

February 17, 2004

Mr. L. M. Stinson
Vice President - Farley Project
Southern Nuclear Operating
Company, Inc.
Post Office Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2 — REQUEST FOR
ADDITIONAL INFORMATION RE: RELIEF REQUEST RR-53 - USE OF
ALTERNATE CODE CASE FOR NEW CONSTRUCTION OF REPLACEMENT
REACTOR VESSEL HEAD ADAPTERS (TAC NOS. MC1598 AND MC1599)

Dear Mr. Stinson:

By letter dated December 19, 2003, Southern Nuclear Operating Company requested relief to use the design stress intensity values of Code Case N-698 for the new construction of replacement reactor vessel head adapters at Farley Nuclear Plant, Units 1 and 2. The Nuclear Regulatory Commission technical staff has reviewed the application and has determined that additional information is required, as identified in the Enclosure.

We discussed these issues with your staff on February 5, 2004. Your staff indicated that you would attempt to provide your response by March 12, 2004.

Please contact me at (301) 415-1842, if you have any other questions on these issues.

Sincerely,

/RA/

Sean E. Peters, Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-348 and 50-364

Enclosure: As stated

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

JOSEPH M. FARLEY NUCLEAR PLANT UNITS, UNITS 1 AND 2

DOCKET NOS. 50-348 AND 50-364

The Nuclear Regulatory Commission (NRC) staff has reviewed the licensee's submittal dated December 19, 2003, regarding use the design stress intensity values of Code Case N-698 for the new construction of replacement reactor vessel head adapters at Farley Nuclear Plant, Units 1 and 2. The NRC staff has identified the following information that is needed to enable the continuation of its review.

The relief request states that, "Based on evaluation of hot-worked UNS N06690 SB-167, Code Case N-698 adjusts the yield strength values for hot-worked UNS N06690 to the evaluated value of 35 ksi minimum and provides associated design stress intensity values that coincide with those of Section II, Part D, Subpart 1 of the 1999 addenda." The request also states that, "The design stress intensities and yield strength of Code Case N-474-2 were incorporated into the 1999 agenda." However, your request does not state the design stress intensities values for hot-worked UNS N06690 tubing that you propose to use in the fabrication of the replacement reactor vessel head. Provide a copy of the pending Code Case N-698 and state the design stress intensity values for the proposed hot-worked UNS N06690 tubing material. These values are needed for the NRC staff to determine whether the proposed stress intensity values are appropriate for the material identified in the relief request.

Enclosure

Joseph M. Farley Nuclear Plant

cc:

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