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December 30, 2003
JAFP-03-0179

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
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Washington, DC 20555-0001

SUBJECT: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333

Proposed Alternatives in Accordance with 10CFR50.55a(g)(6)(ii)(A)(5) and Relief from ASME Section XI Code Regarding Inspection of RPV Vertical Shell Welds pursuant to 10 CFR 50.55a (g)(6)(i)

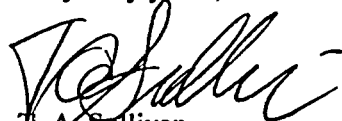
REFERENCE: 1. Entergy Letter (JAFP-03-0111) to NRC, "Proposed Alternatives in Accordance with 10CFR50.55a(g) (6)(ii)(A)(5) and Relief From ASME Section XI Code Regarding Inspection of RPV Vertical Shell Welds pursuant to 10CFR50.55a(g)(6)(i), August 4, 2003

Dear Sir:

Entergy proposed Relief Request #30 to the James A. FitzPatrick (JAF) Inservice Inspection Program (Reference 1). The NRC staff in a recent telephone conversation on December 10, 2003 requested that Entergy provide additional details from previous inspection results documented in Reference 1. This information is provided in Attachment 1 to this letter.

If you have any questions, please contact Mr. Andrew Halliday at 315-349-6055.

Very truly yours,


T. A. Sullivan
Site Vice President

Attachment

A047

cc: Regional Administrator
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Attachment 1 to JAFP-03-0179

Proposed Alternatives in Accordance with 10CFR50.55a(g)(6)(ii)(A)(5) and Relief from ASME Section XI Code Regarding Inspection of RPV Vertical Shell Welds pursuant to 10 CFR 50.55a (g)(6)(i)

From the standpoint of protecting the FitzPatrick RPV against pressurized thermal shock (PTS), the FitzPatrick RPV is a weld-limited vessel. The lower shell axial welds contain the material most susceptible to radiation-induced embrittlement in the RPV (i.e., the limiting material in the vessel). These welds, designated 2-233 A,B,C on vessel drawings and VV-4A, VV-4B and VV-4C on ISI documents, contain heat 27204/12008 (Reference 2) and have a RT_{ndt} @ End of Life (EOL) = 127.9°F (Reference 3 and 4).

Welds VV-4A and VV-4B were examined in RO15 (Phase I) from vessel OD side with 73% of total weld length coverage, including 91% in the beltline region (Reference 1). A review of the ultrasonic examination data acquired on these welds revealed no recordable indications.

Lower shell weld VV-4C will be examined in RO16 (Phase II) from vessel ID side, also with 73% of total projected weld length coverage, and 100% in the beltline region (Reference 1).

Lower intermediate shell axial welds 1-233 A,B,C (heat 13253/12008, (Reference 2), and RT_{ndt} @ EOL = 119.9°F) (Reference 3 and 4) corresponding to welds VV-3A, VV-3B and VV-3C will be examined in RO16 (Phase II) from vessel ID side, with 41%, 86% and 41% of total projected weld length coverage, respectively. These welds are less susceptible to radiation-induced embrittlement than are the limiting welds in the lower shell region of the vessel.

References:

1. Entergy Letter (JAFP-03-0111) to NRC, "Proposed Alternatives in Accordance with 10CFR50.55a(g)(6)(ii)(A)(5) and Relief From ASME Section XI Code Regarding Inspection of RPV Vertical Shell Welds pursuant to 10CFR50.55a(g)(6)(i), dated August 4, 2003
2. GE Report DRF B11-00732-02, "Resolution of Comments for Report GE-NE-B1100732-01, Rev. 1, Plant FitzPatrick RPV Surveillance Materials Testing and Analysis of 120 degree F Capsule at 13.4 EFPY."
3. NYPA Letter (JPN-99-035) to NRC, "Comments on the Reactor Vessel Integrity Database", dated October 15, 1999.
4. Interface Control Document No. JAF-ICD-RPV-03439, "Reactor Vessel Integrity Database (RVID) Comments and Corrections in Response to NRC Request dated July 6, 1999", dated October 8, 1999.