

Mark B. Bezilla Vice President - Nuclear 419-321-7676 Fax: 419-321-7582

Docket Number 50-346

License Number NPF-3

Serial Number 2993

February 4, 2004

United States Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555-0001

Subject: Clarifications Regarding Safety Evaluations for License Amendment No. 253
(License Amendment Request No. 98-0004; TAC No. MA9694) and License Amendment No. 259 (License Amendment Request No. 99-0004; TAC No. MB1679)

Ladies and Gentlemen:

The purpose of this letter is to provide two clarifications regarding the Safety Evaluations associated with two License Amendments approved by the Nuclear Regulatory Commission (NRC). The FirstEnergy Nuclear Operating Company (FENOC) submitted License Amendment Request (LAR) 98-0004 to the NRC by letter dated November 9, 2000 (FENOC Serial No. 2667), proposing a revision to the Davis-Besse Nuclear Power Station (DBNPS) Unit No. 1 Operating License No. NPF-3, Appendix A Technical Specifications. This LAR requested an extension of the allowed outage time for one Low Pressure Injection (LPI) System/Decay Heat Cooler train and one Containment Spray System train. The NRC issued License Amendment No. 253 by letter dated September 17, 2002 (FENOC Log No. 6006).

Section 3.1 of the NRC Safety Evaluation for Amendment No. 253 stated, in part:

In the deterministic evaluation, based on the conclusion of the SER for the topical report and the adequacy of one train of the spray system satisfying the minimum capability requirement for the safety function of the containment spray system, the staff finds the proposed allowable outage time of 7 days acceptable.



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Regarding this statement, FENOC wishes to clarify that one 50% capacity train of the containment spray system, operating in conjunction with one 50% capacity containment air cooler, provides the minimum capability for containment heat removal. DBNPS Updated Safety Analysis Report (USAR) Section 6.2.2.1, "Containment Vessel Heat Removal Systems, Design Bases," states, in part:

Post LOCA containment heat removal is effected through the use of the Containment Air Cooling System and the Containment Spray System. There are two containment sprays and three containment air coolers. The capacity of the containment heat removal systems is based on a containment heat load of 150×10^6 Btu/hr. The LOCA considered to determine the heat load was a split of the hot leg reactor coolant pipe. Each containment spray and each containment air cooler is designed for 50 percent of the heat load (75×10^6 Btu/hr). Two fully redundant heat removal methods composed of one containment spray train and one containment air cooler train are provided for post-LOCA heat removal.

The function of the containment air coolers (or reactor building cooling system) was discussed in the topical report referenced in the NRC's Safety Evaluation. DBNPS Technical Specification Limiting Condition for Operation 3.6.2.2 requires at least two operable independent containment air coolers in Modes 1, 2, and 3 and provides an allowable outage time of 72 hours for one inoperable containment air cooler.

FENOC submitted LAR 99-0004 to the NRC by letter dated April 1, 2001 (FENOC Serial No. 2625), proposing a revision to Technical Specification Surveillance Requirement (SR) 4.3.2.2.2 for the Steam and Feedwater Rupture Control System (SFRCS). The NRC issued License Amendment No. 259 by letter dated September 29, 2003 (FENOC Log No. 6120). Section 4.4 of the NRC Safety Evaluation for Amendment No. 259 stated, in part:

Evaluation: In its submittal, the licensee stated that the proposed change would clarify that the term "total bypass function" for SFAS in current TS SR 4.3.2.2.2 refers to the shutdown bypass . . . The shutdown bypass is the only SFRCS bypass referred to in TS Table 3.3-11 . . . SR 4.3.2.2.2 is accomplished by verifying that the SFRCS bypasses cannot be added prior to decreasing main steam line pressure . . .

The "SFAS" referred to here should be "SFRCS", as SR 4.3.2.2.2 is for the SFRCS instrumentation.

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No response to this letter is requested or required. However, should you have any questions or require additional information, please contact Mr. Gregory A. Dunn, Manager - Regulatory Affairs, at (419) 321-8450.

Very truly yours,

MKL

Enclosure

cc: Regional Administrator, NRC Region III

MIS Byth

J. B. Hopkins, NRC/NRR Senior Project Manager

C. S. Thomas, NRC Region III, DB-1 Senior Resident Inspector

Utility Radiological Safety Board

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COMMITMENT LIST

THE FOLLOWING LIST IDENTIFIES THOSE ACTIONS COMMITTED TO BY THE DAVIS-BESSE NUCLEAR POWER STATION (DBNPS) IN THIS DOCUMENT. ANY OTHER ACTIONS DISCUSSED IN THE SUBMITTAL REPRESENT INTENDED OR PLANNED ACTIONS BY THE DBNPS. THEY ARE DESCRIBED ONLY FOR INFORMATION AND ARE NOT REGULATORY COMMITMENTS. PLEASE NOTIFY THE MANAGER – REGULATORY AFFAIRS (419-321-8450) AT THE DBNPS OF ANY QUESTIONS REGARDING THIS DOCUMENT OR ANY ASSOCIATED REGULATORY COMMITMENTS.

COMMITMENTS	DUE DATE
None.	N/A