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Technical Specification 6.7.A.6

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
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Monticello Nuclear Generating Plant
Docket 50-263
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Annual Report of Safety Relief Valve Failures and Challenges
January 1 through December 31, 2003

In accordance with Monticello Technical Specification Section 6.7.A.6 and NUREG-0737 Item II.K.3.3, the Annual Report of Safety Relief Valve Failures and Challenges is provided in Enclosure 1.

This letter contains no new NRC commitments, nor does it modify any prior commitments. Please contact John Fields, Senior Regulatory Affairs Engineer, at (763) 295-1663 if you require additional information concerning this report.



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Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Monticello, USNRC
Resident Inspector, Monticello, USNRC
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ENCLOSURE 1

Annual Report of Safety Relief Valve Failures and Challenges January 1 through December 31, 2003

In accordance with Monticello Nuclear Generating Plant (MNGP) Technical Specification Section 6.7.A.6 and NUREG-0737 Item II.K.3.3, the following Annual Report of Safety Relief Valve Failures and Challenges is provided:

Failures:

- 05/24/03 MNGP was shutdown and the topworks (pilot and second stage assembly) associated with main steam safety relief valve (SRV) RV-2-71G was changed out due to suspected excessive leakage past the SRV's pilot or second stage seats. It was subsequently determined that the source of the leakage was past the SRV's main stage seat and not the pilot or second stage seats. The SRV main stage assembly was not changed out at this time.
- 06/14/03 MNGP was shutdown and SRV RV-2-71B and RV-2-71G were changed out in their entirety except for the SRV's respective actuators, which were re-used. RV-2-71G was changed out due to excessive leakage past the SRV's main stage seat. RV-2-71B was changed out due to suspected excessive main stage seat leakage as well. However, it was later determined that the source of the leakage past RV-2-71B was past its pilot or second stage seats.

Challenges:

- 05/24/03 Following the 2003 refueling outage, all eight main steam safety relief valves (RV-2-71A through RV-2-71H) were manually opened at a reactor pressure of approximately 150 psig in accordance with a surveillance test performed during plant startup.
- 05/24/03 RV-2-71G was manually opened a 2nd time on the same day at a reactor pressure of approximately 150 psig in accordance with a surveillance test in an attempt to stop leakage past the SRV's discharge.
- 05/26/03 RV-2-71G was manually opened at a reactor pressure greater than 600 psig in accordance with a post maintenance test performed during plant startup following replacement of the SRV's topworks (pilot and second stage assembly).
- 06/16/03 RV-2-71B and RV-2-71G were manually opened at a reactor pressure of approximately 150 psig in accordance with a post maintenance test performed during plant startup following replacement of both SRV's.