FB Flange Bolting	B-G-2 B7.50	C	VT-1	750-0007	X	-	-	-	10/17/2003 - VT-1 performed, no recordable indications. Examined bolts in place under tension. WO 0213395
123300	CVC-02-PSI-2002-25 Pipe to Elbow	R-A R1.20	C	PT	451-0001	X	-	-	-	10/17/2003 - PT performed, no recordable indications. WO 0213397
123400	CVC-02-PSI-2002-26 Elbow to Pipe	R-A R1.20	C	PT	451-0002	X	-	-	-	10/17/2003 - PT performed, no recordable indications. WO 0213397
123500	CVC-02-PSI-2002-27 Pipe to Tee	R-A R1.20	C	PT	451-0003	X	-	-	-	10/17/2003 - PT performed, no recordable indications. WO 0213397
124100	CVC-02-PSI-2002-30-FB Flange Bolting	B-G-2 B7.50	C	VT-1	750-0006	X	-	-	-	10/24/2003 - VT-1 performed, no recordable indications. Examined in place under tension. WO 0213395



POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

## Cold Leg Injection Loop A

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - Pipe SI 2501R-06, Drawing No. ISI-2149										
125150	SIS-02-SI-2004-07A Valve 2SI-845A to Pipe R1.20	R-A	R							10/12/2003 - Weld removed from service due to replacement of valve 2SI-845A. WO 0206562
125160	SIS-02-SI-2004-07B Valve 2SI-845A to Pipe TBD	R-A	B	PT	450816	X	-	-	-	10/12/2003 - Baseline PT performed, no recordable indications. New weld due to replacement of valve 2SI-845A. Shown as weld SW-2 on data sheet. WO 0206562
125250	SIS-02-SI-2004-08B Pipe to Coupling	R-A	R							10/12/2003 - Weld removed from service due to replacement of valve 2SI-845A. WO 0206562
125260	SIS-02-SI-2004-08C Pipe to Coupling	R-A	B	PT	450844	X	-	-	-	10/9/2003 - Baseline PT performed, no recordable indications. New weld added to database due to replacement of valve 2SI-845A. Shown as weld FW-6 on data sheet. WO 020656
126200	2SI-878D-6 Rigid Support	F-A F1.10B	C	VT-3 IDR	754-0002 2003-002	-	-	-	X	10/04/03 - VT-3 performed, gaps found between base plate and wall. Accepted as-is by evaluation. WO 0213398
System ID - Pipe SI 2501R-06, Drawing No. ISI-2150										
128000	2SI-878D-10 Rigid Support	F-A F1.10B	C	VT-3	754-0009	X	-	-	-	10/6/2003 - VT-3 performed, no recordable indications. WO 0213399

DATE: 02/11/04

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

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Letdown from Loop B Cold Leg

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - Pipe RC 2501R-10, Drawing No. ISI-2152										
134500	RC-2501R-10-21PS Pipe Support	F-A F1.10B	C	VT-3	754-0005	X	-	-	-	10/6/2003 - VT-3 performed, no recordable indications. WO 0213382
-----										
134700	29-PRE Spring Hanger	F-A F1.10C	B	VT-3 IDR	754-0006 2003-003	-	-	-	X	10/06/03 - Baseline VT-3 performed, missing bolt on upper base plate, spring can out of plumb, lateral support skewed, lack of full thread engagement bolts 1 and 2, and missing nut on bolts 1 and 2. Accepted as-is during previous outage evaluation. WO 0213382

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

## Excess Letdown to Chemical and Volume Control

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS			DATA SHEET #	N I O N G T O R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD								
System ID - Pipe RC 2501R-13, Drawing No. ISI-2153											
136100	RC-02-DR-2002-02A Pipe to Elbow		N								Weld does not exist. Verified during U2R26. WO 0213401
-----											
136200	RC-02-DR-2002-02B Elbow to Pipe		N								Weld does not exist. Verified during U2R26. WO 0213401
-----											
136900	RC-2501R-13-S5 Guide	F-A F1.10B	B	VT-3	754-0010	X	-	-	-		10/14/2003 - Baseline VT-3 performed, no recordable indications. WO 0213401
-----											
137200	RC-2501R-13-S6 Guide	F-A F1.10B	B	VT-3	754-0011	X	-	-	-		10/14/2003 - Baseline VT-3 performed, no recordable indications. WO 0213401
-----											
137900	RC-2501R-13-S9 Guide	F-A F1.10B	B	VT-3	754-0012	X	-	-	-		10/14/2003 - Baseline VT-3 performed, no recordable indications. WO 0213401
-----											
138300	RC-2501R-13-S10 Guide	F-A F1.10B	B	VT-3	754-0013	X	-	-	-		10/14/2003 - Baseline VT-3 performed, no recordable indications. WO 0213401

DATE: 02/11/04

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 1 PACLEBRN COMPONENTS

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Letdown from Loop B Cold Leg

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - Pipe RC 2501R-10, Drawing No. ISI-2158										
152000	RC-08-DR-2001-01	R-A	L	UT 45	173-0016	X	-	-	-	10/9/2003 - UT performed, no
	Branch Connection to	R1.20		UT 60	173-0016	X	-	-	-	recordable indications. Limited
	Pipe									examination due to branch connection
										configuration limiting access.
										WO 0213400
-----										
152100	RC-08-DR-2001-02	R-A	C	THICK	109-0005	-	-	-	-	10/8/2003 - UT performed, no
	Pipe to Reducer	R1.20		UT 0L	109-0005	X	-	-	-	recordable indications.
				UT 45	173-0017	X	-	-	-	WO 0213400
				UT 60	173-0017	X	-	-	-	
-----										

DATE: 02/11/04

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 2 PACLEBRN COMPONENTS

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Steam Generators

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - 2HX-1B, Drawing No. ISI-PRI-2201										
203600	HS-40	F-A	A	VT-3	2-PT-SS-1	-	-	-	X	10/6/2003 - Angled bolting noted on
	Snubber	F1.20S		CAP	050997	-	-	-	-	two of 12 locations on support.
				IDR	2003-0005	-	-	-	-	Evaluation performed by civil engineering determined these to be acceptable. WO 0208938

Loop B Main Steam Inside Containment

[illegible]

### Loop B Main Steam Outside Containment

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS EXAM METHOD	DATA SHEET #	N I O O N G T R S E H E I O E C G M R	REMARKS WO NUMBER
System ID - Piping EB-01, Drawing No. ISI-2248						
308900	EB-1-2HB	F-A	C VT-3	754-0001	X - - -	10/4/2003 - VT-3 performed, no
	Rigid Support	F1.20A	IDR	2003-001	- - - -	recordable indications. Wrong bolting noted on pipe clamp. Accepted as-is by evaluation. A drawing change has been initiated. WO 9950783

### Feedwater Loop A

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		EXAM METHOD	DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
System ID - EB-09, Drawing No. ISI-2250											
321350	FW-16-FW-2001-28BC	R-A	A	VT-1	NDE-750	-	-	-	X	10/21/2003 - VT-1 performed, linear	
	3" Branch Connection	R1.11		PT	NDE-451	-	-	-	X	indication noted at the weld root.	
Examination performed in response to											
Operating Experience at Kewaunee											
where cracking was seen on the											
inside surface of the inner radius											
section. PT found several rounded											
indications. All were acceptable.											
WO 0305887											



POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 2 PACLEBRN COMPONENTS

## Cold Leg Injection Header

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS			DATA SHEET #	N O R E C	I S I O G	G E M	T H E R	REMARKS WO NUMBER
			EXAM METHOD								
System ID - SI-2501R-06, Drawing No. ISI-2268											
394900	SIS-02-SI-2017-09 Elbow to Pipe		R								10/12/2003 - Weld removed from service due to replacement of valve 2SI-845A. WO 0206562
394910	SIS-02-SI-2017-09A Elbow to Pipe	R-A TBD	B	PT	450844	X	-	-	-		10/9/2003 - Baseline PT performed, no recordable indications. New weld due to replacement of valve 2SI-845A. Shown as weld FW-9 on data sheet. WO 0206562
395100	SIS-02-SI-2017-11 Elbow to Pipe		R								10/12/2003 - Weld removed from service due to replacement of valve 2SI-845A. WO 0206562
395110	SIS-02-SI-2017-11A Elbow to Pipe	R-A TBD	B	PT	450844	X	-	-	-		10/9/2003 - Baseline PT performed, no recordable indications. New weld due to replacement of valve 2SI-845A. Shown as weld FW-7 on data sheet. WO 0206562
395300	SIS-02-SI-2017-12A Pipe to Valve 2SI-845A R1.20	R-A	R								10/12/2003 - Weld removed from service due to replacement of valve 2SI-845A. WO 0206562
395400	SIS-02-SI-2017-12B Pipe to Valve 2SI-845A TBD	R-A	B	PT	450816	X	-	-	-		9/25/2003 - Baseline PT performed, no recordable indications. New weld due to replacement of valve 2SI-845A. Shown as weld SW-1 on data sheet. WO 0206562

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS 2 PACLEBRN COMPONENTS

## Auxiliary Feedwater to Steam Generator A

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		DATA SHEET #	N I O O N G T R S E H E I O E C G M R				REMARKS WO NUMBER
			EXAM METHOD							
System ID - Pipe EB-10, Drawing No. ISI-2281 SH 2										
444700	AF-03-AFW-2001-99 Pipe to Branch Connection	R								10/24/2003 - Weld removed from service due to removal of pipe to examine inside of Branch Connection weld. WO 0306491
-----										
444800	AF-03-AFW-2001-99A Pipe to Branch Connection	R-A TBD	B	RT	RT-2-NMC REV 2 X	-	-	-	-	10/23/2003 - Baseline RT performed, no recordable indications. WO 0306491

## Pressure Test

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		EXAM METHOD	DATA SHEET #	N I O O N G R S E E I O C G M R				REMARKS WO NUMBER
System ID - Auxiliary Feedwater, Drawing No. N/A											
700100	2-PT-AF-1 Auxiliary Feedwater	P	C	VT-2	2-PT-AF-1	-	-	-	X	11/11/2003 - VT-2 performed, leakage noted at mechanical connections. Accepted as-is by evaluation. These will be corrected under plant procedures. WO 0208861	
System ID - Feedwater System, Drawing No. N/A											
700300	2-PT-CS-1 Feedwater (Condensate) System	P	C	VT-2	2-PT-CS-1	X	-	-	-	10/4/2003 - VT-2 complete. No recordable indications. WO 0208862	
System ID - Main Steam, Drawing No. N/A											
701100	2-PT-MS-2 Main Steam Inside Containment	P	C	VT-2	2-PT-MS-2	X	-	-	-	10/4/2003 - VT-2 complete. No recordable indications. WO 0208866	
System ID - Residual Heat Removal, Drawing No. N/A											
701600	IT-536 Containment Sump	P	C	VT-2	IT-536	-	-	-	X	10/18/2003 - VT-2 performed. Leakage noted at mechanical connections. Accepted as-is by evaluation. These will be corrected under plant procedures. WO 0208868	
System ID - Safety Injection, Drawing No. N/A											
701700	2-PT-SI-1 Safety Injection Outside Containment	P	C	VT-2	2-PT-SI-1	-	-	-	X	10/3/2003 - VT-2 complete. Leakage noted at mechanical connections. Accepted as-is by evaluation. These will be corrected under plant procedures. WO 0208869	
701800	2-PT-SI-2 Safety Injection Outside Containment	P	C	VT-2	2-PT-SI-2	-	-	-	X	10/24/2003 - VT-2 performed, evidence of leakage noted at mechanical connections. Accepted as-is by evaluation. These will be corrected under plant procedures. WO 0208870	
System ID - Chemical Volume Control, Drawing No. N/A											

POINT BEACH NUCLEAR PLANT - UNIT 2  
INSERVICE INSPECTION RESULTS SUMMARY  
INTERVAL 4, PERIOD 1, OUTAGE 1 (U2R26)  
CLASS P PACLEBRN COMPONENTS

## Pressure Test

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATGY ITEM NO	STATUS		EXAM METHOD	DATA SHEET #	NON GET ORSEH EIOE CGMR				REMARKS WO NUMBER
System ID - Chemical Volume Control, Drawing No. N/A											
703100	2-PT-CV-1 CVCS Inside/Outside Containment	P	C	VT-2	2-PT-CV-1	-	-	-	X	11/11/2003 - VT-2 performed, evidence of leakage noted at mechanical connections. Accepted as-is by evaluation. These will be corrected under plant procedures. WO 0208863	
-----											
703200	2-PT-CV-2 CVCS Outside Containment	P	C	VT-2	2-PT-CV-2	-	-	-	X	11/11/2003 - VT-2 performed, evidence of leakage noted at mechanical connections. Accepted as-is by evaluation. These will be corrected under plant procedures. WO 0208876	
-----											
System ID - Reactor Coolant, Drawing No. N/A											
705200	2-PT-RCS-1 RCS System Leakage Test	B-P	C	VT-2	2-PT-RCS-1	-	-	-	X	U2R26 - Performed VT-2 on RCS system. Leakage noted at mecahnical connections. Accepted as-is by evaluation. These will be corrected under plant procedures. WO 0216320	

**UNIT 2 REFUELING 26**

**SUMMARY LIST**

**FOR**

**FORM NIS-2**

This report includes a listing of Unit 2 NIS-2 reports and a copy of the forms for the time period of May 14, 2002 through November 21, 2003.

RR Activity	Component	Dated	Work Order No.
2002-0037	SI-1501R-2, SI-1501R-3	April 30, 2002	9914721
2002-0113	MS-05958	October 11, 2002	0206087
2002-0114	MS-05959	November 15, 2002	0206088
2002-0115	SI-00870A	December 11, 2002	9934409
2003-0025	SI-00871B	May 5, 2003	0216629
2003-0027	SI-00845A	July 7, 2003	0206562
2003-0031	SI 0301R-03	June 24, 2003	9610973
2003-0035	C-1-D1V	June 24, 2003	9926799
2003-0036	C-1-D1V	June 24, 2003	9926799
2003-0037	C-2-D1V	June 24, 2003	9926780
2003-0038	C-2-D1V	June 24, 2003	9926780
2003-0039	SI-00853A	September 10, 2003	0215082
2003-0040	SI-00853B	September 10, 2003	0215083
2003-0041	SI-00853C	September 10, 2003	0215084
2003-0042	SI-00853D	September 10, 2003	0215085
2003-0046	CC-00721D	July 21, 2003	0202312
2003-0049	Piping EB-01-SP	August 8, 2003	0303659
2003-0050	Piping EB-01 SP	August 8, 2003	0303684
2003-0053	PS RC 2501R-14-2H1	August 27, 2003	0206026
2003-0055	RH-00715C	August 29, 2003	9937799
2003-0057	Piping SI 0151R	September 20, 2003	0300621
2003-0058	SI-00854A	July 7, 2003	0206562
2003-0059	RC-00434	October 7, 2003	0216385
2003-0060	C-2 outer vent	October 10, 2003	9926780
2003-0060-1	C-2 outer vent	October 10, 2003	9926780
2003-0061	T-01 Bolting	October 10, 2003	9945681
2003-0062	RH-00861C	October 11, 2003	0200287
2003-0064	T-013 Immersion Heater B	October 15, 2003	0203158
2003-0065	C-2 inner vent	October 10, 2003	9926780
2003-0066	RH-00715C	August 29, 2003	9937799
2003-0066-1	RH-00715C	August 29, 2003	9937799
2003-0067	SI-00871B	May 5, 2003	0216629
2003-0068	SI-00851A	October 16, 2003	9948308
2003-0070	HX-001A	October 16, 2003	0200102
2003-0071	AF-00100	October 16, 2003	0200225
2003-0072	RH-00716A	October 15, 2003	0216258
2003-0076	T-013 Immersion Heater B	October 18, 2003	9917877
2003-0077	Piping EB-9, Piping EB-10	October 19, 2003	0306491
2003-0078	RC-2501R-2-RC15	October 28, 2003	0309658
2003-0079	CPP-56	October 28, 2003	0213485
2003-0083	VNPSE-03212	November 8, 2003	0310051, 0310053

**Class 3 RR Activities:**

<b>RR Activity</b>	<b>Component</b>	<b>Dated</b>	<b>Work Order No.</b>
2001-0078	Piping DB-03	May 31, 2002	9914184
2002-0061	AF-00114	August 2, 2002	0210898
2003-0005	2RO-04003	April 1, 2003	0213452, 0215080
2003-0012	2RO-04003	April 1, 2003	0213452, 0215080
2003-0044	Piping HB-19	December 15, 2003	9946700, 9941615
2003-0063	HX-0015A8	November 19, 2003	0301204
2003-0084	Piping HB-19	November 18, 2003	0310113

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date November 18, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 2
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 9926780  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Containment  
Bechtel Spec 6118-C-8
5. (a) Applicable Construction Code ASME Sec. III, 19 65 Edition, 67 Addenda, None Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1992 Edition, 1992 Addenda  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work: Replacement of ball valves and weld replacement support plate.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☒ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure 66.0 psi Test Temp. Ambient °F
9. Remarks RR 2003-0037, 2003-0038, 2003-0060, 2003-0060-1, Bechtel Spec 6118-C-8; CAP053794  
Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

Russell T. Turner  
Owner or Owner's Designee, Title

ISI Coordinator Date

2/14/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period August 8, 2003 to November 18, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 2-16-2004





**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date August 2, 2002  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0210898  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Auxiliary Feedwater  
ANSI Spec B16.5; West 676241
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
AF-00114	Rockwell Mfg. Co.	N/A	N/A	AF-114	Unknown	Corrected	No

7. Description of Work: Buffed out linear indication on valve body.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F
9. Remarks RR 2002-0061

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date \_\_\_\_\_

1/23/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period July 29, 2002 to August 2, 2002, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

A. J. O'Brien  
Inspector's Signature

Commissions WI-100128

National Board, State, Province, and Endorsements

Date: JAN 27, 2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date May 31, 2002  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 9914184  
Repair Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Auxiliary Feedwater  
Bechtel Spec 6118-M-78
5. (a) Applicable Construction Code B31.1 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986 Edition No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
Piping DB-03	Unknown	N/A	N/A	N/A	Unknown	Repair	No

7. Description of Work: Repaired to increase the size of the welds to meet Code requirements and improve resistance to vibration fatigue.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐  
Other ☒ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None
9. Remarks RRM 2001-0078, CAP 028877, See RRM 2002-0061 WO 0210898  
Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in the report are correct and this repair conforms to the rules of the ASME Code, Section XI.  
repair or replacement

Type Code Symbol Stamp: N/A

Certificate of Authorization No. Docket 50-301

Expiration Date N/A

Signed *Michael L. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date 1/3/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Wisconsin and employed by HSB CT have inspected the components described in this Owner's Report during the period October 26, 2001 to May 30, 2002, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*U. S. Chokley*  
Inspector's Signature

Commissions WI-100128

National Board, State, Province, and Endorsements

Date: 1-23-03

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date November 19, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0301204  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Service Water  
PB-653
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
HX-0015A8 bolting	Unknown	N/A	N/A	Bolting	Unknown	Removed	No
HX-0015A8 bolting	Nova Machine Products Corp.	N/A	N/A	Bolting	Unknown	Installed	No

7. Description of Work: Replacement of bolting.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None Required
9. Remarks RR 2003-0063

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

*Russell H. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date

12/1/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 14, 2003 to November 19, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*RR*  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date:

12-2-2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date April 1, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WOs 0213452, 0215080  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Auxiliary Feedwater
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases N/A
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
2RO-04003	Byron Jackson Pump Div.	N/A	N/A	RO-4003	Unknown	Removed	No
2RO-04003	Byron Jackson Pump Div.	N/A	N/A	RO-4003	Unknown	Installed	No

7. Description of Work: Replacement of restricting orifice to address plugging issues.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ Exempt ☐  
Other ☐ Pressure 1380 psi Test Temp. 46/52 °F
9. Remarks RR 2003-0005, 2003-0012, CAP 032045

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

*Donald H. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date

2/11/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period February 27, 2003 to April 1, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*J. S. Oakes*

Inspector's Signature

Commissions WI-100128

National Board, State, Province, and Endorsements

Date:

April 23, 2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date December 15, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 9946700, 9941615  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Service Water  
Bechtel 6118-M-78
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
Piping HB-19	Unknown	N/A	N/A	Piping	Unknown	Removed	No
Piping HB-19	Unknown	N/A	N/A	Piping	Unknown	Installed	No

7. Description of Work: Replacement carbon steel piping with stainless steel piping.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ Exempt ☐  
Other ☐ Pressure 49 psi Test Temp. 53 °F
9. Remarks RR 2003-0044

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

[Signature]  
Owner or Owner's Designee, Title

ISI Coordinator Date

1/23/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period June 27, 2003 to December 15, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date:

1-23-2004

As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation Date November 18, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0310113  
 Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A
4. Identification of System: Service Water  
Spec M-78; Design Guide DG-M03
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
 (b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
 (c) Applicable Section XI Code Cases None

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
Piping HB-19	Unknown	N/A	N/A	piping	Unknown	Removed	No
Piping HB-19	Energy & Process Corp	N/A	N/A	piping	Unknown	Installed	No

7. Description of Work: Replacement of 2' of piping due to leak.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ Exempt ☐  
 Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F
9. Remarks RR 2003-0084

Applicable Manufacturer's Data Reports to be attached.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date 1/12/2004

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period November 11, 2003 to November 18, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature \_\_\_\_\_

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 1-23-2004

# FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

012

As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date November 13, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 9610973  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West Spec. G-676262; Bechtel M-78
5. (a) Applicable Construction Code B31.1, 19 67 Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
SI 0301R-03 Support	Unknown	N/A	N/A	None	Unknown	Corrected	No

7. Description of Work: Corrected pipe support to reduce line vibration.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required.
9. Remarks RR 2003-0031

Applicable Manufacturer's Data Reports to be attached.

## CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

Turnell L. Turner  
Owner or Owner's Designee, Title

ISI Coordinator Date

11/17/2003

## CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period June 24, 2003 to November 13, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date:

11-17-2003



**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date December 16, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 9945681  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Reactor Coolant  
West 676440 Rev. 5
5. (a) Applicable Construction Code B31.1/Sec. III, 19 68 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
Bolting	Westinghouse	Unknown	N/A	None	Unknown	Removed	No
Bolting	Nova/Jos Dyson & Sons	Unknown	N/A	None	Unknown	Installed	No

7. Description of Work: Replacement of bolting.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None Required
9. Remarks RR 2003-0061

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date 1/12/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 10, 2003 to December 16, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Commissions WI-247972

Inspector's Signature

National Board, State, Province, and Endorsements

Date: 1-23-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date January 9, 2004  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0306491  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Condensate System and Auxiliary Feedwater System  
Bechtel 6118-M-78
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
Piping EB-9	Unknown	N/A	N/A	N/A	Unknown	Corrected	No
Piping EB-10	Unknown	N/A	N/A	N/A	Unknown	Corrected	No

7. Description of Work: Removed and reinstalled piping.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
System Leakage Test (2-PT-AF-1)
9. Remarks RR 2003-0077, CAP 052701, CAP 051234

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date \_\_\_\_\_

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 19, 2003 to January 9, 2004, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 1-19-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date January 9, 2004  
700 First Street, Hudson, WI 54016 Sheet: 1 of 2
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0206562  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West Spect G-676262, West Spec G-677264, DG-M02
5. (a) Applicable Construction Code B31.1, 19 67 Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work: Replacement of check valve and associated piping.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ Exempt ☐  
Other ☐ Pressure 2235 psi Test Temp. 535 °F  
System Leakage Test
9. Remarks RR 03-0058, 2003-0027

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date 1/12/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period July 7, 2003 to January 9, 2004, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 1-27-2004

1. Owner	Nuclear Management Corporation 700 First Street, Hudson, WI 54016	Date	<u>January 9, 2004</u>
2. Plant	Point Beach Nuclear Plant 6610 Nuclear Road, Two Rivers, WI 54241	Sheet:	<u>2</u> of <u>2</u>
		Unit:	<u>2</u>
		WO	<u>0206562</u>
Repair/Replacement Organization P.O. No., Job No. Etc.			
3. Work Performed By:	<u>We Energies</u> name <u>6610 Nuclear Road</u> <u>Two Rivers, WI 54241</u> address	Type Code Symbol Stamp	<u>N/A</u>
		Authorization No.	<u>N/A</u>
		Expiration Date	<u>N/A</u>
4. Identification of System:	<u>Safety Injection</u> West Spec G-676262, West Spec G-677264, DG-M02		
5. (a) Applicable Construction Code	<u>B31.1 19 67 Edition, No Addenda, No Code Case</u>		
(b) Applicable Edition of Section XI Used for Repair/Replacement Activity	<u>1998 Ed. Addenda thru 2000</u>		
(c) Applicable Section XI Code Cases	<u>None</u>		
6. Identification of Components			

[illegible]

FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY  
As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation Date December 16, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0206088  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Main Steam  
Spec PB-109
5. (a) Applicable Construction Code Section III, 19 80 Edition, W80 Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
MS-05959	Copes Vulcan Div.	N/A	N/A	MS-5959	Unknown	Removed	No
MS-05959	Copes Vulcan Div.	N/A	N/A	MS-5959	Unknown	Installed	No

7. Description of Work: Replacement of valve trim.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☒ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
ORT-54, IT-745
9. Remarks RR 2002-0114, CAP 052961

Applicable Manufacturer's Data Reports to be attached.

CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

Murphy L. Turner  
Owner or Owner's Designee, Title

ISI Coordinator Date

1/20/2004

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period November 15, 2002 to December 16, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date:

1-21-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date November 18, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0206087  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Main Steam  
Spec PB-109
5. (a) Applicable Construction Code Section III, 19 80 Edition, W80 Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
MS-05958	Copes Vulcan Div.	8120-94532-1	N/A	MS-5958	Unknown	Removed	No
MS-05958	Copes Vulcan Div.	8120-94532-1	N/A	MS-5958	Unknown	Installed	No

7. Description of Work: Replacement of trim assembly.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☒ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
ORT-54, IT-745
9. Remarks RR 2002-0113, CAP 052731

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date 1/14/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 11, 2002 to November 18, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 1-20-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date December 16, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0215083  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West 676241
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
SI-00853B	Velan Valve Corp.	N/A	N/A	SI-853B	Unknown	Corrected	No

7. Description of Work: Seal welded to prevent boric acid leakage.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required
9. Remarks RR 2003-0040

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date \_\_\_\_\_

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period September 10, 2003 to December 16, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Commissions WI-247972

Inspector's Signature \_\_\_\_\_

National Board, State, Province, and Endorsements

Date: 1-19-2004

# FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date December 16, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0215084  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West 676241
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
SI-00853C	Velan Valve Corp.	N/A	N/A	SI-853C	Unknown	Corrected	No

7. Description of Work: Seal welded to prevent boric acid leakage.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required
9. Remarks RR 2003-0041

Applicable Manufacturer's Data Reports to be attached.

## CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

Murphy L. ...  
Owner or Owner's Designee, Title

ISI Coordinator Date

1/19/2003

## CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period September 10, 2003 to December 16, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 1-19-2004



**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date December 16, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0215085  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West 676241
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
SI-00S53D	Velan Valve Corp.	N/A	N/A	SI-853D	Unknown	Corrected	No

7. Description of Work: Seal welded to prevent boric acid leakage.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required
9. Remarks RR 2003-0042

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date 1/19/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period September 10, 2003 to December 16, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature \_\_\_\_\_

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 1-19-2004

# FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241
3. Work Performed By: We Energies  
name  
6610 Nuclear Road  
Two Rivers, WI 54241  
address
4. Identification of System: Containment Purge and Supply Exhaust Heating and Ventillation  
Bechtel Spec 6118-M-189
5. (a) Applicable Construction Code AWWA C-504-66, 19 N/A Edition, None Addenda, None Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1992 Ed. 1992 Addenda  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work: Replace butterfly valves with blank flanges.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☒ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure 63 psi Test Temp. ambient °F
9. Remarks RR 2003-0083

Applicable Manufacturer's Data Reports to be attached.

## CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

*Russell L. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date

12/11/2003

## CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period November 8, 2003 to November 18, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Date: 12-11-2003

Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

**As Required by the Provisions of the ASME Code Section XI**

Sheet: 2 of 2

Repair/Replacement Organization P.O. No., Job No. Etc.

address

(c) Applicable Section XI Code Cases None

[illegible]

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation Date November 13, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 2
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0216629  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West 676258
5. (a) Applicable Construction Code B31.1/USAS 16.5, 19 67/66 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
See Attached.							

7. Description of Work: Seal welded valve and replaced damaged bolting.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required
9. Remarks RRs 2003-0025, 2003-0067

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed *Michael S. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date 11/17/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period May 5, 2003 to November 13, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*Michael S. Turner*  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 11-24-2003

1. Owner	Nuclear Management Corporation 700 First Street, Hudson, WI 54016	Date	<u>November 13, 2003</u>
2. Plant	Point Beach Nuclear Plant 6610 Nuclear Road, Two Rivers, WI 54241	Sheet:	<u>2</u> of <u>2</u>
		Unit:	<u>2</u>
		WO	<u>0216629</u>
Repair/Replacement Organization P.O. No., Job No. Etc.			
3. Work Performed By:	<u>We Energies</u> name <u>6610 Nuclear Road</u> <u>Two Rivers, WI 54241</u> address	Type Code Symbol Stamp	<u>N/A</u>
		Authorization No.	<u>N/A</u>
		Expiration Date	<u>N/A</u>
4. Identification of System:	<u>Safety Injection</u> <u>West 676258</u>		
5. (a)	Applicable Construction Code <u>B31.1/USAS 16.5</u> 19 <u>67/66</u> Edition, <u>N/A</u> Addenda, <u>N/A</u> Code Case		
(b)	Applicable Edition of Section XI Used for Repair/Replacement Activity <u>1998 Ed. Addenda thru 2000</u>		
(c)	Applicable Section XI Code Cases <u>None</u>		
6. Identification of Components			

[illegible]

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date December 15, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0200225  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Auxiliary Feedwater  
Bechtel 6118-M-78; DG M03
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
AF-00100	Enertech	10740	N/A	AF-100	Unknown	Removed	No
AF-00100	Enertech	10741	N/A	AF-100	Unknown	Installed	No

7. Description of Work: Replacement of checkvalve.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure                      psi Test Temp.                      °F  
None Required
9. Remarks RR 2003-0071

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.                     

Expiration Date N/A

Signed                     

Owner or Owner's Designee, Title

ISI Coordinator Date 1/14/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 16, 2003 to December 15, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

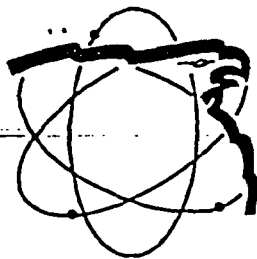
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

                      
Inspector's Signature

Commissions WI-247972

                      
National Board, State, Province, and Endorsements

Date: 1-16-2004



**ENERTECH**  
Curtiss-Wright Flow Control Corporation

*Solutions for Today...Standards for Tomorrow*  
(714) 528-2301

2950 Birch Street, Brea, CA 92821-6235  
FAX (714) 528-0128

## **Certificate of Conformance**

Date: July 2, 2003

Customer Purchase Order No. P031538

Customer: Nuclear Management Company, LLC

EnerTech Project No. 500043

Site: Point Beach Nuclear Plant

*EnerTech certifies that the parts referenced hereon were produced in accordance with the purchase order cited above and that EnerTech has on file the required data to substantiate conformance to applicable procurement specifications.*

*EnerTech further certifies that all tests and inspections required by the customer's purchase order(s) have been performed and that all items supplied herewith conform to the specified requirements.*


<u>P.O. Item No.</u>	<u>Qty.</u>	<u>Part Number</u>	<u>Description</u>
001	1	PD99532	VALVE, NOZZLE CHECK MODEL: 3"-600 ANSI DRV-Z S/N: 10741

**Additional Requirements:**

This item is provided in accordance with the EnerTech Quality Assurance Manual, Fourth Edition, Rev. 4 dated April 16, 2003.

The above item was returned on EnerTech RMA# 3045. The item conforms to the EnerTech Seismic Qualification Report PA99540 Rev A. Further, the Item described herein is designed in compliance with the requirements of USAS B31.1.0 - 1967, and ASA B16.5-1961.

NOTE: Supporting documentation, material test reports, etc. have been previously provided.

  
Signature  
David E. Whitaker  
Name  
Quality Assurance Engineer  
Title

*Performance of installation and associated service required by a certified EnerTech technician.*

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date December 15, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 9937799  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Residual Heat Removal  
WEST 77264
5. (a) Applicable Construction Code B16.34/B31.1, 19 NA/67 Edition, Unknown/NA Addenda, No/No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
RH-00715C	Henry Vogt Machine Co.	Unknown	N/A	RH-715C	Unknown	Removed	No
RH-00715C	Henry Vogt Machine Co.	2-230216	N/A	RH-715C	Unknown	Installed	No

7. Description of Work: Replacement of valve.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. NOT °F
9. Remarks RRs 2003-0055, 2003-0066, 2003-0066-1

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date \_\_\_\_\_

1/14/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period August 29, 2003 to December 15, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature \_\_\_\_\_

1-16-2004

Commissions WI-247972

National Board, State, Province, and Endorsements



# FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date December 15, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0202312  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Component Cooling Water  
MR 84-158
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
CC-00721D bolting	Unknown	N/A	N/A	None	Unknown	Removed	No
CC-00721D bolting	Nova Machine Products Corp.	N/A	N/A	None	Unknown	Installed	No

7. Description of Work: Replacement of valve inlet and outlet bolting.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required
9. Remarks RR 2003-0046

Applicable Manufacturer's Data Reports to be attached.

## CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

Russell H. Turner  
Owner or Owner's Designee, Title

ISI Coordinator Date

1/9/2004

## CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period July 21, 2003 to December 15, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date:

1-13-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date December 16, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0203158  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
Bechtel Spec M-21
5. (a) Applicable Construction Code API 650, 19 65 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
T-013 Immersion Heater B	Chromalox	Unknown	N/A	T-13	Unknown	Removed	No
T-013 Immersion Heater B	Chromalox	Unknown	N/A	T-13	Unknown	Installed	Yes

7. Description of Work: Replacement of immersion heater.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None Required
9. Remarks RR 2003-0064

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

Murphy L. Turner

Owner or Owner's Designee, Title

ISI Coordinator Date

1/19/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 15, 2003 to December 16, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 1-19-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date January 9, 2004  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 9917877  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
Bechtel Spec M-21
5. (a) Applicable Construction Code API 650, 19 65 Edition, N/A Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
T-013 Immersion Heater B	Chromalox	Unknown	No	T-13	Unknown	Removed	No
T-013 Immersion Heater B	Chromalox	Unknown	No	T-13	Unknown	Installed	Yes

7. Description of Work: Replacement of immersion heater.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None Required
9. Remarks RR 2003-0076, CAP 051021

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date \_\_\_\_\_

1/13/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 18, 2003 to January 9, 2004, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 1-13-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date November 18, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0200102  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Reactor Coolant  
WEST412A72
5. (a) Applicable Construction Code Section III, 19 86 Edition, N/A Addenda, N/A Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
HX-001A bolting	Westinghouse	N/A	N/A	Bolting	Unknown	Removed	No
HX-001A bolting	Westinghouse	N/A	N/A	Bolting	Unknown	Installed	No

7. Description of Work: Replaced bolting due to galling.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None Required
9. Remarks RR 2003-0070

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed *Russell E. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date 12/11/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 16, 2003 to November 18, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*[Signature]*  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 12-11-2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date November 10, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0216258  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West Spec 676241
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases \_\_\_\_\_
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
RH-00716A Bolting	Unknown	N/A	N/A	None	Unknown	Removed	No
RH-00716A Bolting	Nova	N/A	N/A	None	Unknown	Installed	No

7. Description of Work: Replaced stud
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☒ 2/14/01 Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required.
9. Remarks RR 2003-0072

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed *William H. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date 12/1/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 16, 2003 to November 10 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*BB*  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 12-2-2003

# FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY

As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date April 22, 2003  
Sheet: 1 of 2
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 9934409  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West. 676258
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases NONE
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
See Attached Page							

7. Description of Work: Installed seal weld and replaced body-to-bonnet bolts and nuts.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None
9. Remarks RR 2002-0115

Applicable Manufacturer's Data Reports to be attached.

## CERTIFICATE OF COMPLIANCE

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed *Russell L. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date 7/19/2003

## CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period December 11, 2002 to April 22, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*W. J. O'Leary*  
Inspector's Signature

Commissions WI-100128

National Board, State, Province, and Endorsements

Date: 8-26-03

1. Owner	Nuclear Management Corporation 700 First Street, Hudson, WI 54016	Date	<u>April 22, 2003</u>
2. Plant	Point Beach Nuclear Plant 6610 Nuclear Road, Two Rivers, WI 54241	Sheet:	<u>2</u> of <u>2</u>
		Unit:	<u>2</u>
		WO	<u>9934409</u>
Repair/Replacement Organization P.O. No., Job No. Etc.			
3. Work Performed By:	<u>We Energies</u> name <u>6610 Nuclear Road</u> <u>Two Rivers, WI 54241</u> address	Type Code Symbol Stamp	<u>N/A</u>
		Authorization No.	<u>N/A</u>
		Expiration Date	<u>N/A</u>
4. Identification of System:	<u>Safety Injection</u> <u>West 676258</u>		
5. (a)	Applicable Construction Code <u>B31.1 19 67</u> Edition, <u>N/A</u> Addenda, <u>No</u> Code Case		
(b)	Applicable Edition of Section XI Used for Repair/Replacement Activity <u>1998 Ed. Addenda thru 2000</u>		
(c)	Applicable Section XI Code Cases <u>None</u>		
6. Identification of Components			

[illegible]

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date October 23, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0303659  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Main Steam  
Bech. 6118-M-01 (West 676262)
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
Piping EB-01-SP	Unknown	None	N/A	N/A	Unknown	Corrected	No

7. Description of Work: Ensure a rounded radiuson inlet to safety valve MS-2010.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None
9. Remarks RR 2003-0049

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date 10/24/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period August 8, 2003 to October 23, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 10-31-03



**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
As Required by the Provisions of the ASME Code Section XI

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016 Date November 6, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241 Unit: 2  
WO 0213485  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Containment
5. (a) Applicable Construction Code B31.1, 19 67 Edition, N/A Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1992 Ed. 1992 Addenda  
(c) Applicable Section XI Code Cases None

6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected. Removed or Installed	ASME Code Stamped (Yes or No)
CPP-56 Stud	Unknown	N/A	N/A	N/A	Unknown	Removed	No
CPP-56 Stud	Accutech	N/A	N/A	N/A	1997	Installed	No

7. Description of Work: Replacement of galled stud.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required.
9. Remarks RR 2003-0079

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed *Russell H. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date 11/7/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 28, 2003 to November 6, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*RB*  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 11-10-2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date November 6, 2003  
Sheet: 1 of 2
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0300621  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West G-676262
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work: Removed and reinstalled piping and added new coupling.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒ Exempt ☐  
Other ☒ 2/1/401 Pressure \_\_\_\_\_ psi Test Temp. Ambient °F  
System Leakage Test performed
9. Remarks RR 2003-0057

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed Musallh Turner  
Owner or Owner's Designee, Title

ISI Coordinator Date 11/7/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period September 30, 2003 to November 6, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 11-10-2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY (CONTINUATION)**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner	Nuclear Management Corporation 700 First Street, Hudson, WI 54016	Date	November 6, 2003
		Sheet:	<u>2</u> of <u>2</u>
2. Plant	Point Beach Nuclear Plant 6610 Nuclear Road, Two Rivers, WI 54241	Unit:	2
			WO 0300621
<hr/>			
Repair/Replacement Organization P.O. No., Job No. Etc.			

3. --Work Performed By:	<u>We Energies</u>	Type Code Symbol Stamp	<u>N/A</u>
	<small>name</small>		
	<u>6610 Nuclear Road</u>	Authorization No.	<u>N/A</u>
	<u>Two Rivers, WI 54241</u>	Expiration Date	<u>N/A</u>
	<small>address</small>		

4. Identification of System: Safety Injection  
WEST G-676262

5. (a) Applicable Construction Code B31.1 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repair/Replacement Activity 1998 Ed. Addenda thru 2000

(c) Applicable Section XI Code Cases None

## 6. Identification of Components

[illegible]

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date October 23, 2003  
Sheet: 1 of 1 --- ---  
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0303684  
Repair/Replacement Organization P.O. No., Job No. Etc.  
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address  
4. Identification of System: Main Steam  
Bech. 6118-M-01 (West 676262)  
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None  
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
Piping EB-01-SP	Unknown	None	N/A	N/A	Unknown	Corrected	No

7. Description of Work: Ensure a rounded radius on inlet to safety valve MS-2005.  
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None  
9. Remarks RR 2003-0050

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed

*Russell L. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date

11/6/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period August 8, 2003 to October 23, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*[Signature]*  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date:

11-7-2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date October 24, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 9948308  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West Spec. 676258
5. (a) Applicable Construction Code USAS 16.5, 19 1966 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
SI-00851A nut	Unknown	N/A	N/A	nut	Unknown	Remove	No
SI-00851A nut	Unknown	N/A	N/A	nut	Unknown	Installed	No

7. Description of Work: Replacement of body to bonnet nut associated with SI-00851A.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None
9. Remarks RR 2003-0068

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date \_\_\_\_\_

11/6/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 16, 2003 to October 24, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 11-7-2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date October 23, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0200287  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Residual Heat
5. (a) Applicable Construction Code ASME III, 1971 Edition, Winter 1971 Class 2 Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
RH-00861C	Crosby	N56225-00-0002	N/A	RH-861C	1972	Removed	Yes
RH-00861C	Crosby	N56225-01-0003	N/A	RH-861C	2003	Installed	Yes

7. Description of Work: Replacement of relief valve.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None
9. Remarks RR 2003-0062

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.

Expiration Date N/A

Signed

*Russell L. Turner*  
Owner or Owner's Designee, Title

ISI Coordinator Date

10/24/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 11, 2003 to October 23, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

*[Signature]*  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date:

11-5-2003

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date November 7, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0206026  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Reactor Coolant  
West G-676262; Bech 6118-M-78
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
PS RC 2501R-14-2H1 Clevis Pin	Grinnell	N/A	N/A	None	Unknown	Removed	No
PS RC 2501R-14-2H1 Clevis Pin	Anvil International Inc.	N/A	N/A	None	Unknown	Installed	No

7. Description of Work: Replaced the clevis pin.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None Required
9. Remarks RR 2003-0053

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date \_\_\_\_\_

11/7/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period August 27, 2003 to November 7, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature \_\_\_\_\_

11-10-2003

Commissions WI-247972

National Board, State, Province, and Endorsements

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation  
700 First Street, Hudson, WI 54016  
Date November 6, 2003  
Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant  
6610 Nuclear Road, Two Rivers, WI 54241  
Unit: 2  
WO 0309658  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Reactor Coolant  
Bechtel 6118-M-78 Rev. 6
5. (a) Applicable Construction Code B31.1, 19 67 Edition, N/A Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
Support RC-2501R-2-RC15	Unknown	N/A	N/A	None	Unknown	Removed	No
Support RC-2501R-2-RC15	Unknown	N/A	N/A	None	Unknown	Installed	No

7. Description of Work: Replacement of threaded rod.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None Required.
9. Remarks RR 2003-0078

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A


Signed   
Owner or Owner's Designee, Title

ISI Coordinator Date 11/7/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 28, 2003 to November 6, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 11-10-2003



**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date December 16, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0215082  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West 676241
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
SI-00853A	Velan Valve Corp.	N/A	N/A	SI-853A	Unknown	Corrected	No

7. Description of Work: Seal welded to prevent boric acid leakage.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required
9. Remarks RR 2003-0039, CAP 52831

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date 1/21/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period September 10, 2003 to December 16, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 2-3-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date July 20, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 1  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0213015  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
Bechtel Spec M-78
5. (a) Applicable Construction Code B31.1, 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
HS-16	Grinnell Co.	3150	N/A	None	Unknown	Removed	No
HS-16	Crinnell Co.	3150	N/A	None	Unknown	Installed	No

7. Description of Work: Like-for-like replacement due to worn fitting connections.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☒  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F
9. Remarks RR 2003-0034

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed Muselli X. Turner  
Owner or Owner's Designee, Title

ISI Coordinator Date 7/22/2003

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period June 18, 2003 to July 20, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

W. J. Oakley  
Inspector's Signature

Commissions WI-100128

National Board, State, Province, and Endorsements

Date: 8-4-03

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date December 16, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 0216385  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Reactor Coolant  
PB-247 Rev. 3
5. (a) Applicable Construction Code Section III, 19 68 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1998 Ed. Addenda thru 2000  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
RC-00434	Crosby	N82732-00-0001	N/A	RC-434	N/A	Removed	Yes
RC-00434	Crosby	XX5950175	N/A	RC-434	N/A	Installed	Yes

7. Description of Work: Replacement of valve.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None Required
9. Remarks RR 2003-0059

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date 2/6/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 7, 2003 to December 16, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 2-12-2004

FORM NVR-1 REPORT OF REPAIR ☒ MODIFICATION ☐ OR REPLACEMENT ☐  
OF NUCLEAR PRESSURE RELIEF DEVICES

1. Work performed by Crane Nuclear, Inc. w/o 102679 / PD 14619  
2825 Cobb International Blvd., Kennesaw, GA 30152-4352
2. Work performed for NUCLEAR MANAGEMENT Co., LLC
3. Owner WISCONSIN ELECTRIC POWER Co.  
6610 NUCLEAR Rd. Two RIVERS, WI 54241 USA
4. Name, address and identification of nuclear power plant POINT BEACH NUCLEAR PLANT
5. a. Repair pressure relief device: S/N XX05950175
- b. Name of manufacturer: CROSBY
- c. Identifying nos. HB 86 BPA XX05950175 N/A STEAM 4 K2 G N/A
- d. Construction Code: ASME SEC VIII 1968 N/A N/A N/A N/A
6. ASME Code Section XI applicable for inservice inspection: N/A N/A N/A
7. ASME Code Section XI used for repairs, modifications, or replacements: 1986 N/A N/A
8. Construction Code used for repairs, modifications, or replacements: N/A N/A N/A
9. Design Responsibility CROSBY VALVE AND GAUGE, Co.
10. Opening pressure: N/A Blowdown (if applicable) N/A % Set pressure and blowdown adjustment made at: N/A using N/A
11. Description of work: (include name and identifying number of replacement parts) DISASSEMBLED, CLEANED, LAPPED SEATS, REPLACED NOZZLES P/N 904-0579, REASSEMBLED
12. Remarks:

CERTIFICATE OF COMPLIANCE

I, LEE H. KELLER, certify that to the best of my knowledge and belief the statements made in this report are correct and the repair, modification or replacement of the pressure relief devices described above conforms to Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

National Board Certificate of Authorization No. 473 to use the "VR" stamp expires July 22, 2005

National Board Certificate of Authorization No. 82 to use the "NR" stamp expires July 22, 2005

Date July 26, 2003 Crane Nuclear, Inc. Signed LEE H. KELLER

(name of certificate holder)

(authorized representative)

(title)

CERTIFICATE OF INSPECTION

I, R.M. WINTON, holding a valid commission issued by The National Board of Boiler and Pressure Vessel Inspectors and certificate of competency issued by the jurisdiction of GREEN and employed by THE HARTFORD STEAM BOILER INSPECTORS OF CONN. OF HARTFORD CONN have inspected the repair, modification or replacement described in this report on 7-26, 2003 and state that to the best of my knowledge and belief, this repair, modification or replacement has been completed in accordance with Section XI of the ASME Code and the National Board Inspection Code "VR" and "NR" rules.

By signing this certificate, neither the undersigned nor my employer makes any warranty, expressed or implied, concerning the repair, modification or replacement described in this report. Furthermore, neither the undersigned nor my employer shall be liable in any manner for any personal injury, property damage or loss of any kind arising from or connected with this inspection.

Date 7-26, 2003 Signed R.M. Winton Commissions NB5272-N, GR 264

(Inspector)

(National Board and jurisdiction)

(and jurisdiction and no.)

**FORM NIS-2 OWNER'S REPORT FOR REPAIRS OR REPLACEMENTS**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date May 1, 2002  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WOs 0203256, 0203257  
Repair Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Safety Injection  
West Spec G676262, ASTM A-312
5. (a) Applicable Construction Code B31.1 19 67 Edition, No Addenda, No Code Case  
(b) Applicable Edition of Section XI Utilized for Repairs or Replacements 1986 Edition No Addenda

6. Identification of Components Repaired or Replaced and Replacement Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Repaired, Replaced, or Replacement	ASME Code Stamped (Yes or No)
SI-1501R-2	N/A	N/A	N/A	N/A	Unknown	Replacement	No
SI-1501R-3	N/A	N/A	N/A	N/A	Unknown	Replacement	No

7. Description of Work: Modify the piping so that bigger holes can be used to thoroughly clean and examine piping.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☒  
Other ☐ Pressure NOP psi Test Temp. NOT °F  
System Leakage Test
9. Remarks RRM 2002-0037, ECR 2002-0074, Code Case N-416-1.  
Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

We certify that the statements made in the report are correct and this replacement conforms to the rules of the ASME Code, Section XI.  
repair or replacement

Type Code Symbol Stamp: N/A

Certificate of Authorization No. Docket 50-301

Expiration Date N/A

Signed Murphy J. Turner  
Owner or Owner's Designee, Title

ISI Coordinator Date 2/12/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Province of Wisconsin and employed by HSB CT have inspected the components described in this Owner's Report during the period May 1, 2002 to May 15, 2002, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature]  
Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 2-12-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date January 9, 2004  
700 First Street, Hudson, WI 54016 Sheet: 1 of 1
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 9926780  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Containment  
Bechtel Spec 6118-C-8
5. (a) Applicable Construction Code ASME Section III, 1992 Edition, 92 Addenda, No Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1992 Ed. Addenda thru 1992  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
C-2 support plate	Unknown	None	N/A	None	Unknown	Corrected	No

7. Description of Work: Moved plate to accommodate new venting device.
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☐ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure \_\_\_\_\_ psi Test Temp. \_\_\_\_\_ °F  
None required
9. Remarks RR 2003-0065; CAP 053794

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No. \_\_\_\_\_

Expiration Date N/A

Signed \_\_\_\_\_

Owner or Owner's Designee, Title

ISI Coordinator Date \_\_\_\_\_

2/14/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period October 15, 2003 to January 9, 2004, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

Inspector's Signature

Commissions WI-247972

National Board, State, Province, and Endorsements

Date: 2-16-2004

**FORM NIS-2 OWNER'S REPORT FOR REPAIR/REPLACEMENT ACTIVITY**  
**As Required by the Provisions of the ASME Code Section XI**

1. Owner Nuclear Management Corporation Date November 18, 2003  
700 First Street, Hudson, WI 54016 Sheet: 1 of 2
2. Plant Point Beach Nuclear Plant Unit: 2  
6610 Nuclear Road, Two Rivers, WI 54241 WO 9926779  
Repair/Replacement Organization P.O. No., Job No. Etc.
3. Work Performed By: We Energies Type Code Symbol Stamp N/A  
name  
6610 Nuclear Road Authorization No. N/A  
Two Rivers, WI 54241 Expiration Date N/A  
address
4. Identification of System: Containment
5. (a) Applicable Construction Code       , 19        Edition,        Addenda,        Code Case  
(b) Applicable Edition of Section XI Used for Repairs/Replacement Activity 1992 Ed. Addenda thru 1992  
(c) Applicable Section XI Code Cases None
6. Identification of Components

Name of Component	Name of Manufacturer	Manufacturer Serial No.	National Board No.	Other Identification	Year Built	Corrected, Removed or Installed	ASME Code Stamped (Yes or No)
See Attached							

7. Description of Work: Replacement of ball valves
8. Tests Conducted: Hydrostatic ☐ Pneumatic ☒ Nominal Operating Pressure ☐ Exempt ☐  
Other ☐ Pressure 66 psi Test Temp. 68 °F
9. Remarks RR 2003-0035, 2003-0036, Contruction Code Bechtel Spec 6118-C-8; CAP 053794

Applicable Manufacturer's Data Reports to be attached.

**CERTIFICATE OF COMPLIANCE**

I certify that the statements made in the report are correct and that this conforms to the rules of the ASME Code, Section XI.

Type Code Symbol Stamp: N/A

Certificate of Authorization No.                     

Expiration Date N/A

Signed                     

Owner or Owner's Designee, Title

ISI Coordinator Date 2/14/2004

**CERTIFICATE OF INSERVICE INSPECTION**

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State of Wisconsin and employed by HSBCT have inspected the components described in this Owner's Report during the period August 8, 2003 to November 18, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and taken corrective measures described in this Owner's Report in accordance with the requirements of ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

                      
Inspector's Signature

Commissions WI-247972

                      
National Board, State, Province, and Endorsements

Date: 2-16-2004

**As Required by the Provisions of the ASME Code Section XI**

1.	Owner	Nuclear Management Corporation 700 First Street, Hudson, WI 54016	Date	<u>January 13, 2004</u>
			Sheet:	<u>2</u> of <u>2</u>
2.	Plant	Point Beach Nuclear Plant 6610 Nuclear Road, Two Rivers, WI 54241	Unit:	<u>2</u>
				<u>WO 9926779</u>
Repair/Replacement Organization P.O. No.. Job No. Etc.				

3. Work Performed By:	<u>We Energies</u>	Type Code Symbol Stamp	<u>N/A</u>
	<u>name</u>		
	<u>6610 Nuclear Road</u>	Authorization No.	<u>N/A</u>
	<u>Two Rivers, WI 54241</u>	Expiration Date	<u>N/A</u>
	<u>address</u>		

4. Identification of System: Containment  
Bechtel Spec 6118-C-8

5. (a) Applicable Construction Code \_\_\_\_\_ 19 \_\_\_\_\_ Edition, \_\_\_\_\_ Addenda, \_\_\_\_\_ Code Case  
(b) Applicable Edition of Section XI Used for Repair/Replacement Activity \_\_\_\_\_ 1992 Ed. Addenda thru 1992

(c) Applicable Section XI Code Cases None

## 6. Identification of Components

[illegible]



NIS-1 Inservice Inspection  
Summary Report for  
U2R26 IWE

Nuclear Management Company, LLC  
700 First Street  
Hudson, Wisconsin 54016

Point Beach Nuclear Plant Unit 2  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

Commercial Operating Date: October 1, 1972

Report Date: February 11, 2004

Written by: Angela E. Gushki Date: 2-17-04  
Reviewed by: Russell E. Turner Date: 2/17/2004  
Approved by: Tom Sherman Date: 2/17/04

## FORM NIS-1 OWNER'S REPORT FOR INSERVICE INSPECTIONS

1. Owner: Nuclear Management Company, 700 First Street  
Hudson, Wisconsin 54016
2. Plant: Point Beach Nuclear Plant, 6610 Nuclear Road, Two Rivers, Wisconsin 54241
3. Plant Unit: 2      4. Owner Certificate of Authorization: Docket 50-301
5. Commercial Service Date: October 1, 1972      6. National Board Number for Unit: N/A
7. Components Inspected

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Containment	Bechtel	N/A	U009972	N/A

Bechtel Group, Inc.  
2280 Bates Avenue  
Concord, CA 94520

8. Examination Dates: May 15, 2002 to November 14, 2003
9. Inspection Period Identification: Second Period
10. Inspection Interval Identification: First Interval
11. Applicable Edition of Section XI: 1992 Addenda 1992
12. Date / Revision of Inspection Plan: August 24, 2001 / rev. 0
13. Abstract of Examinations: Include a list of examinations and a statement concerning status of work required for current Interval.

An Inservice Examination of containment metallic liner Class MC components of Nuclear Management Company (NMC) Point Beach Nuclear Plant (PBNP) Unit 2 was performed during the time period of May 15, 2002 to November 14, 2003. This was the second period of the first ten-year interval.

The containment liner was examined in accordance with the IWE Containment Inspection Program, which was prepared in accordance with the requirements of the American Society of Mechanical Engineers (ASME) Section XI, 1992 Edition, 1992 Addenda in accordance with 10 CFR 50.55a.

No Code Cases were used in the performance of examinations, tests, and repairs and replacements.

Visual examinations were conducted in accordance with ASME Section XI. WE Energies, Nuclear Management Company, Raytheon, and Lambert MacGill Thomas, Inc. personnel performed the examinations. See Attachment 1 for examinations.

There were eight repair/replacements performed on Class MC components during this IWE exam report.

- RR 2003-0035 for 2C-1-D1V replacement of equipment airlock inner door equalizing vent valve with an equalizing vent flange to reduce maintenance.
- RR 2003-0036 for 2C-1-D2V replacement of equipment airlock outer door equalizing vent valve with an equalizing vent flange to reduce maintenance.
- RR 2003-0037 for 2C-2-D1V replacement of personnel airlock inner door equalizing vent valve with an equalizing vent flange to reduce maintenance.
- RR 2003-0038 for 2C-2-D2V replacement of personnel airlock outer door equalizing vent valve with an equalizing vent flange to reduce maintenance.
- RR 2003-0060-1 for 2C-2-D2V relocation of personnel airlock outer door equalizing vent support for installation performed in RR 2003-0036.
- RR 2003-0065-1 for 2C-2-D2V relocation of personnel airlock inner door equalizing vent support for installation performed in RR 2003-0037.
- RR 2003-0079 for 2CPP-56 to replace one flange bolt.
- RR 2003-0083 to replace containment isolation valve on purge supply and exhaust penetrations with blank flanges.

This exam meets the requirements of Inspection Program B of ASME Section XI with approximately 61% examination completed for the interval.

#### 14. Abstract of Conditions Noted:

See Attachment 1 for a summary of examinations performed.

#### Containment Liner Plate

One liner plate contained surface corrosion where paint had been removed as a result of repeated impact such as a scaffolding pole. No loss of base metal was noted.

**Process Piping Penetrations**

Two piping penetrations for the steam generator blowdown lines have noted corrosion starting on the coating. This is the start of a coatings failure and was identified during an augmented examination.

**15. Abstract of Corrective Measures Recommended and Taken.**

**Containment Liner Plates**

Touch up painting to the affected area will be performed. Surface preparation for painting will remove the surface corrosion.

**Process Piping Penetrations**

The process piping will be recoated at next available opportunity to maintain the corrosion protection of the coating.

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI and c) corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. (if applicable): NRC Docket 50-301 Expiration Date: N/A

Date: 2-17-04 Signed: Nuclear Management Co., LLC by Raymond G. Gunkel  
Owner

#### CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Wisconsin and employed by Hartford Steam Boiler of Hartford Connecticut, have inspected the components described in this Owner's Report during the period May 16, 2002 to November 14, 2003, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owner's Report in accordance with the Inspection Plan and as required by the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

  
\_\_\_\_\_  
Inspector's Signature

Commissions WI 247972  
National Board, State, Province,  
and Endorsements

Date 2-17-2004

## Attachment 1

### Unit 2 Refueling Outage 26 IWE Containment Inspection Program Examination Summary

Nuclear Management Company  
700 First Street  
Hudson, Wisconsin 54016  
National Board Number for Unit: N/A

Commercial Service Date: 10/01/72  
Docket 50-301  
Pt. Beach Unit 2

Component	Category	Item	Exam Type	NDE Data Sheet	RRM #	IDR	Notes
2C-1-D1V	E-G	E8.10	VT-1		2003-0035		WO 9926779
2C-1-D2V	E-G	E8.10	VT-1		2003-0036		WO 9926779
2C-2-D1V	E-G	E8.10	VT-1		2003-0037		WO 9926780
2C-2-D2V	E-G	E8.10	VT-1		2003-0038		WO 9926780
2C-3	E-G	E8.20	Torque				RP-1B
2CH-01	E-C	E4.11	VT-1	03U2-760E002		03U2-E002	
	E-C	E4.12	UT	03U2-114E001		03U2-E002	
	E-D	E5.30	VT-3	03U2-760E074			
2CH-02	E-C	E4.11	VT-1	03U2-760E004		03U2-E002	
	E-C	E4.12	UT	03U2-114E002		03U2-E002	
	E-D	E5.30	VT-3	03U2-760E074			
2CH-03	E-C	E4.11	VT-1	03U2-760E006		03U2-E002	
	E-C	E4.12	UT	03U2-114E003		03U2-E002	
	E-D	E5.30	VT-3	03U2-760E074			
2CH-04	E-C	E4.11	VT-1	03U2-760E008		03U2-E002	
	E-C	E4.12	UT	03U2-114E004		03U2-E002	
	E-D	E5.30	VT-3	03U2-760E074			

Component	Category	Item	Exam Type	NDE Data Sheet	RRM #	IDR	Notes
2CH-05	E-C	E4.11	VT-1	03U2-760E009		03U2-E011	
	E-C	E4.12	UT	03U2-114E005		03U2-E011	
	E-D	E5.30	VT-3	03U2-760E010			
2CH-06	E-C	E4.11	VT-1	03U2-760E011		03U2-E011	
	E-C	E4.12	UT	03U2-114E006		03U2-E011	
	E-D	E5.30	VT-3	03U2-760E012			
2CH-07	E-C	E4.11	VT-1	03U2-760E013		03U2-E011	
	E-C	E4.12	UT	03U2-114E007		03U2-E011	
	E-D	E5.30	VT-3	03U2-760E014			
2CP-025	E-A	E1.12	VT-3	03U2-760E015			
2CP-026	E-A	E1.12	VT-3	03U2-760E016			
2CP-027	E-A	E1.12	VT-3	03U2-760E017			
2CP-028	E-A	E1.12	VT-3	03U2-760E018			
2CP-029	E-A	E1.12	VT-3	03U2-760E019			
2CP-030	E-A	E1.12	VT-3	03U2-760E020			
2CP-031	E-A	E1.12	VT-3	03U2-760E021			
2CP-032	E-A	E1.12	VT-3	03U2-760E022			



Component	Category	Item	Exam Type	NDE Data Sheet	RRM #	IDR	Notes
2CP-033	E-A	E1.12	VT-3	03U2-760E023			
2CP-034			UT	03U2-114E028		03U2-E003	
			VT-3	03U2-760E079			
	E-A	E1.12	VT-3	03U2-760E024		03U2-E003	
2CP-035	E-A	E1.12	VT-3	03U2-760E025			
2CP-036	E-A	E1.12	VT-3	03U2-760E026			
2CP-037	E-A	E1.12	VT-3	03U2-760E027			
2CP-038	E-A	E1.12	VT-3	03U2-760E028			
2CP-039	E-A	E1.12	VT-3	03U2-760E029			
2CP-040	E-A	E1.12	VT-3	03U2-760E030			
2CP-041	E-A	E1.12	VT-3	03U2-760E031			
2CP-042	E-A	E1.12	VT-3	03U2-760E032			
2CP-043	E-A	E1.12	VT-3	03U2-760E033			
2CP-044	E-A	E1.12	VT-3	03U2-760E034			
2CP-045	E-A	E1.12	VT-3	03U2-760E035			
2CP-046	E-A	E1.12	VT-3	03U2-760E036			

Component	Category	Item	Exam Type	NDE Data Sheet	RRM #	IDR	Notes
2CP-047	E-A	E1.12	VT-3	03U2-760E037			
2CP-048	E-A	E1.12	VT-3	03U2-760E038			
2CP-049	E-A	E1.12	VT-3	03U2-760E039			Small paint scrapes
2CP-050	E-A	E1.12	VT-3	03U2-760E040			Limited to exposed areas
2CP-051	E-A	E1.12	VT-3	03U2-760E041			Limited to exposed areas
2CP-052	E-A	E1.12	VT-3	03U2-760E042			Small paint scrapes
2CP-053	E-A	E1.12	VT-3	03U2-760E043		03U2-E001	Small paint scrapes
2CP-054	E-A	E1.12	VT-3	03U2-760E044			Small paint scrapes
2CP-055	E-A	E1.12	VT-3	03U2-760E045			Small paint scrapes
2CP-056	E-A	E1.12	VT-3	03U2-760E046			Small paint scrapes
2CP-057	E-A	E1.12	VT-3	03U2-760E047			Small paint scrapes
2CP-058	E-A	E1.12	VT-3	03U2-760E048			Small paint scrapes. Limited to exposed areas
2CP-059	E-A	E1.12	VT-3	03U2-760E049			Small paint scrapes
2CP-060	E-A	E1.12	VT-3	03U2-760E050			Small paint scrapes
2CP-116			VT-3	03U2-760E051			PBNP re-exam

Component	Category	Item	Exam Type	NDE Data Sheet	RRM #	IDR	Notes
2CP-127			VT-3	03U2-760E052		03U2-E009	PBNP re-exam
2CP-128			VT-3	03U2-760E053			PBNP re-exam
2CP-129	E-C	E.12	UT	03U2-114E008		03U2-E008	
	E-C	E4.11	VT-1	03U2-760E054		03U2-E008	
2CP-133	E-C	E4.11	VT-1	03U2-760E055		03U2-E011	
	E-C	E4.12	UT	03U2-114E009		03U2-E011	
2CP-134	E-C	E4.11	VT-1	03U2-760E056		03U2-E011	
	E-C	E4.12	UT	03U2-114E010		03U2-E011	
2CP-135			UT	03U2-114E011		03U2-E011	PBNP (grid)
	E-C	E4.11	VT-1	03U2-760E057		03U2-E011	
	E-C	E4.12	UT	03U2-114E011			
2CP-137	E-C	E4.11	VT-1	03U2-760E058		03U2-E011	
	E-C	E4.12	UT	03U2-114E013		03U2-E011	
2CP-138	E-C	E4.11	VT-1	03U2-760E059		03U2-E011	
	E-C	E4.12	UT	03U2-114E014		03U2-E011	

Component	Category	Item	Exam Type	NDE Data Sheet	RRM #	IDR	Notes
2CP-139			UT	03U2-114E016		03U2-E011	PBNP (grid)
	E-C	E4.11	VT-1	03U2-760E060		03U2-E011	
	E-C	E4.12	UT	03U2-114E016		03U2-E011	
2CP-140	E-C	E4.11	VT-1	03U2-760E061		03U2-E011	
	E-C	E4.12	UT	03U2-114E017		03U2-E011	
2CPP-35	E-C	E4.11	VT-1	03U2-760E062			
	E-C	E4.12	UT	03U2-114E018			
2CPP-36	E-C	E4.11	VT-1	03U2-760E063		03U2-E004	
	E-C	E4.12	UT	03U2-114E019		03U2-E004	
2CPP-39	E-C	E4.11	VT-1	03U2-760E064			
	E-C	E4.12	UT	03U2-114E020			
2CPP-40	E-C	E4.11	VT-1	03U2-760E065			
	E-C	E4.12	UT	03U2-114E021			
2CPP-43	E-C	E4.11	VT-1	03U2-760E066			
	E-C	E4.12	UT	03U2-114E022			
2CPP-44	E-C	E4.11	VT-1	03U2-760E067		03U2-E010	
	E-C	E4.12	UT	03U2-114E023		03U2-E010	
2CPP-47	E-C	E4.11	VT-1	03U2-760E068			
	E-C	E4.12	UT	03U2-114E024			

Component	Category	Item	Exam Type	NDE Data Sheet	RRM #	IDR	Notes
2CPP-48	E-C	E4.11	VT-1	03U2-760E069		03U2-E005	
	E-C	E4.12	UT	03U2-114E025		03U2-E005	
2CPP-50	E-C	E4.11	VT-1	03U2-760E070		03U2-E006	
	E-C	E4.12	UT	03U2-114E026		03U2-E006	
2CPP-51	E-C	E4.11	VT-1	03U2-760E071		03U2-E007	
	E-C	E4.12	UT	03U2-114E027		03U2-E007	
2CPP-56	E-G	E8.10	VT-1	03U2-760E072	2003-0079		(outer flange bolting)
	E-G	E8.20	Torque		2003-0079		(inner and outer flange bolting)
2CPP-57	E-A	E1.12	VT-3	03U2-760E077			
2CPP-58	E-A	E1.12	VT-3	03U2-760E078			
2CPP-V-1	E-A	E1.12	VT-3	03U2-760E080	2003-0083		
	E-A	E1.12	VT-3	03U2-760E075			
	E-G	E8.10	VT-1	03U2-760E081	2003-0083		
2CPP-V-2	E-A	E1.12	VT-3	03U2-760E082	2003-0083		
	E-A	E1.12	VT-3	03U2-760E076			
	E-G	E8.10	VT-1	03U2-760E083	2003-0083		
2RB-01			augmented	03U2-762E003			reference bar measurement

Nuclear Management Company  
700 First Street  
Hudson, Wisconsin 54016  
National Board Number for Unit: N/A

Commercial Service Date: 10/01/72  
Docket 50-301  
Pt. Beach Unit 2

Component	Category	Item	Exam Type	NDE Data Sheet	RRM #	IDR	Notes
2RB-02			augmented	03U2-762E003			reference bar measurement
2RB-03			augmented	03U2-762E003			reference bar measurement
2RB-04			augmented	03U2-762E003			reference bar measurement
2RB-05			augmented	03U2-762E003			reference bar measurement
2RB-06			augmented	03U2-762E003			reference bar measurement
2Z-529	E-D	E5.30	VT-3	03U2-760E074			
U2-Containment	E-A	E1.11	GV	03U2-762E001			exterior
	E-A	E1.11	GV	03U2-762E002			interior

NIS-1 Inservice Inspection  
Summary Report for  
Unit 1 IWL

Nuclear Management Company, LLC  
700 First Street  
Hudson, Wisconsin 54016

Point Beach Nuclear Plant Unit 1  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

Commercial Operating Date: December 21, 1970

Report Date: February 11, 2004

Written by: Lucretia Ghent Date: 2-18-04

Reviewed by: Russell E. Turner Date: 2/18/2004

Approved by: [Signature] Date: 2/18/04

## FORM NIS-1 OWNER'S REPORT FOR INSERVICE INSPECTIONS

1. Owner: Nuclear Management Company, 700 First Street  
Hudson, Wisconsin 54016
2. Plant: Point Beach Nuclear Plant, 6610 Nuclear Road, Two Rivers, Wisconsin 54241
3. Plant Unit: 1 4. Owner Certificate of Authorization: Docket 50-266
5. Commercial Service Date: December 21, 1970 6. National Board Number for Unit: N/A
7. Components Inspected

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Containment	Bechtel	N/A	U009971	N/A

Bechtel Group, Inc.  
2280 Bates Avenue  
Concord, CA 94520

8. Examination Dates: September 10, 2001 to February 11, 2004
9. Inspection Period Identification: N/A
10. Inspection Interval Identification: N/A
11. Applicable Edition of Section XI: 1992 Addenda 1992
12. Date / Revision of Inspection Plan: October 25, 2002 / rev. 1
13. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current Interval.

An Inservice Examination of containment concrete Class CC components of Nuclear Management Company (NMC) Point Beach Nuclear Plant (PBNP) Unit 1 was performed during the time period of September 10, 2001 to February 11, 2004.



The concrete was examined in accordance with the IWL Containment Inspection Program, which was prepared in accordance with the requirements of the American Society of Mechanical Engineers (ASME) Section XI, 1992 Edition, 1992 Addenda and 10 CFR 50.55a.

No Code Cases were used in the performance of examinations, tests, and repairs and replacements.

The visual tendon surveillance was completed during the implementation period in accordance with Technical Specifications. Precision Surveillance Corporation and WE Energies personnel performed this surveillance. See Attachment 1 for examinations.

Visual examinations were conducted in accordance with ASME Section XI to meet examination Category L-A, Item L1.11 for VT-3C and Item L1.12 for VT-1C. WE Energies, Nuclear Management Company, and Lambert MacGill Thomas, Inc. personnel performed the examinations. Visual examinations were conducted at the same time as the tendon surveillance. See Attachment 2 for examinations.

No repairs and replacements have been performed on Class CC components during this IWL implementation period.

All examinations required for 33 year tendon surveillance have been completed. This meets the requirements of ASME Section XI and applicable relief requests.

#### 14. Abstract of Conditions Noted:

- Grease Leakage  
Areas of grease leakage at the tendon grease cans and through concrete leakage were recorded. No appreciable change was noted from conditions recorded during the 2001 concrete examination.
- Exposed Reinforcing Steel  
Areas of exposed reinforcing steel were recorded. There was no measurable difference since the 2001 examinations
- Concrete Cracking  
The concrete cracking recorded showed no change with previously recorded indications. One area of concrete damage was noted that was not previously recorded.

15. Abstract of Corrective Measures Recommended and Taken.

- Grease Leakage  
Additional corrective maintenance will be pursued to correct grease leakage points.
- Exposed Reinforcing Steel  
No corrective measures are recommended for these construction defects.
- Concrete Cracking  
Trending of existing cracks will continue to verify stability of concrete.

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI and c) corrective measures taken conform to the rules of the ASME Code, Section XI.


Certificate of Authorization No. (if applicable): NRC Docket 50-266 Expiration Date: N/A

Date: 2-18-04 Signed: Nuclear Management Co., LLC by Raydel Hanki  
Owner

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Wisconsin and employed by Hartford Steam Boiler of Hartford Connecticut, have inspected the components described in this Owner's Report during the period September 10, 2001 to February 11, 2004, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owner's Report in accordance the Inspection Plan and as required by the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

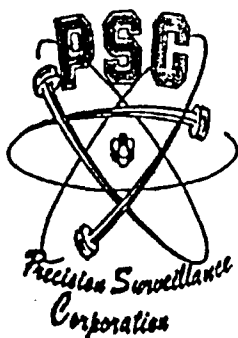
  
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Inspector's Signature

Commissions WI 247972  
National Board, State, Province,  
and Endorsements

Date 2-18-2004

## Attachment 1

### Unit 1 Thirty-three Year Tendon Surveillance Summary



33<sup>rd</sup> YEAR UNIT 1 PHYSICAL AND  
UNIT 2 VISUAL TENDON  
SURVEILLANCE AT THE POINT  
BEACH NUCLEAR PLANT



TABLE I: LABORATORY ANALYSIS OF SHEATHING FILLER UNIT 1

TENDON	END	ION CONCENTRATION (PPM)			% WATER CONTENT	NEUTRAL No. mg KOH/g
		CHLORIDE	SULFIDE	NITRATE		
AC-19	SHOP/BUTT.A.	<0.50	<0.50	<0.50	<0.10	10.5
BF-25	SHOP/BUTT.B	<0.50	<0.50	<0.50	<0.10	<0.50
CE-7	SHOP/BUTT. C	<0.50	<0.50	<0.50	<0.10	<0.50
DF-46	SHOP/BUTT. F	<0.50	<0.50	<0.50	<0.10	<0.50
DF-55	SHOP/BUTT. F	<0.50	<0.50	<0.50	0.14	<0.50
EA-45	SHOP/BUTT.A	<0.50	<0.50	<0.50	<0.10	<0.50
V-5	SHOP/TOP	0.50	<0.50	<0.50	0.32	<0.50
V-32	SHOP/TOP	0.50	<0.50	<0.50	0.32	<0.50
V-88	SHOP/TOP	<0.50	<0.50	<0.50	<0.10	<0.50
V-144	SHOP/TOP	<0.50	<0.50	<0.50	<0.10	<0.50
D1-15	SHOP/NEAR A	0.50	<0.50	<0.50	0.10	<0.50
D1-36	SHOP/NEAR A	0.50	<0.50	<0.50	<0.10	<0.50
D1-38	SHOP/NEAR A	<0.50	<0.50	<0.50	<0.10	<0.50
D2-25	SHOP/NEAR B	0.50	<0.50	<0.50	<0.10	<0.50
D3-9	SHOP/NEAR F	1.00	<0.50	<0.50	0.20	<0.50
D3-10	SHOP/NEAR F	<0.50	<0.50	<0.50	0.15	5.17
D3-15	SHOP/NEAR F	1.00	<0.50	<0.50	<0.10	<0.50

Acceptance Limits

<u>Test</u>	<u>Limits</u>
Water Soluble Chloride	Less than 10.0 ppm
Water Soluble Nitrates	Less than 10.0 ppm
Water Soluble Sulfides	Less than 10.0 ppm
Water Content	Less than 10% Dry Weight
Neutralization No.	Greater than 0 mg KOH/g



**33<sup>rd</sup> YEAR UNIT 1 PHYSICAL AND  
UNIT 2 VISUAL TENDON  
SURVEILLANCE AT THE POINT  
BEACH NUCLEAR PLANT**



**TABLE XI: SUMMARY OF DATA SHEETS SQ 12.1  
GREASE LOSS Vs GREASE REPLACEMENT UNIT 1**

TENDON	GREASE REMOVED			GREASE REPLACED			DIFF. (GAL.)	NET VOLUME	%
	SHOP	FIELD	TOTAL (GAL.)	SHOP	FIELD	TOTAL (GAL.)			
AC-19	2.25	2.25	4.50	2.25	2.50	4.75	+0.25	56.83	+0.43
BF-25	2.25	2.25	4.50	2.50	2.25	4.75	+0.25	56.78	+0.44
CE-7	2.25	2.25	4.50	2.50	2.50	5.00	+0.50	56.55	+0.88
DF-46	1.75	2.25	4.00	2.50	2.50	5.00	+1.00	56.86	+1.70
DF-55	3.25	2.50	5.75	3.50	2.50	6.00	+0.25	56.71*	+0.44**
EA-45	1.75	2.25	4.00	1.75	2.50	4.25	+0.25	56.55	+0.44
V-5	2.25	37.25	39.50	0.00	39.75	39.75	+0.25	66.83	+0.37
V-32	2.25	40.25	42.50	0.00	43.25	43.25	+0.75	66.81	+1.12
V-88	2.25	40.25	42.50	0.00	44.00	44.00	+1.50	67.23	+2.23
V-144	3.25	37.25	40.50	0.00	42.25	42.25	+1.75	66.87	+2.61
D1-15	3.75	4.00	7.75	4.00	5.25	9.25	+1.50	51.04*	+2.93**
D1-36	2.25	2.75	5.00	2.50	2.50	5.00	0.00	51.04*	0.00
D1-38	2.25	2.75	5.00	2.50	2.50	5.00	0.00	51.05	0.00
D2-25	3.25	3.25	6.50	3.50	3.50	7.00	+0.50	54.04	+0.92
D3-9	2.25	2.25	4.50	2.75	2.50	5.25	+0.75	48.21	+1.56
D3-10	2.25	2.25	4.50	2.50	2.50	5.00	+0.50	50.86	+0.98
D3-15	3.25	3.75	7.00	3.50	4.50	8.00	+1.00	51.04*	+1.96**

\* Group average.

\*\* Acceptable base upon group average



33<sup>rd</sup> YEAR UNIT 1 PHYSICAL AND  
UNIT 2 VISUAL TENDON  
SURVEILLANCE AT THE POINT  
BEACH NUCLEAR PLANT



TABLE VIII: SUMMARY OF DATA SHEETS SQ 9.0 - TENDON LIFTOFFS UNIT 1

TENDON	END	EFFECT. WIRES	JACK No.	PRESS.	LIFTOFF	AVE. LIFTOFF	LOWER LIMIT	95% LOWER	90% LOWER	UPPER LIMIT	ACCEPT
AC-19	SHOP/ BUTT A	90	6002	3460	653.7	648.2	601.2	571.1	541.1	664.2	YES
	FIELD/BUTT.C	90	6001	3390	642.8						
BF-25	SHOP/BUTT. B	90	6001	3220	610.3	618.9	601.2	571.1	541.1	664.2	YES
	FIELD/BUTT.F	90	6001	3310	627.5						
CE-7	SHOP/BUTT. C	90	6001	3460	656.2	658.7	601.2	571.1	541.1	664.2	YES
	FIELD/BUTT. E	90	6002	3500	661.3						
DF-55	SHOP/BUTT.F	90	6001	3360	637.1	633	601.2	571.1	541.1	664.2	YES
	FIELD/BUTT D	90	6002	3330	628.9						
EA-45	SHOP/BUTT.A	90	6002	3360	634.6	628.8	601.2	571.1	541.1	664.2	YES
	FIELD/BUTT.E	90	6002	3300	623.1						
V-5	SHOP/TOP	90	6002	3600	680.3	680.3	628.2	596.8	565.4	672.3	YES
V-32	SHOP/TOP	90	6001	3530	669.6	669.6	628.2	596.8	565.4	672.3	YES
V-88	SHOP/TOP	90	6001	3540	671.5	671.5	628.2	596.8	565.4	672.3	YES
V-144	SHOP/TOP	90	6002	3360	634.6	634.6	628.2	596.8	565.4	672.3	YES

**NMC**  
*Committed to Nuclear Excellence*



33<sup>rd</sup> YEAR UNIT 1 PHYSICAL AND  
UNIT 2 VISUAL TENDON  
SURVEILLANCE AT THE POINT  
BEACH NUCLEAR PLANT



TABLE IX: SUMMARY OF DATA SHEETS SQ 10.2 & 10.3-VISUAL INSPECTION AND TENSILE TESTING OF WIRES-UNIT 1

TENDON	SAMPLE No.	CORROSION LEVEL	SAMPLE LOCATION (FT)	DIAMETER (IN)	YIELD STRENGTH (PSI)	ULTIMATE STRENGTH (PSI)	ELONGATION (IN)	ACCEPTABLE
DF-55	1	A	20'-29'	0.250	217,654	259,656	4.20	YES
	2	A	60'-69'	0.250	218,927	260,292	4.05	YES
	3	A	100'-109'	0.250	216,382	261,565	4.30	YES
V-88	1	A	20'-29'	0.249	228,425	264,997	4.10	YES
	2	A	70'-79'	0.249	226,500	265,638	4.25	YES
	3	A	120'-129'	0.249	225,217	264,355	4.10	YES
D3-15	1	A	20'-29'	0.251	219,727	266,448	4.85	YES
	2	A	50'-59'	0.251	220,358	266,448	4.65	YES
	3	A	80'-89'	0.251	222,253	268,974	5.05	YES





33<sup>rd</sup> YEAR UNIT 1 PHYSICAL AND  
UNIT 2 VISUAL TENDON  
SURVEILLANCE AT THE POINT  
BEACH NUCLEAR PLANT



TABLE X: SUMMARY OF DATA SHEETS SQ 11.0  
TENDON RETENSIONING UNIT 1

WIRE REMOVAL TENDONS

TENDON	END	ORIGINAL ELONGATION		OBSERVED ELONGATION		% VARI.	ACCEPT	AVE. LIFTOFF BEFORE RETEN.	RETENSIONING			% VARI.	ACCEPT.
		EACH	TOTAL	EACH	TOTAL				JACK	PRESS.	AVE. L/OFF		
DF-55	SHOP	—	8.3125	4.4	8.3	-0.15	YES	633	6001	3660	684.9	+8.20*	YES
	FIELD	—	—	3.90	—	—	—	—	6002	3573	—	—	—
V-88	SHOP	—	9.375	9.4	9.4	+0.26	YES	671.5	6001	3620	686.8	+2.28	YES
D3-15	SHOP	—	7.125	3.55	7.15	+0.35	YES	644.4	6001	3550	682.6	+5.93	YES
	FIELD	—	—	3.6	—	—	—	—	6002	3660	—	—	—

\* Within 6% of Upper Limit Predicted Force per SQ 9.0 Section 9.8.7.4

## Attachment 2

### Unit 1 IWL Containment Concrete Examination Summary

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
	Tendon Gallery Ceiling	V-055, V-056	UT, supplemental	03U1-L002
03U1-761L001	Tendon Gallery Ceiling	Tendon Gallery Ceiling	VT-1C, direct	03U1-L003
03U1-761L002	Tendon Gallery Ceiling	Tendon Gallery Ceiling	VT-1C, direct	03U1-L003
03U1-761L003	Tendon Gallery Ceiling	Tendon Gallery	VT-3C, direct	03U1-L003
03U1-761L004	Tendon Gallery Inner Wall	Tendon Gallery	VT-3C, direct	03U1-L002, 03U1-L003
03U1-761L005	Tendon Gallery Outer Wall	Tendon Gallery	VT-3C, direct	03U1-L002, 03U1-L003
03U1-761L006	Dome	Top of Dome	VT-3C, direct	03U1-L001, 03U1-L002
03U1-761L007	Dome	LDR-A-1	VT-3C, direct	
03U1-761L008	Dome	PLTF-A-1	VT-3C, direct	03U1-L001
03U1-761L009	Dome	PLTF-A-1	VT-3C, remote	03U1-L001
03U1-761L010	Dome	LDR-B-4	VT-3C, direct	
03U1-761L011	Dome	PLTF-B-3	VT-3C, direct	03U1-L001
03U1-761L012	Dome	PLTF-B-3	VT-3C, remote	03U1-L001
03U1-761L013	Dome	LDR-C-1	VT-3C, direct	
03U1-761L014	Dome	PLTF-C-1	VT-3C, direct	03U1-L001
03U1-761L015	Dome	PLTF-C-1	VT-3C, remote	03U1-L001
03U1-761L016	Dome	LDR-D-2	VT-3C, direct	
03U1-761L017	Dome	PLTF-D-3	VT-3C, direct	03U1-L001
03U1-761L018	Dome	PLTF-D-3	VT-3C, remote	03U1-L001
03U1-761L019	Dome	LDR-E-1	VT-3C, direct	
03U1-761L020	Dome	PLTF-E-1	VT-3C, direct	03U1-L001
03U1-761L021	Dome	PLTF-E-1	VT-3C, remote	03U1-L001
03U1-761L022	Dome	LDR-F-7	VT-3C, direct	
03U1-761L023	Dome	PLTF-F-3	VT-3C, direct	03U1-L001, 03U1-L003
03U1-761L024	Dome	PLTF-F-3	VT-3C, remote	03U1-L001
03U1-761L025	Dome	Dome	VT-1C, direct	03U1-L003
03U1-761L026	Buttress A	Facade 6.5 ft el	VT-3C, direct	03U1-L001, 03U1-L002
03U1-761L027	Buttress A	Facade 6.5 ft el	VT-3C, remote	03U1-L001, 03U1-L002
03U1-761L028	Between A&B Buttress	Facade 6.5 ft el	VT-3C, direct	03U1-L003

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U1-761L029	Between A&B Buttress	Facade 6.5 ft el	VT-3C, remote	03U1-L002
03U1-761L030	Buttress B	Facade 6.5 ft el	VT-3C, direct	03U1-L001
03U1-761L031	Buttress B	Facade 6.5 ft el	VT-3C, remote	03U1-L001, 03U1-L002, 03U1-L003
03U1-761L032	Between B&C Buttress	Facade 6.5 ft el	VT-3C, direct	03U1-L001, 03U1-L003
03U1-761L033	Between B&C Buttress	Facade 6.5 ft el	VT-3C, remote	03U1-L001, 03U1-L002
03U1-761L034	Buttress C	Facade 6.5 ft el	VT-3C, direct	03U1-L001
03U1-761L035	Buttress C	Facade 6.5 ft el	VT-3C, remote	03U1-L001
03U1-761L036	Between C&D Buttress	Facade 6.5 ft el	VT-3C, direct	03U1-L003
03U1-761L037	Between C&D Buttress	Facade 6.5 ft el	VT-3C, remote	
03U1-761L038	Buttress D	Facade 6.5 ft el	VT-3C, direct	03U1-L001
03U1-761L039	Buttress D	Facade 6.5 ft el	VT-3C, remote	03U1-L001
03U1-761L040	Between D&E Buttress	Facade 6.5 ft el	VT-3C, direct	03U1-L001, 03U1-L003
03U1-761L041	Between D&E Buttress	Facade 6.5 ft el	VT-3C, remote	
03U1-761L042	Buttress E	Facade 6.5 ft el	VT-3C, direct	03U1-L001
03U1-761L043	Buttress E	Facade 6.5 ft el	VT-3C, remote	03U1-L001
03U1-761L044	Between E&F Buttress	Facade 6.5 ft el	VT-3C, direct	03U1-L003
03U1-761L045	Between E&F Buttress	Facade 6.5 ft el	VT-3C, remote	03U1-L001
03U1-761L046	Buttress F	Facade 6.5 ft el	VT-3C, direct	03U1-L003
03U1-761L047	Buttress F	Facade 6.5 ft el	VT-3C, remote	03U1-L001
03U1-761L048	Between F&A Buttress	Facade 6.5 ft el	VT-3C, direct	03U1-L001
03U1-761L049	Between F&A Buttress	Facade 6.5 ft el	VT-3C, remote	03U1-L002, 03U1-L003
03U1-761L050	Buttress A	Platform T-21A	VT-3C, remote	03U1-L001
03U1-761L051	Between A&B Buttress	Platform T-21A	VT-3C, remote	03U1-L001
03U1-761L052	Buttress B	Platform T-21A	VT-3C, remote	03U1-L001
03U1-761L053	Between B&C Buttress	Platform T-21A	VT-3C, remote	03U1-L001
03U1-761L054	Buttress C	Platform T-21A	VT-3C, remote	03U1-L001
03U1-761L055	Between C&D Buttress	Platform T-21A	VT-3C, remote	
03U1-761L056	Buttress B	LDR-B-1	VT-3C, direct	03U1-L001
03U1-761L057	Buttress B	PLTF-B-1	VT-3C, direct	03U1-L001
03U1-761L058	Between B&C Buttress	PLTF-B-1	VT-3C, direct	
03U1-761L059	Buttress B	LDR-B-2	VT-3C, direct	

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U1-761L060	Buttress B	PLTF-B-1	VT-3C, remote	03U1-L001
03U1-761L061	Between B&C Buttress	PLTF-B-1	VT-3C, remote	03U1-L001
03U1-761L062	Buttress B	LDR-B-3	VT-3C, direct	
03U1-761L063	Between A&B Buttress	PLTF-B-2	VT-3C, direct	
03U1-761L064	Buttress B	PLTF-B-2	VT-3C, direct	03U1-L001
03U1-761L065	Between A&B Buttress	PLTF-B-2	VT-3C, remote	
03U1-761L066	Buttress B	PLTF-B-2	VT-3C, remote	03U1-L001
03U1-761L067	Between B&C Buttress	Equipment Hatch Platform	VT-3C, direct	
03U1-761L068	Between B&C Buttress	Equipment Hatch Platform	VT-3C, remote	
03U1-761L069	Buttress C	Equipment Hatch Platform	VT-3C, remote	03U1-L001
03U1-761L070	Buttress E	Door 149 platform	VT-3C, remote	03U1-L001
03U1-761L071	Between D&E Buttress	Door 149 platform	VT-3C, remote	
03U1-761L072	Between E&F Buttress	LDR-F-1	VT-3C, direct	
03U1-761L073	Buttress E	Transfer Canal Platform	VT-3C, remote	03U1-L001
03U1-761L074	Between E&F Buttress	Transfer Canal Platform	VT-3C, remote	
03U1-761L075	Buttress F	Transfer Canal Platform	VT-3C, remote	03U1-L001
03U1-761L076	Between E&F Buttress	PLTF-F-1	VT-3C, direct	03U1-L003
03U1-761L077	Buttress F	PLTF-F-1	VT-3C, direct	03U1-L001
03U1-761L078	Between E&F Buttress	PLTF-F-1	VT-3C, remote	03U1-L001, 03U1-L003
03U1-761L079	Buttress F	PLTF-F-1	VT-3C, remote	03U1-L001
03U1-761L080	Buttress F	LDR-F-3	VT-3C, direct	
03U1-761L081	Buttress F	LDR-F-4	VT-3C, direct	
03U1-761L082	Buttress F	LDR-F-5	VT-3C, direct	
03U1-761L083	Buttress F	LDR-F-6	VT-3C, direct	
03U1-761L084	Buttress F	PLTF-F-2	VT-3C, direct	03U1-L001
03U1-761L085	Between F&A Buttress	PLTF-F-2	VT-3C, direct	
03U1-761L086	Buttress F	PLTF-F-2	VT-3C, remote	03U1-L001
03U1-761L087	Between F&A Buttress	PLTF-F-2	VT-3C, remote	03U1-L001
03U1-761L088	Between E&F Buttress	Radwaste Compactor Landing	VT-3C, remote	03U1-L001
03U1-761L089	Buttress F	Radwaste Compactor Landing	VT-3C, remote	03U1-L001
03U1-761L090	Between F&A Buttress	Radwaste Compactor Landing	VT-3C, remote	03U1-L001

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U1-761L091	Between C&D Buttress	WKWY-CD-1	VT-3C, direct	03U1-L001, 03U1-L002
03U1-761L092	Buttress C	WKWY-CD-1	VT-3C, remote	03U1-L001
03U1-761L093	Between C&D Buttress	WKWY-CD-1	VT-3C, remote	
03U1-761L094	Buttress D	WKWY-CD-1	VT-3C, remote	03U1-L001
03U1-761L095	Between C&D Buttress	WKWY-CD-2	VT-3C, direct	03U1-L001
03U1-761L096	Buttress C	WKWY-CD-2	VT-3C, remote	03U1-L001
03U1-761L097	Between C&D Buttress	WKWY-CD-2	VT-3C, remote	03U1-L001
03U1-761L098	Buttress D	WKWY-CD-2	VT-3C, remote	03U1-L001
03U1-761L099	Between D&E Buttress	Pipeway #2 Upper (on top)	VT-3C, direct	
03U1-761L100	Between D&E Buttress	Pipeway #2 Upper (on top)	VT-3C, remote	
03U1-761L101	Buttress E	Pipeway #2 Upper (on top)	VT-3C, remote	03U1-L001
03U1-761L102	Buttress C	WKWY-CD-3	VT-3C, remote	03U1-L001
03U1-761L103	Between C&D Buttress	WKWY-CD-3	VT-3C, remote	03U1-L001
03U1-761L104	Buttress D	WKWY-CD-3	VT-3C, remote	03U1-L001
03U1-761L105	Between C&D Buttress	WKWY-CD-4	VT-3C, direct	
03U1-761L106	Buttress C	WKWY-CD-4	VT-3C, remote	03U1-L001
03U1-761L107	Between C&D Buttress	WKWY-CD-4	VT-3C, remote	03U1-L001
03U1-761L108	Buttress D	WKWY-CD-4	VT-3C, remote	03U1-L001
03U1-761L109	Buttress C	Mezzanine 66 ft el.	VT-3C, remote	03U1-L001
03U1-761L110	Between C&D Buttress	Mezzanine 66 ft el.	VT-3C, remote	
03U1-761L111	Between C&D Buttress	WKWY-CD-5	VT-3C, direct	03U1-L001, 03U1-L003
03U1-761L112	Buttress D	WKWY-CD-5	VT-3C, direct	03U1-L001, 03U1-L003
03U1-761L113	Buttress D	WKWY-CD-5	VT-1C, direct	03U1-L001, 03U1-L003
03U1-761L114	Between D&E Buttress	WKWY-CD-5	VT-3C, direct	03U1-L001, 03U1-L003
03U1-761L115	Buttress C	WKWY-CD-5	VT-3C, remote	03U1-L001
03U1-761L116	Between C&D Buttress	WKWY-CD-5	VT-3C, remote	03U1-L001
03U1-761L117	Buttress D	WKWY-CD-5	VT-3C, remote	03U1-L001
03U1-761L118	Between D&E Buttress	WKWY-CD-5	VT-3C, remote	03U1-L001
03U1-761L119	Buttress E	WKWY-CD-5	VT-3C, remote	03U1-L001
03U1-761L120	Between C&D Buttress	WKWY-CD-6	VT-3C, direct	03U1-L003
03U1-761L121	Buttress D	WKWY-CD-6	VT-3C, direct	03U1-L001, 03U1-L003

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U1-761L122	Buttress C	WKWY-CD-6	VT-3C, remote	03U1-L001
03U1-761L123	Between C&D Buttress	WKWY-CD-6	VT-3C, remote	03U1-L001
03U1-761L124	Buttress D	WKWY-CD-6	VT-3C, remote	03U1-L001
03U1-761L125	Between C&D Buttress	PLTF-D-2	VT-3C, direct	
03U1-761L126	Buttress D	PLTF-D-2	VT-3C, direct	03U1-L001
03U1-761L127	Between C&D Buttress	PLTF-D-2	VT-3C, remote	
03U1-761L128	Buttress D	PLTF-D-2	VT-3C, remote	03U1-L001
03U1-761L129	Buttress D	PLTF-D-1	VT-3C, direct	03U1-L001
03U1-761L130	Between D&E Buttress	PLTF-D-1	VT-3C, direct	
03U1-761L131	Buttress D	PLTF-D-1	VT-3C, remote	03U1-L001
03U1-761L132	Between D&E Buttress	PLTF-D-1	VT-3C, remote	
03U1-761L133	Between D&E Buttress	PLTF-DE-1	VT-3C, remote	03U1-L001
03U1-761L134	Buttress E	PLTF-DE-1	VT-3C, remote	03U1-L001
03U1-761L135	Between C&D Buttress	Pipeway #1	VT-3C, direct	
03U1-761L136	Between D&E Buttress	Pipeway #2 Lower	VT-3C, direct	03U1-L001
03U1-761L137	Between D&E Buttress	Pipeway #2 Upper	VT-3C, direct	03U1-L001, 03U1-L002
03U1-761L138	End Anchorage	V-5 (Dome)	VT-1C, direct	
03U1-761L139	End Anchorage	V-5 (Tendon Gallery)	VT-1C, direct	03U1-L002
03U1-761L140	End Anchorage	V-32 (Dome)	VT-1C, direct	
03U1-761L141	End Anchorage	V-32 (Tendon Gallery)	VT-1C, direct	
03U1-761L142	End Anchorage	V-88 (Dome)	VT-1C, direct	
03U1-761L143	End Anchorage	V-88 (Tendon Gallery)	VT-1C, direct	
03U1-761L144	End Anchorage	V-144 (Dome)	VT-1C, direct	
03U1-761L145	End Anchorage	V-144 (Tendon Gallery)	VT-1C, direct	
03U1-761L146	End Anchorage	AC-19 (Buttress A)	VT-1C, direct	
03U1-761L147	End Anchorage	AC-19 (Buttress C)	VT-1C, direct	
03U1-761L148	End Anchorage	BF-25 (Buttress B)	VT-1C, direct	
03U1-761L149	End Anchorage	BF-25 (Buttress F)	VT-1C, direct	
03U1-761L150	End Anchorage	CE-7 (Buttress C)	VT-1C, direct	
03U1-761L151	End Anchorage	CE-7 (Buttress E)	VT-1C, direct	
03U1-761L152	End Anchorage	DF-46 (Buttress D)	VT-1C, direct	

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U1-761L153	End Anchorage	DF-46 (Buttress F)	VT-1C, direct	
03U1-761L154	End Anchorage	EA-45 (Buttress A)	VT-1C, direct	
03U1-761L155	End Anchorage	EA-45 (Buttress E)	VT-1C, direct	
03U1-761L156	End Anchorage	D1-38 (Field end)	VT-1C, direct	
03U1-761L157	End Anchorage	D1-38 (Shop end)	VT-1C, direct	
03U1-761L158	End Anchorage	D2-25 (Field end)	VT-1C, direct	
03U1-761L159	End Anchorage	D2-25 (Shop end)	VT-1C, direct	
03U1-761L160	End Anchorage	D3-9 (Field end)	VT-1C, direct	
03U1-761L161	End Anchorage	D3-9 (Shop end)	VT-1C, direct	
03U1-761L162	End Anchorage	DF-55 (Buttress D)	VT-1C, direct	
03U1-761L163	End Anchorage	DF-55 (Buttress F)	VT-1C, direct	
03U1-761L164	End Anchorage	D3-15 (Field end)	VT-1C, direct	
03U1-761L165	End Anchorage	D3-15 (Shop end)	VT-1C, direct	
03U1-761L166	End Anchorage	D1-36 (Field end)	VT-1C, direct	
03U1-761L167	End Anchorage	D1-36 (Shop end)	VT-1C, direct	
03U1-761L168	End Anchorage	V-5 (Dome)	VT-1, direct	
03U1-761L169	End Anchorage	V-5 (Tendon Gallery)	VT-1, direct	
03U1-761L170	End Anchorage	V-32 (Dome)	VT-1, direct	
03U1-761L171	End Anchorage	V-32 (Tendon Gallery)	VT-1, direct	
03U1-761L172	End Anchorage	V-88 (Dome)	VT-1, direct	
03U1-761L173	End Anchorage	V-88 (Tendon Gallery)	VT-1, direct	
03U1-761L174	End Anchorage	V-144 (Dome)	VT-1, direct	
03U1-761L175	End Anchorage	V-144 (Tendon Gallery)	VT-1, direct	
03U1-761L176	End Anchorage	AC-19 (Buttress A)	VT-1, direct	
03U1-761L177	End Anchorage	AC-19 (Buttress C)	VT-1, direct	
03U1-761L178	End Anchorage	BF-25 (Buttress B)	VT-1, direct	
03U1-761L179	End Anchorage	BF-25 (Buttress F)	VT-1, direct	
03U1-761L180	End Anchorage	CE-7 (Buttress C)	VT-1, direct	
03U1-761L181	End Anchorage	CE-7 (Buttress E)	VT-1, direct	
03U1-761L182	End Anchorage	DF-46 (Buttress D)	VT-1, direct	
03U1-761L183	End Anchorage	DF-46 (Buttress F)	VT-1, direct	03U1-L004



Nuclear Management Power Company  
700 First Street  
Hudson, Wisconsin 54016  
National Board Number for Unit: N/A

Commercial Service Date: 12/21/70  
Docket 50-266  
Pt. Beach Unit 1

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U1-761L184	End Anchorage	EA-45 (Buttress E)	VT-1, direct	
03U1-761L185	End Anchorage	EA-45 (Buttress A)	VT-1, direct	
03U1-761L186	End Anchorage	D1-38 (Field end)	VT-1, direct	
03U1-761L187	End Anchorage	D1-38 (Shop end)	VT-1, direct	
03U1-761L188	End Anchorage	D2-25 (Field end)	VT-1, direct	
03U1-761L189	End Anchorage	D2-25 (Shop end)	VT-1, direct	
03U1-761L190	End Anchorage	D3-9 (Field end)	VT-1, direct	
03U1-761L191	End Anchorage	D3-9 (Shop end)	VT-1, direct	
03U1-761L192	End Anchorage	DF-55 (Buttress D)	VT-1, direct	
03U1-761L193	End Anchorage	DF-55 (Buttress F)	VT-1, direct	
03U1-761L194	End Anchorage	D3-15 (Field End)	VT-1, direct	
03U1-761L195	End Anchorage	D3-15 (Shop End)	VT-1, direct	
03U1-761L196	End Anchorage	D1-36 (Field End)	VT-1, direct	
03U1-761L197	End Anchorage	D1-36 (Shop End)	VT-1, direct	

NIS-1 Inservice Inspection  
Summary Report for  
Unit 2 IWL

Nuclear Management Company, LLC  
700 First Street  
Hudson, Wisconsin 54016

Point Beach Nuclear Plant Unit 2  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

Commercial Operating Date: October 1, 1972

Report Date: February 11, 2004

Written by: Luzile C. Amicki Date: 2-18-04  
Reviewed by: Musall K. Turner Date: 2/17/2004  
Approved by: A. A. Sherwood Date: 2/18/04

## FORM NIS-1 OWNER'S REPORT FOR INSERVICE INSPECTIONS

1. Owner: Nuclear Management Company, 700 First Street  
Hudson, Wisconsin 54016
2. Plant: Point Beach Nuclear Plant, 6610 Nuclear Road, Two Rivers, Wisconsin 54241
3. Plant Unit: 2      4. Owner Certificate of Authorization: Docket 50-301
5. Commercial Service Date: October 1, 1972      6. National Board Number for Unit: N/A
7. Components Inspected

Component or Appurtenance	Manufacturer or Installer	Manufacturer or Installer Serial No.	State or Province No.	National Board No.
Containment	Bechtel	N/A	U009972	N/A

Bechtel Group, Inc.  
2280 Bates Avenue  
Concord, CA 94520

8. Examination Dates: September 10, 2001 to February 11, 2004.
9. Inspection Period Identification: N/A
10. Inspection Interval Identification: N/A
11. Applicable Editions of Section XI: 1992 Edition Addenda 1992
12. Date / Revision of Inspection Plan: October 25, 2002 / rev. 1
13. Abstract of Examinations. Include a list of examinations and a statement concerning status of work required for current Interval.

An Inservice Examination of containment concrete Class CC components of Nuclear Management Company (NMC) Point Beach Nuclear Plant (PBNP) Unit 2 was performed during the time period of September 10, 2001 to February 11, 2004.

The concrete was examined in accordance with the IWL Containment Inspection Program, which was prepared in accordance with the requirements of the American Society of Mechanical Engineers (ASME) Section XI, 1992 Edition, 1992 Addenda and 10 CFR 50.55a.

No Code Cases were used in the performance of examinations, tests, and repairs and replacements.

The visual tendon surveillance was completed during the implementation period in accordance with Technical Specifications. Precision Surveillance Corporation and WE Energies personnel performed this surveillance. See Attachment 1 for test results.

Visual examinations were conducted in accordance with ASME Section XI to meet examination Category L-A, Item L1.11 for VT-3C and Item L1.12 for VT-1C. WE Energies, Nuclear Management Company, and Lambert MacGill Thomas, Inc. personnel performed the examinations. Visual examinations were not conducted at the same time as the tendon surveillance due to incorporation of relief requests in the ASME Section XI visual examination program. See Attachment 2 for test results.

No repairs and replacements have been performed on Class CC components during this IWL implementation period.

All examinations required for the 33<sup>rd</sup> year tendon surveillance been completed. This meets the requirements of ASME Section XI and applicable relief requests.

#### 14. Abstract of Conditions Noted:

- **Grease Leakage**  
Areas of grease leakage at the tendon grease cans and through concrete leakage were recorded. No appreciable change was noted from conditions recorded during the 2001 concrete examination.
- **Grease Voids**  
Grease voids greater than 5% duct volume were noted in five Unit 2 tendons with three of these exceeding 10% duct volume.
- **Exposed Reinforcing Steel**  
Areas of exposed reinforcing steel were recorded. There was no measurable difference since the 2001 examinations
- **Concrete Cracking**  
The concrete cracking recorded showed no change with previously recorded indications. One area of concrete damage was noted that was not previously recorded.

15. Abstract of Corrective Measures Recommended and Taken.

- Grease Leakage  
Additional corrective maintenance will be pursued to correct grease leakage points.
- Grease Voids  
Tendons with the grease voids were refilled after completion of the visual examinations.
- Exposed Reinforcing Steel  
No corrective measures are recommended for these construction defects.
- Concrete Cracking  
Trending of existing cracks will continue to verify stability of concrete.

We certify that a) the statements made in this report are correct, b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI and c) corrective measures taken conform to the rules of the ASME Code, Section XI.

Certificate of Authorization No. (if applicable): NRC Docket 50-301 Expiration Date: N/A

Date: 2-18-04 Signed: Nuclear Management Co., LLC by Lucretia Hunkler  
Owner

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of Wisconsin and employed by Hartford Steam Boiler of Hartford Connecticut, have inspected the components described in this Owner's Report during the period September 10, 2001 to February 11, 2004, and state that to the best of my knowledge and belief, the Owner has performed examinations and tests and taken corrective measures described in this Owner's Report in accordance with the Inspection Plan and as required by the ASME Code, Section XI.

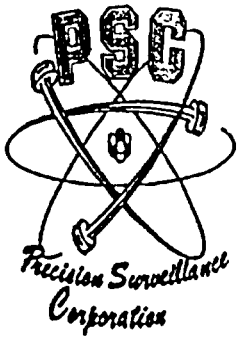
By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, and corrective measures described in this Owner's Report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

[Signature] Commissions WI 247972  
Inspector's Signature National Board, State, Province,  
and Endorsements

Date 2-18-2004

## Attachment 1

### Unit 2 Thirty-three Year Tendon Surveillance Summary



33<sup>rd</sup> YEAR UNIT 1 PHYSICAL AND  
UNIT 2 VISUAL TENDON  
SURVEILLANCE AT THE POINT  
BEACH NUCLEAR PLANT



TABLE I: LABORATORY ANALYSIS OF SHEATHING FILLER UNIT 2

TENDON	END	ION CONCENTRATION (PPM)			% WATER CONTENT	NEUTRAL No. mg KOH/g
		CHLORIDE	SULFIDE	NITRATE		
HK-10	SHOP/BUTT.H.	<0.50	<0.50	<0.50	0.19	<0.50
KM-60	SHOP/BUTT.K	<0.50	<0.50	<0.50	<0.10	0.510
MH-56	SHOP/BUTT. H	<0.50	<0.50	<0.50	<0.10	8.24
JL-30	SHOP/BUTT. J	<0.50	<0.50	<0.50	0.18	3.08
GL-7	SHOP/BUTT. G	<0.50	<0.50	<0.50	<0.10	11.4
V-217	SHOP/TOP	<0.50	<0.50	<0.50	0.30	<0.50
V-243	SHOP/TOP	<0.50	<0.50	<0.50	0.20	2.58
V-273	SHOP/TOP	<0.50	<0.50	<0.50	0.30	<0.50
V-328	SHOP/TOP	<0.50	<0.50	<0.50	0.28	<0.50
V-337	SHOP/TOP	<0.50	<0.50	<0.50	<0.10	<0.50
D1-216	SHOP/NEAR H	<0.50	<0.50	<0.50	0.20	<0.50
D2-230	SHOP/NEAR G	0.50	<0.50	<0.50	0.29	<0.50
D3-221	SHOP/NEAR J	0.50	<0.50	<0.50	<0.10	<0.50
D3-226	SHOP/NEAR J	<0.50	<0.50	<0.50	0.20	5.69

Acceptance Limits

<u>Test</u>	<u>Limits</u>
Water Soluble Chloride	Less than 10.0 ppm
Water Soluble Nitrates	Less than 10.0 ppm
Water Soluble Sulfides	Less than 10.0 ppm
Water Content	Less than 10% Dry Weight
Neutralization No.	Greater than 0 mg KOH/g



33<sup>rd</sup> YEAR UNIT 1 PHYSICAL AND  
UNIT 2 VISUAL TENDON  
SURVEILLANCE AT THE POINT  
BEACH NUCLEAR PLANT



TABLE XI: SUMMARY OF DATA SHEETS SQ 12.1  
GREASE LOSS Vs GREASE REPLACEMENT UNIT 2

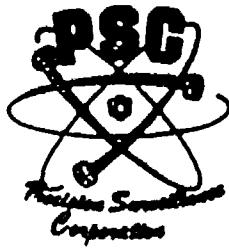
TENDON	GREASE REMOVED			GREASE REPLACED			DIFF. (GAL.)	NET VOLUME	%
	SHOP	FIELD	TOTAL (GAL.)	SHOP	FIELD	TOTAL (GAL.)			
HK-10	1.75	1.75	3.50	2.0	2.0	4.0	+0.50	56.27	+0.88
KM-60	2.25	1.75	4.00	2.50	1.75	4.25	+0.25	56.65	+0.44
MH-56	2.25	2.50	4.75	2.50	2.50	5.00	+0.25	56.31	+0.44
JL-30	1.75	1.75	3.50	2.00	2.00	4.00	+0.50	56.73	+0.88
GL-7	1.75	1.75	3.50	2.00	2.00	4.00	+0.50	56.53	+0.88
V-217	1.25	17.25	18.50	0.25	24.75	25.00	6.50	67.02	+9.69♣
V-243	2.75	32.25	35.00	0.25	39.00	39.25	4.25	67.10*	+6.30♣
V-273	2.25	20.25	22.50	0.25	35.50	35.75	13.25	67.45	+19.60♣
V-328	1.25	31.25	32.50	0.25	42.50	42.75	10.25	67.14	+15.26♣
V-337	3.25	27.25	30.50	0.25	31.75	32.00	1.50	66.79	+2.60
D1-216	5.75	4.00	9.75	6.00	4.25	10.25	0.50	53.18	+0.94
D2-230	18.00	6.75	24.75	0.25	25.75	26.00	1.25	55.03	+2.27
D3-221	6.75	4.25	11.00	7.25	4.75	12.00	1.00	54.77*	+1.80
D3-226	17.25	N/A	17.25	25.50	0.00	25.50	8.25	56.10	+14.70♦

♣ Ref. NCR# FN885-001

♣ Ref. NCR# FN885-002

♦ Ref. NCR# FN885-003





33<sup>rd</sup> YEAR UNIT 1 PHYSICAL AND  
UNIT 2 VISUAL TENDON  
SURVEILLANCE AT THE POINT  
BEACH NUCLEAR PLANT



TABLE VIII: SUMMARY OF DATA SHEETS SQ 9.0 - TENDON LIFTOFFS UNIT 2

TENDON	END	EFFECT. WIRES	JACK No.	PRESS.	LIFTOFF	AVE. LIFTOFF	LOWER LIMIT	95% LOWER	90% LOWER	UPPER LIMIT	ACCEPT
MH-56	SHOP/ BUTT.H	89	6001	3360	637.1	638.7	594.5	564.8	535.1	656.8	YES
	FIELD/BUTT.M	89	6002	3390	640.3						
V-337	SHOP/TOP	90	6002	3420	646	646	628.2	596.8	565.4	672.3	YES
D2-230	SHOP/NEAR G	90	6002	3350	632.7	633.6	613.8	583.1	552.4	666.0	YES
	FIELD/NEAR K	90	6002	3360	634.6						

## Attachment 2

### Unit 2 IWL Containment Concrete Examination Summary

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U2-761L001	Tendon Gallery Ceiling	Tendon Gallery	VT-1C, direct	03U2-L003
03U2-761L002	Tendon Gallery Ceiling	Tendon Gallery	VT-1C, direct	03U2-L003
03U2-761L003	Tendon Gallery Ceiling	Tendon Gallery	VT-1C, direct	03U2-L003
03U2-761L004	Tendon Gallery Ceiling	Tendon Gallery	VT-1C, direct	03U2-L003
03U2-761L005	Tendon Gallery Ceiling	Tendon Gallery	VT-3C, direct	03U2-L001, 03U2-L003
03U2-761L006	Tendon Gallery Inner Wall	Tendon Gallery	VT-3C, direct	03U2-L002
03U2-761L007	Tendon Gallery Outer Wall	Tendon Gallery	VT-3C, direct	03U2-L002
03U2-761L008	Dome	Dome	VT-3C, direct	03U2-L001, 03U2-L002
03U2-761L009	Dome	LDR-G-1	VT-3C, direct	
03U2-761L010	Dome	PLTF-G-1	VT-3C, direct	03U2-L001
03U2-761L011	Dome	PLTF-G-1	VT-3C, remote	03U2-L001
03U2-761L012	Dome	LDR-H-5	VT-3C, direct	
03U2-761L013	Dome	PLTF-H-3	VT-3C, direct	03U2-L001
03U2-761L014	Dome	PLTF-H-3	VT-3C, remote	03U2-L001
03U2-761L015	Dome	LDR-J-1	VT-3C, direct	
03U2-761L016	Dome	PLTF-J-1	VT-3C, direct	03U2-L001, 03U1-L002
03U2-761L017	Dome	PLTF-J-1	VT-3C, remote	
03U2-761L018	Dome	LDR-K-4	VT-3C, direct	
03U2-761L019	Dome	PLTF-K-3	VT-3C, direct	03U2-L001
03U2-761L020	Dome	PLTF-K-3	VT-3C, remote	03U2-L001
03U2-761L021	Dome	LDR-L-1	VT-3C, direct	
03U2-761L022	Dome	PLTF-L-1	VT-3C, direct	
03U2-761L023	Dome	PLTF-L-1	VT-3C, remote	
03U2-761L024	Dome	LDR-M-2	VT-3C, direct	
03U2-761L025	Dome	PLTF-M-3	VT-3C, direct	03U2-L002
03U2-761L026	Dome	PLTF-M-3	VT-3C, remote	
03U2-761L027	Buttress G	Facade 6.5 ft el	VT-3C, direct	03U2-L001
03U2-761L028	Between G&H Buttress	Facade 6.5 ft el	VT-3C, direct	03U2-L003
03U2-761L029	Buttress H	Facade 6.5 ft el	VT-3C, direct	03U2-L001

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U2-761L030	Between H&J Buttress	Facade 6.5 ft el	VT-3C, direct	
03U2-761L031	Buttress J	Facade 6.5 ft el	VT-3C, direct	03U2-L001
03U2-761L032	Between J&K Buttress	Facade 6.5 ft el	VT-3C, direct	03U2-L001
03U2-761L033	Buttress K	Facade 6.5 ft el	VT-3C, direct	
03U2-761L034	Between K&L Buttress	Facade 6.5 ft el	VT-3C, direct	
03U2-761L035	Buttress L	Facade 6.5 ft el	VT-3C, direct	03U2-L001
03U2-761L036	Between L&M Buttress	Facade 6.5 ft el	VT-3C, direct	03U2-L001, 03U2-L002, 03U2-L003
03U2-761L037	Buttress M	Facade 6.5 ft el	VT-3C, remote	03U2-L001
03U2-761L038	Between M&G Buttress	Facade 6.5 ft el	VT-3C, direct	03U2-L001, 03U2-L003
03U2-761L039	Buttress G	Facade 6.5 ft el	VT-3C, remote	03U2-L001, 03U2-L002
03U2-761L040	Between G&H Buttress	Facade 6.5 ft el	VT-3C, remote	
03U2-761L041	Buttress H	Facade 6.5 ft el	VT-3C, remote	03U2-L001
03U2-761L042	Between H&J Buttress	Facade 6.5 ft el	VT-3C, remote	
03U2-761L043	Buttress J	Facade 6.5 ft el	VT-3C, remote	03U2-L001
03U2-761L044	Between J&K Buttress	Facade 6.5 ft el	VT-3C, remote	
03U2-761L045	Buttress K	Facade 6.5 ft el	VT-3C, remote	03U2-L001
03U2-761L046	Between K&L Buttress	Facade 6.5 ft el	VT-3C, remote	
03U2-761L047	Buttress L	Facade 6.5 ft el	VT-3C, remote	
03U2-761L048	Between L&M Buttress	Facade 6.5 ft el	VT-3C, remote	
03U2-761L049	Buttress M	Facade 6.5 ft el	VT-3C, direct	03U2-L001, 03U2-L002
03U2-761L050	Between M&G Buttress	Facade 6.5 ft el	VT-3C, remote	03U2-L001
03U2-761L051	Between G&H Buttress	PLTF-H-1	VT-3C, direct	
03U2-761L052	Between G&H Buttress	PLTF-H-1	VT-3C, remote	
03U2-761L053	Buttress H	LDR-H-1	VT-3C, direct	
03U2-761L054	Buttress H	PLTF-H-1	VT-3C, direct	03U2-L001
03U2-761L055	Buttress H	PLTF-H-1	VT-3C, remote	03U2-L001
03U2-761L056	Buttress H	LDR-H-2	VT-3C, direct	
03U2-761L058	Buttress H	LDR-H-3	VT-3C, direct	
03U2-761L059	Buttress H	LDR-H-4	VT-3C, direct	
03U2-761L060	Buttress H	PLTF-H-2	VT-3C, direct	03U2-L001
03U2-761L061	Buttress H	PLTF-H-2	VT-3C, remote	03U2-L001, 03U2-L002

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U2-761L062	Between H&J Buttress	PLTF-H-2	VT-3C, direct	03U2-L002
03U2-761L063	Between H&J Buttress	PLTF-H-2	VT-3C, remote	
03U2-761L064	Buttress J	evap bldg ladder	VT-3C, direct	
03U2-761L065	Buttress J	evap bldg landing	VT-3C, direct	
03U2-761L066	Between J&K Buttress	evap bldg landing	VT-3C, direct	03U2-L001
03U2-761L067	Between H&J Buttress	evap bldg landing	VT-3C, remote	
03U2-761L068	Buttress J	evap bldg landing	VT-3C, remote	
03U2-761L069	Between J&K Buttress	evap bldg landing	VT-3C, remote	
03U2-761L070	Between J&K Buttress	PLTF-K-2	VT-3C, direct	03U2-L001
03U2-761L071	Between J&K Buttress	PLTF-K-2	VT-3C, remote	
03U2-761L072	Buttress K	LDR-K-1	VT-3C, direct	
03U2-761L073	Buttress K	LDR-K-2	VT-3C, direct	
03U2-761L074	Buttress K	PLTF-K-1	VT-3C, direct	03U2-L003
03U2-761L075	Between K&L Buttress	PLTF-K-1	VT-3C, direct	03U2-L003
03U2-761L076	Between K&L Buttress	PLTF-K-1	VT-3C, remote	03U2-L001
03U2-761L077	Buttress K	PLTF-K-1	VT-3C, remote	
03U2-761L078	Buttress K	LDR-K-3	VT-3C, direct	
03U2-761L079	Buttress K	PLTF-K-2	VT-3C, direct	
03U2-761L080	Buttress K	PLTF-K-2	VT-3C, remote	03U2-L001
03U2-761L081	Between L&M Buttress	WKWY-LM-1	VT-3C, direct	03U2-L001, 03U2-L003
03U2-761L082	Buttress L	WKWY-LM-1	VT-3C, remote	03U2-L001
03U2-761L083	Between L&M Buttress	WKWY-LM-1	VT-3C, remote	03U2-L001
03U2-761L084	Buttress M	WKWY-LM-1	VT-3C, remote	
03U2-761L085	Between L&M Buttress	WKWY-LM-2	VT-3C, remote	
03U2-761L086	Between K&L Buttress	Pipeway #3 Upper (on top)	VT-3C, direct	
03U2-761L087	Between K&L Buttress	Pipeway #3 Upper (on top)	VT-3C, remote	03U2-L001, 03U2-L002
03U2-761L088	Between L&M Buttress	WKWY-LM-3	VT-3C, direct	03U2-L001
03U2-761L089	Buttress L	WKWY-LM-3	VT-3C, remote	03U2-L001
03U2-761L090	Between L&M Buttress	WKWY-LM-3	VT-3C, remote	03U2-L001
03U2-761L091	Buttress M	WKWY-LM-3	VT-3C, remote	
03U2-761L092	Between L&M Buttress	WKWY-LM-4	VT-3C, direct	

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U2-761L093	Buttress L	WKWY-LM-4	VT-3C, remote	03U2-L001
03U2-761L094	Between L&M Buttress	WKWY-LM-4	VT-3C, remote	03U2-L001
03U2-761L095	Buttress M	WKWY-LM-4	VT-3C, remote	03U2-L001
03U2-761L096	Between K&L Buttress	WKWY-LM-5	VT-3C, direct	03U2-L001, 03U2-L003
03U2-761L097	Buttress L	WKWY-LM-5	VT-3C, direct	03U2-L001
03U2-761L098	Between L&M Buttress	WKWY-LM-5	VT-3C, direct	03U2-L002, 03U2-L003
03U2-761L099	Buttress K	WKWY-LM-5	VT-3C, remote	03U2-L001
03U2-761L100	Between K&L Buttress	WKWY-LM-5	VT-3C, remote	03U2-L001, 03U2-L002, 03U2-L003
03U2-761L101	Buttress L	WKWY-LM-5	VT-3C, remote	03U2-L001
03U2-761L102	Between L&M Buttress	WKWY-LM-5	VT-3C, remote	
03U2-761L103	Buttress M	WKWY-LM-5	VT-3C, remote	03U2-L001
03U2-761L104	Between L&M Buttress	WKWY-LM-6	VT-3C, direct	03U2-L002, 03U2-L003
03U2-761L105	Buttress L	WKWY-LM-6	VT-3C, remote	03U2-L001
03U2-761L106	Between L&M Buttress	WKWY-LM-6	VT-3C, remote	
03U2-761L107	Buttress M	WKWY-LM-6	VT-3C, remote	
03U2-761L108	Between L&M Buttress	PLTF-M-2	VT-3C, direct	
03U2-761L109	Buttress M	PLTF-M-2	VT-3C, direct	03U2-L001
03U2-761L110	Between L&M Buttress	PLTF-M-2	VT-3C, remote	
03U2-761L111	Buttress M	PLTF-M-2	VT-3C, remote	03U2-L001
03U2-761L112	Buttress M	LDR-M-1	VT-3C, direct	
03U2-761L113	Buttress M	PLTF-M-1	VT-3C, direct	03U2-L001
03U2-761L114	Between M&G Buttress	PLTF-M-1	VT-3C, direct	
03U2-761L115	Buttress M	PLTF-M-1	VT-3C, remote	03U2-L001
03U2-761L116	Between M&G Buttress	PLTF-M-1	VT-3C, remote	
03U2-761L117	Between M&G Buttress	Equipment hatch walkway	VT-3C, direct	
03U2-761L118	Between M&G Buttress	Equipment hatch walkway	VT-3C, remote	
03U2-761L119	Buttress M	Equipment hatch walkway	VT-3C, remote	03U2-L001
03U2-761L120	Buttress M	gas stripper bldg. Platform	VT-3C, remote	03U2-L001
03U2-761L121	Between M&G Buttress	gas stripper bldg. Platform	VT-3C, remote	03U2-L001
03U2-761L122	Between K&L Buttress	Pipeway #3 Lower	VT-3C, direct	03U2-L001
03U2-761L123	Between L&M Buttress	Pipeway #4	VT-3C, direct	03U2-L001

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U2-761L124	Between K&L Buttress	Pipeway #3 Upper	VT-3C, direct	03U2-L001
03U2-761L125	End Anchorage	V-217 (Dome)	VT-1C, direct	
03U2-761L126	End Anchorage	V-217 (Tendon Gallery)	VT-1C, direct	
03U2-761L127	End Anchorage	V-273 (Dome)	VT-1C, direct	
03U2-761L128	End Anchorage	V-273 (Tendon Gallery)	VT-1C, direct	03U2-L001
03U2-761L129	End Anchorage	V-328 (Dome)	VT-1C, direct	
03U2-761L130	End Anchorage	V-328 (Tendon Gallery)	VT-1C, direct	
03U2-761L131	End Anchorage	V-337 (Dome)	VT-1C, direct	
03U2-761L132	End Anchorage	V-337 (Tendon Gallery)	VT-1C, direct	03U2-L002
03U2-761L133	End Anchorage	GL-7 (Buttress L)	VT-1C, direct	
03U2-761L134	End Anchorage	GL-7 (Buttress G)	VT-1C, direct	
03U2-761L135	End Anchorage	HK-10 (Buttress H)	VT-1C, direct	
03U2-761L136	End Anchorage	HK-10 (Buttress K)	VT-1C, direct	03U2-L001
03U2-761L137	End Anchorage	JL-30 (Buttress J)	VT-1C, direct	
03U2-761L138	End Anchorage	JL-30 (Buttress L)	VT-1C, direct	
03U2-761L139	End Anchorage	KM-60 (Buttress M)	VT-1C, direct	
03U2-761L140	End Anchorage	KM-60 (Buttress K)	VT-1C, direct	03U2-L002
03U2-761L141	End Anchorage	MH-56 (Buttress M)	VT-1C, direct	
03U2-761L142	End Anchorage	MH-56 (Buttress H)	VT-1C, direct	
03U2-761L143	End Anchorage	D1-216 (Field end)	VT-1C, direct	
03U2-761L144	End Anchorage	D1-216 (Shop end)	VT-1C, direct	
03U2-761L145	End Anchorage	D2-230 (Field end)	VT-1C, direct	
03U2-761L146	End Anchorage	D2-230 (Shop end)	VT-1C, direct	
03U2-761L147	End Anchorage	D3-226 (Field end)	VT-1C, direct	
03U2-761L148	End Anchorage	D3-226 (Shop end)	VT-1C, direct	03U2-L004
03U2-761L149	End Anchorage	V-217 (Dome)	VT-1, direct	
03U2-761L150	End Anchorage	V-217 (Tendon Gallery)	VT-1, direct	
03U2-761L151	End Anchorage	V-273 (Dome)	VT-1, direct	
03U2-761L152	End Anchorage	V-273 (Tendon Gallery)	VT-1, direct	
03U2-761L153	End Anchorage	V-328 (Dome)	VT-1, direct	
03U2-761L154	End Anchorage	V-328 (Tendon Gallery)	VT-1, direct	

NDE Data Sheet #	Concrete Area	Examination Location	Exam Type	IDR
03U2-761L155	End Anchorage	V-337 (Dome)	VT-1, direct	
03U2-761L156	End Anchorage	V-337 (Tendon Gallery)	VT-1, direct	
03U2-761L157	End Anchorage	GL-7 (Buttress L)	VT-1, direct	
03U2-761L158	End Anchorage	GL-7 (Buttress G)	VT-1, direct	
03U2-761L159	End Anchorage	HK-10 (Buttress H)	VT-1, direct	
03U2-761L160	End Anchorage	HK-10 (Buttress K)	VT-1, direct	
03U2-761L161	End Anchorage	JL-30 (Buttress J)	VT-1, direct	
03U2-761L162	End Anchorage	JL-30 (Buttress L)	VT-1, direct	
03U2-761L163	End Anchorage	KM-60 (Buttress M)	VT-1, direct	
03U2-761L164	End Anchorage	KM-60 (Buttress K)	VT-1, direct	
03U2-761L165	End Anchorage	MH-56 (Buttress M)	VT-1, direct	03U2-L004
03U2-761L166	End Anchorage	MH-56 (Buttress H)	VT-1, direct	03U2-L004
03U2-761L167	End Anchorage	D1-216 (Field end)	VT-1, direct	
03U2-761L168	End Anchorage	D1-216 (Shop end)	VT-1, direct	
03U2-761L169	End Anchorage	D2-230 (Field end)	VT-1, direct	
03U2-761L170	End Anchorage	D2-230 (Shop end)	VT-1, direct	
03U2-761L171	End Anchorage	D3-226 (Field end)	VT-1, direct	
03U2-761L172	End Anchorage	D3-226 (Shop end)	VT-1, direct	