

September 17, 2003

Mr. R. T. Ridenoure
Division Manager - Nuclear Operations
Omaha Public Power District
Fort Calhoun Station FC-2-4 Adm.
Post Office Box 550
Fort Calhoun, NE 68023-0550

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 - UPCOMING STEAM GENERATOR
TUBE INSERVICE INSPECTION (TAC NO. MC0266)

Dear Mr. Ridenoure:

Inservice inspections of steam generator (SG) tubes play a vital role in assuring that adequate structural integrity of the tubes is maintained. As required by the plant's technical specifications, reporting requirements range from submitting a special report, within 15 days following completion of each inservice inspection of SG tubes, that identifies the number of tubes plugged and/or repaired; to submitting a special report, within 12 months following completion of the inspection, that provides complete results of the SG tube inservice inspection. The special report containing the complete results of the inspection shall include the following:

1. Number and extent of tubes inspected.
2. Location and percent of wall-thickness penetration for each indication of an imperfection.
3. Identification of tubes plugged and/or repaired.

A phone conference will be arranged with members of your staff to discuss the ongoing results of the SG tube inspections to be conducted during the upcoming Fort Calhoun Station, Unit 1 refueling outage. We would like to have this phone call after the majority of the tubes have been inspected, but before the SG inspection activities have been completed. The preferable time would be when the SG inspection is approximately 75 percent complete. Enclosed is a list of discussion points to facilitate this phone conference.

R. Ridenoure

-2-

The staff plans to document a brief summary of the conference call as well as any material that you may have provided to the staff in support of the call.

Sincerely,

/RA/

Alan B. Wang, Project Manager, Section 2
Project Directorate IV
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosure: List of Discussion Points

cc w/encl: See next page

R. Ridenoure

-2-

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Enclosure: List of Discussion Points

cc w/encl: See next page

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NRR-106

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STEAM GENERATOR TUBE INSPECTION DISCUSSION POINTS

PREPARED BY THE OFFICE OF NUCLEAR REACTOR REGULATION

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT 1

DOCKET NO. 50-285

The following discussion points have been prepared to facilitate the telephone conference to be arranged with the Omaha Public Power District (OPPD/the licensee) to discuss the results of the steam generator (SG) tube inspections to be conducted during the upcoming Fort Calhoun Station, Unit 1 refueling outage. This phone call will be scheduled towards the end of the planned SG tube inspection interval, but before the unit exits its refueling outage.

The staff plans to document a brief summary of the conference call as well as any material that you may have provided to the staff in support of the call.

1. Discuss whether any primary-to-secondary leakage existed in this unit prior to shutdown.
2. Discuss the results of secondary side pressure tests.
3. For each steam generator, provide a description of areas examined, including the expansion criteria utilized and type of probe used in each area. Also, be prepared to discuss your inspection of the tube within the tubesheet, particularly the portion of the tube below the expansion/transition region.
4. Discuss any exceptions taken to the industry guidelines.
5. Provide a summary of the number of indications identified to date of each degradation mode and steam generator tube location (e.g., tube support plate, top-of-tubesheet, etc.). Also provide information, such as voltages, and estimated depths and lengths of the most significant indications.
6. Describe repair/plugging plans for the SG tubes that meet the repair/plugging criteria.
7. Discuss the previous history of SG tube inspection results, including any "look backs" performed; specifically for significant indications or indications where look backs are used in support of dispositioning (e.g., manufacturing burnish marks).
8. Discuss, in general, new inspection findings (e.g., degradation mode or location of degradation new to this unit).

9. Discuss your use or reliance on inspection probes (eddy current or ultrasonic) other than bobbin and typical rotating probes, if applicable.
10. Describe in-situ pressure test plans and results, if applicable and available, including tube selection criteria.
11. Describe tube pull plans and preliminary results, if applicable and available; include tube selection criteria.
12. Discuss the assessment of tube integrity for the previous operating cycle (i.e., condition monitoring).
13. Provide the schedule for steam generator-related activities during the remainder of the current outage.
14. Discuss the following regarding loose parts:
 - what inspections are performed to detect loose parts,
 - a description of any loose parts detected and their location within the SG,
 - if the loose parts were removed from the SG,
 - indications of tube damage associated with the loose parts, and
 - the source or nature of the loose parts if known.
15. Once Through Steam Generators - if you have Babcock and Wilcox (B&W) welded plugs installed in the steam generators, be prepared to discuss the actions taken in response to Framatome's notification of the effect of tubesheet hole dilation on the service life of B&W welded plugs.
16. Once Through Steam Generators - describe your inspection/plugging plans with respect to the industry identified severed tube issue (NRC Information Notice (IN) 2002-02, "Recent Experience with Plugged Steam Generator Tubes" and IN 2002-02, Supplement 1).
17. If steam generators contain thermally treated tubing (Alloy 600 or 690), discuss actions taken (if any) based on Seabrook's recent findings (Reference Information Notice (IN) 2002-21, "Axial Outside Diameter Cracking Affecting Thermally Treated Alloy 600 Steam Generator Tubing")?

Ft. Calhoun Station, Unit 1

cc:

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