January 14, 2004

Mr. John L. Skolds, President
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2 - CORRECTION TO ISSUANCE OF AMENDMENTS (TAC NOS. MB9888 AND MB9889)

Dear Mr. Skolds:

On January 9, 2004, the Nuclear Regulatory Commission (Commission) issued Amendment No. 164 to Facility Operating License No. NPF-11 and Amendment No. 150 to Facility Operating License No. NPF-18 for the LaSalle County Station, Units 1 and 2, (LaSalle) respectively. The amendments consisted of changes to the Technical Specifications (TS) in response to your application dated July 1, 2003, as supplemented by letter dated December 10, 2003.

Based on discussions between your staff and the Commission staff on January 12, 2004, we are reissuing a corrected copy of page 5 of the Safety Evaluation dated January 9, 2004. The lower end of the Inlet Sub-Cooling range and the lower end of the R-Factor were incorrectly listed as 0.1 Btu/lbm and 1.03, respectively. These values should be 6 and 1.02, consistent with your supplemental letter dated December 10, 2003. The third paragraph is also corrected to reference the use of additional data provided in the December 10 letter to support these values rather than the use of extrapolated data.

We apologize for any inconvenience this may have caused.

Sincerely,

/RA/

William A. Macon, Jr., Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos.: 50-373 and 50-374

Enclosure: As stated

cc w/encl: See next page

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(d) A final set of additive constants are determined by adjusting the preliminary additive constants, subject to minimizing the difference between the R-Factors.

The range of application for the GEXL97 correlation is the same as the range of the hypothetical database from which the correlation is derived. The applicable ranges are presented below:

i) Pressure: 800 to 1300 psia
ii) Mass Flux: 0.1×10^6 to 1.5×10^6 lb/hr-ft^2
iii) Inlet Sub-cooling: 6 to 100 Btu/lbm
iv) R-Factor: 1.02 -- 1.20

This application range covers the complete range of expected operation of the ATRIUM-10 fuel during normal steady state and transient conditions in the LSCS BWR reload cores.

The lower end of the Inlet Sub-cooling range differs slightly from that presented in the licensee's original submittal dated July 1, 2003. In that document, the lower end of the range was extrapolated from 10 down to zero for the Inlet Sub-cooling and the R-Factor range was also extrapolated from 1.03 down to 1.02. Following discussions with the NRC staff (documented in Ref. 2), the licensee and GNF acquired additional subcooling and R-Factor data to justify the desired ranges within the limits of the development database of the GEXL97 correlation.

With respect to the ranges stated above, GNF provided the results of analysis of the reference loading pattern for the LaSalle Unit 1, Cycle 11 mixed core reload, which has core operating characteristics that are representative of the conditions that will be encountered during this cycle. The CPR was extracted for all the ATRIUM-10 fuel throughout the entire cycle.

In summary, the NRC staff reviewed the analyses and results presented in NEDC-33106P, "GEXL97 Correlation for ATRIUM-10 Fuel," and determined that the analyses and results are in accordance with the guidance and limitations of 10 CFR 50.34, and the applicable sections of NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants." In addition, the staff concludes that the analysis presented in NEDC-33106P is acceptable because:

(a) The total uncertainty in the correlation's critical power predictions appropriately takes into account the fact that the uncertainty in the new correlation's fit to the hypothetical database and the uncertainty in the hypothetical database with respect to the underlying experimental data are appropriately treated;

(b) Generating the hypothetical database using the approved SPCB correlation encoded in the Framatome subchannel code XCOBRA-T is a reasonable engineering approach to dealing with mixed reload core fuel, where the experimental database and critical power correlation for the previous vendor's fuel is not available to the new fuel vendor;

(c) GNF intends to utilize the new GEXL97 correlation within the limits of the hypothetical data base, without extrapolation outside the approved limits of the data base, as specified in this safety evaluation (SE); and