

DEC 12 2003

LRN-03-0418
LCR H02-01



U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

**REQUEST FOR CHANGE TO TECHNICAL SPECIFICATIONS
CONTROL ROOM EMERGENCY FILTRATION SYSTEM
HOPE CREEK GENERATING STATION
FACILITY OPERATING LICENSE NPF-57
DOCKET NO. 50-354**

Reference: Letter LR-N03-0003, Request For Change To Technical Specifications Relaxation of Secondary Containment Operability Requirements and Elimination of FRVS Recirculation Charcoal Filters, dated January 18, 2003.

Pursuant to 10 CFR 50.90, PSEG Nuclear LLC (PSEG) hereby requests a revision to the Technical Specifications (TS) for Hope Creek Generating Station. In accordance with 10CFR50.91(b)(1), a copy of this submittal has been sent to the State of New Jersey.

Amendment 146 issued April 15, 2003 in response to the referenced letter, contained revisions to TS 3.7.2, CONTROL ROOM EMERGENCY FILTRATION SYSTEM for incorporation of Alternative Source Term (AST) for the fuel handling accident and consistency with TSTF-51, Rev. 2. However, it did not address all of the required changes. Changes to TS Table 3.3.7.1-1, RADIATION MONITORING INSTRUMENTATION and Table 4.3.7.1-1, RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS are also required. The proposed amendment would revise TS 3.7.2, Table 3.3.7.1-1, and Table 4.3.7.1-1 to reflect both changes required by TSTF-51, Rev. 2, and changes to be consistent with NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants.

PSEG has evaluated the proposed changes in accordance with 10CFR50.91(a)(1), using the criteria in 10CFR50.92(c) and has determined that this request involves no significant hazards considerations. An evaluation of the requested changes is provided in Attachment 1 to this letter. In addition, there is no significant increase in the amounts or types of any effluents that may be released offsite, and there is no significant increase in individual or cumulative occupational radiation exposure. Consequently, the

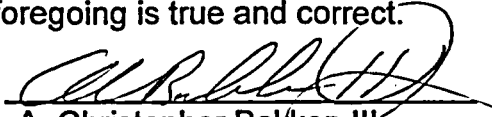
A001

proposed amendment satisfies the criteria of 10CFR51.22(c)(9) for categorical exclusion from the requirement for an environmental assessment. The marked up Technical Specification pages affected by the proposed change are provided in Attachment 2. Re-typed Technical Specification pages affected by the proposed change are provided in Attachment 3.

If you have any questions or require additional information, please contact Mr. Michael Mosier at (856) 339-5434.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on 12/12/03


A. Christopher Bakken III
Sr. Vice President – Site Operations

Attachments (3)

DEC 12 2003

**C: Mr. H. Miller, Administrator – Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406**

**Mr. J. Boska, Project Manager – Hope Creek
U. S. Nuclear Regulatory Commission
Mail Stop 08B1
Washington, DC 20555-0001**

USNRC Senior Resident Inspector – Hope Creek (X24)

**Mr. K. Tosch, Manager IV
Bureau of Nuclear Engineering
PO Box 415
Trenton, New Jersey 08625**

**HOPE CREEK GENERATING STATION
FACILITY OPERATING LICENSE NPF-57
DOCKET NO. 50-354**

**REQUEST FOR CHANGE TO TECHNICAL SPECIFICATIONS
CONTROL ROOM EMERGENCY FILTRATION SYSTEM**

| | |
|---|---|
| 1. DESCRIPTION..... | 2 |
| 2. PROPOSED CHANGE..... | 2 |
| 3. BACKGROUND | 4 |
| 4. TECHNICAL ANALYSIS | 5 |
| 4.1 TSTF-51, Rev. 2 | 5 |
| 4.2 Revision 2 to NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants..... | 5 |
| 5. REGULATORY SAFETY ANALYSIS | 6 |
| 5.1 No Significant Hazards Consideration Determination..... | 6 |
| 5.2 Applicable Regulatory Requirements/Criteria..... | 9 |
| 6. ENVIRONMENTAL IMPACT EVALUATION | 9 |
| 7. REFERENCES..... | 9 |

REQUEST FOR CHANGE TO TECHNICAL SPECIFICATIONS CONTROL ROOM EMERGENCY FILTRATION SYSTEM

1. DESCRIPTION

This letter is a request to amend Facility Operating License NPF-57 for the Hope Creek Generating Station (HCGS). In our letter dated January 18, 2003 PSEG requested changes to the Technical Specifications (TS) to provide flexibility in scheduling outage tasks and to modify unnecessarily restrictive containment closure and ventilation system requirements. The elimination of the selected TS Engineered Safety Feature (ESF) requirements during core alterations and during the movement of sufficiently decayed irradiated fuel is consistent with TSTF-51, Rev. 2. Two additional changes, omitted from the original submittal, are included in Section 2 of this LCR as items a and b.

Changes to be consistent with NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants are proposed as items a through g of Section 2 of this LCR. Changes involving the deletion of Operational Condition 5 (Refueling) from the Control Room Ventilation Radiation Monitor are included as items c and d. Changes involving the deletion of Operational Condition 4 (Cold Shutdown) from the Control Room Emergency Filtration (CREF) System are included as items e and f. Also, the addition of operations with the potential for draining the reactor vessel (OPDRV) is included in items a, b and g.

2. PROPOSED CHANGE

The proposed changes to the TS are included in Attachment 2 of this submittal. In summary, it is requested that:

a. TABLE 3.3.7.1-1, RADIATION MONITORING INSTRUMENTATION (PAGE 3/4 3-64), under TABLE NOTATION currently states:

***When irradiated fuel is being handled in the secondary containment.**

is revised to read:

***When recently irradiated fuel is being handled in the secondary containment and during operations with the potential for draining the reactor vessel.**

b. TABLE 4.3.7.1-1, RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS (PAGE 3/4 3-67), under TABLE NOTATION currently states:

*When irradiated fuel is being handled in the secondary containment.

is revised to read:

***When recently irradiated fuel is being handled in the secondary containment and during operations with the potential for draining the reactor vessel.**

c. TABLE 3.3.7.1-1, RADIATION MONITORING INSTRUMENTATION (PAGE 3 /4 3-63), currently states:

| <u>INSTRUMENTATION</u> | <u>APPLICABLE CONDITIONS</u> |
|---|------------------------------|
| 1. Control Room Ventilation Radiation Monitor | 1,2,3,5 and* |

is revised to read:

| <u>INSTRUMENTATION</u> | <u>APPLICABLE CONDITIONS</u> |
|---|------------------------------|
| 1. Control Room Ventilation Radiation Monitor | 1,2,3 and* |

d. TABLE 4.3.7.1-1, RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS (PAGE 3/4 3-66): CURRENTLY STATES:

| <u>INSTRUMENTATION</u> | <u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u> |
|---|---|
| 1. Control Room Ventilation Radiation Monitor | 1, 2, 3, 5 and * |

Is revised to read:

INSTRUMENTATION

OPERATIONAL
CONDITIONS FOR
WHICH SURVEILLANCE
REQUIRED

1. Control Room Ventilation
Radiation Monitor

1, 2, 3, and *

- e. **Limiting Condition for Operation 3.7.2 (PAGE 3/4 7-6) under APPLICABILITY currently states:**

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, 4, and *.

is revised to read:

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, and *.

- f. **Limiting Condition for Operation 3.7.2 (PAGE 3/4 7-6) under ACTION b. currently states:**

IN OPERATIONAL CONDITION 4 or *

is revised to read:

IN OPERATIONAL CONDITION *

- g. **Limiting Condition for Operation 3.7.2 (PAGE 3/4 7-6) under footnote * currently states:**

*When recently irradiated fuel is being handled in the secondary containment.

is revised to read:

*When recently irradiated fuel is being handled in the secondary containment and during operations with a potential for draining the reactor vessel.

3. BACKGROUND

By letter dated January 18, 2003 PSEG requested changes to the Technical Specifications (TS) to provide flexibility in scheduling outage tasks and to modify unnecessarily restrictive containment closure and ventilation system requirements. The elimination of the selected TS Engineered Safety Feature (ESF) requirements during core alterations and the movement of sufficiently decayed irradiated fuel is consistent with TSTF-51, Rev. 2. This change allows the flexibility

to move personnel and equipment and perform work that would affect containment OPERABILITY during the handling of irradiated fuel. However, two additional changes, omitted from the original submittal, are included in Section 2 of this LCR as items a and b.

Changes to be consistent with NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants are proposed as items a through g of Section 2 of this LCR. Changes involving the deletion of Operational Condition 5 (Refueling) from the Control Room Ventilation Radiation Monitor are included as items c and d. Changes involving the deletion of Operational Condition 4 (Cold Shutdown) from the CREF System are included as items e and f. Also, the addition of operations with the potential for draining the reactor vessel (OPDRV) is included in items a, b and g.

4. TECHNICAL ANALYSIS

4.1 TSTF-51, Rev. 2

The changes associated with items a and b of Section 2 are being made for consistency with TSTF-51, Rev. 2.

Following reactor shutdown, decay of the short-lived fission products greatly reduces the fission product inventory present in irradiated fuel. Amendment 146 was based on Alternative Source Term (AST) methodology for the fuel handling accident. The analysis also assumed a decay period of 24 hours to take advantage of the reduced radionuclide inventory available for release in the event of a fuel handling accident. Following this period, the primary success path for mitigating the fuel handling accident no longer includes the functioning of the active containment systems. Therefore, the OPERABILITY requirements of the TS are modified to reflect that water level and decay time are the primary success paths for mitigating a fuel handling accident.

4.2. NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants.

Changes to be consistent with NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants are proposed as items a through g of Section 2 of this LCR. Changes involving the deletion of Operational Condition 5 (Refueling) from the Control Room Ventilation Radiation Monitor are included as items c and d. Changes involving the deletion of Operational Condition 4 (Cold Shutdown) from the CREF System are included as items e and f. Also, the addition of operations with the potential for draining the reactor vessel (OPDRV) is included in items a, b and g.

Section 9.4.1.1.3 of the UFSAR states that the CREF system is designed to maintain control room habitability by providing filtration of fresh air and recirculated air during any accident that may release high radioactivity. In Operational Conditions 1, 2, and 3, a loss of coolant accident (LOCA) could lead to a fission product release to primary containment that leaks to secondary containment. In Operational Conditions 4 and 5, the probability and consequences of a LOCA are reduced because of the pressure and temperature limitations. Therefore, CREF system operability is not required except for other situations under which significant releases of radioactive material can be postulated, such as operations with the potential for draining the reactor vessel or movement of recently irradiated fuel assemblies in secondary containment.

The Control Room Ventilation Radiation Monitor is required to be OPERABLE in MODES 1, 2, and 3 and during OPDRVs and movement of recently irradiated fuel assemblies in secondary containment, to ensure that control room personnel are protected during a LOCA, FHA, or vessel draindown event. During MODES 4 and 5, when these specified conditions are not in progress, the probability of a LOCA is low; thus, the monitor is not required.

5. REGULATORY SAFETY ANALYSIS

5.1 No Significant Hazards Consideration Determination

PSEG Nuclear LLC (PSEG) has evaluated whether or not a significant hazards consideration is involved with the proposed amendment by focusing on the three standards set forth in 10CFR50.92, "Issuance of Amendment," as discussed below:

1. Does the change involve a significant increase in the probability or consequences of an accident previously analyzed?

Response: No.

The proposed changes to Table 3.3.7.1-1, Radiation Monitoring Instrumentation, and Table 4.3.7.1-1, Radiation Monitoring Instrumentation Surveillance Requirements, adds "recently" to modify irradiated fuel in the "*" footnote to provide consistency with TSTF-51, Rev. 2. Proposed changes to eliminate Operational Condition 5 from Tables 3.3.7.1-1 and 4.3.7.1-1, Control Room Ventilation Radiation Monitor, Operational Condition 4 from Control Room Emergency Filtration (CREF) System and adding operations with the potential for draining the reactor vessel (OPDRV) to Tables 3.3.7.1-1 and 4.3.7.1-1 footnote "*" and the CREF System are consistent with NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants.

The proposed changes associated with the fuel handling accident (FHA) do not involve a change to structures, components, or systems that would affect the probability of an accident previously evaluated in the Hope Creek Updated Final Safety Analysis Report (UFSAR). The FHA for the Hope Creek Generating Station (HCGS) is defined as a drop of a fuel assembly over irradiated assemblies in the reactor core 24 hours after reactor shutdown. Alternative Source Term (AST) is used to evaluate the dose consequences of a postulated accident. The FHA has been analyzed without credit for Secondary Containment, Filtration Recirculation and Ventilation System (FRVS), and CREF system. The resultant radiological consequences are within the acceptance criteria set forth in 10CFR50.67 and Regulatory Guide (RG) 1.183. This amendment does not alter the methodology or equipment used in fuel handling operations. The equipment hatch, personnel air locks, other containment penetrations, or any component thereof is not an accident initiator. Actual fuel handling operations are not affected by the proposed changes.

Consequently the probability of a previously analyzed FHA is not affected by the proposed amendment. No other accident initiator is affected by the proposed changes.

Therefore, this proposed amendment does not involve a significant increase in the probability of occurrence or radiological consequences of an accident previously analyzed.

2. Does the change create the possibility of a new or different kind of accident from any accident previously analyzed?

Response: No

The proposed changes to Table 3.3.7.1-1, Radiation Monitoring Instrumentation, and Table 4.3.7.1-1, Radiation Monitoring Instrumentation Surveillance Requirements, adds "recently" to modify irradiated fuel in the "*" footnote provides consistency with TSTF-51, Rev. 2. Proposed changes to eliminate Operational Condition 5 from Tables 3.3.7.1-1 and 4.3.7.1-1, Control Room Ventilation Radiation Monitor, Operational Condition 4 from CREF System and adding OPDRV to Table 3.3.7.1-1 and 4.3.7.1-1 footnote "*" and the CREF System are consistent with NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants.

The proposed amendment will not create the possibility for a new or different type of accident from any accident previously evaluated because changes to the allowable activity in the primary and secondary systems do not result in changes to the design or operation of these systems. The evaluation of the effects of the proposed changes indicates that all design standard and applicable safety

criteria limits are met. Equipment important to safety will continue to operate as designed. Component integrity is not challenged. The changes do not result in any event previously deemed incredible being made credible. The changes do not result in more adverse conditions or result in any increase in the challenges to safety systems. The systems affected by the changes are used to mitigate the consequences of an accident that has already occurred. The proposed TS changes do not significantly affect the mitigative function of these systems.

Therefore, the proposed changes would not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the change involve a significant reduction in the margin of safety?

Response: No

The proposed changes to Table 3.3.7.1-1, Radiation Monitoring Instrumentation, and Table 4.3.7.1-1, Radiation Monitoring Instrumentation Surveillance Requirements, adds "recently" to modify irradiated fuel in the "*" footnote provides consistency with TSTF-51, Rev. 2. Proposed changes to eliminate Operational Condition 5 from Tables 3.3.7.1-1 and 4.3.7.1-1, Control Room Ventilation Radiation Monitor, Operational Condition 4 from CREF System and adding OPDRV to Table 3.3.7.1-1 and 4.3.7.1-1 footnote "*" and the CREF System are consistent with NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants.

The proposed changes revise the TS to establish operational conditions where specific activities represent situations during which significant radioactive releases can be postulated. These operational conditions are consistent with the design basis analysis and are established such that the radiological consequences remain at or below the regulatory guidelines. Safety margins and analytical conservatisms are retained to ensure that the analysis adequately bounds all postulated event scenarios. The proposed TS continue to ensure that the total effective dose equivalent (TEDE) for the control room (CR), the exclusion area boundary (EAB), and low population zone (LPZ) boundaries are below the corresponding acceptance criteria specified in 10CFR50.67 and RG 1.183.

Therefore, these changes do not involve a significant reduction in margin of safety.

Based on the above, PSEG concludes that the proposed amendment presents no significant hazards consideration under the standards set forth in 10CFR50.92(c), and, accordingly, a finding of "no significant hazards consideration" is justified.

5.2 Applicable Regulatory Requirements/Criteria

The proposed amendment would revise TS 3.7.2, Table 3.3.7.1-1, and Table 4.3.7.1-1 to reflect both changes required by TSTF-51, Rev. 2, and changes associated with OPERATIONAL CONDITIONS to be consistent with NUREG-1433 Vol. 1, Rev. 2, Standard Technical Specifications, General Electric Plants. The FHA dose analysis was performed using AST and TEDE dose criteria in accordance with the guidance provided in RG 1.183.

In conclusion, based on the considerations discussed above, (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

6. ENVIRONMENTAL IMPACT EVALUATION

PSEG has determined that the proposed amendment would change a requirement with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR 20, or would change an inspection or surveillance requirement. However, the proposed amendment does not involve (i) a significant hazards consideration, (ii) a significant change in the types or significant increase in the amounts of any effluent that may be released offsite, or (iii) a significant increase in individual or cumulative occupational radiation exposure. Accordingly, the proposed amendment meets the eligibility criterion for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the proposed amendment.

7. REFERENCES

- 7.1. Letter LR-N02-0002, Request For Change To Technical Specifications Relaxation Of Secondary Containment Operability Requirements And Elimination Of FRVS Recirculation Charcoal Filters, dated June 28, 2002.
- 7.2. USNRC Regulatory Guide 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors", July 2000.
- 7.3. 10CFR50.67, "Accident Source Term".
- 7.4. TSTF-51, Rev. 2.

**7.5. Hope Creek Amendment 146, April 15, 2003, Relaxation Of Secondary
Containment Operability Requirements And Elimination Of FRVS Recirculation
Charcoal Filters.**

HOPE CREEK GENERATING STATION
FACILITY OPERATING LICENSE NPF-57
DOCKET NO. 50-354
REVISIONS TO THE TECHNICAL SPECIFICATIONS (TS)

TECHNICAL SPECIFICATION PAGE WITH PROPOSED CHANGE

The following are marked-up Technical Specifications for Facility Operating License NPF-57 affected by this change request:

| <u>Technical Specification</u> | <u>Page</u> |
|--------------------------------|-------------|
| Table 3.3.7.1-1 | 3/4 3-63 |
| Table 3.3.7.1-1 | 3/4 3-64 |
| Table 4.3.7.1-1 | 3/4 3-66 |
| Table 4.3.7.1-1 | 3/4 3-67 |
| 3/4.7.2 | 3/4 7-6 |

TABLE 3.3.7.1-1

RADIATION MONITORING INSTRUMENTATION

| <u>INSTRUMENTATION</u> | <u>MINIMUM CHANNELS OPERABLE</u> | <u>APPLICABLE CONDITIONS</u> | <u>ALARM/TRIP SETPOINT</u> | <u>ACTION</u> |
|---|--------------------------------------|----------------------------------|---|---------------|
| 1. Control Room Ventilation Radiation Monitor | 2/intake | 1, 2, 3, 4 and * | $\leq 2 \times 10^{-5} \mu\text{C/cc}^{**}$ | 71 |
| 2. Area Monitors | | | | |
| a. Criticality Monitors | | | | |
| 1) New Fuel Storage Vault | 1 | # | $\geq 5 \text{ mR/hr}$ and $\leq 20 \text{ mR/hr}^{(a)}$ | 72 |
| 2) Spent Fuel Storage Pool | 1 | ## | $\geq 5 \text{ mR/hr}$ and $\leq 20 \text{ mR/hr}^{(a)}$ | 72 |
| b. Control Room Direct Radiation Monitor | 1 | At all times | $2.5 \text{ mR/hr}^{(a)}$ | 72 |
| 3. Reactor Auxiliaries Cooling Radiation Monitor | 1 | At all times | $9 \times 10^{-5} \mu\text{C/cc}^{(a)}$ | 73 |
| 4. Safety Auxiliaries Cooling Radiation Monitor | 1/loop | At all times | $6 \times 10^{-5} \mu\text{C/cc}^{(a)}$ | 73 |
| 5. Offgas Pre-treatment Radiation Monitor | 1 | *** | (b) | 74 |

TABLE 3.3.7.1-1 (Continued)

RADIATION MONITORING INSTRUMENTATION

TABLE NOTATION

- recently*
- *When ^{recently}irradiated fuel is being handled in the secondary containment, and during operations with the potential for draining the reactor vessel.
 - **Activates control room emergency filtration system.
 - ***When the offgas treatment system is operating.
 - #With fuel in the new fuel storage vault.
 - ##With fuel in the spent fuel storage pool.
- (a) Alarm only.
- (b) Alarm setpoint to be set in accordance with Specification 3.11.2.7.

TABLE 4.3.7.1-1

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

| <u>INSTRUMENTATION</u> | <u>CHANNEL CHECK</u> | <u>CHANNEL FUNCTIONAL TEST</u> | <u>CHANNEL CALIBRATION</u> | <u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u> |
|--|----------------------|--------------------------------|----------------------------|---|
| 1. Control Room Ventilation Radiation Monitor | S | Q | R | 1, 2, 3, 4 and * |
| 2. Area Monitors | | | | |
| a. Criticality Monitors | | | | |
| 1) New Fuel Storage Vault | S | Q | R | # |
| 2) Spent Fuel Storage Pool | S | Q | R | ## |
| b. Control Room Direct Radiation Monitor | S | Q | R | At all times |
| 3. Reactor Auxiliaries Cooling Radiation Monitor | S | Q | R | At all times |
| 4. Safety Auxiliaries Cooling Radiation Monitor | S | Q | R | At all times |
| 5. Offgas Pre-treatment Radiation Monitor | S | Q | R | ** |



TABLE 4.3.7.1-1 (Continued)

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TABLE NOTATION

- recently*
- #With fuel in the new fuel storage vault.
 - ##With fuel in the spent fuel storage pool.
 - *When irradiated fuel is being handled in the secondary containment
 - **When the offgas treatment system is operating.

and during operations with the potential for draining the reactor vessel.

PLANT SYSTEMS

3/4.7.2 CONTROL ROOM EMERGENCY FILTRATION SYSTEM

LIMITING CONDITION FOR OPERATION

=====

3.7.2 Two independent control room emergency filtration system subsystems shall be OPERABLE with each subsystem consisting of:

- a) One control room supply unit,
- b) One filter train, and
- c) One control room return air fan.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, ~~4~~ and *.

ACTION:

- a. In OPERATIONAL CONDITION 1, 2 or 3 with one control room emergency filtration subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION ~~4~~ *:
 - 1. With one control room emergency filtration subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or initiate and maintain operation of the OPERABLE subsystem in the pressurization/recirculation mode of operation.
 - 2. With both control room emergency filtration subsystems inoperable, suspend handling of recently irradiated fuel in the secondary containment and operations with a potential for draining the reactor vessel.
- c. The provisions of Specification 3.0.3 are not applicable in Operational Condition *.

SURVEILLANCE REQUIREMENTS

=====

4.7.2 Each control room emergency filtration subsystem shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 85°F*.
- b. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, the control area chilled water pump, flow

*When recently irradiated fuel is being handled in the secondary containment, *and during operations with the potential for draining the reactor vessel.*

*This does not require starting the non-running control emergency filtration subsystem.

HOPE CREEK GENERATING STATION
FACILITY OPERATING LICENSE NPF-57
DOCKET NO. 50-354
REVISIONS TO THE TECHNICAL SPECIFICATIONS (TS)

TECHNICAL SPECIFICATION PAGE WITH PROPOSED CHANGE

The following are retyped Technical Specifications for Facility Operating License NPF-57 affected by this change request:

| <u>Technical Specification</u> | <u>Page</u> |
|--------------------------------|-------------|
| Table 3.3.7.1-1 | 3/4 3-63 |
| Table 3.3.7.1-1 | 3/4 3-64 |
| Table 4.3.7.1-1 | 3/4 3-66 |
| Table 4.3.7.1-1 | 3/4 3-67 |
| 3/4.7.2 | 3/4 7-6 |

TABLE 3.3.7.1-1

RADIATION MONITORING INSTRUMENTATION

| <u>INSTRUMENTATION</u> | <u>MINIMUM CHANNELS OPERABLE</u> | <u>APPLICABLE CONDITIONS</u> | <u>ALARM/TRIP SETPOINT</u> | <u>ACTION</u> |
|---|--------------------------------------|----------------------------------|---|---------------|
| 1. Control Room Ventilation Radiation Monitor | 2/intake | 1,2,3 and * | $\leq 2 \times 10^{-5} \mu\text{C}/\text{cc}^{**}$ | 71 |
| 2. Area Monitors | | | | |
| a. Criticality Monitors | | | | |
| 1) New Fuel Storage Vault | 1 | # | $\geq 5 \text{ mR/hr}$ and $\leq 20 \text{ mR/hr}^{(a)}$ | 72 |
| 2) Spent Fuel Storage Pool | 1 | ## | $\geq 5 \text{ mR/hr}$ and $\leq 20 \text{ mR/hr}^{(a)}$ | 72 |
| b. Control Room Direct Radiation Monitor | 1 | At all times | $2.5 \text{ mR/hr}^{(a)}$ | 72 |
| 3. Reactor Auxiliaries Cooling Radiation Monitor | 1 | At all times | $9 \times 10^{-5} \mu\text{C}/\text{cc}^{(a)}$ | 73 |
| 4. Safety Auxiliaries Cooling Radiation Monitor | 1/loop | At all times | $6 \times 10^{-5} \mu\text{C}/\text{cc}^{(a)}$ | 73 |
| 5. Offgas Pre-treatment Radiation Monitor | 1 | *** | (b) | 74 |

TABLE 3.3.7.1-1 (Continued)

RADIATION MONITORING INSTRUMENTATION

TABLE NOTATION

- *When recently irradiated fuel is being handled in the secondary containment and during operations with the potential for draining the reactor vessel.
- **Activates control room emergency filtration system.
- ***When the offgas treatment system is operating.
 - #With fuel in the new fuel storage vault.
 - ##With fuel in the spent fuel storage pool.
 - (a)Alarm only.
 - (b)Alarm setpoint to be set in accordance with Specification 3.11.2.7.

TABLE 4.3.7.1-1

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

| <u>INSTRUMENTATION</u> | <u>CHANNEL CHECK</u> | <u>CHANNEL FUNCTIONAL TEST</u> | <u>CHANNEL CALIBRATION</u> | <u>OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED</u> |
|--|----------------------|--------------------------------|----------------------------|---|
| 1. Control Room Ventilation Radiation Monitor | S | Q | R | 1, 2, 3, and * |
| 2. Area Monitors | | | | |
| a. Criticality Monitors | | | | |
| 1) New Fuel Storage Vault | S | Q | R | # |
| 2) Spent Fuel Storage Pool | S | Q | R | ## |
| b. Control Room Direct Radiation Monitor | S | Q | R | At all times |
| 3. Reactor Auxiliaries Cooling Radiation Monitor | S | Q | R | At all times |
| 4. Safety Auxiliaries Cooling Radiation Monitor | S | Q | R | At all times |
| 5. Offgas Pre-treatment Radiation Monitor | S | Q | R | ** |

TABLE 4.3.7.1-1 (Continued)

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

TABLE NOTATION

- #With fuel in the new fuel storage vault.
- ##With fuel in the spent fuel storage pool.
- *When recently irradiated fuel is being handled in the secondary containment and during operations with the potential for draining the reactor vessel.
- **When the offgas treatment system is operating.

PLANT SYSTEMS

3/4.7.2 CONTROL ROOM EMERGENCY FILTRATION SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.2 Two independent control room emergency filtration system subsystems shall be OPERABLE with each subsystem consisting of:

- a) One control room supply unit,
- b) One filter train, and
- c) One control room return air fan.

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, and *.

ACTION:

- a. In OPERATIONAL CONDITION 1, 2 or 3, with one control room emergency filtration subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION *:
 - 1. With one control room emergency filtration subsystem inoperable, restore the inoperable subsystem to OPERABLE status within 30 days or initiate and maintain operation of the OPERABLE subsystem in the pressurization/recirculation mode of operation.
 - 2. With both control room emergency filtration subsystems inoperable, suspend handling of recently irradiated fuel in the secondary containment and operations with a potential for draining the reactor vessel.
- c. The provisions of Specification 3.0.3 are not applicable in Operational Condition *.

SURVEILLANCE REQUIREMENTS

4.7.2 Each control room emergency filtration subsystem shall be demonstrated OPERABLE:

- a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 85°F[#].
- b. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, the control area chilled water pump, flow

[#]When recently irradiated fuel is being handled in the secondary containment and during operations with a potential for draining the reactor vessel.

[#]This does not require starting the non-running control emergency filtration subsystem.