

December 18, 2003

Mr. Lew W. Myers  
Chief Operating Officer  
FirstEnergy Nuclear Operating Company  
Davis-Besse Nuclear Power Station  
5501 North State Route 2  
Oak Harbor, OH 43449-9760

SUBJECT: DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1 - ISSUANCE OF  
AMENDMENT RE: MOTOR DRIVEN FEEDWATER PUMP FLOW INDICATION  
(TAC NO. MB9017)

Dear Mr. Myers:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 261 to Facility Operating License No. NPF-3 for the Davis-Besse Nuclear Power Station, Unit 1. The amendment revises the Technical Specifications (TSs) in response to your application dated May 19, 2003, as supplemented October 27, 2003.

This amendment revises the TSs to require "flow indication," rather than "safety-grade flow indication," to satisfy Surveillance Requirement 4.7.1.7.e.2 for the motor driven feedwater pump.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

**/RA/**

Carl F. Lyon, Project Manager, Section 2  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-346

Enclosures: 1. Amendment No. 261 to  
License No. NPF-3  
2. Safety Evaluation

cc w/encls: See next page

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OFFICIAL RECORD COPY

Davis-Besse Nuclear Power Station, Unit 1

cc:

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The Honorable Dennis J. Kucinich, Member  
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FIRSTENERGY NUCLEAR OPERATING COMPANY

DOCKET NO. 50-346

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 261  
License No. NPF-3

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the FirstEnergy Nuclear Operating Company (the licensee) dated May 19, 2003, as supplemented October 27, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. NPF-3 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 261 , are hereby incorporated in the license. FirstEnergy Nuclear Operating Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 120 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*/RA/*

Anthony J. Mendiola, Chief, Section 2  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical  
Specifications

Date of Issuance: December 18, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 261

FACILITY OPERATING LICENSE NO. NPF-3

DOCKET NO. 50-346

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove

3/4 7-12c

Insert

3/4 7-12c

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 261 TO FACILITY OPERATING LICENSE NO. NPF-3

FIRSTENERGY NUCLEAR OPERATING COMPANY

DAVIS-BESSE NUCLEAR POWER STATION, UNIT 1

DOCKET NO. 50-346

## 1.0 INTRODUCTION

By application dated May 19, 2003, as supplemented by letter dated October 27, 2003, FirstEnergy Nuclear Operating Company (the licensee) requested changes to the Technical Specifications (TSs) for the Davis-Besse Nuclear Power Station, Unit 1 (DBNPS). The supplement dated October 27, 2003, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on June 10, 2003 (68 FR 34669).

The proposed changes would revise TS Surveillance Requirement (SR) 4.7.1.7, "Motor Driven Feedwater Pump System," to remove the requirement that safety grade flow indication be used to satisfy SR 4.7.1.7.e.2. Specifically, the proposed changes would revise:

### 1.1 SR 4.7.1.7.e.2

SR 4.7.1.7.e states, in part,

"After any modification or repair to the Motor Driven Feedwater Pump System that could affect the system's capability to deliver water from the Condensate Storage Tanks to the Auxiliary Feedwater System, the affected flow path shall be demonstrated available as follows:

1. ...
2. If the modification or repair is upstream of the Motor Driven Feedwater Pump test flow line tie-in, the Motor Driven Feedwater Pump shall pump water from the Condensate Storage Tanks to the test flow line and the flow path availability will be verified by Motor Driven Feedwater Pump Safety Grade Flow Indication."

The licensee proposes to revise the wording of SR 4.7.1.7.e.2 to read,

"2. If the modification or repair is upstream of the Motor Driven Feedwater Pump test flow line tie-in, the Motor Driven Feedwater Pump shall pump water from the Condensate Storage Tanks to the test flow line and the flow path availability will be verified by Motor Driven Feedwater Pump flow indication."

## 2.0 REGULATORY EVALUATION

The staff finds that the licensee in Sections 3.0 and 5.2 of its submittal identified the applicable regulatory requirements. The motor driven feedwater pump (MDFWP) is described in DBNPS Updated Safety Analysis Report (USAR) Section 9.2.8. The regulatory requirements for which the staff based its acceptance are TS 3/4.7.1.7 and USAR Section 9.2.8.

## 3.0 TECHNICAL EVALUATION

### 3.1 Background

The MDFWP was installed by the licensee as part of the corrective actions following the loss of feedwater and subsequent steam generator dryout event at DBNPS in 1985. The MDFWP system provides feedwater to the steam generators during normal plant startup and shutdown. It also provides a backup supply of feedwater to the steam generators in the event of a total loss of both auxiliary feedwater (AFW) and the main feedwater. The MDFWP system is non-safety-related, as described in Section 9.2.8 of the USAR and Sections 3.3.1.2 and 3.3.1.3 of NUREG-1177, "Safety Evaluation Report Related to the Restart of [DBNPS] Following the Event of June 9, 1985." However, the MDFWP provides a diverse method of supplying AFW to the steam generators and so functions as a backup to the safety-related AFW system.

Performance of SR 4.7.1.7.e.2 demonstrates the availability of the flowpath from the condensate storage tanks to the test flow line tie-in following any modification or repair activity upstream of the test flow line tie-in that could affect the MDFWP system's capability to supply water from the tanks to the AFW system. SR 4.7.1.7.e.2 currently requires the flow indication used to verify flow path availability be safety-grade. The original wording of the SR required only the use of the "[MDFWP] flow indication" to verify flow path availability. The wording was changed to "[MDFWP] safety grade flow indication" by Amendment No. 193, dated October 18, 1994, which was issued in response to the licensee's application for amendment dated April 5, 1994.

### 3.2 Evaluation

The staff has reviewed the licensee's technical analyses in support of its proposed license amendment, which is described in Sections 3.0 and 4.0 of the licensee's May 19, 2003, application, as supplemented by letter dated October 27, 2003.

The licensee states that the change in wording in SR 4.7.1.7.e.2 from "[MDFWP] flow indication" to "[MDFWP] safety grade flow indication" was made due to an error in its application for Amendment No. 193. The licensee confirmed that the flow indication for the MDFWP was not originally installed as safety-grade equipment and has not been upgraded to safety-related standards. The licensee states that the flow indication for the MDFWP is not required during design-basis accidents. There are no accidents analyzed in the USAR that credit the MDFWP system. If both AFW pumps were not available and the MDFWP system was being used to supply water to the steam generators, the MDFWP system flow indication would not be the relied-upon indication. Instead, flow would be verified by other safety-grade flow indications in the AFW system that are periodically tested by other SRs. The licensee states that the proposed correction to the wording in SR 4.7.1.7.e.2 does not involve any physical alteration of

the plant or a change in the methods of normal plant operation. No modification or repair activity has been identified as having been performed upstream of the MDFWP test tie-in since Amendment No. 193 was implemented; therefore, SR 4.7.1.7.e.2 has not been required.

Since (1) the MDFWP is a non-safety-related system that is not assumed in any accident analysis and (2) safety-grade flow indication in the AFW system would be relied upon if the AFW pumps were not available and the MDFWP was supplying flow to the steam generators, there is no need to require that the MDFWP flow indication be safety-grade. Therefore, the licensee's proposed change to the wording of SR 4.7.1.7.e.2 is acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Ohio State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes an inspection or a surveillance requirement. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (68 FR 34669). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Lyon

Date: December 18, 2003