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December 5, 2003

U. S. Nuclear Regulatory Commission Washington, DC 20555

ATTENTION: Document Control Desk

- SUBJECT:Calvert Cliffs Nuclear Power Plant
Unit Nos. 1 & 2; Docket Nos. 50-317 & 50-318
Withdrawal of Request for Exemption to 10 CFR 50.44 and 10 CFR Part 50,
Appendix E, Section VI
- **REFERENCE:** (a) Letter from Mr. P. E. Katz (CCNPP) to Document Control Desk (NRC) dated March 28, 2003, "Request for Exemption to 10 CFR 50.44 and 10 CFR Part 50, Appendix E, Section VI and Proposed .License Amendments for Relaxation of Post-Accident Hydrogen Monitoring and Control Requirements"

Reference (a) requested an exemption from the combustible gas monitoring and control of 10 CFR 50.44 and 10 CFR Part 50, Appendix E, Section VI. The definition of a design-basis loss-of-coolant accident hydrogen release and the requirements for hydrogen control systems to mitigate such a release have now been removed from the regulations by a Final Rule published in 68 Federal Register 54123, dated Tuesday, September 16, 2003. As a result, the exemption requested in Reference (a) is no longer needed. Therefore, we wish to withdraw our request for exemption.

The second part of Reference (a) is a proposed license amendment to change the Calvert Cliffs Technical Specifications to relax the post-accident hydrogen monitoring and control requirements. This request is not withdrawn.

The background and justification presented in Attachment (1) to Reference (a) supports the Safety Evaluation put forth in the model application made available in 68 Federal Register 55416, dated Thursday, September 25, 2003. Additionally, the No Significant Hazards Discussion presented in Attachment (2) to Reference (a) is consistent with the No Significant Hazards Discussion in 68 Federal Register 55416. The marked-up Technical Specification pages were presented in Attachment (3) to Reference (a). The final Technical Specification pages are included with this letter as Attachment (1).

The retention of hydrogen monitors as non-safety-related equipment was noted in Reference (a) as a regulatory commitment. This letter does not withdraw that commitment. Additionally, Calvert Cliffs does not have an inerted Containment, therefore oxygen monitors are not required.

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We request that the license amendment proposed in Reference (a) be approved and issued by March 1, 2004. This date is a convenience to Calvert Cliffs Nuclear Power Plant and is not an operational necessity.

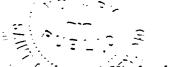
Should you have questions regarding this matter, we will be pleased to discuss them with you.

Very truly yours,

STATE OF MARYLAND : : TO WIT: COUNTY OF CALVERT :

I, George Vanderheyden, being duly sworn, state that I am Vice President - Calvert Cliffs Nuclear Power Plant, Inc. (CCNPP), and that I am duly authorized to execute and file this License Amendment Request on behalf of CCNPP. To the best of my knowledge and belief, the statements contained in this document are true and correct. To the extent that these statements are not based on my personal knowledge, they are based upon information provided by other CCNPP employees and/or consultants. Such information has been reviewed in accordance with company practice and I believe it to be reliable.

Subscribed and sworn before me, a Notary Rublic in and for the State of Maryland and County of <u>St. Mary 5</u>, this <u>5</u> day of <u>Lecenser</u>, 2003.



WITNESS my Hand and Notarial Seal:

My Commission Expires:

GV/EMT/bjd

Attachment: (1) Final Technical Specification Pages

cc: J. Petro, Esquire J. E. Silberg, Esquire Director, Project Directorate I-1, NRC G. S. Vissing, NRC H. J. Miller, NRC Resident Inspector, NRC R. I. McLean, DNR

ATTACHMENT (1)

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FINAL TECHNICAL SPECIFICATION PAGES

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| 3.5.5 | Trisodium Phosphate (TSP) | 3.5.5-1 |
|--------|---|----------|
| 3.6 | CONTAINMENT SYSTEMS | 3.6.1-1 |
| 3.6.1 | Containment | 3.6.1-1 |
| 3.6.2 | Containment Air Locks | 3.6.2-1 |
| 3.6.3 | Containment Isolation Valves | 3.6.3-1 |
| 3.6.4 | Containment Pressure | 3.6.4-1 |
| 3.6.5 | Containment Air Temperature | 3.6.5-1 |
| 3.6.6 | Containment Spray and Cooling Systems | 3.6.6-1 |
| 3.6.7 | Deleted | |
| 3.6.8 | Iodine Removal System (IRS) | 3.6.8-1 |
| 3.7 | PLANT SYSTEMS | 3.7.1-1 |
| 3.7.1 | Main Steam Safety Valves (MSSVs) | 3.7.1-1 |
| 3.7.2 | Main Steam Isolation Valves (MSIVs) | 3.7.2-1 |
| 3.7.3 | Auxiliary Feedwater (AFW) System | 3.7.3-1 |
| 3.7.4 | Condensate Storage Tank (CST) | 3.7.4-1 |
| 3.7.5 | Component Cooling (CC) System | 3.7.5-1 |
| 3.7.6 | Service Water (SRW) System | 3.7.6-1 |
| 3.7.7 | Saltwater (SW) System | 3.7.7-1 |
| 3.7.8 | Control Room Emergency Ventilation System (CREVS) | 3.7.8-1 |
| 3.7.9 | Control Room Emergency Temperature System (CRETS) | 3.7.9-1 |
| 3.7.10 | Emergency Core Cooling System (ECCS) Pump Room | |
| | Exhaust Filtration System (PREFS) | 3.7.10-1 |
| 3.7.11 | Spent Fuel Pool Exhaust Ventilation System | |
| | (SFPEVS) | 3.7.11-1 |
| 3.7.12 | Penetration Room Exhaust Ventilation System | |
| | (PREVS) | 3.7.12-1 |
| 3.7.13 | Spent Fuel Pool (SFP) Water Level | 3.7.13-1 |
| 3.7.14 | Secondary Specific Activity | 3.7.14-1 |
| 3.7.15 | Main Feedwater Isolation Valves (MFIVs) | |
| 3.7.16 | Spent Fuel Pool (SFP) Boron Concentration | |
| 3.7.17 | Spent Fuel Pool (SFP) Storage | 3./.1/-1 |
| 3.8 | ELECTRICAL POWER SYSTEMS | 3.8.1-1 |
| 3.8.1 | AC Sources-Operating | 3.8.1-1 |
| 3.8.2 | AC Sources-Shutdown | 3.8.2-1 |
| 3.8.3 | Diesel Fuel Oil | 3.8.3-1 |
| 3.8.4 | DC Sources-Operating | 3.8.4-1 |
| 3.8.5 | DC Sources-Shutdown | 3.8.5-1 |
| 3.8.6 | Battery Cell Parameters | 3.8.6-1 |
| 3.8.7 | Inverters—Operating | 3.8.7-1 |

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ACTIONS (continued)

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| - | CONDITION | | REQUIRED ACTION | COMPLETION TIME |
|----|--|--------------------------|--|---------------------|
| C. | One or more Functions with two required indication channels inoperable. | C.1 | Restore one indication channel to OPERABLE status. | 7 days |
| D. | Required Action and associated Completion Time of Condition C not met. | D.1 | Enter the Condition referenced in Table 3.3.10-1 for the channel. | Immediately |
| Ε. | As required by Required Action D.1 and referenced in Table 3.3.10-1. | E.1 <u>AND</u> E.2 | Be in MODE 3. Be in MODE 4. | 6 hours 12 hours |
| F. | As required by Required Action D.1 and referenced in Table 3.3.10-1. | F.1 | Initiate action in accordance with Specification 5.6.7. | Immediately |

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SURVEILLANCE REQUIREMENTS

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These Surveillance Requirements apply to each PAM instrumentation Function in Table 3.3.10-1.

| | SURVEILLANCE | FREQUENCY | |
|-------------|---|-----------|---|
| SR 3.3.10.1 | Perform CHANNEL CHECK for each required indication channel that is normally energized. | 31 days | |
| SR 3.3.10.2 | Deleted | | - |
| SR 3.3.10.3 | Neutron detectors, Core Exit Thermocouples, and Reactor Vessel Level Monitoring System are excluded from CHANNEL CALIBRATION. | | _ |
| | Perform CHANNEL CALIBRATION on each indication channel. | 24 months | |

CALVERT CLIFFS - UNIT 1 CALVERT CLIFFS - UNIT 2

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| | FUNCTION | REQUIRED INDICATION CHANNELS | CONDITIONS REFERENCED FROM REQUIRED ACTION E.1 |
|-----|---|---|---|
| 1. | Wide Range Logarithmic Neutron Flux | 2 | F |
| 2. | Reactor Coolant Outlet Temperature | 2 | F |
| 3. | Reactor Coolant Inlet Temperature | 2 | F |
| 4. | RCS Subcooled Margin Monitor | 1 | N/A |
| 5. | Reactor Vessel Water Level | 2 | G |
| 6. | Containment Water Level (wide range) | 2 | F |
| 7. | Containment Pressure | 2 | F |
| 8. | Containment Isolation Valve Position | 2 per penetration flow path ^{(*)(b)} | F |
| 9. | Containment Area Radiation (high range) | 2 | G |
| 10. | Pressurizer Pressure (wide range) | 2 | F |
| 11. | Steam Generator Pressure | 2 per steam generator | F |
| 12. | Pressurizer Level | 2 | F |
| 13. | Steam Generator Water Level (wide range) | 2 per steam generator | F |

Table 3.3.10-1 (page 1 of 2) Post-Accident Monitoring Instrumentation

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| | FUNCTION | REQUIRED INDICATION CHANNELS | CONDITIONS REFERENCED FROM REQUIRED ACTION E.1 |
|-----|----------------------------------|------------------------------------|---|
| 14. | Condensate Storage Tank Level | 2 | F |
| 15. | Core Exit Temperature-Quadrant 1 | 2 ^(c) | F |
| 16. | Core Exit Temperature-Quadrant 2 | 2 ^(c) | F |
| 17. | Core Exit Temperature-Quadrant 3 | 2 ^(c) | F |
| 18. | Core Exit Temperature-Quadrant 4 | 2 ^(c) | F |
| 19. | Pressurizer Pressure (low range) | 2 | F |

Table 3.3.10-1 (page 2 of 2) Post-Accident Monitoring Instrumentation

- (a) Not required for isolation values whose associated penetration is isolated by at least one closed and de-activated automatic value, closed manual value, check value with flow through the value secured, blind flange, or equivalent.
- ^(b) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.
- (c) A channel consists of two or more core exit thermocouples.

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