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L-04-017

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555-0001

Subject: Beaver Valley Power Station, Unit No. 1 and No. 2
BV-1 Docket No. 50-334, License No. DPR-66
BV-2 Docket No. 50-412, License No. NPF-73
Additional Information Regarding Bulletin 2002-01

References:

- 1) NRC Bulletin 2002-01, Reactor Pressure Vessel Head Degradation and Reactor Coolant Pressure Boundary Integrity, dated March 18, 2002
- 2) BVPS 60-day Response to Bulletin 2002-01, L-02-055 dated May 17, 2002
- 3) NRC Request for Additional Information (RAI) dated November 19, 2002
- 4) BVPS Reply to RAI re: Bulletin 2002-01, L-03-004 dated January 24, 2003

On January 24, 2003, the Beaver Valley Power Station (BVPS) submitted a reply (reference 4) to a Request for Additional Information (reference 3) regarding the BVPS 60-day response (reference 2) to NRC Bulletin 2002-01 (reference 1).

We have determined that additional information should be included with the BVPS Unit 1 information that was submitted in reply for RAI items 1 and 2. RAI items 1 and 2 had requested details regarding the examination of Alloy 600 pressure boundary material and dissimilar metal Alloy 82/182 welds and connections in the reactor coolant pressure boundary. The BVPS reply to the RAI included tables (Table 1-A for BVPS Unit 1 and Table 1-B for BVPS Unit 2) that provided the requested information for the various locations of the specified materials. As noted in the reply, these locations were based on a review of component drawings and a referenced Westinghouse report.

In January 2004, BVPS received information from Westinghouse which indicated that Alloy 82/182 weld material was used to weld the Unit 1 Pressurizer spray and relief valve nozzles. This is contrary to the previous information supplied by Westinghouse, which stated that stainless steel weld material was used for those nozzle welds. Upon BVPS request, Westinghouse subsequently confirmed that Alloy 82/182 weld material had been

used on the spray and relief valve nozzles on the BVPS Unit 1 pressurizer. This issue was entered into the BVPS corrective action program.

Therefore, an "Addition to Table 1-A" is hereby provided as a supplement to the originally supplied Table 1-A, to address the recently discovered additional locations of Alloy 82/182 weld material at BVPS Unit 1. No changes to the BVPS Unit 2 locations listed in Table 1-B were identified by our review.

There are no new regulatory commitments contained in this submittal. If there are any questions concerning this matter, please contact Mr. Larry R. Freeland, Manager, Regulatory Affairs/Performance Improvement at 724-682-5284.

I declare under penalty of perjury that the following is true and correct. Executed on February 11, 2004.

Sincerely,



L. William Pearce

Enclosure

- c: Mr. T. G. Colburn, NRR Project Manager
Mr. P. C. Cataldo, NRC Sr. Resident Inspector
Mr. H. J. Miller, NRC Region I Administrator
Mr. D. A. Allard, Director BRP/DEP
Mr. L. E. Ryan (BRP/DEP)
Ms. C. O'Clair, Ohio Emergency Management Agency

**Addition to Table 1-A
BVPS UNIT 1**

(Additional) RC Pressure Boundary Alloy 600 Base Material or Alloy 82/182 Weld Material Locations						
Component	Inspection Techniques	Personnel Qualifications	Extent of Coverage	Frequency (minimum)	Degree of Insulation Removal/Insulation Type	Corrective Action
Pressurizer vessel top head – (5) nozzle to safe-end welds: (1) spray nozzle-to-safe-end weld (4) - Code Safety and relief nozzle-to-safe-end welds	Visual, VT-2, UT (Volumetric), PT (Surface)	Visual, VT-2, UT – ASME XI-1995 Edition Appendix VIII), PT – ASME XI, 1989 Edition	VT-2 inspection for evidence of leakage; UT – Exam Volume per PDI UT Procedure PDI-UT-10, PT – Exam area per Table IWB-2500 (Category B-F).	Per Table IWB-2500-1 (Category B-F). In addition, VT-2 is planned for next refueling outage.	Mirror insulation to be removed for bare metal VT-2 examinations in 1R16 (Fall 2004). Mirror insulation also removed for UT and PT examinations when scheduled.	VT-Evidence of leakage from pressure boundary (weld) is unacceptable UT, PT – indications exceeding IWB-3514 acceptance criteria are unacceptable.

Notes:

Additional locations were based on information provided in Westinghouse Proprietary WCAP-16199-P, "PWSCC Susceptibility Assessment of the Alloy 600 and Alloy 82/182 Components in Beaver Valley Units 1 and 2."

Information within the "Addition to Table 1-A" reflects the current status of these examinations and includes guidance and actions driven by circumstances that have occurred subsequent to the January 2003 submittal.