

December 4, 2003

Mr. Rick A. Muench  
President and Chief Executive Officer  
Wolf Creek Nuclear Operating Corporation  
Post Office Box 411  
Burlington, KS 66839

SUBJECT: WOLF CREEK GENERATING STATION - EVALUATION OF RESULTS OF  
INSERVICE INSPECTIONS OF STEAM GENERATORS IN 2002 REFUELING  
OUTAGE (TAC NO. MB7164)

Dear Mr. Muench:

By letters dated April 21, 2002 (ET 02-0019) and December 11, 2002 (ET 02-0046), you submitted reports summarizing the results of the steam generator (SG) tube inspections performed during the 2002 refueling outage for the Wolf Creek Generating Station (WCGS). By letter dated July 23, 2003 (ET 03-0003), you responded to the staff's request for additional information pertaining to the reports.

WCGS has four Westinghouse Model F SGs, each of which has 5626 thermally-treated Alloy 600 tubes. The tubes have an outside diameter of 0.688-inch, a wall thickness of 0.040-inch and are supported by stainless steel tube supports with quatrefoil-shaped holes and V-shaped chrome plated Alloy 600 anti-vibration bars (AVBs).

During the 2002 SG tube inspection, Wolf Creek Nuclear Operating Corporation (WCNOC) performed tube inspections in SGs B and C. The scope and results of the inspections are provided in the above letters. WCNOC stated it plugged six tubes in SG B and three tubes in SG C. One tube in SG B was plugged due to a single circumferential anomaly and the remaining tubes in both SGs were plugged due to AVB wear that exceeded the tube plugging limit. The circumferential anomaly was inspected using an ultrasonic technique (UT) and WCNOC concluded no crack-like indications were present. WCNOC detected a wear indication at a cold leg tube support in addition to the indications of wear at the AVBs. WCNOC indicated that the cold leg wear indication was likely the result of a loose part, although it stated that no loose part was identified at this location.

During their 1999 SG tube inspection and again in 2002, the licensee detected a number of volumetric indications near the top of the tubesheet on the hot leg side. These indications were reported as "wear like" and the through-wall depth detected in both outages has not changed significantly. One of these indications was inspected by UT and was confirmed to be a volumetric indication. The licensee indicated that the UT results support their interpretation that these indications are volumetric and are not crack or crack precursor signals.

R. Muench

- 2 -

Based on the staff's review of the above information provided by WCNOG, the staff concludes that the information required to be submitted by the technical specifications was provided and that no additional follow-up is required at this time. This closes out the subject TAC.

Sincerely,

***/RA/***

Jack Donohew, Senior Project Manager, Section 2  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-482

cc: See next page

R. Muench

- 2 -

Based on the staff's review of the above information provided by WCNOG, the staff concludes that the information required to be submitted by the technical specifications was provided and that no additional follow-up is required at this time. This closes out the subject TAC.

Sincerely,

**/RA/**

Jack Donohew, Senior Project Manager, Section 2  
Project Directorate IV  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-482

cc: See next page

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\* See memorandum dated October 1, 2003

**ACCESSION NO.: ML033500187**

**NRR-106**

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