

**Mail Envelope Properties (3F9EAC1F.AA1 : 18 : 20163)**

**Subject:** Staff Comments on OI 5.2.3-3  
**Creation Date:** 10/28/2003 12:49PM  
**From:** Joelle Starefos

**Created By:** JLS1@nrc.gov

Recipients	Action	Date & Time
nrc.gov		
owf2_po.OWFN_DO	Delivered	10/28/03 12:49PM
EJS BC (Edmund Sullivan)	Opened	10/28/03 12:51PM
	Deleted	10/28/03 12:52PM
	Undeleted	10/28/03 03:42PM
	Deleted	10/29/03 10:21AM
JPS1 CC (John Segala)	Opened	10/28/03 12:50PM
nrc.gov		
owf4_po.OWFN_DO	Delivered	10/28/03 12:49PM
JXC1 CC (Joseph Colaccino)	Opened	10/28/03 12:49PM
	Replied	10/28/03 12:50PM
westinghouse.com	Transferred	10/28/03 12:49PM
Vijukrp (Vijukrp@westinghouse.com)		
<b>Post Office</b>	<b>Delivered</b>	<b>Route</b>
owf2_po.OWFN_DO	10/28/03 12:49PM	nrc.gov
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		westinghouse.com
<b>Files</b>	<b>Size</b>	<b>Date &amp; Time</b>
Open Item 523-3 Staff Comments.wpd	2997	10/28/03 12:46PM
MESSAGE	994	10/28/03 12:49PM

**Options**

**Auto Delete:** No  
**Expiration Date:** None  
**Notify Recipients:** Yes  
**Priority:** Standard  
**Reply Requested:** No  
**Return Notification:** None

**Concealed Subject:** No  
**Security:** Standard

**To Be Delivered:**  
**Status Tracking:**

**Immediate**  
**All Information**

**From:** Joelle Starefos  
**To:** Vijukrp@westinghouse.com  
**Date:** 10/28/2003 12:49PM  
**Subject:** Staff Comments on OI 5.2.3-3

Ron,  
Please see attached comments on Open Item 5.2.3-3. Please contact me if you need further clarification.  
Thanks, Joelle

Joelle L. Starefos  
Project Manager, AP1000  
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(301) 415-8488  
jls1@nrc.gov

**CC:** Colaccino, Joseph; Segala, John

**Open Item 5.2.3-3 Staff Comments**

The staff has reviewed your September 8, 2003, response on lowered fracture toughness of Alloy 690/52/152 materials after exposure to hydrogenated water. For the staff to evaluate this information, the staff needs the following additional information.

- a. Provide the H<sub>2</sub> concentration of the reactor coolant at normal plant operating conditions.
- b. For a simulated Pressurized Thermal Shock transient, what temperature levels would the most susceptible bi-metallic welds reach? What bi-metallic welds in the reactor coolant system would experience the most significant cooldown effect?
- c. Provide a schematic drawing of welds at these locations. For example, describe whether these welds would consist of Alloy 52 material through the entire wall, or Alloy 52 in contact with the reactor coolant and Alloy 82/182 for the remainder of the wall thickness.
- d. Assume a small ID surface breaking flaw in the bi-metallic weld identified in parts b. and c. above. Evaluate what conditions and effects the flaw would see as a result of a simulated Pressurized Thermal Shock transient (i.e., hydrogen concentration, final temperature, loading rate, and failure potential).