

January 20, 2004

Mr. Michael Kansler
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SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT, INDIAN POINT NUCLEAR GENERATING UNITS NO. 2 AND NO. 3, PILGRIM NUCLEAR POWER STATION, VERMONT YANKEE NUCLEAR POWER STATION - REQUEST FOR ADDITIONAL INFORMATION CONCERNING RELIEF REQUEST TO USE AMERICAN SOCIETY OF MECHANICAL ENGINEERS BOILER AND PRESSURE VESSEL CODE (ASME CODE) CASE N-600 (TAC NOS. MC0303, MC0336, MC0337, MC0338, AND MC0339)

Dear Mr. Kansler:

The Nuclear Regulatory Commission (NRC) staff has reviewed your submittal dated August 11, 2003, concerning the proposed relief requests to use ASME Code Case N-600 for the subject nuclear power plants. The NRC staff has determined that it needs additional information to continue its review. The enclosed request for additional information identifies the information needed to continue the review. The staff discussed the issue with your staff on November 3, 2003, and your staff indicated that you could respond within 60 days from receipt of this letter.

Sincerely,

/RA/

Guy S. Vissing, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Enclosure: As stated

Docket Nos. 50-333, 50-246, 50-286,
50-293, and 50-271

cc w/encl: See next page

FitzPatrick Nuclear Power Plant

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January 20, 2004

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT, INDIAN POINT NUCLEAR GENERATING UNITS NO. 2 AND NO. 3, PILGRIM NUCLEAR POWER STATION, VERMONT YANKEE NUCLEAR POWER STATION - REQUEST FOR ADDITIONAL INFORMATION CONCERNING RELIEF REQUEST TO USE AMERICAN SOCIETY OF MECHANICAL ENGINEERS BOILER AND PRESSURE VESSEL CODE (ASME CODE) CASE N-600 (TAC NOS. MC0303, MC0336, MC0337, MC0338, AND MC0339)

Dear Mr. Kansler:

The Nuclear Regulatory Commission (NRC) staff has reviewed your submittal dated August 11, 2003, concerning the proposed relief requests to use ASME Code Case N-600 for the subject nuclear power plants. The NRC staff has determined that it needs additional information to continue its review. The enclosed request for additional information identifies the information needed to continue the review. The staff discussed the issue with your staff on November 3, 2003, and your staff indicated that you could respond within 60 days from receipt of this letter.

Sincerely,
/RA/

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Enclosure: As stated

Docket Nos. 50-333, 50-246, 50-286,
50-293, and 50-271

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ACCESSION NUMBER: ML033320089

*See previous concurrence

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DATE	12/24/03	1/14/04	1/14/04	1/14/04	12/24/03	1/8/04	1/15/04

OFFICIAL RECORD COPY

REQUEST FOR ADDITIONAL INFORMATION (RAI)

JAMES A. FITZPATRICK NUCLEAR POWER PLANT

INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND NO. 3

PILGRIM NUCLEAR POWER STATION

VERMONT YANKEE NUCLEAR POWER STATION

RELIEF REQUEST TO USE AMERICAN SOCIETY OF MECHANICAL ENGINEERS BOILER

AND PRESSURE VESSEL CODE (ASME CODE) CASE N-600

Please note that although this appears to be one request for additional information, the response should be specific for each plant as we intend to issue a safety evaluation specific for each plant.

Relief Request RR-32, Rev. 0, James A. FitzPatrick Nuclear Power Plant

Relief Request RR-64, Indian Point Nuclear Generating Unit No. 2

Relief Request RR-3-33(A), Indian Point Nuclear Generating Unit No. 3

Relief Request PRR-33, Pilgrim Nuclear Power Station

Relief Request ISI-012, Vermont Yankee Nuclear Power Station

1. Each specific relief request must be a stand-alone document and must state the Code of record for the specific plant. Please modify each relief request to provide the exact Code of record applicable for each plant.
2. Each relief request shall state the technical basis for the relief requested, i.e. how does the proposed alternative provide an acceptable level of quality and safety? The proposed relief requests do not provide enough detail to justify the conclusion concerning the acceptable level of quality and safety. Please provide a more detailed basis for justification that the proposed alternative provides an acceptable level of quality and safety for each plant.
3. Paragraph E.8 in Relief Requests, RR-32, RR-64, RR-3-33(A), and PRR-33 states, "Entergy will comply with the Quality Assurance requirements of ASME Section XI, Article IWA-1400." Justify this deviation from ASME Code, Section XI, Code Case N-600, paragraph (h) which states, "The Owner accepting the WPQ/BPQ shall comply with the Quality Assurance requirements of IWA-4142(a)." (Relief Request ISI-012, for the Vermont Yankee Nuclear Power Station contains the reference shown in Code Case N-600)