

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

January 23, 2004

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. 04-042
NAPS/MPW
Docket Nos. 50-338/339
License Nos. NPF-4/7

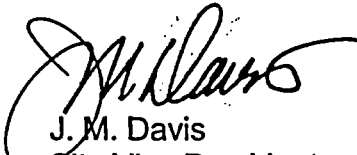
Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION UNITS 1 AND 2
REVISIONS TO EMERGENCY PLAN IMPLEMENTING PROCEDURES

Pursuant to 10 CFR 50.54(q), enclosed is a recent revision to the North Anna Power Station Emergency Plan Implementing Procedure. This revision does not implement actions that decrease the effectiveness of our Emergency Plan. The Emergency Plan and Implementing Procedures continue to meet the standards of 10 CFR 50.47(b).

Please update your manual by performing the actions described in Attachment 1, Tabulation of Changes.

Very truly yours,



J. M. Davis
Site Vice President

Commitments Stated or Implied: None.

Enclosures

cc: U.S. Nuclear Regulatory Commission (2 copies)
Region II
Atlanta Federal Center
61 Forsyth St., SW, Suite 23T85
Atlanta, GA 30303

Mr. M. T. Widmann
NRC Senior Resident Inspector
North Anna Power Station

A045

**ATTACHMENT 1
TABULATION OF CHANGES**

**VIRGINIA ELECTRIC AND POWER COMPANY
REVISIONS TO NORTH ANNA POWER STATION
EMERGENCY PLAN IMPLEMENTING PROCEDURE**

Enclosed is a recent revision to the North Anna Power Station Emergency Plan Implementing Procedure (EPIP). Please take the following actions in order to keep your manual updated.

REMOVE AND DESTROY	DATED	INSERT	EFFECTIVE DATE
EPIP - 1.01, Rev. 38	07/01/03	EPIP - 1.01, Rev. 39	01/15/04

Emergency Plan Privacy and Proprietary Material has been removed. Reference Generic Letter No. 81-27.

NORTH ANNA POWER STATION
LIST OF NAPS EMERGENCY PLAN IMPLEMENTATION PROCEDURES
CHECK THIS FOR LATEST DOCUMENT INFORMATION

DOCUMENT NUMBER	REV	APPROVAL **DATE**	EFFECT** **DATE**	DOCUMENT TITLE
EPIP-1.01	039	01/15/04	01/15/04	EMERGENCY MANAGER CONTROLLING PROCEDURE
EPIP-1.02	011	09/07/99	10/01/99	RESPONSE TO NOTIFICATION OF UNUSUAL EVENT
EPIP-1.03	014	09/07/99	10/01/99	RESPONSE TO ALERT
EPIP-1.04	014	09/07/99	10/01/99	RESPONSE TO SITE AREA EMERGENCY
EPIP-1.05	016	09/07/99	10/01/99	RESPONSE TO GENERAL EMERGENCY
EPIP-1.06	004	09/05/01	09/05/01	PROTECTIVE ACTION RECOMMENDATIONS
EPIP-2.01	025	08/13/02	08/28/02	NOTIFICATION OF STATE AND LOCAL GOVERNMENTS
EPIP-2.02	015	08/13/02	08/28/02	NOTIFICATION OF NRC
EPIP-3.02	021	03/04/03	03/17/03	ACTIVATION OF TECHNICAL SUPPORT CENTER
EPIP-3.03	013	03/04/03	03/17/03	ACTIVATION OF OPERATIONAL SUPPORT CENTER
EPIP-3.04	015	07/14/98	07/20/98	ACTIVATION OF LOCAL EMERGENCY OPERATIONS FACILITY
EPIP-3.05	002	04/02/03	04/08/03	AUGMENTATION OF EMERGENCY RESPONSE ORGANIZATION
EPIP-4.01	020	06/24/03	07/01/03	RADIOLOGICAL ASSESSMENT DIRECTOR CONTROLLING PROCEDURE
EPIP-4.02	015	06/24/03	07/01/03	RADIATION PROTECTION SUPERVISOR CONTROLLING PROCEDURE
EPIP-4.03	011	12/20/93	01/01/94	DOSE ASSESSMENT TEAM CONTROLLING PROCEDURE
EPIP-4.04	009	11/21/94	11/28/94	EMERGENCY PERSONNEL RADIATION EXPOSURE
EPIP-4.05	009	01/28/00	02/04/00	RESPIRATORY PROTECTION AND KI ASSESSMENT
EPIP-4.06	009	12/21/95	12/28/95	PERSONNEL MONITORING AND DECONTAMINATION
EPIP-4.07	014	09/29/00	10/06/00	PROTECTIVE MEASURES
EPIP-4.08	014	05/10/02	06/19/02	INITIAL OFFSITE RELEASE ASSESSMENT
EPIP-4.09	014	06/24/03	07/01/03	SOURCE TERM ASSESSMENT
EPIP-4.10	011	08/13/02	08/28/02	DETERMINATION OF X/Q
EPIP-4.13	009	09/29/00	10/06/00	OFFSITE RELEASE ASSESSMENT WITH ENVIRONMENTAL DATA

NORTH ANNA POWER STATION
LIST OF NAPS EMERGENCY PLAN IMPLEMENTATION PROCEDURES
CHECK THIS FOR LATEST DOCUMENT INFORMATION

DOCUMENT NUMBER	REV	APPROVAL **DATE**	EFFECT** **DATE**	DOCUMENT TITLE
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EPIP-4.14	007	12/20/93	01/01/94	INPLANT MONITORING
EPIP-4.15	011	02/18/00	02/28/00	ONSITE MONITORING
EPIP-4.16	015	12/13/02	12/18/02	OFFSITE MONITORING
EPIP-4.17	016	12/13/02	12/18/02	MONITORING OF EMERGENCY RESPONSE FACILITIES
EPIP-4.18	013	12/13/02	12/18/02	MONITORING OF LEOF
EPIP-4.21	008	12/20/93	01/01/94	EVACUATION AND REMOTE ASSEMBLY AREA MONITORING
EPIP-4.24	013	04/02/03	04/08/03	GASEOUS EFFLUENT SAMPLING DURING AN EMERGENCY
EPIP-4.26	012	06/24/03	07/01/03	HIGH LEVEL ACTIVITY SAMPLE ANALYSIS
EPIP-4.28	007	01/09/97	01/14/97	TSC/LEOF RADIATION MONITORING SYSTEM
EPIP-4.30	005	04/05/02	04/09/02	USE OF MIDAS CLASS A MODEL
EPIP-4.31	003	06/20/94	06/20/94	USE OF MIDAS CLASS B MODEL
EPIP-4.33	003	11/28/00	11/30/00	HEALTH PHYSICS NETWORK COMMUNICATIONS
EPIP-4.34	003	12/13/02	12/18/02	FIELD TEAM RADIO OPERATOR INSTRUCTIONS
EPIP-4.35	000	06/24/03	07/01/03	CHEMISTRY SAMPLING
EPIP-5.01	011	12/11/96	12/17/96	TRANSPORTATION OF CONTAMINATED INJURED PERSONNEL
EPIP-5.03	016	02/18/00	02/28/00	PERSONNEL ACCOUNTABILITY
EPIP-5.04	010	03/04/03	03/17/03	ACCESS CONTROL
EPIP-5.05	013	06/25/96	07/02/96	SITE EVACUATION
EPIP-5.07	012	06/24/03	07/01/03	ADMINISTRATION OF RADIOPROTECTIVE DRUGS
EPIP-5.08	008	04/02/03	04/08/03	DAMAGE CONTROL GUIDELINE
EPIP-5.09	004	08/02/02	08/15/02	SECURITY TEAM LEADER CONTROLLING PROCEDURE
EPIP-6.01	007	05/12/99	05/17/99	RE-ENTRY/RECOVERY GUIDELINE

VIRGINIA POWER
NORTH ANNA POWER STATION
EMERGENCY PLAN IMPLEMENTING PROCEDURE

NUMBER EPIP-1.01	PROCEDURE TITLE EMERGENCY MANAGER CONTROLLING PROCEDURE (With 3 Attachments)	REVISION 39
		PAGE 1 of 7

PURPOSE

To assess potential emergency conditions and initiate corrective actions.

LEVEL 2 COPY
THIS DOCUMENT SHALL BE VERIFIED
TO A CONTROLLED SOURCE AS
REQUIRED TO PERFORM WORK

ENTRY CONDITIONS

Any of the following:

1. Another station procedure directs initiation of this procedure.
2. A potential emergency condition is reported to the Shift Manager.

Approvals on File

Effective Date

1/15/2004

NUMBER EPIP-1.01	PROCEDURE TITLE EMERGENCY MANAGER CONTROLLING PROCEDURE	REVISION 39 PAGE 2 of 7
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
<p>*****</p> <p>CAUTION: Declaration of the highest emergency class for which an Emergency Action Level is exceeded shall be made.</p> <p>*****</p> <p>NOTE: The PCS is potentially unreliable in the event of an earthquake. Therefore, PCS parameters should be evaluated for accuracy should this situation occur.</p> <p>_____ 1 EVALUATE EMERGENCY ACTION LEVELS:</p> <p>a) Determine event category using Attachment 1, EMERGENCY ACTION LEVEL TABLE INDEX</p> <p>b) Review EAL Tab associated with event category</p> <p>c) Use Control Room monitors, PCS, and outside reports to get indications of emergency conditions listed in the EAL Table</p> <p>d) Verify EAL - CURRENTLY EXCEEDED</p>		
		<p>d) <u>IF</u> basis for EAL no longer exists when discovered <u>AND</u> no other reasons exist for an emergency declaration, <u>THEN</u> do the following:</p> <ul style="list-style-type: none"> • RETURN TO procedure in effect. • GO TO VPAP-2802, NOTIFICATIONS AND REPORTS, to make one-hour, non-emergency reports for classification without declaration. <p><u>IF</u> EAL was <u>NOT</u> exceeded, <u>THEN</u> RETURN TO procedure in effect.</p>
<p>(STEP 1 CONTINUED ON NEXT PAGE)</p>		

NUMBER EPIP-1.01	PROCEDURE TITLE EMERGENCY MANAGER CONTROLLING PROCEDURE	REVISION 39 PAGE 3 of 7
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
1	<p>EVALUATE EMERGENCY ACTION LEVELS: (Continued)</p> <p>e) Record procedure initiation:</p> <ul style="list-style-type: none"> By: _____ Date: _____ Time: _____ <p>f) Initiate a chronological log of events</p> <p>g) Declare position of Station Emergency Manager</p> <p>NOTE: Assembly, accountability and/or initiation of facility staffing may not be desired during certain situations (e.g., security event, severe weather, anticipated grid disturbance) or may have already been completed. These activities should be implemented as quickly as achievable given the specific situation.</p>	
2	<p>CHECK - CONDITIONS ALLOW FOR NORMAL IMPLEMENTATION OF EMERGENCY RESPONSE ACTIONS</p>	<p><u>IF</u> deviation from normal emergency response actions warranted, <u>THEN</u> do the following:</p> <p>a) Refer to Attachment 3, Considerations for Operations Response Under Abnormal Conditions.</p> <p>b) Consider applicability of 50.54(x).</p> <p>c) <u>IF</u> classification/assembly announcement deferred, <u>THEN</u> GO TO Step 4.</p>

NUMBER EPIP-1.01	PROCEDURE TITLE EMERGENCY MANAGER CONTROLLING PROCEDURE	REVISION 39 <hr/> PAGE 4 of 7
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
<p>3</p>	<p>NOTIFY PLANT STAFF OF ALERT OR HIGHER CLASSIFICATION:</p> <p>a) Check classification - ALERT OR HIGHER</p> <p>b) Check if emergency assembly and accountability - PREVIOUSLY CONDUCTED</p> <p>c) Have Control Room sound EMERGENCY alarm and make announcement on station Gai-Tronics system as follows:</p> <p>“(Emergency classification) has been declared as the result of _____” (event)</p> <p>d) Repeat Step 3.c</p>	<p>a) GO TO Step 4.</p> <p>b) Do the following:</p> <p>1) Have Control Room sound EMERGENCY alarm and make announcement on station Gai-Tronics system as follows:</p> <p>“(Emergency classification) has been declared as the result of _____” (event)</p> <p>“All Emergency Response personnel report to your assigned stations”</p> <p>“All contractor personnel not responding to the emergency and all visitors report to the Security Building”</p> <p>“All other personnel report to your Emergency Assembly Areas”</p> <p>2) Repeat RNO Step 3.b.1.</p> <p>3) GO TO Step 4.</p>

NUMBER EPIP-1.01	PROCEDURE TITLE EMERGENCY MANAGER CONTROLLING PROCEDURE	REVISION 39 PAGE 5 of 7
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
<p>*****</p>		
	<p>CAUTION: Continue through this and all further instructions unless otherwise directed to hold.</p>	
<p>*****</p>		
<p>4</p>	<p>INITIATE SUPPORTING PROCEDURES:</p>	
	<p>a) Direct Emergency Communicators to initiate the following procedures:</p>	
	<p>1) EPIP-2.01, NOTIFICATION OF STATE AND LOCAL GOVERNMENTS</p>	
	<p>2) EPIP-2.02, NOTIFICATION OF NRC</p>	
	<p>b) Direct HP to initiate EPIP-4.01, RADIOLOGICAL ASSESSMENT DIRECTOR CONTROLLING PROCEDURE</p>	
	<p>c) Establish communications with Security Team Leader:</p>	
	<p>1) Provide Security with current emergency classification</p>	
	<p>2) Notify Security which Operations Shift is designated for coverage</p>	
	<p>3) Direct Security to initiate EPIP-5.09, SECURITY TEAM LEADER CONTROLLING PROCEDURE</p>	

NUMBER EPIP-1.01	PROCEDURE TITLE EMERGENCY MANAGER CONTROLLING PROCEDURE	REVISION 39 PAGE 6 of 7
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
5	CHECK TSC - ACTIVATED	<p>IF TSC <u>NOT</u> activated, <u>THEN</u> do the following:</p> <ul style="list-style-type: none"> a) Have STA report to the Control Room. b) Notify Manager Nuclear Operations or Operations Manager On Call. c) Consider having Radiological Assessment Director report to the Control Room. d) <u>WHEN</u> relief SEM arrives, <u>THEN</u> perform turnover using EPIP-1.01, Attachment 2. Turnover Checklist.
6	<p>IMPLEMENT EPIP FOR EMERGENCY CLASSIFICATION IN EFFECT:</p> <ul style="list-style-type: none"> • Notification of Unusual Event - GO TO EPIP-1.02, RESPONSE TO NOTIFICATION OF UNUSUAL EVENT • Alert - GO TO EPIP-1.03, RESPONSE TO ALERT • Site Area Emergency - GO TO EPIP-1.04, RESPONSE TO SITE AREA EMERGENCY • General Emergency - GO TO EPIP-1.05, RESPONSE TO GENERAL EMERGENCY 	

NUMBER EPIP-1.01	PROCEDURE TITLE EMERGENCY MANAGER CONTROLLING PROCEDURE	REVISION 39 <hr/> PAGE 7 of 7
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STEP	ACTION/EXPECTED RESPONSE	RESPONSE NOT OBTAINED
7	<p>NOTIFY OFFSITE AUTHORITIES OF EMERGENCY TERMINATION:</p> <ul style="list-style-type: none"> a) State and local governments (made by LEOF or CEOF when activated) b) NRC 	
8	<p>NOTIFY STATION PERSONNEL ABOUT THE FOLLOWING:</p> <ul style="list-style-type: none"> • Emergency termination • Facility de-activation • Selective release of personnel • Completion and collection of procedures • Recovery 	
9	<p>TERMINATE EPIP-1.01:</p> <ul style="list-style-type: none"> • Give completed EIPs, forms and other applicable records to Nuclear Emergency Preparedness (TSC Emergency Procedures Coordinator if TSC activated) • Completed By: _____ Date: _____ Time: _____ 	

-END-

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	INDEX	PAGE
1		1 of 42

- CAUTION:**
- Declaration of the highest emergency class for which an EAL is exceeded shall be made.
 - Emergency Action Levels shall be conservatively classified based on actual or anticipated plant conditions.

<u>EVENT CATEGORY:</u>	<u>TAB</u>
1. Safety, Shutdown, or Assessment System Event.....	A
2. Reactor Coolant System Event.....	B
3. Fuel Failure or Fuel Handling Accident.....	C
4. Containment Event.....	D
5. Radioactivity Event.....	E
6. DELETED	
7. Loss of Secondary Coolant.....	G
8. Electrical Failure.....	H
9. Fire.....	I
10. Security Event.....	J
11. Hazard to Station Operation.....	K
12. Natural Events.....	L
13. Miscellaneous Abnormal Events.....	M

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB A)	PAGE
1	SAFETY, SHUTDOWN, OR ASSESSMENT SYSTEM EVENT	2 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

CAUTION: EAL C.2 is duplicated below for cross-reference/comparison to EAL A.1:

C.2. Probable large radioactivity release initiated by loss of heat sink leading to core degradation

MODES 1, 2, 3 & 4

Loss of Main Feedwater System, Condensate System and Auxiliary Feedwater System

GENERAL
EMERGENCY

1. Loss of function needed for unit HSD condition

MODES 1, 2, 3 & 4

• Total loss of the Charging/SI System

OR

Total loss of the Main Feedwater and Auxiliary Feedwater systems

SITE AREA
EMERGENCY

2. Failure of the Reactor Protection System to initiate and complete a required trip while at power

MODES 1 & 2

• Reactor trip setpoint and coincidences - EXCEEDED

AND

• Automatic trip from RPS - FAILED

AND

• Manual trip from Control Room - FAILED

SITE AREA
EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB A) SAFETY, SHUTDOWN, OR ASSESSMENT SYSTEM EVENT	39
ATTACHMENT		PAGE
1		3 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
<p>3. Inability to monitor a significant transient in progress</p> <p>MODES 1, 2, 3 & 4</p>	<ul style="list-style-type: none"> Most (>75%) or all annunciator alarms on panels "A" to "K" - NOT AVAILABLE <p><u>AND</u></p> <ul style="list-style-type: none"> All computer monitoring capability (e.g., PCS) - NOT AVAILABLE <p><u>AND</u></p> <ul style="list-style-type: none"> Significant transient - IN PROGRESS (e.g., reactor trip, SI actuation, turbine runback >25% thermal reactor power, thermal power oscillations >10%) <p><u>AND</u></p> <ul style="list-style-type: none"> Inability to directly monitor any one of the following using Control Room indications: <ul style="list-style-type: none"> Subcriticality Core Cooling Heat Sink Vessel Integrity Containment Integrity 	<p>SITE AREA EMERGENCY</p>
<p>4. Evacuation of Main Control Room with control not established within 15 minutes</p> <p>ALL MODES</p>	<p>Evacuation of the Control Room with local shutdown control not established within 15 minutes</p>	<p>SITE AREA EMERGENCY</p>

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB A) SAFETY, SHUTDOWN, OR ASSESSMENT SYSTEM EVENT	39
ATTACHMENT		PAGE
1		4 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
5. Total loss of function needed for unit CSD condition MODES 5 & 6	<ul style="list-style-type: none"> Secondary system cooling capability - UNAVAILABLE <p><u>AND</u></p> <ul style="list-style-type: none"> Loss of any of the following systems: <ul style="list-style-type: none"> Service Water Component Cooling RHR <p><u>AND</u></p> <ul style="list-style-type: none"> RCS temperature GREATER THAN 140 °F 	ALERT
6. Failure of the Reactor Protection System to complete a trip which takes the Reactor Subcritical MODES 1 & 2	<ul style="list-style-type: none"> Reactor trip setpoint and coincidences - EXCEEDED <p><u>AND</u></p> <ul style="list-style-type: none"> Automatic trip from RPS - FAILED <p><u>AND</u></p> <ul style="list-style-type: none"> Manual trip - REQUIRED <p><u>AND</u></p> <ul style="list-style-type: none"> Manual trip from Control Room - SUCCESSFUL 	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB A) SAFETY, SHUTDOWN, OR ASSESSMENT SYSTEM EVENT	39
ATTACHMENT		PAGE
1		5 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

7. Unplanned loss of safety system annunciators with compensatory indicators unavailable or a transient in progress

MODES 1, 2, 3 & 4

- Unplanned loss of most (>75%) or all annunciator alarms on panels "A" to "K" for GREATER THAN 15 minutes

AND

- All computer monitoring capability (e.g., PCS)
- NOT AVAILABLE

OR

Significant transient -
INITIATED OR IN PROGRESS
(e.g., reactor trip, SI,
turbine runback > 25%
thermal reactor power,
thermal power oscillations
> 10%)

ALERT

8. Evacuation of Main Control Room required

ALL MODES

Evacuation of the Control Room with shutdown control established within 15 minutes

ALERT

9. Inability to reach required mode within technical specification limits

MODES 1, 2, 3 & 4

- Intentional reduction in power, load or temperature IAW T.S. Action Statement - HAS COMMENCED

AND

- T.S. Action Statement time limit for mode change - CANNOT BE MET

NOTIFICATION
OF UNUSUAL
EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB A)	39
ATTACHMENT	SAFETY, SHUTDOWN, OR ASSESSMENT SYSTEM EVENT	PAGE
1		6 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

10. Failure of a safety or relief valve to close after pressure reduction, which may affect the health and safety of the public

MODES 1, 2, 3, 4 & 5

• RCS

- RCS pressure - LESS THAN 2000 psig

OR

NDT Protection System - IN SERVICE

AND

- Any indication after lift or actuation that Pressurizer Safety or PORV - REMAINS OPEN

AND

- Flow - UNISOLABLE

• Main Steam

- Excessive Steam Generator Safety, PORV or Decay Heat Release flow as indicated by rapid RCS cooldown rate

AND

- Main Steam pressure greater than 100 psi below setpoint of affected valve

NOTIFICATION
OF UNUSUAL
EVENT

11. Unplanned loss of most or all safety system annunciators for greater than 15 minutes

MODES 1, 2, 3 & 4

- Unplanned loss of most (>75%) or all annunciators on panels "A" to "K" for GREATER THAN 15 minutes

NOTIFICATION
OF UNUSUAL
EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB A)	39
ATTACHMENT 1	SAFETY, SHUTDOWN, OR ASSESSMENT SYSTEM EVENT	PAGE 7 of 42

CONDITION/APPLICABILITY

12. Loss of
communications
capability

ALL MODES

INDICATION

- Station PBX phone system -
FAILED

AND

- Station Gai-tronics
system - FAILED

AND

- Station UHF radio system -
FAILED

CLASSIFICATION

NOTIFICATION
OF UNUSUAL
EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB B) REACTOR COOLANT SYSTEM EVENT	39
ATTACHMENT		PAGE
1		8 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
1. Loss of 2 of 3 fission product barriers with potential loss of 3rd barrier ALL MODES	<p>Any two of a), b) or c) exist and the third is imminent:</p> <p>a) Fuel clad integrity failure as indicated by any of the following:</p> <ul style="list-style-type: none"> RCS specific activity greater than or equal to 300.0 $\mu\text{Ci}/\text{gram}$ dose equivalent I-131 <p><u>OR</u></p> <p>5 or more core exit thermocouples greater than 1200 °F</p> <p><u>OR</u></p> <p>Containment High Range Radiation Monitor</p> <div style="border: 1px solid black; padding: 5px; width: fit-content; margin: 10px auto;"> RM-RMS-165, -166 or RM-RMS-265, -266 GREATER THAN 1.88x10² R/hr </div> <p>b) Loss of RCS integrity as indicated by any of the following:</p> <ul style="list-style-type: none"> RCS pressure greater than 2735 psig <p><u>OR</u></p> <p>Loss of Reactor Coolant in progress</p> <p>c) Loss of containment integrity as indicated by any of the following:</p> <ul style="list-style-type: none"> Containment pressure greater than 60 psia and not decreasing <p><u>OR</u></p> <p>Release path to environment -EXISTS</p>	GENERAL EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB B) REACTOR COOLANT SYSTEM EVENT	39
ATTACHMENT		PAGE
1		9 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
2. Fuel failure with steam generator tube rupture ALL MODES	<p>Any two of a), b) or c) exist and the third is imminent:</p> <p>a) Fuel clad integrity failure as indicated by any of the following:</p> <ul style="list-style-type: none"> RCS specific activity greater than 300 $\mu\text{Ci}/\text{gram}$ dose equivalent I-131 <p style="text-align: center;"><u>OR</u></p> <p>5 or more core exit thermocouples GREATER THAN 1200 °F</p> <p style="text-align: center;"><u>OR</u></p> <p>High Range Letdown radiation monitor</p> <div style="border: 1px solid black; padding: 5px; width: fit-content; margin: 10px auto;"> 1-CH-RI-128 or 2-CH-RI-228 GREATER THAN 5.9×10^4 mR/hr </div> <p>b) Steam Generator tube rupture as indicated by both of the following:</p> <ul style="list-style-type: none"> SI coincidence - SATISFIED <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Steam Generator tube rupture -IN PROGRESS <p>c) Loss of secondary integrity associated with ruptured steam generator pathway as indicated by any of the following:</p> <ul style="list-style-type: none"> Steam Generator PORV - OPEN <p style="text-align: center;"><u>OR</u></p> <p>Main Steam Code Safety Valve - OPEN</p> <p style="text-align: center;"><u>OR</u></p> <p>Loss of secondary coolant outside containment - IN PROGRESS</p>	GENERAL EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB B)	39
ATTACHMENT 1	REACTOR COOLANT SYSTEM EVENT	PAGE 10 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

3. RCS leak rate exceeds
makeup capacity

MODES 1, 2, 3, & 4

- Primary system leak (LOCA)
- IN PROGRESS

AND

- Safety Injection - REQUIRED

AND

- RCS subcooling based on
Core Exit Thermocouples -
LESS THAN 30° F

OR

RCS inventory cannot be
maintained based on pressurizer
level or RVLIS indication

SITE AREA
EMERGENCY

4. Gross primary to
secondary leakage
with loss of offsite
power

MODES 1, 2, 3, & 4

- Steam Generator Tube
Rupture - IN PROGRESS

AND

- Safety Injection - REQUIRED

AND

- Vent Vent A MGPI Monitor

RM-VG-179 GREATER THAN
1.25 x 10⁸ µCi/sec

OR

Steam Generator Blowdown
monitor on affected pathway

RM-SS-122, -222
RM-SS-123, -223
RM-SS-124, -224
GREATER THAN 1x10⁶ cpm

AND

- A subsequent loss of
offsite power indicated by
zero volts on voltmeters
for 4160V buses D, E, & F

SITE AREA
EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB B) REACTOR COOLANT SYSTEM EVENT	39
ATTACHMENT		PAGE
1		11 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
5. RCS leak rate limit - EXCEEDED MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Pressurizer level cannot be maintained greater than 20% with one (1) Charging/SI pump in operation <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> RCS inventory balance indicates leakage - greater than 50 gpm 	ALERT
6. Gross primary to secondary leakage MODES 1, 2, 3, & 4	Steam Generator Tube Rupture - IN PROGRESS <p style="text-align: center;"><u>AND</u></p> Safety Injection - REQUIRED	ALERT
7. Excessive primary to secondary leakage with loss of offsite power MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Intentional reduction in power, load or temperature IAW T.S. 3.4.13 primary-to-secondary leakage LCO Action Statement <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Vent Vent A MGPI Monitor <div style="border: 1px solid black; padding: 5px; margin: 5px 0;"> RM-VG-179 GREATER THAN $1.73 \times 10^6 \mu\text{Ci/sec}$ </div> <p style="text-align: center;"><u>OR</u></p> Steam Generator Blowdown monitor on affected pathway <div style="border: 1px solid black; padding: 5px; margin: 5px 0;"> RM-SS-122, -222 RM-SS-123, -223 RM-SS-124, -224 GREATER THAN 1×10^5 cpm </div> <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> A subsequent loss of offsite power indicated by zero volts on voltmeters for 4160V buses D, E, & F 	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB B) REACTOR COOLANT SYSTEM EVENT	39
ATTACHMENT		PAGE
1		12 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
8. RCS operational leakage requiring plant shutdown IAW T.S. 3.4.13 MODES 1, 2, 3, & 4	Intentional reduction in power load or temperature IAW T.S. 3.4.13 leakage limit action statement - HAS COMMENCED	NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB C)	PAGE
1	FUEL FAILURE OR FUEL HANDLING ACCIDENT	13 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

1. Probable large radioactivity release initiated by LOCA with ECCS failure leading to core degradation

ALL MODES

- Loss of reactor coolant in progress
- AND
- RCS specific activity - greater than 300 $\mu\text{Ci}/\text{gram}$ dose equivalent I-131

GENERAL
EMERGENCY

OR

Containment High Range
Radiation Monitor

RM-RMS-165, -166 or
 RM-RMS-265, -266
 GREATER THAN
 1.88×10^2 R/hr

AND

- High or low head ECCS flow not being delivered to the core (if expected by plant conditions)

CAUTION: EAL A.1 is duplicated below for cross-reference/comparison to EAL C.2:

- A.1. Loss of function needed for unit HSD condition

- Total loss of the Charging/SI System

SITE AREA
EMERGENCY

MODES 1, 2, 3 & 4

OR

Total loss of the Main
Feedwater and Auxiliary
Feedwater systems

2. Probable large radioactivity release initiated by loss of heat sink leading to core degradation

MODES 1, 2, 3 & 4

Loss of Main Feedwater System,
Condensate System and Auxiliary
Feedwater System

GENERAL
EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB C)	39
ATTACHMENT 1	FUEL FAILURE OR FUEL HANDLING ACCIDENT	PAGE 14 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
3. Probable large radioactivity release initiated by failure of protection system to bring Rx subcritical and causing core degradation ALL MODES	<ul style="list-style-type: none"> Rx nuclear power after a trip - greater than 5% <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> RCS pressure greater than or equal to 2485 psig <p style="text-align: center;"><u>OR</u></p> <p>Containment pressure and temperature rapidly increasing</p>	GENERAL EMERGENCY
4. Probable large radioactivity release initiated by loss of AC power and all feedwater ALL MODES	<ul style="list-style-type: none"> Loss of all onsite and offsite AC power <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Turbine Driven Auxiliary Feedwater Pump not operable <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Restoration of either of the above not likely within 2 hours 	GENERAL EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB C)	PAGE
1	FUEL FAILURE OR FUEL HANDLING ACCIDENT	15 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
5. Probable large radioactivity release initiated by LOCA with loss of ECCS and containment cooling ALL MODES	<ul style="list-style-type: none"> Loss of reactor coolant in progress <p><u>AND</u></p> <ul style="list-style-type: none"> High or low head ECCS flow not being delivered to the core (if expected by plant conditions) <p><u>AND</u></p> <ul style="list-style-type: none"> Containment RS sump temperature greater than 190°F and NOT decreasing <p><u>OR</u></p> <p>All Quench Spray and Recirculation Spray systems - NOT OPERABLE</p>	GENERAL EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB C)	PAGE
1	FUEL FAILURE OR FUEL HANDLING ACCIDENT	16 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
6. Core damage with possible loss of coolable geometry MODES 1, 2, 3, & 4	<p>a) Fuel clad failure as indicated by any of the following:</p> <ul style="list-style-type: none"> RCS Specific activity greater than 60 $\mu\text{Ci}/\text{gram}$ dose equivalent I-131 <p><u>OR</u></p> <p>High Range Letdown radiation monitor</p> <div style="border: 1px solid black; padding: 5px; width: fit-content; margin: 10px auto;"> 1-CH-RI-128 or 2-CH-RI-228 GREATER THAN 1.2×10^4 mR/hr </div> <p><u>AND</u></p> <p>b) Loss of cooling as indicated by any of the following:</p> <ul style="list-style-type: none"> 5 confirmed core exit thermocouples greater than 1200 °F <p><u>OR</u></p> <p>Core delta T - zero</p> <p><u>OR</u></p> <p>Core delta T - rapidly diverging</p>	SITE AREA EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB C)	PAGE
1	FUEL FAILURE OR FUEL HANDLING ACCIDENT	17 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
7. Major fuel damage accident with radioactivity release to containment or fuel buildings ALL MODES	<ul style="list-style-type: none"> Water level in Rx vessel during refueling below the top of core <p style="text-align: center;"><u>OR</u></p> <p>Water level in spent fuel pool below top of spent fuel</p> <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Verified damage to irradiated fuel resulting in readings on Vent Vent "B" MGPI monitor <div style="border: 1px solid black; padding: 5px; margin: 10px auto; width: fit-content;"> RM-VG-180 GREATER THAN $2.69 \times 10^8 \mu\text{Ci/sec}$ </div>	SITE AREA EMERGENCY
8. Severe Fuel Clad Damage MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> High Range Letdown radiation monitor <div style="border: 1px solid black; padding: 5px; margin: 10px auto; width: fit-content;"> 1-CH-RI-128 or 2-CH-RI-228 Increases to GREATER THAN Hi Hi Alarm setpoint (representing 1% fuel failure) within 30 minutes and remains for at least 15 minutes </div> <p style="text-align: center;"><u>OR</u></p> <ul style="list-style-type: none"> RCS specific activity - greater than $300 \mu\text{Ci/gram}$ dose equivalent I-131 	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB C)	PAGE
1	FUEL FAILURE OR FUEL HANDLING ACCIDENT	18 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
9. Fuel damage accident with release of radioactivity to containment or fuel buildings ALL MODES	<ul style="list-style-type: none"> Verified accident involving damage to irradiated fuel <p><u>AND</u></p> <ul style="list-style-type: none"> Health Physics confirms fission product release from fuel <p><u>OR</u></p> <p>Vent Vent "B" MGPI monitor</p> <div style="border: 1px solid black; padding: 5px; width: fit-content; margin: 0 auto;"> RM-VG-180 GREATER THAN $1.99 \times 10^6 \mu\text{Ci/sec}$ </div>	ALERT
10. Potential for fuel damage to occur during refueling MODE 6	Continuing uncontrolled decrease of water level in Reactor Refueling Cavity or Spent Fuel Pool	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB C) FUEL FAILURE OR FUEL HANDLING ACCIDENT	39
ATTACHMENT		PAGE
1		19 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
11. Fuel clad damage indication MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Intentional reduction in power, load or temperature IAW reactor coolant activity T.S. Action Statement - HAS COMMENCED <p style="text-align: center;"><u>OR</u></p> <p>High Range Letdown radiation monitor</p> <div style="border: 1px solid black; padding: 5px; margin: 10px auto; width: fit-content;"> <p>1-CH-RI-128 or 2-CH-RI-228 Increases to GREATER THAN Hi Alarm setpoint (representing 0.1% fuel failure) within 30 minutes and remains for for at least 15 minutes</p> </div>	NOTIFICATION OF UNUSUAL EVENT
12. Independent Spent Fuel Storage Installation (ISFSI) event ALL MODES	<ul style="list-style-type: none"> Verified Sealed Surface Storage Cask (SSSC) seal leakage <p style="text-align: center;"><u>OR</u></p> <p>Sealed Surface Storage Cask (SSSC) dropped or mishandled</p>	NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB D) CONTAINMENT EVENT	39
ATTACHMENT		PAGE
1		20 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
1. Extremely high containment radiation, pressure and temperature MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Containment High Range radiation monitor <div style="border: 1px solid black; padding: 5px; margin: 10px 0;"> RM-RMS-165, -166 or RM-RMS-265, -266 GREATER THAN 3.76×10^2 R/hr </div> <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Containment pressure greater than 45 psia and not decreasing <p style="text-align: center;"><u>OR</u></p> Containment temperature greater than 280°F	GENERAL EMERGENCY
2. High-high containment radiation, pressure, and temperature MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Containment High Range radiation monitor <div style="border: 1px solid black; padding: 5px; margin: 10px 0;"> RM-RMS-165, -166 or RM-RMS-265, -266 GREATER THAN 1.88×10^2 R/hr </div> <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Containment pressure - greater than 27.75 psia and not decreasing <p style="text-align: center;"><u>OR</u></p> Containment temperature - greater than 200 °F	SITE AREA EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB D) CONTAINMENT EVENT	39
ATTACHMENT		PAGE
1		21 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
3. High Containment radiation, pressure and temperature MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Containment High Range radiation monitor <div style="border: 1px solid black; padding: 5px; margin: 10px 0;"> RM-RMS-165, -166 or RM-RMS-265, -266 GREATER THAN 81.5 R/hr </div> <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Containment pressure -- greater than 17 psia <p style="text-align: center;"><u>OR</u></p> Containment temperature - greater than 150°F	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB E)	PAGE
1	RADIOACTIVITY EVENT	22 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

1. Release imminent or in progress and site boundary doses projected to exceed 1.0 Rem TEDE or 5.0 Rem Thyroid CDE

- HP assessment indicates actual or projected doses at or beyond site boundary greater than 1.0 Rem TEDE or 5.0 Rem Thyroid CDE

GENERAL
EMERGENCY

ALL MODES

2. Release imminent or in progress and site boundary doses projected to exceed 0.1 Rem TEDE or 0.5 Rem Thyroid CDE

- HP assessment indicates actual or projected dose at or beyond Site Boundary exceeds 0.1 Rem TEDE or 0.5 Rem Thyroid CDE

SITE AREA
EMERGENCY

ALL MODES

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB E) RADIOACTIVITY EVENT	39
ATTACHMENT		PAGE
1		23 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
3. Effluent release greater than 10 times ODCM allowable limit ALL MODES	<p>a) Any of the following monitors indicate valid readings above the specified values for greater than 15 minutes</p> <ul style="list-style-type: none"> Clarifier Effluent <div>RM-LW-111 GREATER THAN 4.8 x 10⁵ cpm</div> Discharge Canal <div>RM-SW-130 or -230 GREATER THAN 5 x 10⁴ cpm</div> Vent Vent A MGPI <div>RM-VG-179 GREATER THAN 1.73 x 10⁶ µCi/sec</div> Vent Vent B MGPI <div>RM-VG-180 GREATER THAN 1.99 x 10⁶ µCi/sec</div> Process Vent MGPI <div>RM-GW-178 GREATER THAN 1.35 x 10⁷ µCi/sec</div> <p><u>OR</u></p> <p>b) HP assessment (sample results or dose projections) indicate greater than 10 times ODCM allowable limit</p>	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB E) RADIOACTIVITY EVENT	39
ATTACHMENT		PAGE
1		24 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
<p>4. High radiation or airborne contamination levels indicate a severe degradation in control of radioactive material</p> <p>ALL MODES</p>	<p>Valid readings on any of the following monitors have increased by a factor of 1000 and remain for at least 15 minutes:</p> <ul style="list-style-type: none"> Ventilation Vent Multi-sample gaseous or particulate monitor RM-VG-106 or -105 Control Room Area RMS-157 Aux. Bldg. Control Area RMS-154 Decon. Bldg. Area RMS-151 Fuel Pool Bridge Area RMS-153 New fuel storage Area RMS-152 Laboratory Area RMS-158 Sample Room Area RMS-156 	<p>ALERT</p>

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB E) RADIOACTIVITY EVENT	39
ATTACHMENT		PAGE
1		25 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
5. Effluent release greater than ODCM allowable limit ALL MODES	<p>a) Any of the following monitors indicate valid readings above the specified value for more than 1 hour:</p> <ul style="list-style-type: none"> Clarifier Effluent <div>RM-LW-111 GREATER THAN 4.8×10^4 cpm</div> Discharge Canal <div>RM-SW-130 or -230 GREATER THAN 5×10^3 cpm</div> Vent Vent A MGPI <div>RM-VG-179 GREATER THAN 1.73×10^5 μCi/sec</div> Vent Vent B MGPI <div>RM-VG-180 GREATER THAN 1.99×10^5 μCi/sec</div> Process Vent MGPI <div>RM-GW-178 GREATER THAN 1.35×10^6 μCi/sec</div> <p style="text-align: center;"><u>OR</u></p> <p>b) HP assessment (sample results or dose projections) indicates greater than ODCM allowable limit</p>	NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB G) LOSS OF SECONDARY COOLANT	39
ATTACHMENT		PAGE
1		26 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
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1. Major secondary line break with significant primary to secondary leakage and fuel damage indicated

MODES 1, 2, 3, & 4

Conditions a) and b) exist with c):
a) Uncontrolled loss of secondary coolant - IN PROGRESS

AND

b) RCS specific activity exceeds limits of T.S. Figure 3.4.16-1

OR

High Range Letdown radiation monitor

1-CH-RI-128 or 2-CH-RI-228
GREATER THAN Hi Alarm setpoint

AND

c) Vent Vent A MGPI Monitor

RM-VG-179 GREATER THAN
6.21 x 10⁷ µCi/sec

OR

Affected pathway Steam Generator Blowdown monitor

RM-SS-122, -123, -124,
-222, -223, -224
GREATER THAN 1 x 10⁶ cpm

OR

Affected pathway Main Steam Line High Range monitor

RM-MS-170, -171, -172,
-270, -271, -272
GREATER THAN 12.2 mR/hr

SITE AREA
EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB G) LOSS OF SECONDARY COOLANT	39
ATTACHMENT		PAGE
1		27 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
2. Major secondary line break with significant primary to secondary leakage MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Uncontrolled loss of secondary coolant - IN PROGRESS <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Vent Vent A MGPI Monitor <div style="border: 1px solid black; padding: 5px; margin: 5px;"> RM-VG-179 GREATER THAN $1.76 \times 10^6 \mu\text{Ci/sec}$ </div> <p style="text-align: center;"><u>OR</u></p> <p>Steam Generator Blowdown monitor on affected pathway</p> <div style="border: 1px solid black; padding: 5px; margin: 5px;"> RM-SS-122, -123, -124 RM-SS-222, -223, -224 GREATER THAN 1×10^5 cpm </div> <p style="text-align: center;"><u>OR</u></p> <p>Main Steam Line High Range monitor on affected pathway</p> <div style="border: 1px solid black; padding: 5px; margin: 5px;"> RM-MS-170, -171, -172 RM-MS-270, -271, -272 GREATER THAN 0.14 mR/hr </div>	ALERT
3. Major secondary line break MODES 1, 2, 3, & 4	Uncontrolled loss of secondary coolant - IN PROGRESS	NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB H) ELECTRICAL FAILURE	39
ATTACHMENT		PAGE
1		28 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
<p>1. Loss of offsite and onsite AC power for more than 15 minutes</p> <p>ALL MODES</p>	<p>The following conditions exist for greater than 15 minutes:</p> <ul style="list-style-type: none"> • Ammeters for 4160V Reserve Station Service Buses D, E, & F all indicate - zero (0) amps <p><u>AND</u></p> <ul style="list-style-type: none"> • Ammeters for 4160V Station Service Buses A, B, & C all indicate - zero (0) amps <p><u>AND</u></p> <ul style="list-style-type: none"> • Ammeters for 4160V Emergency Buses H & J both indicate - zero (0) amps 	<p>SITE AREA EMERGENCY</p>
<p>2. Loss of all onsite DC power for greater than 15 minutes</p> <p>ALL MODES</p>	<p>The following conditions exist for greater than 15 minutes:</p> <ul style="list-style-type: none"> • All station battery voltmeters indicate zero (0) volts <p><u>AND</u></p> <ul style="list-style-type: none"> • No light indication available to Reserve Station Service breakers 15D1, 15E1 and 15F1 	<p>SITE AREA EMERGENCY</p>

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB H) ELECTRICAL FAILURE	39
ATTACHMENT		PAGE
1		29 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

CAUTION: EAL A.1 is duplicated below for cross-reference/comparison to EAL H.3:

A.1. Loss of function
needed for unit
HSD condition

MODES 1, 2, 3 & 4

- Total loss of the
Charging/SI System

SITE AREA
EMERGENCY

OR

Total loss of the Main
Feedwater and Auxiliary
Feedwater Systems

3. Loss of all offsite
and onsite AC power

ALL MODES

- Ammeters for 4160V Reserve
Station Service Buses D, E,
& F all indicate - zero (0)
amps

ALERT

AND

- Ammeters for 4160V Station
Service Buses A, B, & C all
indicate - zero (0) amps

AND

- Ammeters for 4160V
Emergency Buses H and J
both indicate - zero (0)
amps

4. Loss of all onsite
DC power

ALL MODES

- All station battery
voltmeters indicate - zero
(0) volts

ALERT

AND

- No light indication
available to Reserve
Station Service Breakers
15D1, 15E1 and 15F1

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB H)	PAGE
1	ELECTRICAL FAILURE	30 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
5. Loss of offsite power or onsite AC power capability	<ul style="list-style-type: none"> Unit main generator and both emergency diesel generators out of service 	NOTIFICATION OF UNUSUAL EVENT
ALL MODES	<p><u>OR</u></p> <p>Loss of all 34.5 KV reserve station service buses</p>	

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB I) FIRE	39
ATTACHMENT		PAGE
1		31 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
1. Fire resulting in degradation of safety systems MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Fire which causes major degradation of a safety system function required for protection of the public <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Affected systems are caused to be <u>NOT</u> operable as defined by Tech. Specs. 	SITE AREA EMERGENCY
2. Fire potentially affecting station safety systems MODES 1, 2, 3, & 4	Fire which has potential for causing a safety system not to be operable as defined by Tech. Specs.	ALERT
3. Fire lasting greater than 10 minutes in Protected Area or Service Water Pump/Valve House ALL MODES	Fire within the Protected Area or Service Water Pump/Valve House which is not under control within 10 minutes after Fire Brigade - DISPATCHED	NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB J)	PAGE
1	SECURITY EVENT	32 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
1. Loss of physical Station control ALL MODES	<ul style="list-style-type: none"> Shift Manager/Station Emergency Manager has been informed that the security force has been neutralized by attack, resulting in loss of physical control of station <p style="text-align: center;"><u>OR</u></p> <p>Shift Manager/Station Emergency Manager has been informed of intrusion into one or more Vital Areas which are occupied or controlled by an aggressor</p>	GENERAL EMERGENCY
2. Imminent loss of physical Station control ALL MODES	Security Shift Supervisor has notified the Operations Shift Manager/Station Emergency Manager of imminent intrusion into a Vital Area	SITE AREA EMERGENCY
3. Ongoing Security compromise ALL MODES	Security Shift Supervisor has notified the Operations Shift Manager/Station Emergency Manager of a confirmed unneutralized intrusion into the Protected Area or ISFSI	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB J)	PAGE
1	SECURITY EVENT	33 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
4. Security threat, unauthorized attempted entry, or attempted sabotage ALL MODES	Any of the following when determined to have potential for degrading the level of safety of the plant or ISFSI <ul style="list-style-type: none"> • Receipt of a credible site-specific threat from Security, NRC or FBI • Confirmed hostage situation • Civil disturbance • Discovery of a bomb device (other-than on or near a safety- related system which represents an on-going security compromise) • Confirmed attempted intrusion (Protected Area or ISFSI) • Attempted sabotage 	NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB K) HAZARD TO STATION OPERATION	39
ATTACHMENT		PAGE
1		34 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
1. Aircraft damage to vital plant systems MODES 1, 2, 3, & 4	Aircraft crash which affects vital structures by impact or fire	SITE AREA EMERGENCY
2. Severe explosive damage MODES 1, 2, 3, & 4	Explosion which results in severe degradation of any of the following systems required for safe shutdown: <ul style="list-style-type: none"> CVCS System <u>OR</u> ECCS System <u>OR</u> Main/Auxiliary Feedwater System 	SITE AREA EMERGENCY
3. Entry of toxic or flammable gases into plant vital areas other than the Control Room MODES 1, 2, 3, & 4	<ul style="list-style-type: none"> Uncontrolled release of toxic or flammable agents greater than life threatening or explosive limits in Vital Areas <u>AND</u> Evacuation of Vital Area other than Control Room - REQUIRED <u>OR</u> Significant degradation of plant safety systems resulting in loss of a safety system function required for protection of the public 	SITE AREA EMERGENCY

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB K) HAZARD TO STATION OPERATION	39
ATTACHMENT		PAGE
1		35 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
4. Severe missile damage to safety systems MODES 1, 2, 3, & 4	Missile impact causing severe degradation of safety systems required for unit shutdown	SITE AREA EMERGENCY
5. Aircraft crash on the facility ALL MODES	Aircraft crash within the Protected Area or Switchyard	ALERT
6. Explosion damage to facility ALL MODES	Unplanned explosion resulting in damage to plant structure or equipment that affects plant operations	ALERT
7. Entry of toxic or flammable gases or liquids into plant facility ALL MODES	Notification of uncontrolled release of toxic or flammable agent which causes: <ul style="list-style-type: none"> Evacuation of personnel from plant areas <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Safety related equipment is rendered inoperable 	ALERT
8. Turbine failure or missile impact MODES 1 & 2	Failure of turbine/generator rotating equipment resulting in casing penetration	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB K) HAZARD TO STATION OPERATION	39
ATTACHMENT		PAGE
1		36 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
9. Missile damage to safety related equipment or structures MODES 1, 2, 3, & 4	Notification of missile impact causing damage to safety related equipment or structures	ALERT
10. Aircraft crash or unusual aircraft activity ALL MODES	<ul style="list-style-type: none"> Confirmed notification of aircraft crash within the site boundary <p style="text-align: center;"><u>OR</u></p> <p>Unusual aircraft activity in the vicinity of the site as determined by the Operations Shift Manager/ Station Emergency Manager or the Security Shift Supervisor</p>	NOTIFICATION OF UNUSUAL EVENT
11. Train derailment within Protected Area ALL MODES	Confirmed report of train derailment within Protected Area	NOTIFICATION OF UNUSUAL EVENT
12. Explosion within Protected Area ALL MODES	Confirmed report of unplanned explosion within Protected Area	NOTIFICATION OF UNUSUAL EVENT
13. Onsite or nearsite release of toxic or flammable liquids or gases ALL MODES	Notification of unplanned release of toxic or flammable agents which may affect safety of station personnel or equipment	NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB K)	PAGE
1	HAZARD TO STATION OPERATION	37 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

14. Turbine rotating
component failure
with no casing
penetration

Failure of turbine/generator
rotating equipment resulting in
immediate unit shutdown

NOTIFICATION
OF UNUSUAL
EVENT

MODES 1 & 2

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB L)	PAGE
1	NATURAL EVENTS	38 of 42

CONDITION/APPLICABILITY

INDICATION

CLASSIFICATION

1. Earthquake greater than or equal to DBE levels

MODES 1, 2, 3, & 4

- Confirmed earthquake which activates the Event Indicator on the Strong Motion Accelerograph

AND

- Alarms on the Peak Shock Annunciator indicate a horizontal motion of greater than or equal to 0.12 g or a vertical motion of greater than or equal to 0.08g

SITE AREA
EMERGENCY

2. Sustained winds in excess of design levels experienced or projected

MODES 1, 2, 3, & 4

Sustained winds 150 mph
OR GREATER experienced
or projected

SITE AREA
EMERGENCY

3. NOT USED

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB L) NATURAL EVENTS	39
ATTACHMENT		PAGE
1		39 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
4. Earthquake greater than or equal to OBE levels ALL MODES	<ul style="list-style-type: none"> Confirmed earthquake which activates Event Indicator on the Strong Motion Accelerograph <p style="text-align: center;"><u>AND</u></p> <ul style="list-style-type: none"> Alarms on the Peak Shock Annunciator indicate a horizontal motion of greater than or equal to 0.06 g or a vertical motion of greater than or equal to 0.04g 	ALERT
5. Tornado striking facility ALL MODES	Tornado visually detected striking structures within the Protected Area or Switchyard	ALERT
6. Hurricane winds near design basis level experienced or projected ALL MODES	Hurricane winds 120 mph OR GREATER experienced or projected	ALERT
7. Flood near design levels ALL MODES	Flood in the Lake Anna Reservoir with indicated level - greater than 263 feet MSL	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE (TAB L) NATURAL EVENTS	39
ATTACHMENT		PAGE
1		40 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
8. Earthquake detected ALL MODES	Confirmed earthquake which activates the Event Indicator on the Strong Motion Accelerograph	NOTIFICATION OF UNUSUAL EVENT
9. Tornado within Protected Area or Switchyard ALL MODES	Tornado visually detected within Protected Area or Switchyard	NOTIFICATION OF UNUSUAL EVENT
10. Hurricane force winds projected onsite within 12 hours ALL MODES	<ul style="list-style-type: none"> Confirmation by Weather Center that hurricane force winds (greater than 73 mph) projected onsite within 12 hours 	NOTIFICATION OF UNUSUAL EVENT
11. 50 year flood ALL MODES	Flood in the Lake Anna Reservoir with indicated level - greater than 254 feet MSL	NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB M)	PAGE
1	MISCELLANEOUS ABNORMAL EVENTS	41 of 42

<u>CONDITION/APPLICABILITY</u>	<u>INDICATION</u>	<u>CLASSIFICATION</u>
1. Any major internal or external events which singly or in combination cause massive damage to station facilities or may warrant evacuation of the public ALL MODES	Shift Manager/Station Emergency Manager judgement	GENERAL EMERGENCY
2. Station conditions which may warrant notification of the public near the site ALL MODES	Shift Manager/Station Emergency Manager judgement	SITE AREA EMERGENCY
3. Station conditions which have the potential to degrade or are actually degrading the level of safety of the station ALL MODES	Shift Manager/Station Emergency Manager judgement	ALERT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	EMERGENCY ACTION LEVEL TABLE	39
ATTACHMENT	(TAB M)	PAGE
1	MISCELLANEOUS ABNORMAL EVENTS	42 of 42

CONDITION/APPLICABILITY

4. Station conditions which warrant increased awareness of state and/or local authorities

ALL MODES

INDICATION

Shift Manager/Station Emergency Manager judgement that any of the following exist:

- Unit shutdown is other than a controlled shutdown

OR

Unit is in an uncontrolled condition during operation

OR

A condition exists which has the potential for escalation and therefore warrants notification

CLASSIFICATION

NOTIFICATION OF UNUSUAL EVENT

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	TURNOVER CHECKLIST	39
ATTACHMENT		PAGE
2		1 of 1

Conduct a turnover between the onshift and relief SEM in accordance with the following checklist. Use placekeeping aid at left of item, "____", to track completion.

- ____ 1. Determine the status of primary responder notification.
- ____ 2. Determine the status of "Report of Emergency to State and Local Governments," EPIP-2.01, Attachment 2. Get completed copies if available.
- ____ 3. Determine status of the "Report of Radiological Conditions to the State," EPIP-2.01, Attachment 3. Get completed copy if available.
- ____ 4. Determine status of Emergency Notification System (ENS) communications and completion status of NRC Event Notification Worksheet (EPIP-2.02 Attachment 1).
- ____ 5. Review classification and initial PAR status.
- ____ 6. Review present plant conditions and status. Get copy of Critical Safety Functions form.
- ____ 7. Review status of station firewatches and re-establish if conditions allow.
- ____ 8. Determine readiness of TSC for activation.
- ____ 9. After all information is obtained, transfer location to TSC. (Consider direct transfer of State & local notifications to LEOF/CEOF.)
- ____ 10. Call the Control Room and assess any changes that may have occurred during transition to the TSC.
- ____ 11. When sufficient personnel are available, the relief SEM is to assume the following responsibilities from the onshift Station Emergency Manager:
 - a. Reclassification.
 - b. Protective Action Recommendations until LEOF activated.
 - c. Notifications (i.e., state, local, & NRC). Upon LEOF activation, transfer notification responsibilities except for the NRC ENS.
 - d. Site evacuation authorization.
 - e. Emergency exposure authorization.
 - f. Command/control of onsite response.
- ____ 12. Formally relieve the Interim SEM and assume control in the TSC. Announce name and facility activation status to facility.

NUMBER	ATTACHMENT TITLE	REVISION
EPIP-1.01	CONSIDERATIONS FOR OPERATIONS RESPONSE UNDER ABNORMAL CONDITIONS	39
ATTACHMENT		PAGE
3		1 of 1

This attachment provides procedural guidance for controlling selected emergency response actions when their implementation would have adverse results.

Station Emergency Manager (SEM) approval is required before any required action is postponed, suspended or modified. The guidance below is not all-inclusive.

UNANTICIPATED HAZARD EXISTS (e.g., security event, tornado or toxic release):

IF implementation of emergency response actions could compromise Security Plan response strategies, THEN consider postponing or suspending emergency response actions until threat has been resolved, e.g., on-site announcement directing assembly and emergency response facility activation, pager activation and call-out per EPIP-3.05, AUGMENTATION OF EMERGENCY RESPONSE ORGANIZATION, dispatch of Security Team members to the LEOF per EPIP-3.04, ACTIVATION OF LOCAL EMERGENCY OPERATIONS FACILITY, and staging of road blocks per EPIP-5.04, ACCESS CONTROL.

IF assembling on-site personnel for accountability or activation of emergency response facilities could endanger plant personnel, THEN consider postponing emergency assembly until hazardous conditions are resolved. (Consider having Corporate Security notify corporate emergency response organization only using CPIP-3.4, INNSBROOK SECURITY SUPPORT, and notifying personnel in unaffected areas on-site selectively.)

IF notifying augmentation could create a safety hazard for personnel coming to the station, THEN consider postponing augmentation notification. (Consider having Corporate Security notify corporate emergency response organization only using CPIP-3.4, INNSBROOK SECURITY SUPPORT, or deferring notifications until hazardous conditions are resolved.)

ANTICIPATED SITUATION (e.g., forecasted severe weather or grid disturbance):

IF all or part of the ERO has been staged in anticipation of a predicted event, THEN notify Security to omit performance of augmentation notification (as described in EPIP-3.05, AUGMENTATION OF EMERGENCY RESPONSE ORGANIZATION).

IF adequate controls have been established to continually account for personnel staged in anticipation of a predicted event, THEN notify Security to omit performance of initial accountability (as described in EPIP-5.03, PERSONNEL ACCOUNTABILITY).

IF a decision has been made to staff the Central EOF in lieu of the LEOF, THEN notify Security that performance of EPIP-3.04, ACTIVATION OF LOCAL EMERGENCY OPERATIONS FACILITY, is not required.

IF environmental conditions are hazardous, THEN consult with Security Team Leader about suspending procedural requirements for staging road blocks (IAW EPIP-5.04, ACCESS CONTROL).