

Status of RAMA Fluence Methodology

Bob Carter

EPRI

Contains BWRVIP Proprietary Information

EPRI

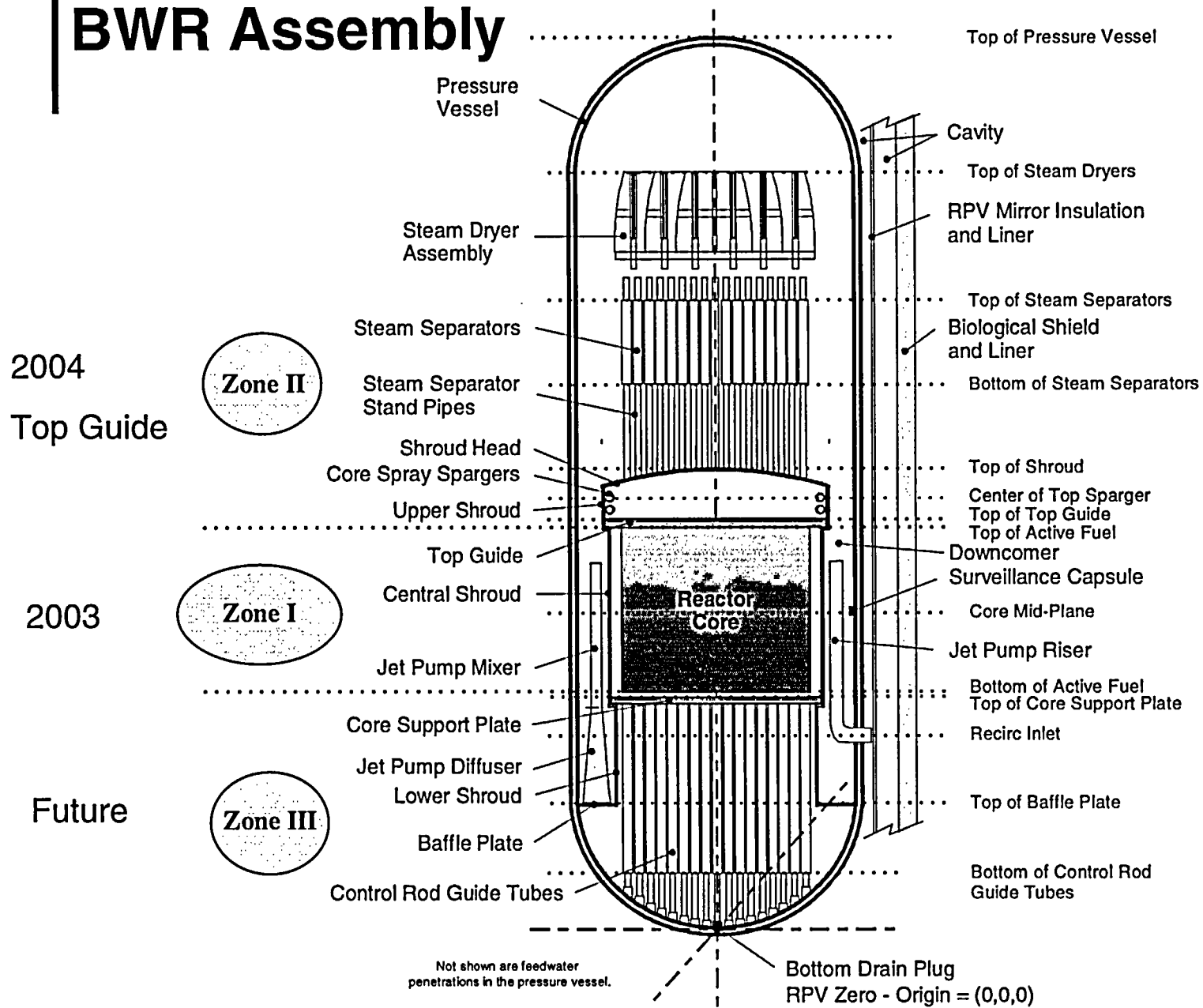
Overview of RAMA

- Calculates neutron and gamma flux distributions
- Uses 3-D modeling
 - actual representation of component shapes
 - capable of modeling entire reactor system
- Integral transport solution in 3-D
- Detailed (p7) anisotropic scattering treatment

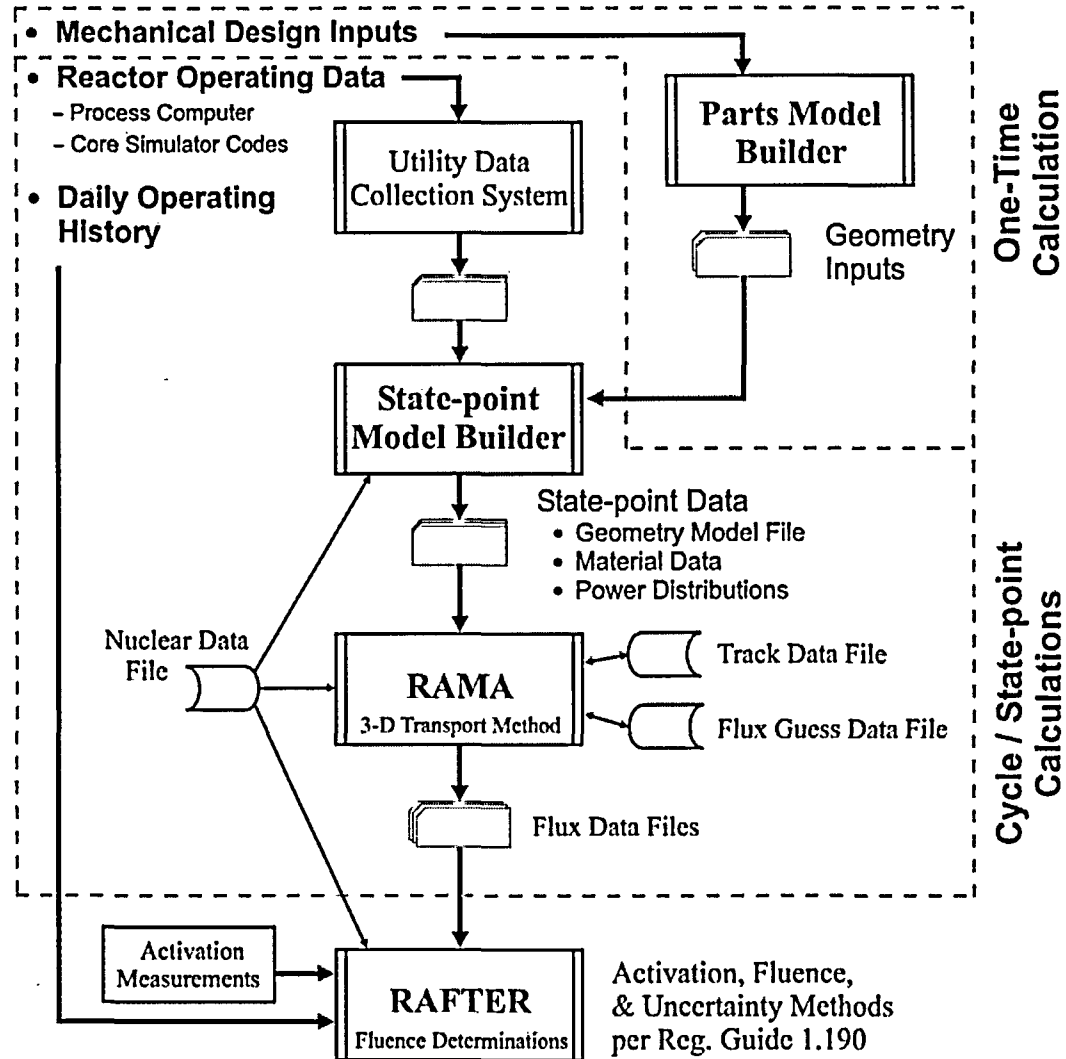
Overview of RAMA – Modeling

- Accurate 3-D Modeling
 - Straight and Curved Surfaces
 - Symmetry Modeling Capability
- Automated Geometry Modeling Tools Available
 - Parts Model Builder (PMB)
 - Generates geometry models of standard components
 - User specified dimensions and meshing
 - Each component built separately and combined later
 - State-point Model Builder
 - Uses the reactor operating data (i.e., state point data) to process inputs for the geometry model built by the PMB code

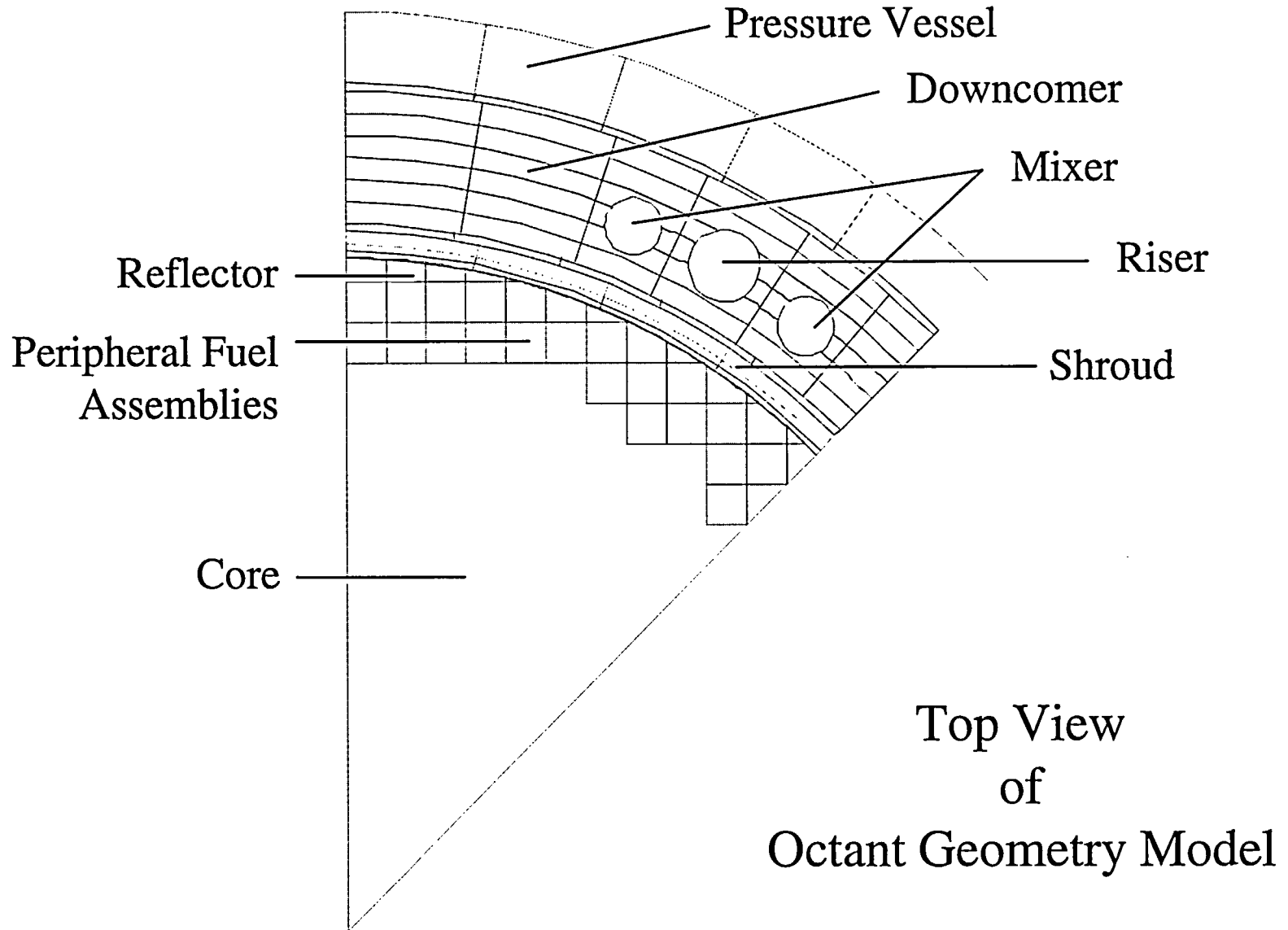
BWR Assembly



RAMA Fluence Methodology Calculation Flow Diagram

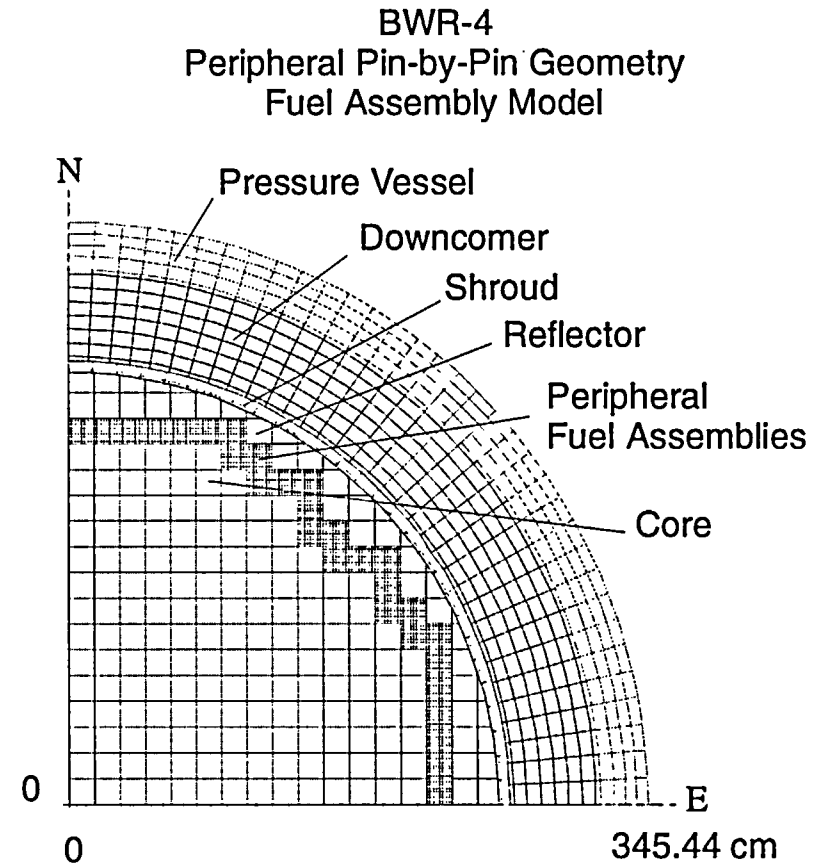
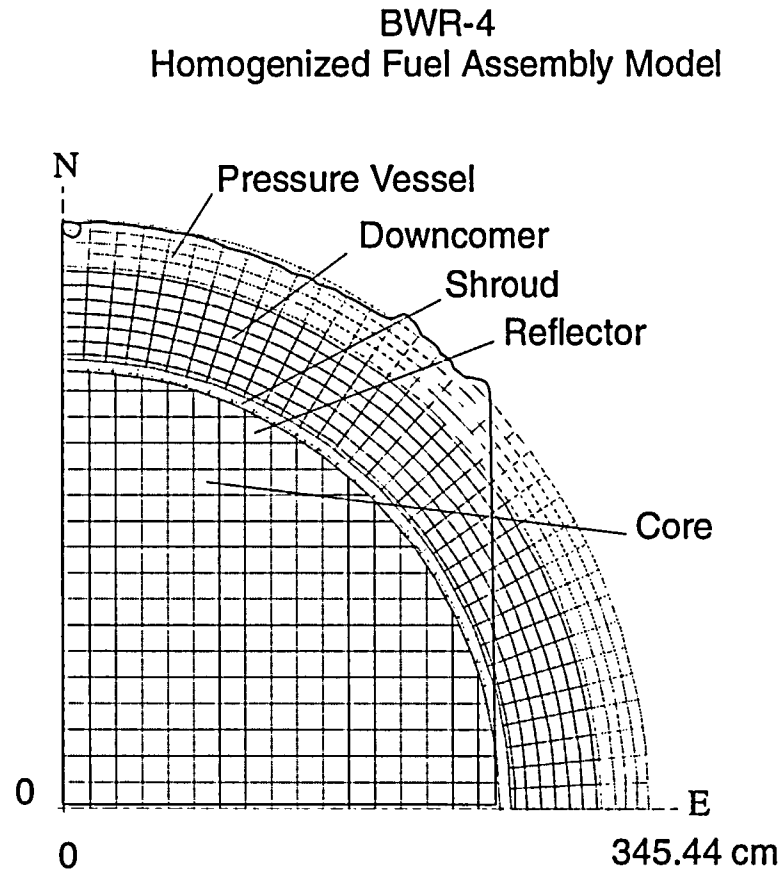


RAMA Model

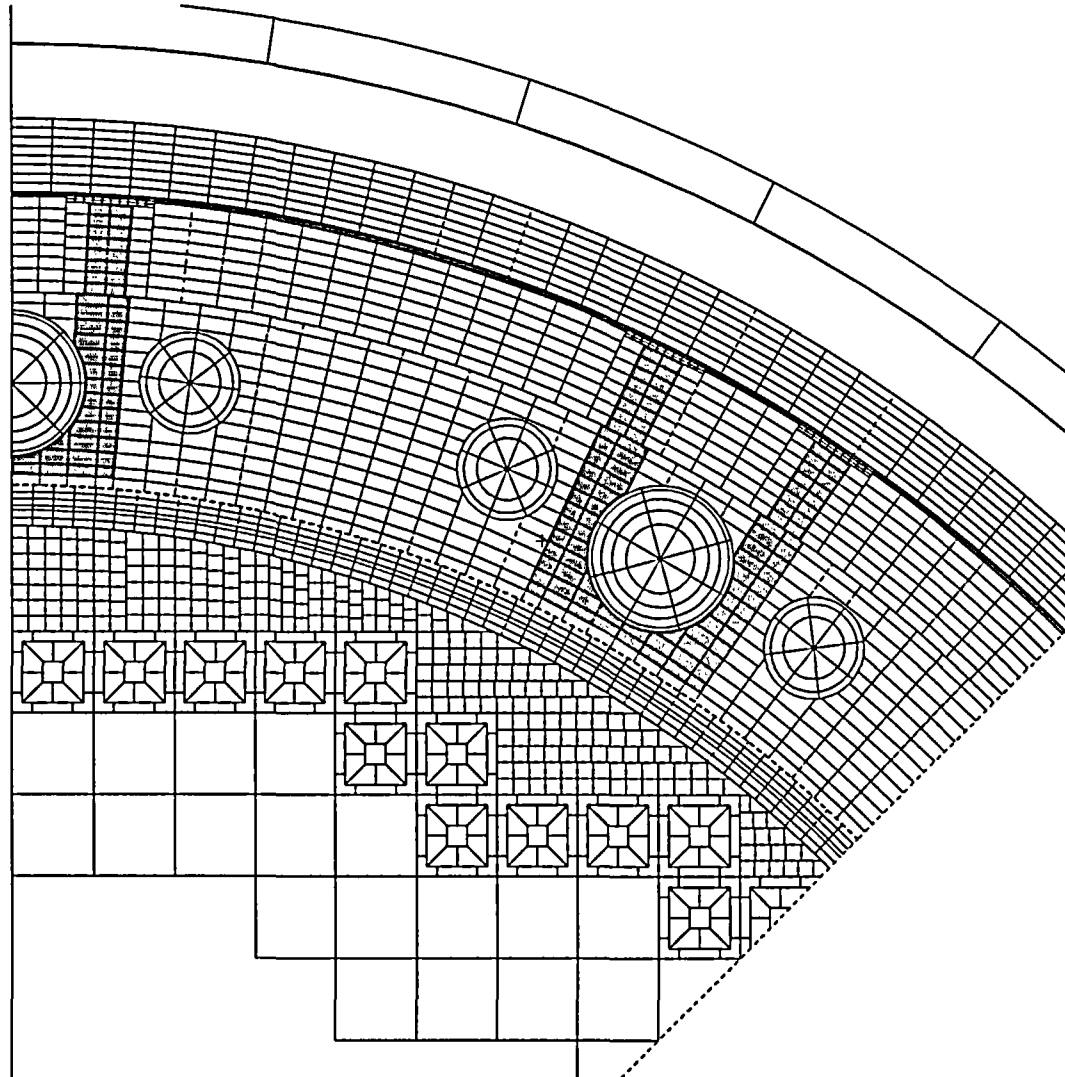


RAMA Model

Peripheral Pin-by-Pin Vs Coarse Mesh



Example of Hatch 1 Model



Benchmarking of RAMA in 2003

- Regulatory Guide 1.190 Benchmarks
 - Poolside Critical Assembly
 - Venus-3
 - H.B. Robinson
 - BWR Numerical Benchmark
- Plant Applications
 - Susquehanna Unit 2 first surveillance capsule
 - Hope Creek first cycle dosimetry
 - Hatch Unit 1 jet pump riser brace boat sample

Results of Benchmarking

- In general, all RAMA benchmarking calculations performed to date are in good agreement with measurements
 - RAMA does not agree well with some aspects of the BWR numerical benchmark
 - Possible reasons include differences in required inputs between 2-D code (DORT) and RAMA

RAMA Reports in 2003

- BWRVIP-114: BWR Vessel and Internals Project, RAMA Fluence Methodology Theory Manual
- BWRVIP-115: BWR Vessel and Internals Project, RAMA Fluence Methodology Benchmark Manual - Evaluation of Regulatory Guide 1.190 Benchmark Problems
- BWRVIP-117: BWR Vessel and Internals Project, RAMA Fluence Methodology Plant Application – Susquehanna U2
- BWRVIP-121: BWR Vessel and Internals Project, RAMA Fluence Methodology Procedure Manual
- BWRVIP-xxx: BWR Vessel and Internals Project, RAMA Fluence Methodology Plant Application – Hope Creek

Planned Activities for RAMA in 2004

- NRC report on comparison of DORT and RAMA for jet pump riser brace location at Hatch 1
- Fluence reanalysis of SSP Capsules D, G and H (BWRVIP-87)
- Fluence of SSP Capsules A, B and C from Cooper plant
 - Includes reanalysis of 2 original capsules previously withdrawn
- Benchmarking of core shroud and top guide samples removed from Susquehanna U2
- Address NRC comments on RAMA fluence methodology

Summary

- RAMA is a robust methodology to facilitate the building of complex reactor problems, and to perform the three-dimensional flux calculation, fluence analysis, and uncertainty analysis.
- Development of software completed
- RAMA calculations are in very good agreement with benchmark problems indicating the methodology is accurately predicting neutron flux, dosimetry measurements, and component fluence.
- Several 2004 activities planned to further benchmark RAMA and address NRC comments