

November 5, 2003

Mr. Ronald A. Jones
Vice President, Oconee Site
Duke Energy Corporation
P. O. Box 1439
Seneca, SC 29679

SUBJECT: OCONEE NUCLEAR STATION, UNITS 1, 2 AND 3 RE: ISSUANCE OF
AMENDMENTS (TAC NOS. MC0116, MC0117, AND MC0118)

Dear Mr. Jones:

The Nuclear Regulatory Commission has issued the enclosed Amendment Nos. 336, 336, and 337 to Renewed Facility Operating Licenses DPR-38, DPR-47, and DPR-55, respectively, for the Oconee Nuclear Station, Units 1, 2, and 3. The amendments consist of changes to the Technical Specifications in response to your application dated July 10, 2003.

The amendments remove requirements that are no longer applicable because the implementation of the automatic feedwater isolation system modifications has been completed on all three Oconee units.

A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Leonard N. Olshan, Senior Project Manager, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, and 50-287

Enclosures:

1. Amendment No. 336 to DPR-38
2. Amendment No. 336 to DPR-47
3. Amendment No. 337 to DPR-55
4. Safety Evaluation

cc w/encls: See next page

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DUKE ENERGY CORPORATION

DOCKET NO. 50-269

OCONEE NUCLEAR STATION, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 336
Renewed License No. DPR-38

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 1 (the facility) Renewed Facility Operating License No. DPR-38 filed by the Duke Energy Corporation (the licensee) dated July 10, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Renewed Facility Operating License No. DPR-38 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 336, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: November 5, 2003

DUKE ENERGY CORPORATION

DOCKET NO. 50-270

OCONEE NUCLEAR STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 336
Renewed License No. DPR-47

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 2 (the facility) Renewed Facility Operating License No. DPR-47 filed by the Duke Energy Corporation (the licensee) dated July 10, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Renewed Facility Operating License No. DPR-47 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 336, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: November 5, 2003

DUKE ENERGY CORPORATION

DOCKET NO. 50-287

OCONEE NUCLEAR STATION, UNIT 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 337
Renewed License No. DPR-55

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Oconee Nuclear Station, Unit 3 (the facility) Renewed Facility Operating License No. DPR-55 filed by the Duke Energy Corporation (the licensee) dated July 10, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.B of Renewed Facility Operating License No. DPR-55 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 337, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

John A. Nakoski, Chief, Section 1
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: November 5, 2003

ATTACHMENT TO LICENSE AMENDMENT NO. 336
RENEWED FACILITY OPERATING LICENSE NO. DPR-38
DOCKET NO. 50-269
AND
TO LICENSE AMENDMENT NO. 336
RENEWED FACILITY OPERATING LICENSE NO. DPR-47
DOCKET NO. 50-270
AND
TO LICENSE AMENDMENT NO. 337
RENEWED FACILITY OPERATING LICENSE NO. DPR-55
DOCKET NO. 50-287

Replace the following pages of the Appendix A Technical Specifications and associated Bases with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
ii (TS Section)	ii (TS Section)
3.3.11-1	3.3.11-1
3.3.12-1	3.3.12-1
3.3.13-1	3.3.13-1
3.3.25-1	3.3.25-1
3.3.25-2	-
3.3.26-1	3.3.26-1
3.3.26-2	-
3.3.27-1	3.3.27-1
3.3.27-2	-
ii (Bases Section)	ii (Bases Section)
B 3.3.11-1	B 3.3.11-1
B 3.3.11-2	B 3.3.11-2
B 3.3.11-3	B 3.3.11-3
B 3.3.11-4	B 3.3.11-4
B 3.3.11-5	B 3.3.11-5
B 3.3.12-1	B 3.3.12-1

Remove

Insert

B 3.3.12-2
B 3.3.13-1
B 3.3.13-2
B 3.3.13-3
B 3.3.13-4
B 3.3.25-1
B 3.3.25-2
B 3.3.25-3
B 3.3.25-4
B 3.3.25-5
B 3.3.25-6
B 3.3.26-1
B 3.3.26-2
B 3.3.26-3
B 3.3.27-1
B 3.3.27-2
B 3.3.27-3

B 3.3.12-2
B 3.3.13-1
B 3.3.13-2
B 3.3.13-3
B 3.3.13-4
B 3.3.25-1
-
-
-
-
-
B 3.3.26-1
-
-
B 3.3.27-1
-
-

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO
AMENDMENT NO. 336 TO RENEWED FACILITY OPERATING LICENSE DPR-38
AMENDMENT NO. 336 TO RENEWED FACILITY OPERATING LICENSE DPR-47
AND AMENDMENT NO. 337 TO RENEWED FACILITY OPERATING LICENSE DPR-55
DUKE ENERGY CORPORATION
OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3
DOCKET NOS. 50-269, 50-270, AND 50-287

1.0 INTRODUCTION

By letter dated July 10, 2003, Duke Energy Corporation (the licensee) submitted a request for changes to the Oconee Nuclear Station, Units 1, 2, and 3, Technical Specifications (TS). The requested changes would remove requirements that are no longer applicable because the implementation of the automatic feedwater isolation system (AFIS) modifications has been completed on all three units.

2.0 REGULATORY EVALUATION

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include technical specifications (TSs) as part of the license. The Commission's regulatory requirements related to the content of TSs are stated in *Title 10 of the Code of Federal Regulations* (10 CFR) Section 50.36, "Technical specifications." These TSs, as stated in 10 CFR 50.36, should include safety limits, limiting safety system settings, and limiting control settings (c)(1); limiting conditions for operation (c)(2); surveillance requirements (c)(3); design features (c)(4); administrative controls (c)(5); decommissioning (c)(6); initial notification (c)(7); and written reports (c)(8).

In following the requirements constituted in 10 CFR 50.36(c)(2)(ii), to establish technical specification limiting conditions for operation of a nuclear reactor, one or more of the following criteria must be met.

1. Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary. (c)(2)(ii)(A)
2. A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier. (c)(2)(ii)(B)

3. A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier. (c)(2)(ii)(C)
4. A structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety. (c)(2)(ii)(D)

The Automatic Feedwater Isolation System (AFIS) is designed to provide automatic termination of feedwater flow to limit the effects of a Main Steam Line Break (MSLB). Main feedwater (MFW) is terminated by tripping both MFW pumps and closing the affected steam generators main and startup feedwater control valves and block valves. The AFIS instrumentation, manual initiation, and logic channels satisfies Criterion 3 of 10 CFR 50.36 (c)(2)(ii)(C). The AFIS manual initiation circuitry satisfies the manual initiation and single-failure criterion requirements of IEEE-279-1971.

3.0 TECHNICAL EVALUATION

3.1 TS LCO 3.3.11, LCO 3.3.12, AND LCO 3.3.13

These three TSs, which apply to AFIS, currently have a note that states, "Not applicable on each Unit until after completion of the Automatic Feedwater Isolation System modification." As previously stated, the AFIS modification has been completed on all three units. Therefore, the NRC staff concludes that this note is no longer needed and that removing it from LCO 3.3.11, LCO 3.3.12, and LCO 3.3.13 is acceptable.

3.2 TS LCO 3.3.25, LCO 3.3.26, AND LCO 3.3.27

These three TSs apply to the main steam line break (MSLB) detection and feedwater isolation system and currently have a note that states, "Applicable on each Unit until after completion of the Automatic Feedwater Isolation System modification on each respective Unit." The AFIS replaced the MSLB detection and feedwater isolation system. Since the AFIS modification has been completed on all three units, the NRC staff concludes that removal of LCO 3.3.25, LCO 3.3.26, and LCO 3.3.27 is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the South Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (68 FR 49816). Accordingly, the amendments meet the eligibility criteria for categorical

exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: L. Olshan

Date: November 5, 2003

Oconee Nuclear Station

cc:

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