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3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.1 Recirculation Loops Operating

LCO 3.4.1 Two recirculation loops with matched flows shall be in operation with a THERMAL POWER/core flow condition outside of Regions I and II of the Power Flow Map as specified in the COLR.

OR

One recirculation loop may be in operation provided the following limits are applied when the associated LCO is applicable with a THERMAL POWER/core flow condition outside of Regions I and II of the Power Flow Map as specified in the COLR.

- a. LCO 3.2.1, "AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR)," single loop operation limits specified in the COLR;
- b. LCO 3.2.2, "MINIMUM CRITICAL POWER RATIO (MCPR)," single loop operation limits specified in the COLR;
- c. LCO 3.2.3, "LINEAR HEAT GENERATION RATE (LHGR)," single loop operation limits specified in the COLR, and
- d. LCO 3.3.1.1, "Reactor Protection System (RPS) Instrumentation," Function 2.b (Average Power Range Monitors Flow Biased Simulated Thermal Power—High), Allowable Value of Table 3.3.1.1-1 is reset for single loop operation.
- e. Recirculation pump speed is \leq 80%.

-----Note-----

Required limit and setpoint resets for single recirculation loop operation may be delayed for up to 12 hours after transition from two recirculation loop operation to single recirculation loop operation.

APPLICABILITY: MODES 1 and 2.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. Total core flow as a function of THERMAL POWER within Region I of the Power Flow Map as specified in the COLR.</p> <p><u>OR</u></p> <p>No recirculation loops operating while in MODE 1.</p>	<p>A.1 Place reactor mode switch in the shutdown position.</p>	<p>Immediately</p>
<p>B. -----NOTE----- Only applicable when in Region II of the Power Flow Map as specified in the COLR.</p> <p><u>OR</u></p> <p>Two or more APRM readings oscillating with one or more oscillating $\geq 10\%$ of RTP peak-to-peak.</p> <p><u>OR</u></p> <p>Two or more LPRM upscale alarms activating and deactivating with a period ≥ 1 second and ≤ 5 seconds.</p> <p><u>OR</u></p> <p>Sustained LPRM oscillations $> 10 \text{ w/cm}^2$ peak-to-peak with a period ≥ 1 second and ≤ 5 seconds.</p> <p><u>OR</u></p>	<p>B.1 Place the reactor mode switch in the shutdown position.</p>	<p>Immediately</p> <p>(continued)</p>

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>B. (continued) Less than 50% of required LPRM upscale alarms OPERABLE.</p>		
<p>C. Total core flow as a function of THERMAL POWER within Region II of the Power Flow Map as specified in the COLR.</p>	<p>C.1 Initiate action to restore total core flow as a function of THERMAL POWER outside of Region II.</p>	<p>Immediately</p>
<p>D. Recirculation loop flow mismatch not within limits.</p>	<p>D.1 Declare the recirculation loop with lower flow to be "not in operation."</p>	<p>2 hours</p>
<p>E. No recirculation loops in operation while in MODE 2.</p> <p><u>OR</u></p> <p>Single Recirculation Loop required limits and setpoints not established within required time.</p>	<p>E.1 Be in MODE 3.</p>	<p>12 hours</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.1.1	<p>-----NOTE-----</p> <p>Not required to be performed until 24 hours after both recirculation loops are in operation.</p> <p>-----</p> <p>Verify recirculation loop jet pump flow mismatch with both recirculation loops in operation is:</p> <p>a. ≤ 10 million lbm/hr when operating at < 75 million lbm/hr total core flow; and</p> <p>b. ≤ 5 million lbm/hr when operating at ≥ 75 million lbm/hr total core flow.</p>	24 hours
SR 3.4.1.2	Verify total core flow as a function of THERMAL POWER is outside of Region I and II of the Power Flow Map as specified in the COLR.	24 hours
SR 3.4.1.3	<p>-----NOTE-----</p> <p>Only required to be met during single loop operations.</p> <p>-----</p> <p>Verify recirculation pump speed is within the limit specified in the LCO.</p>	24 hours



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5.6 Reporting Requirements (continued)

5.6.4 Monthly Operating Reports

Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the main steam safety/relief valves, shall be submitted on a monthly basis no later than the 15th of each month following the calendar month covered by the report.

5.6.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, and shall be documented in the COLR for the following:
1. The Average Planar Linear Heat Generation Rate for Specification 3.2.1;
 2. The Minimum Critical Power Ratio for Specification 3.2.2;
 3. The Linear Heat Generation Rate for Specification 3.2.3;
 4. The Average Power Range Monitor (APRM) Gain and Setpoints for Specification 3.2.4; and
 5. The Shutdown Margin for Specification 3.1.1.
 6. The stability related regions of the Power Flow Map for Specification 3.4.1.
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC.

When an initial assumed power level of 102 percent of rated power is specified in a previously approved method, this refers to the power level associated with the design basis analyses, or 3510 MWt. The power level of 3510 MWt is 100.6% of the rated thermal power level of 3489 MWt. The RTP of 3489 MWt may only be used when feedwater flow measurement (used as input to the reactor thermal power measurement) is provided by the Leading Edge Flow Meter (LEFMTM) as described in the LEFMTM Topical Report and supplement referenced below. When feedwater flow measurements from the LEFMTM system are not available, the core thermal power level may not exceed the originally approved RTP of 3441 MWt, but the value of 3510 MWt.

(continued)

5.6 Reporting Requirements

5.6.5 COLR (continued)

9. ANF-91-048(P)(A), "Advanced Nuclear Fuels Corporation Methodology for Boiling Water Reactors EXEM BWR Evaluation Model.
 10. XN-NF-80-19(P)(A), "Exxon Nuclear Methodology for Boiling Water Reactors.
 11. XN-NF-79-71(P)(A), "Exxon Nuclear Plant Transient Methodology for Boiling Water Reactors."
 12. EMF-1997(P)(A), "ANFB-10 Critical Power Correlation."
 13. Caldon, Inc., "TOPICAL REPORT: Improving Thermal Power Accuracy and Plant Safety While Increasing Operating Power Level Using the LEFM[✓]™ System," Engineering Report - 80P.
 14. Caldon, Inc., "Supplement to Topical Report ER-80P: Basis for a Power Uprate with the LEFM[✓]™ or LEFM CheckPlus™ System," Engineering Report ER-160P.
 15. EMF-85-74(P), "RODEX 2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model."
 16. EMF-CC-074(P)(A), Volume 4, "BWR Stability Analysis: Assessment of STAIF with Input from MICROBURN-B2."
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

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3.1.4	Control Rod Scram Times	3.1-12
3.1.5	Control Rod Scram Accumulators	3.1-15
3.1.6	Rod Pattern Control.....	3.1-18
3.1.7	Standby Liquid Control (SLC) System	3.1-20
3.1.8	Scram Discharge Volume (SDV) Vent and Drain Valves	3.1-25
3.2	POWER DISTRIBUTION LIMITS	3.2-1
3.2.1	Average Planar Linear Heat Generation Rate (APLHGR).....	3.2-1
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3.3.1.1	Reactor Protection System (RPS) Instrumentation	3.3-1
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3.3.6.1	Primary Containment Isolation Instrumentation.....	3.3-52
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(continued)

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.1 Recirculation Loops Operating

LCO 3.4.1 Two recirculation loops with matched flows shall be in operation with a THERMAL POWER/core flow condition outside of Regions I and II of the Power Flow Map as specified in the COLR.

OR

One recirculation loop may be in operation with a THERMAL POWER/core flow condition outside of Regions I and II of the Power Flow Map as specified in the COLR, provided the following limits are applied when the associated LCO is applicable:

- a. LCO 3.2.1, "AVERAGE PLANAR LINEAR HEAT GENERATION RATE (APLHGR)," single loop operation limits specified in the COLR;
- b. LCO 3.2.2, "MINIMUM CRITICAL POWER RATIO (MCPR)," single loop operation limits specified in the COLR;
- c. LCO 3.2.3, "LINEAR HEAT GENERATION RATE (LHGR)," single loop operation limits specified in the COLR, and
- d. LCO 3.3.1.1, "Reactor Protection System (RPS) Instrumentation," Function 2.b (Average Power Range Monitors Flow Biased Simulated Thermal Power—High), Allowable Value of Table 3.3.1.1-1 is reset for single loop operation.
- e. Recirculation pump speed is $\leq 80\%$.

Note

Required limit and setpoint resets for single recirculation loop operation may be delayed for up to 12 hours after transition from two recirculation loop operation to single recirculation loop operation.

APPLICABILITY: MODES 1 and 2.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. Total core flow as a function of THERMAL POWER within Region 1 of the Power Flow Map as specified in the COLR.</p> <p><u>OR</u></p> <p>No recirculation loops operating while in MODE 1.</p>	<p>A.1 Place reactor mode switch in the shutdown position.</p>	<p>Immediately</p>
<p>B. -----NOTE----- Only applicable when in Region II of the Power Flow Map as specified in the COLR. -----</p> <p>Two or more APRM readings oscillating with one or more oscillating $\geq 10\%$ of RTP peak-to-peak.</p> <p><u>OR</u></p> <p>Two or more LPRM upscale alarms activating and deactivating with a period ≥ 1 second and ≤ 5 seconds.</p> <p><u>OR</u></p> <p>Sustained LPRM oscillations > 10 w/cm² peak-to-peak with a period ≥ 1 second and ≤ 5 seconds.</p> <p><u>OR</u></p>	<p>B.1 Place the reactor mode switch in the shutdown position.</p>	<p>Immediately</p> <p style="text-align: right;">(continued)</p>

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>B. (continued) Less than 50% of required LPRM upscale alarms OPERABLE.</p>		
<p>C. Total core flow as a function of THERMAL POWER within Region II of the Power Flow Map as specified in the COLR.</p>	<p>C.1 Initiate action to restore total core flow as a function of THERMAL POWER outside of Region II.</p>	<p>Immediately</p>
<p>D. Recirculation loop flow mismatch not within limits.</p>	<p>D.1 Declare the recirculation loop with lower flow to be "not in operation."</p>	<p>2 hours</p>
<p>E. No recirculation loops in operation while in MODE 2. <u>OR</u> Single Recirculation Loop required limits and setpoints not established within required time.</p>	<p>E.1 Be in MODE 3.</p>	<p>12 hours</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.1.1	<p style="text-align: center;">-----NOTE-----</p> <p>Not required to be performed until 24 hours after both recirculation loops are in operation.</p> <hr/> <p>Verify recirculation loop jet pump flow mismatch with both recirculation loops in operation is:</p> <p>a. \leq 10 million lbm/hr when operating at < 75 million lbm/hr total core flow; and</p> <p>b. \leq 5 million lbm/hr when operating at \geq 75 million lbm/hr total core flow.</p>	24 hours
SR 3.4.1.2	Verify total core flow as a function of THERMAL POWER is outside of Region I and II of the Power Flow Map as specified in the COLR.	24 hours
SR 3.4.1.3	<p style="text-align: center;">-----NOTE-----</p> <p>Only required to be met during single loop operations.</p> <hr/> <p>Verify recirculation pump speed is within the limit specified in the LCO.</p>	24 hours



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5.6 Reporting Requirements

5.6.4 Monthly Operating Reports

Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the main steam safety/relief valves, shall be submitted on a monthly basis no later than the 15th of each month following the calendar month covered by the report.

5.6.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, and shall be documented in the COLR for the following:
 1. The Average Planar Linear Heat Generation Rate for Specification 3.2.1;
 2. The Minimum Critical Power Ratio for Specification 3.2.2;
 3. The Linear Heat Generation Rate for Specification 3.2.3;
 4. The Average Power Range Monitor (APRM) Gain and Setpoints for Specification 3.2.4; and
 5. The Shutdown Margin for Specification 3.1.1.
 6. The stability related regions of the Power Flow Map for Specification 3.4.1.
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC.

When an initial assumed power level of 102 percent of rated power is specified in a previously approved method, this refers to the power level associated with the design basis analyses, or 3510 MWt. The power level of 3510 MWt is 100.6% of the rated thermal power level of 3489 MWt. The RTP of 3489 MWt may only be used when feedwater flow measurement (used as input to the reactor thermal power measurement) is provided by the Leading Edge Flow Meter (LEFM[✓]TM) as described in the LEFM[✓]TM Topical Report and supplement referenced below. When feedwater flow measurements from the LEFM[✓]TM system are not available, the core thermal power level may not exceed the originally approved RTP of 3441 MWt, but the value of 3510 MWt (102% of 3441 MWt) remains the initial power level for the bounding licensing analysis.

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5.6 Reporting Requirements

5.6.5 COLR (continued)

11. Caldon, Inc., "Supplement to Topical Report ER-80P: Basis for a Power Uprate with the LEFMTM or LEFM CheckPlusTM System," Engineering Report ER-160P.
 12. EMF-85-74(P)(A), "RODEX 2A (BWR) Fuel Rod Thermal-Mechanical Evaluation Model."
 13. EMF-2158(P)(A), "Siemens Power Corporation Methodology for Boiling Water Reactors: Evaluation and Validation of CASMO-4/Microburn-B2," Siemens Power Corporation.
 14. EMF-CC-074(P)(A), Volume 4, "BWR Stability Analysis: Assessment of STAIF with Input from MICROBURN-B2."
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6.6 EDG Failures Report

If an individual emergency diesel generator (EDG) experiences four or more valid failures in the last 25 demands, these failures and any nonvalid failures experienced by that EDG in that time period shall be reported within 30 days. Reports on EDG failures shall include the information recommended in Regulatory Guide 1.9, Revision 3, Regulatory Position C.4.

5.6.7 PAM Report

When a report is required by Condition B or F of LCO 3.3.3.1, "Post Accident Monitoring (PAM) Instrumentation," a report shall be submitted within the following 14 days. The report shall outline the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the Function to OPERABLE status.
