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December 22, 2003  
E910-03-048

U.S. Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, DC 20555

Gentlemen,

Subject: Saxton Nuclear Experimental Corporation (SNEC)  
Operating License No., DPR-4  
Docket No. 50-146  
Containment Vessel (CV) Radiation Survey

In our December 5, 2002 letter (E910-02-058) GPU Nuclear provided responses to seven (7) requests for additional information (RAI) questions. These questions and responses were related to the SNEC Facility Technical Specifications Change Request (TSCR No. 62) to allow removal of the CV upper dome following Final Status Survey of the below grade portions of the CV. From that letter the GPU Nuclear committed to taking the following action to RAI question 7.

#### Question 7

Please provide survey information on radiation levels on the operating floor level of the CV after backfilling and isolation activities are complete.

#### Response:

GPU Nuclear agrees to provide survey information on radiation levels on the "operating floor" level of the CV after backfilling and isolation activities are complete. Note that the term "operating floor" has traditionally referred to the original 818'/812' elevation concrete floor of the CV. Since all the concrete has been removed from the CV this floor is no longer in place. Following back fill and isolation activities, a cover will be placed on top of the back fill to prevent cross contamination during removal of the upper CV dome. This cover, and any exposed portion of the CV liner, which will remain following license termination, will be included in the Final Status Survey (FSS).

The purpose of this letter is to provide you the information on radiation levels on the operating floor of the CV since backfilling and isolation activities have been completed. A nominal six-inch concrete cap has been poured over the backfill materials at approximately the 803.9' elevation. This elevation represents the new operating floor level. Radiation readings were taken on this elevation.

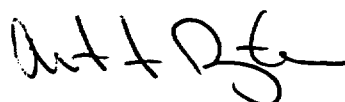
Twenty nine (29) general area radiation readings were taken at waist level. Survey information is provided in Attachment 1. The radiation survey results are consistent with natural background and are listed as follows:

Average radiation dose rate: 0.0035 +/- 0.0016 mr/hr  
Range of readings: 0.002 - 0.005 mr/hr

A020

If you have any questions on this information, please contact Mr. Art Paynter at (814) 635-4384.

Sincerely,

 Art Paynter SNECRSD for

G. A. Kuehn  
Program Director, SNEC

cc: NRC Project Manager, NRR  
NRC Project Scientist, Region 1

**Attachments:**

Attachment 1 – CV Radiation Survey Results

**CV Radiation Survey Results**

**Location:** New CV Ops Floor @803.9' elevation - post concrete cap installation  
**Survey Date:** 12/18/2003  
**Technician:** Bill Horton  
**Instrument:** Bicron microrem meter  
**Serial #:** C233J  
**Cal. Due Date:** 3/11/2004

Item	Reading (mr/hr)
1	0.004
2	0.004
3	0.003
4	0.004
5	0.003
6	0.003
7	0.002
8	0.004
9	0.004
10	0.003
11	0.003
12	0.003
13	0.004
14	0.003
15	0.002
16	0.005
17	0.003
18	0.003
19	0.003
20	0.003
21	0.004
22	0.004
23	0.005
24	0.003
25	0.004
26	0.005
27	0.004
28	0.003
29	0.003
<b>Mean</b>	<b>0.0035</b>
<b>Uncertainty (2<math>\sigma</math>)</b>	<b>0.0016</b>

**Comments:**

1. Concrete cap poured on 12/17/03. Radiation readings required per NRC RAI Q7 for TSCR#62.
2. Survey readings taken at waist level.

**ORIGINAL**

<b>SNEC FSS RADIOLOGICAL SURVEY</b>		Survey Unit #	CV1-1	Survey #	200-03-1658
Location	Containment Vessel General Area at elevation 808 ft				
Grid #	N/A		Area Classification	N/A	SR # N/A
Reason For Survey	General Area Survey after Concrete Cap Installation		Date of Survey	12/18/03	Time of Survey 1315
Technician	W. Horton <i>W<sup>m</sup> D. Horton</i>		Technician	N/A	
GRCS Review	R. Shepherd <i>R. Shepherd</i>		Date Of Review	12-18-03	
<b>Radiological Instrument Data</b>		<b>Radiological Instrument Data</b>		<b>Radiological Instrument Data</b>	
Inst./Probe Type	micro Rem	Inst./Probe Type	N/A	Inst./Probe Type	N/A
Serial Number(s)	C233J	Serial Number(s)		Serial Number(s)	
Cal. Due Date(s)	3/11/04	Cal. Due Date(s)		Cal. Due Date(s)	
Efficiency (%)	N/A	Efficiency (%)	B.Y.	Efficiency (%)	B.Y.
ABCR (cpm)	N/A	ABCR (cpm)		ABCR (cpm)	
BRA Average	N/A	BRA Average		BRA Average	
BRA Location	N/A	BRA Location	↓	BRA Location	↓
Source Checks	Sat. <input checked="" type="checkbox"/> Unsat. <input type="checkbox"/>	Source Checks	Sat. <input type="checkbox"/> Unsat. <input type="checkbox"/>	Source Checks	Sat. <input type="checkbox"/> Unsat. <input type="checkbox"/>
<b>Comments</b>					
All readings are general area at approximately waist level.					
<b>Survey Map</b>					
Please refer to the attached map.					
<b>NOTES:</b> <input type="checkbox"/> = Location where general area dose rates were taken					

ORIGINAL

# Containment vessel radiation survey

Survey # 200-03-1658

Technician: W. Horton

Instrument: C233J microR

Date: 12/18/03

