LICENSE AUTHORITY FILE
CORRECTIONS TO LICENSE AND TECHNICAL SPECIFICATION PAGES
BASED ON NRC STAFF REVIEW OF THE LICENSE AUTHORITY FILE
FOR R.E. GINNA NUCLEAR POWER PLANT

DOCKET NO. 50-244

Replace the following pages of Appendix A Technical Specifications with the attached revised pages.

Remove	Insert	Amendment/Correction Letter
N/A	3.1.3-1	Amendment 80
N/A	3.1.3-2	Amendment 80
3.3.6-1	3.3.6-1	Amendment 83
3.3.6-2	3.3.6-2	Amendment 83
3.3.6-3	3.3.6-3	Amendment 83

Attachment 1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 26, 2001

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

SUBJECT: R. E. GINNA NUCLEAR POWER PLANT - AMENDMENT RE: IMPROVED TECHNICAL SPECIFICATION FORMATING CHANGE AND REVISION TO 10 CFR 50.59 (TAC NO. MB1184)

Dear Dr. Mecredy:

The Commission has issued the enclosed Amendment No. 80 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated February 14, 2001.

The amendment makes minor revisions in the Ginna Station Improved Technical Specifications (ITS) format to allow for maintaining, viewing, and publishing them with a different software package. The amendment also includes a revision to ITS Section 5.5.13, "Technical Specifications (TS) Bases Control Program," to provide consistency with the changes to 10 CFR 50.59 as published in the Federal Register (64 FR 53852) dated October 4, 1999.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Guy S. Vissing, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 80 to License No. DPR-18
2. Safety Evaluation

cc w/encls: See next page

3.1 REACTIVITY CONTROL SYSTEMS

3.1.3 Moderator Temperature Coefficient (MTC)

LCO 3.1.3 The MTC shall be maintained within the limits specified in the COLR. The maximum upper limit shall be less than or equal to 5 pcm/°F for power levels below 70% RTP and less than or equal to 0 pcm/°F at or above 70% RTP.

APPLICABILITY: MODE 1 and MODE 2 with $k_{eff} \geq 1.0$ for the upper MTC limit, MODES 1, 2, and 3 for the lower MTC limit.

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	MTC not within upper limit.	A.1	Establish administrative withdrawal limits for control banks to maintain MTC within limit.	24 hours
B.	Required Action and associated Completion Time of Condition A not met.	B.1	Be in MODE 2 with k_{eff} < 1.0.	6 hours
C.	- NOTE - Required Action C.1 must be completed whenever Condition C is entered. Projected end of cycle life (EOL) MTC not within lower limit.		- NOTE - LCO 3.0.4 is not applicable. C.1 Re-evaluate core design and safety analysis, and determine that the reactor core is acceptable for continued operation.	Once prior to reaching the equivalent of an equilibrium RTP all rods out (ARO) boron concentration of 300 ppm
D.	Required Action and associated Completion Time of Condition C not met.	D.1	Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.1.3.1	Verify MTC is within upper limit.	Once prior to entering MODE 1 after each refueling
SR 3.1.3.2	Confirm that MTC will be within limits at 70% RTP.	Once prior to entering MODE 1 after each refueling
SR 3.1.3.3	Confirm that MTC will be within limits at EOL.	Once prior to entering MODE 1 after each refueling.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 29, 2003

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

SUBJECT: R. E. GINNA NUCLEAR POWER PLANT - AMENDMENT RE: CONTROL ROOM EMERGENCY AIR TREATMENT SYSTEM ACTUATION INSTRUMENTATION CHANGE (TAC NO. MB1887)

Dear Dr. Mecredy:

The Commission has issued the enclosed Amendment No. 83 to Facility Operating License No. DPR-18 for the R. E. Ginna Nuclear Power Plant. This amendment is in response to your application dated May 3, 2001, as supplemented August 7, 2001, October 29, 2001, May 3, 2002, October 7, 2002, November 5, 2002 and June 6, 2003.

The amendment revises the Improved Technical Specifications to reflect design changes to the actuation circuitry associated with the Control Room Emergency Air Treatment System. The proposed design changes consist of replacing the current radiation monitors with two Geiger-Mueller tubes powered from two independent safety-related power supplies which are then configured into two redundant actuation logic trains. The actuation logic trains utilize safety-grade digital instrumentation which meet Class 1E safety system requirements.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Robert Clark, Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures: 1. Amendment No. 83 to License No. DPR-18
2. Safety Evaluation

cc w/encls: See next page

3.3 INSTRUMENTATION

3.3.6 Control Room Emergency Air Treatment System (CREATS) Actuation Instrumentation

LCO 3.3.6 The CREATS actuation instrumentation for each Function in Table 3.3.6-1 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4,
During movement of irradiated fuel assemblies,
During CORE ALTERATIONS.

ACTIONS

- NOTE -
Separate Condition entry is allowed for each Function.

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more Functions with one channel or train inoperable.	A.1 - NOTE - The control room may be unisolated for ≤ 1 hour every 24 hours while in this condition. Place CREATS in Mode F.	7 days
B. One or more Functions with two channels or two trains inoperable.	B.1 - NOTE - The control room may be unisolated for ≤ 1 hour every 24 hours while in this condition. Place CREATS in Mode F.	Immediately
C. Required Action and associated Completion Time of Condition A or B not met in MODE 1, 2, 3, or 4.	C.1 Be in MODE 3.	6 hours
	AND	
	C.2 Be in MODE 5.	36 hours

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition A or B not met during movement of irradiated fuel assemblies or during CORE ALTERATIONS.	D.1 Suspend CORE ALTERATIONS. AND D.2 Suspend movement of irradiated fuel assemblies.	Immediately Immediately

SURVEILLANCE REQUIREMENTS

- NOTE -
Refer to Table 3.3.6-1 to determine which SRs apply for each CREATS Actuation Function.

SURVEILLANCE		FREQUENCY
SR 3.3.6.1	Perform CHANNEL CHECK.	12 hours
SR 3.3.6.2	Perform COT.	92 days
SR 3.3.6.3	- NOTE - Verification of setpoint is not required. Perform TADOT.	 24 months
SR 3.3.6.4	Perform CHANNEL CALIBRATION.	24 months
SR 3.3.6.5	Perform ACTUATION LOGIC TEST.	24 months

Table 3.3.6-1
CREATS Actuation Instrumentation

	FUNCTION	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1.	Manual Initiation	2 trains	SR 3.3.6.3	NA
2.	Automatic Actuation Logic and Actuation Relays	2 trains	SR 3.3.6.5	NA
3.	Control Room Radiation Intake Monitors	2	SR 3.3.6.1 SR 3.3.6.2 SR 3.3.6.4	≤ .5 mR/hr

NRR FILE
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 26, 2001

Dr. Robert C. Mecredy
Vice President, Nuclear Operations
Rochester Gas and Electric Corporation
89 East Avenue
Rochester, NY 14649

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Guy S. Vissing, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-244

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APPLICABILITY: MODE 1 and MODE 2 with $k_{eff} \geq 1.0$ for the upper MTC limit, MODES 1, 2, and 3 for the lower MTC limit.

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	MTC not within upper limit.	A.1	Establish administrative withdrawal limits for control banks to maintain MTC within limit.	24 hours
B.	Required Action and associated Completion Time of Condition A not met.	B.1	Be in MODE 2 with k_{eff} < 1.0.	6 hours
C.	- NOTE - Required Action C.1 must be completed whenever Condition C is entered. Projected end of cycle life (EOL) MTC not within lower limit.	C.1	- NOTE - LCO 3.0.4 is not applicable. Re-evaluate core design and safety analysis, and determine that the reactor core is acceptable for continued operation.	Once prior to reaching the equivalent of an equilibrium RTP all rods out (ARO) boron concentration of 300 ppm
D.	Required Action and associated Completion Time of Condition C not met.	D.1	Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.1.3.1	Verify MTC is within upper limit.	Once prior to entering MODE 1 after each refueling
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