L-PI-03- 098
10 CFR 50 Appendix A, GDC-4

November 18, 2003

U S Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
DOCKET 50-282
LICENSE No. DPR-42

APPLICATION OF LEAK-BEFORE-BREAK TECHNOLOGY TO REPLACEMENT STEAM GENERATOR DESIGN (TAC NO. MB7509)

By letter dated January 29, 2003 the Nuclear Management Company, LLC (NMC) submitted a letter requesting the Nuclear Regulator Commission (NRC) provide confirmation that the replacement steam generators may be designed on the basis of the reduced loads resulting from the application of leak-before-break criteria to postulated breaks in the primary reactor coolant loop piping. In that letter the NMC outlined the licensing justification for designing the replacement steam generators based on the reduced loads. By letter dated April 23, 2003, the NRC identified that additional information was needed. In accordance with the request in the NRC's April 23, 2003 letter, the NMC provided the additional information in letter dated June 2, 2003.

Subsequent to the June 2, 2003 letter, the NRC verbally informed the NMC that the NRC would no longer be providing the type of confirmation requested in the January 29, 2003 letter. Therefore the NMC is withdrawing the request that the NRC provide confirmation that the replacement steam generators may be designed on the basis of the reduced loads resulting from the application of leak-before-break criteria to postulated breaks in the primary reactor coolant loop piping.
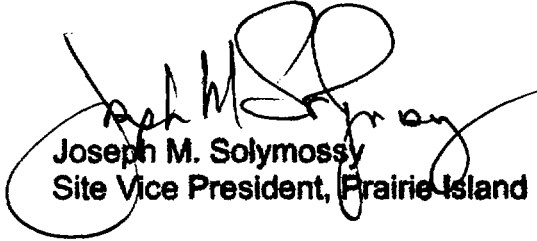
This letter contains no new commitments and no revisions to existing commitments.

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NUCLEAR MANAGEMENT COMPANY, LLC

Please address any comments or questions regarding this letter to Mr. H Oley Nelson at 1-651-388-1121.

Joseph M. Solymossy
Site Vice President, Prairie Island Nuclear Generating Plant

CC   Regional Administrator, USNRC, Region III
      Project Manager, Prairie Island Nuclear Generating Plant, USNRC, NRR
      NRC Resident Inspector - Prairie Island Nuclear Generating Plant