
Industry/TSTF Standard Technical Specification Change Traveler

Delete EFPD Definition

Priority/Classification 2) Consistency/Standardization

NUREGs Affected: 1430 1431 1432 1433 1434

Description:

Delete the definition of EFFECTIVE FULL POWER DAYS.

Justification:

This definition is unnecessary, and inconsistent with the other ISTS NUREGs. EFPD is a very well understood, common term which does not require definition. This term is consistently used throughout the nuclear industry, and has been consistently applied for many years without a TS definition. Additionally, the term is not used as a defined term in the Bases.

A search of the ISTS NUREGs reveals that the term "EFPD" is used in 37 Specifications or Bases in the PWR NUREGs. It is only defined in one occurrence (NUREG-1430, LCO 3.2.2 Bases). EFPD is not a defined term in the other ISTS NUREGs. Therefore, eliminating the defined term from NUREG-1430 and eliminating the one occurrence of it's definition from the NUREG-1430 Bases makes NUREG-1430 internally consistent in its usage and with the other NUREGs (both definitions, Specifications, and Bases).

Revision History

OG Revision 0

Revision Status: Closed

Revision Proposed by: ANO-1

Revision Description:
Original Issue

Owners Group Review Information

Date Originated by OG: 15-Dec-95

Owners Group Comments
1/15/96 - Approved by TE
1/16/96 - Approved by FPC

Owners Group Resolution: Approved Date: 01-Feb-96

TSTF Review Information

TSTF Received Date: 01-Jul-96 Date Distributed for Review 31-Jul-96

OG Review Completed: BWO WOG CEOG BWROG

TSTF Comments:

CEOG - Not applicable
WOG - NA
BWROG - NA

TSTF Resolution: Approved Date: 10-Oct-96

2/17/98

NRC Review Information

NRC Received Date: 22-Jan-97 NRC Reviewer: Harbuck, C.

NRC Comments:

2/7/97 - Reviewer recommended approval.

2/13/97 - To C. Grimes for disposition.

3/18/97 C. Grimes modified pkg. The proposed changes are intended to establish consistent terminology in the B&W STS. However, it is not clear from the justification how the changes achieve consistency both within the B&W STS, and between B&W and the other STS, relative to the proper terminology and use of defined terms.

4/17/97 NRC report states that TSTF preparing revision package.

Final Resolution: Superseded by Revision

Final Resolution Date: 17-Apr-97

TSTF Revision 1

Revision Status: Active

Next Action: NRC

Revision Proposed by: TSTF

Revision Description:

NRC requested additional information on how the proposed change made the B&W NUREG more consistent with the other NUREGs and internally. A search of the ISTS NUREGs reveals that the term "EFPD" is used in 37 Specifications or Bases in the PWR NUREGs. It is only defined in one occurrence (NUREG-1430, LCO 3.2.2 Bases). EFPD is not a defined term in the other ISTS NUREGs. Therefore, eliminating the defined term from NUREG-1430 and eliminating the one occurrence of it's definition from the NUREG-1430 Bases makes NUREG-1430 internally consistent in its usage and with the other NUREGs (both definitions, Specifications, and Bases).

TSTF Review Information

TSTF Received Date: 11-Jan-97 Date Distributed for Review 15-Jan-98

OG Review Completed: BWOG WOG CEOG BWROG

TSTF Comments:

(No Comments)

TSTF Resolution: Approved Date: 05-Feb-98

Incorporation Into the NUREGs

File to BBS/LAN Date:

TSTF Informed Date:

TSTF Approved Date:

NUREG Rev Incorporated:

Affected Technical Specifications

1.1 Definitions - EFPD

Change Description: Definition Deleted

LCO 3.2.2 Bases APSR Insertion Limits

2/17/98

Definitions
1.1

1.1 Definitions

CORE ALTERATION (continued)	ALTERATIONS shall not preclude completion of movement of a component to a safe position.
CORE OPERATING LIMITS REPORT (COLR)	The COLR is the unit specific document that provides cycle specific parameter limits for the current reload cycle. These cycle specific limits shall be determined for each reload cycle in accordance with Specification 5.6.5. Plant operation within these limits is addressed in individual Specifications.
DOSE EQUIVALENT I-131	DOSE EQUIVALENT I-131 shall be that concentration of I-131 (microcuries/gram) that alone would produce the same thyroid dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The thyroid dose conversion factors used for this calculation shall be those listed in [Table III of TID-14844, AEC, 1962, "Calculation of Distance Factors for Power and Test Reactor Sites," or those listed in Table E-7 of Regulatory Guide 1.109, Rev. 1, NRC, 1977, or ICRP 30, Supplement to Part 1, page 192-212, Table titled, "Committed Dose Equivalent in Target Organs or Tissues per Intake of Unit Activity"].
E—AVERAGE DISINTEGRATION ENERGY	E shall be the average (weighted in proportion to the concentration of each radionuclide in the reactor coolant at the time of sampling) of the sum of the average beta and gamma energies per disintegration (in MeV) for isotopes, other than iodines, with half lives > [15] minutes, making up at least 95% of the total noniodine activity in the coolant.
EFFECTIVE FULL POWER DAY (EFPD)	EFPD shall be the ratio of the number of hours of production of a given THERMAL POWER to 24 hours, multiplied by the ratio of the given THERMAL POWER to the RTP. One EFPD is equivalent to the thermal energy produced by operating the reactor core at RTP for one full day.
EMERGENCY FEEDWATER INITIATION AND CONTROL (EFIC) RESPONSE TIME	The EFIC RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its EFIC actuation setpoint at the channel sensor until the emergency feedwater equipment is

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BASES

LCO
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controlling the power distribution within an acceptable range.

The fuel cycle design assumes APSR withdrawal at the ~~effective full power day (EFPD)~~ burnup window specified in the COLR. Prior to this window, the APSRs cannot be maintained fully withdrawn in steady state operation. After this window, the APSRs are not allowed to be reinserted for the remainder of the fuel cycle.

Error adjusted maximum allowable setpoints for APSR insertion are provided in the COLR. The setpoints are derived by adjustment of the measurement system independent limits to allow for THERMAL POWER level uncertainty and rod position errors.

Actual alarm setpoints implemented in the unit may be more restrictive than the maximum allowable setpoint values to allow for additional conservatism between the actual alarm setpoints and the measurement system independent limits.

APPLICABILITY

The APSR physical insertion limits shall be maintained with the reactor in MODES 1 and 2. These limits maintain the power distribution within the range assumed in the accident analyses. In MODE 1, the limits on APSR insertion specified by this LCO maintain the axial fuel burnup design conditions assumed in the reload safety evaluation analysis. In MODE 2, applicability is required because $k_{eff} > 0.99$. Applicability in MODES 3, 4, and 5 is not required, because the power distribution assumptions in the accident analyses would not be exceeded in these MODES.

ACTIONS

For steady state power operation, a normal position for APSR insertion is specified in the station operating procedures. The APSRs may be positioned as necessary for transient AXIAL POWER IMBALANCE control until the fuel cycle design requires them to be fully withdrawn. (Not all fuel cycles may incorporate APSR withdrawal.) APSR position limits are not imposed for gray APSRs, with two exceptions. If the fuel cycle design incorporates an APSR withdrawal (usually near end of cycle (EOC)), the APSRs may not be maintained in the fully withdrawn position prior to the fuel cycle burnup for

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