

March 5, 2004

Mr. Christopher M. Crane, President
and Chief Nuclear Officer
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2, ISSUANCE OF AMENDMENTS
RE: PRIMARY CONTAINMENT ISOLATION INSTRUMENTATION (TAC NOS.
MB8462 AND MB8463)

Dear Mr. Crane:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 166 to Facility Operating License No. NPF-11 and Amendment No. 152 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2, respectively. The amendments are in response to your application dated April 18, 2003.

The amendments revise the Technical Specifications (TS). Specifically, the change modifies TS Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation," to add the requirement to perform a Channel Check in accordance with Surveillance Requirement (SR) 3.3.6.1.1 to thirteen listed instrument functions. The change is the result of the replacement of existing plant equipment with equipment that has the capability of permitting the performance of a Channel Check with the plant in MODES 1, 2, and 3. The change is consistent with the wording specified in NUREG-1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

William A. Macon, Jr., Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos.: 50-373 and 50-374

Enclosures: 1. Amendment No. 166 to NPF-11
2. Amendment No. 152 to NPF-18
3. Safety Evaluation

cc w/encls: See next page

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EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 166
License No. NPF-11

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Exelon Generation Company, LLC (the licensee), dated April 18, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 166, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by DPickett for/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 5, 2004

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 152
License No. NPF-18

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Exelon Generation Company, LLC (the licensee), dated April 18, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 152, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA by DPickett for/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 5, 2004

ATTACHMENT TO LICENSE AMENDMENT NOS. 166 AND 152

FACILITY OPERATING LICENSE NOS. NPF-11 AND NPF-18

DOCKET NOS. 50-373 AND 50-374

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove Pages

3.3.6.1-6
3.3.6.1-7
3.3.6.1-8

Insert Pages

3.3.6.1-6
3.3.6.1-7
3.3.6.1-8

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 166 TO FACILITY OPERATING LICENSE NO. NPF-11
AND AMENDMENT NO. 152 TO FACILITY OPERATING LICENSE NO. NPF-18
EXELON GENERATION COMPANY, LLC
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By application dated April 18, 2003 (Ref. 1), Exelon Generation Company, LLC (the licensee) requested changes to the Technical Specifications (TS) for the LaSalle County Station (LSCS), Units 1 and 2.

The proposed change will modify TS Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation," to add the requirement to perform a Channel Check in accordance with Surveillance Requirement (SR) 3.3.6.1.1 to thirteen listed instrument functions. The proposed changes are the result of the replacement of existing plant equipment with equipment that has the capability of permitting the performance of a Channel Check with the plant in MODES 1, 2, and 3. The proposed changes are consistent with the wording specified in NUREG-1434, "Standard Technical Specifications General Electric Plants, BWR/6," Revision 2, dated June 2001. The thirteen instrument functions in TS Table 3.3.6.1-1 that would require the performance of a Channel Check with the plant in MODES 1, 2, and 3 are as follows:

- 1.e Main Steam Line Tunnel Differential Temperature - High
- 3.e RCIC Equipment Room Temperature - High
- 3.f RCIC Equipment Room Differential Temperature - High
- 3.g RCIC Steam Line Tunnel Temperature - High
- 3.h RCIC Steam Line Tunnel Differential Temperature - High
- 4.c RWCU Heat Exchanger Area Temperature - High
- 4.d RWCU Heat Exchanger Area Ventilation Differential Temperature - High
- 4.e RWCU Pump and Valve Area Temperature - High
- 4.f RWCU Pump and Valve Area Differential Temperature - High
- 4.g RWCU Holdup Pipe Area Temperature - High
- 4.h RWCU Holdup Pipe Area Ventilation Differential Temperature - High
- 4.i RWCU Filter/Demineralizer Valve Room Area Temperature - High
- 4.j RWCU Filter/Demineralizer Valve Room Area Ventilation Differential Temperature - High

2.0 REGULATORY EVALUATION

The staff finds that the licensee in Section 4.0 of Attachment 2 of its submittal identified the applicable regulatory requirements. The licensee uses Section 50.36(c)(2)(ii)(c), Criterion 3, of Title 10 of the *Code of Federal Regulations* (10 CFR) as its regulatory basis for this license amendment. According to 10 CFR 50.36(c)(2)(ii)(c), "Criterion 3," requires that a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier must be included in a licensee's TS.

The staff used the following regulatory basis for its evaluation of the licensee's amendment request:

10 CFR 50.55a(h)(2):

Paragraph (h)(2) of 10 CFR 50.55a, "Codes and Standards," states, in part, that for protection and safety systems at nuclear power plants with construction permits issued between January 1, 1971 and May 13, 1999, protection systems must satisfy the requirements stated in either IEEE Std. 279, "Criteria for Protection Systems for Nuclear Power Generating Stations," or in IEEE Std. 603-1991, "Criteria for Safety Systems for Nuclear Power Generating Stations."

The proposed amendments incorporate a change into the LSCS TS Table 3.3.6.1-1 that is consistent with the wording and intent of NUREG-1434, Rev. 2, and, therefore, does not rely upon the previous issuance of amendments to other licensees as precedent.

3.0 TECHNICAL EVALUATION

The staff has reviewed the licensee's regulatory and technical analyses in support of its proposed license amendments which are described in Sections 5.0 and 6.0 of Attachment 2 of the licensee's submittal. The detailed evaluation below will support the conclusion that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

The licensee has proposed to add a requirement for a Channel Check of thirteen instrument functions to SR 3.3.6.1.1 consistent with NUREG-1434, Rev. 2. The current TS Table 3.3.6-1 for LSCS does not require the performance of a Channel check for the above listed thirteen instrument functions in MODES 1, 2, and 3. The current TS requirements were based on the capability of the previously installed thermocouple monitors. These monitors required the toggle switches to be manipulated by the operator in order to observe the different leak detection channels. However, this manipulation of the toggle switches could cause a risk of inadvertent isolation from channel spiking as the individual channel switches were selected. The replacement of the thermocouple monitors with new monitors with continuous reading, simultaneous digital displays, now allows operators to perform a Channel Check for the thirteen instrument functions.

The leak detection system at LSCS uses ambient or differential temperature increases to detect small primary coolant boundary leaks in the Main Steam Line (MSL) Tunnel and in the various rooms of the Reactor Core Isolation Cooling (RCIC) System and the Reactor Water Cleanup (RWCU) System. The new leak detection system thermocouple monitors have already replaced the monitors in Unit 2 during a refueling outage in January/February 2003. In March 2003, Unit 2 was returned to service and the operators were able to verify that the new thermocouple monitors would be able to provide the capability of performing a Channel Check for the above thirteen instrument functions. Unit 1 completed its refueling outage in February 2004 and the licensee replaced its existing thermocouple monitors with the same new thermocouple monitors as installed in Unit 2. The proposed changes will provide further assurance that the instrumentation is performing as designed.

The staff reviewed the licensee's regulatory and technical analysis in support of its proposed license amendments and, during a conference call with the licensee on November 24, 2003, confirmed the regulatory requirements of the new thermocouple monitors. The licensee confirmed during the conference call that the new thermocouple monitors were installed for the leak detection system under 10 CFR 50.55a (h)(2) using IEEE Std. 279-1971. This standard is referenced in Chapter 7, "Instrumentation and Control Systems", of the LSCS Updated Final Safety Analysis Report (UFSAR). UFSAR Table 7.1-8 indicates that high temperature and differential temperature components for the MSL, RCIC, and RWCU systems meet the requirements of IEEE Std. 279-1971. Based on the licensee's installation of new equipment that satisfies the requirements of IEEE Std. 279 and the proposed change to TS Table 3.3.6-1 being consistent with the wording specified in NUREG 1434, Rev. 2, the staff finds the proposed TS changes acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (68 FR 34667). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the

Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCE

1. Letter from K. R. Jury, Exelon Generation Company, to NRC, "Request for Amendment to Technical Specifications Table 3.3.6.1-1, "Primary Containment Isolation Instrumentation,"" dated April 18, 2003 (ADAMS Accession No. ML031190561).

Principal Contributor: G. Armstrong, NRR/DE/EEIB-A

Date: March 5, 2004

LaSalle County Station Units 1 and 2

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