(WOG-48, Rev. 0) TST	F-8 9
Industry/TSTF Standard Technical Specification Change Traveler	
Change to Frequency of SR 3.1.8.1	
Priority/Classification 3) Improve Specifications	
NUREGS Affected: 1430 🗹 1431 🗌 1432 🗌 1433 🗌 1434	
Description: Change frequency of SR 3.1.8.1 from "[18 months]" to "Once prior to criticality after each removal of the reactor ves head"	sel
Justification: SR 3.1.8.1 verifies that each DRPI agrees within 12 steps of the group demand position for the full indicated range of rod travel. This surveillance is performed during a plant outage or plant startup since there is potential for unnecessar plant transients if the SR is performed with the reactor at power. By not specifying a fixed frequency for this SR, any unit shutdown and reactor vessel head removal would require that the SR be performed again to verify that the operability of the rod position indicator systems has not been affected.	агу
Revision History	
OG Revision 0 Revision Status: Active Next Action:	
Revision Proposed by: Braidwood	
Revision Description: Original Issue	
Owners Group Review Information	
Date Originated by OG: 02-Apr-96	
Owners Group Comments Transmitted and approved by J. Andrachek	
Owners Group Resolution: Approved Date: 02-Apr-96	
TSTF Review Information	
TSTF Received Date: 12-Apr-96 Date Distributed for Review 12-Apr-96	
OG Review Completed: 🗹 BWOG 🗹 WOG 🗹 CEOG 🗹 BWROG	
TSTF Comments: NA CEOG, BWOG, BWROG TSTF Resolution: Approved Date: 28-May-96	
NRC Review Information	
NRC Received Date: 17-Jul-96 NRC Reviewer: R. Tjader	
NRC Comments: 9/18/96 - Approved	
Final Resolution: NRC Approves Final Resolution Date: 18-Sep-96	

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Incorporation Into the NUREGs

			(WOG-48, Rev. 0)	TSTF-89
File to BBS/LAN Date:		TSTF Informed Date:	TSTF Approved Date:	
NUREG Rev Incor	porated:			
Affected Techn	nical Specification	15		
SR 3.1.8.1	Rod Position Indicat			
SR 3.1.8.1 Bases	Rod Position Indicat	ion		

Rod Position Indication 3.1.8 TSTF-89

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SURVEILLANCE REQUIREMENTS

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	SURVEILLANCE	FREQUENCY
SR 3.1.8.1	Verify each [D]RPI agrees within [12] steps of the group demand position for the [full indicated range] of rod travel.	[18 months]
	Or Criteac of head	nce prior to ticality after h removal the reactor

BASES

ACTIONS (continued)

Reduction of THERMAL POWER to \leq 50% RTP puts the core into a condition where rod position is not significantly affecting core peaking factor limits (Ref. 3). The allowed Completion Time of 8 hours provides an acceptable period of time to verify the rod positions per Required Actions C.1.1 and C.1.2 or reduce power to \leq 50% RTP.

<u>D.1</u>

<u>C.2</u>

If the Required Actions cannot be completed within the associated Completion Time, the plant must be brought to a MODE in which the requirement does not apply. To achieve this status, the plant must be brought to at least MODE 3 within 6 hours. The allowed Completion Time is reasonable, based on operating experience, for reaching the required MODE from full power conditions in an orderly manner and without challenging plant systems.

SURVEILLANCE <u>SR 3.1.8.1</u> REQUIREMENTS

Verification that the DRPI agrees with the demand position within [12] steps ensures that the DRPI is operating correctly. Since the DRPI does not display the actual shutdown rod positions between 18 and 210 steps, only points within the indicated ranges are required in comparison.



The [18 month] Frequency is based on the need to perform this Surveillance under the conditions that apply during a plant outage and the potential for unnecessary plant transients if the SR were performed with the reactor at power. Operating experience has shown these components isually pass the SR when performed at a Frequency of once every [18 months.] Therefore, the Frequency was concluded to be acceptable from a reliability standpoint.

REFERENCES

1. 10 CFR 50, Appendix A, GDC 13.

- 2. FSAR, Chapter [15].
- 3. FSAR, Chapter [15].

<u>Insert</u>

This surveillance is perform prior to reactor criticality after each removal of the reactor head as there is the potential for unnecessary plant transients if the SR were performed with the reactor at power.