October 21, 2003

Mr. J. A. Stall
Senior Vice President, Nuclear and
Chief Nuclear Officer
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

SUBJECT: TURKEY POINT UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS REGARDING ADMINISTRATIVE UPDATE OF TECHNICAL SPECIFICATIONS (TAC NOS. MB7148 AND MB7149)

Dear Mr. Stall:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 224 to Renewed Facility Operating License No. DPR-31 and Amendment No. 219 to Renewed Facility Operating License No. DPR-41 for the Turkey Point Plant, Units Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your application dated December 20, 2002, as supplemented by letter dated August 15, 2003.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice. If you have any questions regarding these issues, please feel free to contact me at (301) 415-2315.

Sincerely,

/RA/

Eva A. Brown, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-250 and 50-251

Enclosures:
1. Amendment No. 224 to DPR-31
2. Amendment No. 219 to DPR-41
3. Safety Evaluation

cc w/enclosures: See next page

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FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 224
License No. DPR 31

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated December 20, 2002, as supplemented by letter dated August 15, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

   (B) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 224, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days within issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Allen G. Howe, Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 21, 2003

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 219
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated December 20, 2002, as supplemented by letter dated August 15, 2003, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

   (B) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 219, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days within issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Allen G. Howe, Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 21, 2003

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 224 RENEWED FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 219 RENEWED FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

Remove pages	Insert pages
xiv, 5-4, 5-5, 5-6, 5-7	xiv, 5-4, 5-5, 5-6, 5-7
i, ii, 1-2, 1-3, 1-4, 1-5, 1-6, 1-6a	i, ii, 1-2, 1-3, 1-4, 1-5, 1-6
2-7	2-7
2-9	2-9
iv, 3/4 1-16	iv, 3/4 1-16
3/4 4-32	3/4 4-32
3/4 4-36	3/4 4-36
3/4 6-1	3/4 6-1
3/4 7-11	3/4 7-11
3/4 9-7	3/4 9-7
3/4 9-12	3/4 9-12

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR

REGULATION RELATED TO AMENDMENT NO. 224 TO RENEWED

FACILITY OPERATING LICENSE NO. DPR-31 AND AMENDMENT NO. 219

TO RENEWED FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

By application dated December 20, 2002, as supplemented by letter dated August 15, 2003, Florida Power and Light Company (FPL, the licensee) requested amendments that would revise Renewed Facility Operating Licenses DPR-31 and DPR-41 for the Turkey Point Nuclear Plant, Units 3 and 4. The proposed revisions represent administrative changes to the Technical Specifications (TSs) that would: (1) correct the Index, page xiv, (2) delete definitions, reorder definitions alphabetically, and renumber affected entries, (3) replace the equation on page 2-7, (4) replace the equation on page 2-9, (5) remove "Heat Tracing . . . 3/4 1-16" and entire page 3/4-1-16, (6) change the title of Figure 3.4-32, (7) correct page 3/4 4-36, Limiting Condition for Operation 3.4.9.3, (8) correct page 3/4 6-1, Surveillance Requirement 4.6.1.1 a, (9) correct page 3/4 7-11 and 3.7.1.6 ACTION Statement b, (10) correct page 3/4 9-7, and (11) correct page 3/4 9-12. These proposed changes, as supplemented, will improve the overall consistency and readability of the TSs without altering the content or intended interpretation.

The licensee's supplementary submittal dated August 15, 2003, did not affect the original proposed no significant hazards determination, or expand the scope of the request as noticed in the *Federal Register* on February 4, 2003 (68 FR 5677).

2.0 REGULATORY EVALUATION

The licensee proposed to make the administrative changes in accordance with Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.90:

> Whenever a holder of a license or construction permit desires to amend the license (including the Technical Specifications incorporated into the license) or permit, application for an amendment must be filed with the Commission, as specified in 10 CFR 50.4, fully describing the changes desired, and following as far as applicable, the form prescribed for original applications.

The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. This regulation requires that the TSs include items in five specific categories. These categories include 1) safety limits, limiting safety system settings and limiting control settings, 2) limiting conditions for operation, 3) surveillance requirements, 4) design features, and 5) administrative controls. However, the regulation does not specify the particular TSs to be included in a plant's license.

Additionally, 10 CFR 50.36(c)(2)(ii) sets forth four criteria to be used in determining whether a limiting condition for operation (LCO) is required to be included in the TS. These criteria are as follows:

1. Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.

2. A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

3. A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

4. A structure, system or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

Existing LCOs included as TS requirements which satisfy any of the criteria stated above must be retained in the TSs. Those TS requirements which do not satisfy these criteria may be relocated to other, licensee-controlled documents.

3.0 TECHNICAL EVALUATION

Each of the proposed changes is stated and evaluated below in detail.

3.1 Correct TS Index, page xiv

FPL proposed to delete "5.5 METEOROLOGICAL TOWER LOCATION . . . 5-4" from the TS Index, page xiv, and then renumber the affected sections.

The licensee indicated that on February 12, 2002, the U.S. Nuclear Regulatory Commission (NRC) issued a Safety Evaluation (SE) entitled "Turkey Point Units 3 and 4 - Issuance of Amendments Regarding Removal of Site Area and Plant Area Maps from Technical Specifications (TAC Nos. MB1968 and MB1969)." This evaluation amended the license to allow the relocation of TS Figure 5.1-1, "Site Area Map" and Figure 5.1.3 "Map Defining Unrestricted Area and Site Boundary for Radioactive Gaseous and Liquid Effluents" from the TSs to the Turkey Point Updated Final Safety Analysis Report (UFSAR). The licensee indicated the changes, as proposed, would remove outdated references, which should provide greater clarity and consistency.

The NRC staff reviewed the associated amendment and verified that the "METEOROLOGICAL TOWER LOCATION" figures had been deleted from the TSs and relocated to the UFSAR. As such, the proposed deletion "5.5 METEOROLOGICAL TOWER LOCATION . . . 5-4" from the TS Index, page xiv, along with the renumbering of affected sections, removes outdated information and provides greater consistency in the TSs. The NRC staff finds the changes to be administrative in nature and are, therefore, acceptable.

3.2 Delete definitions, reorder definitions alphabetically, and renumber affected entries

FPL proposed to: (1) delete the references to previously deleted definitions 1.15, 1.16, and 1.21, reorder definitions 1.37 and 1.38 alphabetically, and renumber affected entries on TS Index pages i and ii, (2) remove deleted definitions 1.15, 1.16, and 1.21 on TS pages 1-3 and 1-4, and (3) relocate the [DIGITAL CHANNEL OPERATIONAL TEST] and [CORE OPERATING LIMITS REPORT] definitions 1.37 and 1.38 to insert them alphabetically in the DEFINITIONS text.

The NRC staff verified that definitions 1.15, 1.16, and 1.21 were deleted in previous license amendments and are not needed in the updated TSs. Also, definitions 1.37 and 1.38 were found to have been added out of alphabetical order in previous amendments, thus, reordering them in alphabetical order to conform to the format of the DEFINITIONS section will improve consistency with the TS Index and Definitions. The NRC staff, therefore, finds the proposed changes to be administrative in nature and thus acceptable.

3.3 Replace equation on TS page 2-7

FPL proposed to replace the following equation from TS page 2-7:

$$\Delta T \left\{\frac{1+\tau_1 S}{1+\tau_2 S}\right\} \left(\frac{1}{1+\tau_3 S}\right) \leq \Delta T_0 \{K_1 - K_2 \frac{(1+\tau_4 S)}{(1+\tau_5 S)} [T \left(\frac{1}{1+\tau_6 S}\right) - T'] + K_3 (P - P') - f_1(\Delta I)\}$$

with:

$$\Delta T \frac{(1+\tau_1 S)}{(1+\tau_2 S)} \left(\frac{1}{1+\tau_3 S}\right) \leq \Delta T_0 \left\{K_1 - K_2 \frac{(1+\tau_4 S)}{(1+\tau_5 S)} \left[T \frac{1}{(1+\tau_6 S)} - T'\right] + K_3 (P - P') - f_1(\Delta I)\right\}$$

The NRC staff reviewed the changes, regarding the proposed replacement of the equation on TS page 2-7, and verified that the new equation is in a more appropriate mathematical form due to improved word-processing capabilities. The changes are also consistent with the equation formatting of the Overtemperature ΔT formula as stated on page 3.3.1-17 of NUREG-1431, Vol. 1, Rev. 2, "Standard Technical Specifications Westinghouse Plants." The NRC staff finds that these proposed changes are administrative in nature. They improve the consistency and readability without altering the content or the intended interpretation of the formula in the TS and are, therefore, acceptable.

3.4 Replace equation on TS page 2-9

FPL proposed to replace the following equation from TS page 2-9:

$$\Delta T \quad \left\{\frac{1+\tau_1 S}{1+\tau_2 S}\right\} \quad \frac{\left(\quad 1 \quad\right)}{\left(1+\tau_3 S\right)} \leq \Delta T_0 \left\{K_4 - K_5 \frac{\left(\tau_7 S\right)}{1+\tau_7 S} \quad \frac{\left(\quad 1 \quad\right)}{\left(1+\tau_6 S\right)} T - K_6 \quad \left[T \frac{\left(\quad 1 \quad\right)}{\left(1+\tau_6 S\right)} - T''\right] - f_2\left(\Delta I\right)\right\}$$

with:

$$\Delta T \quad \frac{\left(1+\tau_1 S\right)}{\left(1+\tau_2 S\right)} \left(\frac{1}{1+\tau_3 S}\right) \leq \Delta T_0 \left\{K_4 - K_5 \quad \frac{\tau_7 S}{1+\tau_7 S} \left(\frac{1}{1+\tau_6 S}\right) T - K_6 \left[T \frac{1}{1+\tau_6 S} - T''\right] - f_2\left(\Delta I\right)\right\}$$

The NRC staff reviewed the changes, regarding the proposed replacement of the equation on TS page 2-9, and verified that the new equation is in a more appropriate mathematical form due to improved word-processing capabilities. The changes are also consistent with the equation formatting of the Overtemperature ΔT formula as stated on page 3.3.1-18 of NUREG-1431, Vol. 1, Rev. 2, "Standard Technical Specifications Westinghouse Plants." The NRC staff finds that these proposed changes are administrative in nature. They improve the consistency and readability without altering the content or the intended interpretation of the formula in the TS and are, therefore, acceptable.

3.5 Remove "Heat Tracing . . . 3/4 1-16" and entire page 3/4 1-16

FPL proposed the removal of "Heat Tracing . . . 3/4 1-16" from TS Index page iv and also the removal of page 3/4 1-16 in its entirety.

The licensee indicated that on July 16, 1991, the NRC issued SE "Turkey Point Units 3 and 4 Issuance of Amendments Re: Boric Acid Concentration Reduction (TAC Nos. 79192 and 79193)." This evaluation amended the license to allow the reduction of boric acid concentration and the removal of heat tracing requirements from boric acid tanks, boric acid makeup system piping, and valves. The licensee also indicated, as specified in TS 3.1.2.6, that the TS would no longer be applicable once boric acid tanks inventory and boric acid source and flow paths inventories have been diluted to less than or equal to 3.5 weight percent (wt%).

The NRC staff reviewed the associated amendments and verified that the concentration of boric acid in the boric acid system for both Units 3 and 4 has been reduced to less than 3.5 wt% due to plant changes/modifications completed on August 3, 1991. As the changes have diluted the boric acid concentration to less than or equal to 3.5 wt%, in accordance with the footnote of TS 3.1.2.6, the NRC staff finds that LCO 3.1.2.6 no longer meets the requirements of 10 CFR 50.36(c)(2)(ii) and can be removed from the TSs. Therefore, the removal of the applicable references to the boric acid heat tracing system requirements and nonexistent specifications are administrative in nature and are acceptable.

3.6 Change the title of Figure 3.4-3

FPL proposed, on TS page 3/4 4-32, to change the title of Figure 3.4-3 from "FPL 32 EFPY HEATUP CURVES" to "FPL 32 EFPY COOLDOWN CURVES."

The licensee indicated that Figure 3.4-3, on page 3/4-32, was erroneously labeled as a heatup curve when it is actually intended to be used for cooldown limitations requirements. The NRC staff examined Figure 3.4-3 and verified that the title "FPL 32 EFPY COOLDOWN CURVES" is consistent with the intended meaning of the graph. As such, the NRC staff finds that the title change is administrative in nature and is, therefore, acceptable.

3.7 Correct TS page 3/4 4-36, Limiting Condition for Operation TS 3.4.9.3

The licensee proposed to delete the words "below an average coolant temperature of 275°F" from the Limiting Condition for Operation (LCO) TS 3.4.9.3 on page 3/4 4-36. The licensee also stated that the wording of TS 3.4.9.3 was not clear as written, and that deleting the words "below an average coolant temperature of 275°F" would achieve improved consistency and content in the TS.

During the review, the NRC staff noted an inconsistency between the licensee's TS wording and the nomenclature in other documents. The phrase "below an average coolant temperature of 275°F," in the TS 3.4.9.3 LCO, APPLICABILITY, and ACTION statements, was inconsistent with the applicable phrases of the TS BASES, 10 CFR Section 50, Appendix G, and the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) Section XI, Appendix G. The TSs use the nomenclature of "below an average coolant temperature of 275°F" while the TS BASES, 10 CFR, and the ASME Code all use "less than or equal to a coolant temperature of 275°F." As "less than or equal to" is not equivalent to "below an average," the NRC staff concludes that the change is more than administrative in nature.

The NRC staff reviewed previously approved changes made to TS 3.4.9.3, such as those made in an SE dated June 28, 1994, in which the licensee requested to adopt the TS suggested in Generic Letter (GL) 90-06, "Resolution of Generic Issue 70, 'Power-Operated Relief Valve Reliability'." In the 1994 submittal, the licensee proposed editorial changes to reflect implementation of the GL. From this review the NRC staff determined that the change from "less than or equal to" to "below an average" was inconsistent with the language in the GL, and had not been reviewed and approved in the associated SE.

Upon notification of the inconsistency in their nomenclature, FPL acknowledged this condition and entered it into their corrective action program. Subsequently, the licensee submitted improved TS pages that conformed applicable TS 3.4.9.3 statements to the "less than or equal to" nomenclature. All applicable statements will now read "when the temperature of any RCS cold leg is less than or equal to 275°F," thus, returning the TSs to the existing licensing basis. The NRC staff finds that this change improves the consistency and readability of the TSs, makes the TS 3.4.9.3 statements consistent with the GL-90-06 license amendment approval, the existing TS BASES, 10 CFR, and the ASME Code, and is, therefore, acceptable.

The NRC staff reviewed the licensee proposed change and verified that the TS 3.4.9.3 LCO phrase "below an average coolant temperature of 275°F" is specified in the APPLICABILITY section and is, thus, redundant. As such, the NRC staff finds the deletion of the LCO phrase to be administrative in nature. The deletion would improve the consistency and readability without altering the content or the intended interpretation of the TSs and is, therefore, acceptable.

3.8 Correct TS page 3/4 6.1, Surveillance Requirement 4.6.1.1 a

FPL proposed, on TS page 3/4 6-1 of Surveillance Requirement 4.6.1.1 a., to replace the phrase [secured in their positions;] with [secured in their closed positions;].

The licensee indicated that by specifying that the components should be "secured in their closed positions," it would remove any confusion as to what the intended final position of any valves, blind flanges, or deactivated automatic valves should be in.

The NRC staff reviewed the proposed change and determined that the addition of the word "closed," in the applicable statement above, is conservative in nature and removes any confusion as to the intended final position of the valves, blind flanges, or deactivated automatic valves. Additionally, the NRC staff has determined that the addition of the word "closed" further conforms to the content of ACTION A.1 of TS 3.6.3, Containment Isolation Valves, of NUREG-1431, Vol. 1, Rev. 2, of the Standard Technical Specifications for Westinghouse Plants. The NRC staff finds this change to be consistent with the intent, therefore, it is administrative in nature and acceptable.

3.9 Correct TS page 3/4 7-11 and TS 3.7.1.6 ACTION Statement b

FPL proposed, on TS page 3/4 7-11, TS 3.7.1.6 Action Statement b., to replace the phrase [With both Standby Steam Generator Feedwater Pumps, restore at least one pump to OPERABLE status . . .] to [With both Standby Steam Generator Feedwater Pumps inoperable, restore at least one pump to OPERABLE status . . .].

The licensee indicated that the addition of the word "inoperable" to the TS page 3/4 7-11 and TS 3.7.1.6 ACTION Statement b, would grammatically correct the phrase and would provide greater consistency and readability to the TSs.

The NRC staff reviewed the change and determined that the addition of the word "inoperable" is consistent with the intent of the TS and is administrative in nature. The NRC staff finds that the change will provide greater consistency and readability to the TS and is, therefore, acceptable.

3.10 Correct TS page 3/4 9-7

FPL proposed, on TS page 3/4 9-7, to delete [*] from the TS 3.9.7 and associated footnote at the bottom of page 3/4 9-7.

The licensee indicated that on August 29, 1984, the NRC approved License Amendments 108 and 102 for the prohibition of travel of heavy loads over irradiated fuel assemblies with the exception of a temporary crane for use during proposed reracking of the spent fuel pools. The licensee also indicated that due to changes/modifications that completed the installation of new high-density fuel racks in the Unit 3 and 4 spent fuel pits on September 7, 1989, the relevant exception to the TSs is no longer needed.

The NRC staff reviewed the associated amendments and verified that License Amendments 108 and 102 and the changes/modifications that installed new high-density fuel racks in Units 3 and 4 were relevant documentation associated with the footnote at the bottom of page 3/4 9-7.

Also, the NRC staff verified that due to the completed rerack in the Units 3 and 4 spent fuel pits on September 7, 1989, the relevant exception to the TSs is no longer needed. The NRC staff finds that this change is administrative in nature, and maintains the intent of the TSs and is, therefore, acceptable.

3.11 Correct TS page 3/4 9-12

FPL proposed that the note at the bottom of TS page 3/4 9-12, replace [safety evaluation] with [10 CFR 50.59 evaluation].

The licensee proposed to replace the words “safety evaluation” with “10 CFR 50.59 evaluation” in order to ensure that the intent of the TS will be met and that a complete and adequate safety assessment will be performed in accordance with the requirements of 10 CFR 50.59.

The NRC staff has reviewed the change and has determined that the wording “10 CFR 50.59 evaluation” will not alter the intent of the TS. The NRC staff finds that the change is administrative in nature and will improve the clarity and intent of the TS note. The change is, therefore, acceptable.

4.0 STATE CONSULTATION

Based upon a letter dated May 2, 2003, from Michael N. Stephens of the Florida Department of Health, Bureau of Radiation Control, to Brenda L. Mozafari, Senior Project Manager, U.S. Nuclear Regulatory Commission, the State of Florida does not desire notification of issuance of license amendments.

5.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an Environmental Assessment and Finding of No Significant Impact was published in the *Federal Register* on August 29, 2003 (68 FR 52063). Accordingly, based on the Environmental Assessment, the Commission has determined that the issuance of these amendments will not result in any environmental impacts other than those evaluated in the Final Environmental Statement. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (68 FR 5677).

6.0 CONCLUSION

Based on the evaluation in Section 3.0 above, the NRC staff concludes that the proposed TS changes will improve the consistency and readability of the TSs without altering their content or the intended interpretation, and is therefore acceptable.

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Michael I. Dudek, NRR
Eva A. Brown, NRR

Date: October 21, 2003

Mr. J. A. Stall
Florida Power and Light Company

TURKEY POINT PLANT

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