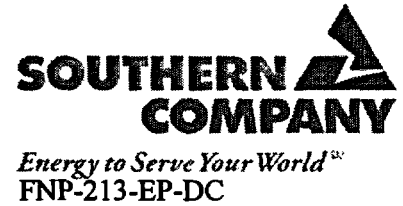


Southern Nuclear
Operating Company, Inc.
Post Office Drawer 470
Ashford, Alabama 36312

Date: September 11, 2003



Director, Office of NRC
US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attention: Mr. Jim McKnight

Dear Sir:

ATTACHED YOU WILL FIND THE NEW REVISIONS FOR THE FOLLOWING PROCEDURES
FOR FARLEY NUCLEAR PLANT.

FNP-0-EP-0.0 REVISION 39
FNP-0-EIP-9.5 REVISION 8
FNP-0-EIP-10.0 REVISION 34
FNP-0-EIP-14.0 REVISION 18
FNP-0-EIP-20.0 REVISION 7
FNP-0-EIP-26.0 REVISION 45
FNP-0-EIP-27.0 REVISION 34
FNP-0-EIP-27.1 REVISION 6
FNP-0-EIP-28.0 REVISION 5
FNP-0-EIP-28.1 REVISION 2
FNP-0-EIP-29.0 REVISION 8

PLEASE REPLACE YOUR COPIES WITH THE ATTACHED REVISED COPIES.
IF YOU HAVE ANY QUESTIONS, PLEASE CALL ME AT 334-988-5256 EXTENSION 3439.

Sincerely,

A handwritten signature in black ink, appearing to read "Joey Hudspeth".

Joey Hudspeth
Document Control Supervisor

JBH:llb
RTYPE: A4.54

A045

04/08/03 16:08:33

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FNPP-0-EIP-28.0
March 17, 2003
Revision 5

FARLEY NUCLEAR PLANT

EMERGENCY PLAN IMPLEMENTING PROCEDURE 28.0

FNPP-0-EIP-28.0

DE-ESCALATION

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PROCEDURE USAGE REQUIREMENTS per FNPP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	FIGURES 1 & 2
Information Use	ALL OTHERS

Approved:



Nuclear Plant General Manager

Date Issued

8-29-03

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DE-ESCALATION
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DE-ESCALATION

1.0 Purpose

The purpose of this procedure is to provide guidance for de-escalating from one emergency classification to a lower level emergency classification, or to close out all emergency classifications completely. This procedure will normally be entered after assessing plant and radiological conditions in FNP-0-EIP-9.0.

2.0 References

See Table 1

NOTE: PRIOR TO DE-ESCALATING OR CLOSING OUT ANY EMERGENCY CLASSIFICATION, THE EMERGENCY DIRECTOR AND THE EOF MANAGER MUST CONCUR ON THE DECISION.

3.0 DETERMINE THE HIGHEST REQUIRED EMERGENCY CLASSIFICATION

- 3.1 Using the plant status criteria listed in Guideline 1 through Guideline 4 of FNP-0-EIP-9.0, "Emergency Classification and Actions", determine the highest emergency classification which is currently indicated by plant conditions. Indicate the required emergency classification at step A on Figure 1.
- 3.2 Determine the highest emergency classification which is currently indicated by plant effluents. Use the EDCM dose/dose rate values from one of the following procedures, listed in the order of preference and comparing to the criteria in Guideline 1 and Guideline 2 of FNP-0-EIP-9.0, "Emergency Classification and Actions". Indicate the required emergency classification at step B on Figure 1.
 - 3.2.1 EIP-9.1, "Automated Dose Assessment Method"
 - 3.2.2 EIP-9.3, "Personal Computer-Automated Dose Assessment Method"
- 3.3 If a General Emergency or Site Area Emergency is indicated by EDCM per step 3.2, go to step 3.5.
- 3.4 If a General Emergency or Site Area Emergency is NOT indicated by EDCM per step 3.2 above, go to EIP-9.5, "Emergency Classification Based on ODCM" and compare the ODCM values obtained to the criteria in Guideline 3 and Guideline 4 of FNP-0-EIP-9.0, "Emergency Classification and Actions". Indicate the required emergency classification at step C on Figure 1.
- 3.5 Review the required emergency classification listed in steps A, B and C of Figure 1, and list the highest required emergency classification at step D of Figure 1.

4.0 DETERMINE IF THE CORRECT EMERGENCY CLASSIFICATION IS CURRENTLY BEING USED.

4.1 Determine the currently declared emergency classification being used in the plant and list in step E of Figure 1.

4.2 If the required emergency classification (Fig. 1, step D) is higher than the currently declared emergency classification (Fig. 1, step E), return to FNP-0-EIP-9.0 and declare the higher emergency classification.

**4.3 If the required emergency classification (Figure 1, step D) is lower than the currently declared emergency classification (Fig. 1, step E),
and**

**plant conditions are stable or improving,
and**

**applicable plant effluent monitors are stable or decreasing,
then**

de-escalation to a lower classification based on acceptable plant conditions and radiological effluents may be considered.

**4.4 If an emergency classification is NOT REQUIRED (Figure 1, step D),
and**

**plant conditions are stable or improving,
and**

**applicable plant effluent monitors are stable or decreasing,
then**

close-out of the emergency classification, based on acceptable plant conditions and radiological effluents, may be considered.

4.5 If the decision is made to consider downgrading or closing out the current emergency classification, then proceed to step 5.

4.6 If the decision is made to remain in the currently declared emergency classification, return to the appropriate guideline in FNP-0-EIP-9.0.

5.0 DOWNGRADE OR CLOSE OUT THE EMERGENCY CLASSIFICATION

NOTE: FIGURE 2 MAY BE USED TO DOCUMENT THE FOLLOWING STEPS OF THIS PROCEDURE.

5.1 IF the currently required emergency classification (Fig. 1, step D) is lower than the currently declared emergency classification (Fig. 1, step E), or there is no required emergency classification (Fig. 1, step D), THEN obtain verbal concurrence from the EOF Manager and the Emergency Director to downgrade or closeout the currently declared emergency classification.

5.2 IF the decision is made to downgrade or closeout an emergency classification, THEN:

5.2.1 Verbally, brief offsite authorities:

- A. NRC**
- B. Alabama Radiation Control Division**
- C. Houston Co. Emergency Management Agency**
- D. Georgia Emergency Management Agency**
- E. Early Co. Emergency Management Agency**
- F. Alabama Emergency Management Agency**
- G. Florida Department of Emergency Management**

5.2.2 Transmit a follow-up written report to (Figure 6 of FNP-0-EIP-9.0):

- A. Alabama Radiation Control Division**
- B. Houston Co. Emergency Management Agency**
- C. Georgia Emergency Management Agency**
- D. Early Co. Emergency Management Agency**
- E. Alabama Emergency Management Agency**
- F. Florida Department of Emergency Management**

5.2.3 Inform the following SNC or APCo organizations or individuals as applicable:

- A. Control Room staff
- B. Plant staff (public address)
- C. SNC Duty Manager
- D. TSC staff
- E. EOF staff
- F. Emergency Coordinator
- G. CEOC Staff
- H. Public Information Site Coordinator

5.3 If there is still a declared emergency, return to the appropriate guideline of FNP-0-EIP-9.0. The initial notification portion of the Guideline does not have to be done if the classification has been downgraded.

5.4 If the emergency classification has been closed out, then proceed to FNP-0-EIP-28.1 ("Recovery") for further actions.

TABLE 1

REFERENCES

1. Joseph M. Farley Nuclear Plant Emergency Plan
2. FNP-0-EIP-9.0, Emergency Classification and Actions
3. FNP-0-EIP-9.1, Automated Dose Assessment Method
4. FNP-0-EIP-9.3, Personnel Computer - Automated Dose Assessment Method
5. FNP-0-EIP-9.5, Emergency Classification Based on ODCM

FIGURE 1**REQUIRED EMERGENCY CLASSIFICATION**

- A. EMERGENCY CLASSIFICATION REQUIRED BY PLANT CONDITIONS (step 3.1)**
- ☐ GENERAL EMERGENCY
 - ☐ SITE AREA EMERGENCY
 - ☐ ALERT
 - ☐ NOTIFICATION OF UNUSUAL EVENT
 - ☐ NONE REQUIRED
- B. EMERGENCY CLASSIFICATION REQUIRED BY EDCM (step 3.2)**
- ☐ GENERAL EMERGENCY
 - ☐ SITE AREA EMERGENCY
 - ☐ NONE REQUIRED
- C. EMERGENCY CLASSIFICATION REQUIRED BY ODCM (step 3.4)**
- ☐ ALERT
 - ☐ NOTIFICATION OF UNUSUAL EVENT
 - ☐ NONE REQUIRED
- D. HIGHEST REQUIRED EMERGENCY CLASSIFICATION BASED ON PLANT OR RADIOLOGICAL CONDITIONS FROM A, B OR C (step 3.5)**
- ☐ GENERAL EMERGENCY
 - ☐ SITE AREA EMERGENCY
 - ☐ ALERT
 - ☐ NOTIFICATION OF UNUSUAL EVENT
 - ☐ NONE REQUIRED
- E. CURRENTLY DECLARED EMERGENCY CLASSIFICATION (step 4.1)**
- ☐ GENERAL EMERGENCY
 - ☐ SITE AREA EMERGENCY
 - ☐ ALERT
 - ☐ NOTIFICATION OF UNUSUAL EVENT
 - ☐ NONE REQUIRED

(FIGURE 1 - CONTINUED)

F. RESULTS

- ☐ **UPGRADE TO A HIGHER EMERGENCY CLASSIFICATION. GO TO THE APPROPRIATE FNP-0-EIP-9.0 GUIDELINE. (step 4.2)**
- ☐ **REMAIN IN THE CURRENT EMERGENCY CLASSIFICATION GO TO THE APPROPRIATE FNP-0-EIP-9.0 GUIDELINE. (step 4.6)**
- ☐ **CONSIDER DE-ESCALATING TO A LOWER EMERGENCY CLASSIFICATION. GO TO FIGURE 2. (step 4.3)**
- ☐ **CONSIDER CLOSING OUT CURRENT EMERGENCY CLASSIFICATION. GO TO FIGURE 2. (step 4.4)**

FIGURE 2**DOWNGRADE OR CLOSEOUT
EMERGENCY CLASSIFICATION****A. RECOMMENDATION FROM FIGURE 1 STEP F**

- ☐ UPGRADE TO A HIGHER EMERGENCY CLASSIFICATION. GO TO THE APPROPRIATE FNP-0-EIP-9.0 GUIDELINE. (step 4.2)
- ☐ REMAIN IN THE CURRENT EMERGENCY CLASSIFICATION GO TO THE APPROPRIATE FNP-0-EIP-9.0 GUIDELINE. (step 4.6)
- ☐ CONSIDER DE-ESCALATING TO A LOWER EMERGENCY CLASSIFICATION. (step 4.3)
- ☐ CONSIDER CLOSING OUT CURRENT EMERGENCY CLASSIFICATION. (step 4.4)

B. VERBAL CONCURRENCE (BOTH ED AND EOF MGR REQUIRED) (step 5.1)EOF MANAGER _____
nameEMERGENCY DIRECTOR _____
name**CAUTION: STEP BE MUST BE AGREED TO BY BOTH THE ED AND EOF Manager
PRIOR TO PROCEEDING.****C. VERBALLY BRIEF OFFSITE AUTHORITIES (step 5.2.1).**

- | | | |
|-------------------------------|---------------------------------|-------------------------------|
| <input type="checkbox"/> NRC | <input type="checkbox"/> HC EMA | <input type="checkbox"/> FDEM |
| <input type="checkbox"/> ARCD | <input type="checkbox"/> EC EMA | |
| <input type="checkbox"/> GEMA | <input type="checkbox"/> AEMA | |

D. TRANSMIT A FOLLOW-UP WRITTEN REPORT fig. 6 FNP-0-EIP-9.0) TO (step 5.2.1).

- | | | |
|-------------------------------|---------------------------------|-------------------------------|
| <input type="checkbox"/> ARCD | <input type="checkbox"/> HC EMA | <input type="checkbox"/> AEMA |
| <input type="checkbox"/> GEMA | <input type="checkbox"/> EC EMA | <input type="checkbox"/> FDEM |

(FIGURE 2 - CONTINUED)

E. INFORM THE FOLLOWING SNC OR APCO ORGANIZATIONS OR INDIVIDUALS, AS APPLICABLE (step 5.2.3).

- | | |
|---|---|
| <input type="checkbox"/> CONTROL ROOM STAFF | <input type="checkbox"/> PLANT STAFF (PUBLIC ADDRESS) |
| <input type="checkbox"/> SNC DUTY MANAGER | <input type="checkbox"/> TSC STAFF |
| <input type="checkbox"/> EOF STAFF | <input type="checkbox"/> EC IN EOC |
| <input type="checkbox"/> CEOC STAFF | <input type="checkbox"/> PIEOC |

F. IF THE EMERGENCY CLASSIFICATION HAS BEEN CLOSED OUT, THEN PROCEED TO FNP-0-EIP-28.1, "RECOVERY", FOR FURTHER ACTIONS. (step 5.4).

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FNPP-0-EIP-28.1
March 17, 2003
Version 2

FARLEY NUCLEAR PLANT

EMERGENCY PLAN IMPLEMENTING PROCEDURE 28.1

FNPP-0-EIP-28.1

RECOVERY

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PROCEDURE USAGE REQUIREMENTS PER FNPP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	
Information Use	All

Approved:



Nuclear Plant General Manager

Date Issued

8-29-03

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RECOVERY

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RECOVERY

1.0 Purpose

This procedure delineates the actions to be taken to restore the plant and site to its pre-emergency status as a result of an emergency classification. This procedure will normally be used after the emergency classification has been closed out, per FNP-0-EIP-28.0.

2.0 References

2.1 Joseph M. Farley Nuclear Plant Emergency Plan

2.2 Title 10, Code of Federal Regulations, Part 20

2.3 FNP-0-EIP-28.0

3.0 Planning

3.1 Due to the unforeseeable conditions that would exist in an emergency condition, specific recovery criteria and procedures will be developed when required. Maximum protection for plant personnel and the general public will be priority, and reasonable efforts will be made to restore the affected unit and continue operation of the unaffected unit.

3.2 The Nuclear Plant General Manager, with the aid of the plant organization, shall develop specific recovery procedures considering such activities as repair, decontamination, disposal, test and startup of restored facilities.

3.3 As a prerequisite for planning recovery actions, damages and radiological conditions in the affected area(s) must be assessed.

3.4 The company emergency organization shall be available to provide administrative, technical and logistical support to the plant emergency organization. The following support is available.

3.4.1 Logistics

The Administrative Support Supervisor will locate and procure supplies, tools, machinery, and vehicles.

3.4.2 Manpower

The Administrative Support Supervisor will locate and provide company and outside sources of maintenance, instrument and labor personnel.

3.4.3 Health Physics

The Recovery Support Supervisor will provide assistance in the areas of environmental monitoring and radiation protection.

3.4.4 Engineering Resources

The Engineering Supervisor will provide for offsite engineering resources directed toward design modification, major repair and engineering evaluations.

3.4.5 Licensing Issues

The Licensing Supervisor will coordinate resolution of regulatory matters and licensing issues.

3.4.6 Medical

The Medical Advisor will provide assistance in handling casualties.

3.4.7 Legal

The Legal Advisor will provide advice on all legal matters concerning the emergency.

4.0 TSC

- 4.1** Turn over to the Control Room those functions that are still in progress in the TSC that would normally be done in the Control Room in a non-emergency condition.
- 4.2** Turn over to the EOF those functions being performed in the TSC that are still required to be performed
- 4.3** When all functions in the TSC have been terminated or turned over, deactivate the TSC and staff, and restore to the pre-emergency condition.

5.0 EOF

- 5.1** If there is a need to have a formal recovery organization on site, it should be located in the EOF.
- 5.2** On-site personnel will normally be relieved of EOF duties to return to their normal functions of operating the plant.

- 5.3 During recovery, the EOF Manager position will normally be filled by corporate staff.
- 5.4 The normal staff for the EOF, as described in FNP-0-EIP-27.0, is not required during the recovery phase.
- 5.5 EOF staff can be prescribed by the EOF Manager. Consideration should be given to those functions and priorities listed in step 3.0.
- 5.6 The decision to totally deactivate the EOF can be made by the EOF Manager when, in his opinion, the EOF is no longer required.
- 5.7 When the decision to deactivate the EOF is made, it should be restored to its normal pre-emergency condition.

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FNP-0-EIP-29.0
March 17, 2003
Revision 8

FARLEY NUCLEAR PLANT

EMERGENCY PLAN IMPLEMENTING PROCEDURE 29.0

FNP-0-EIP-29.0

LONG TERM DOSE ASSESSMENT

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PROCEDURE USAGE REQUIREMENTS PER FNP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	
Information Use	ALL

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Nuclear Plant General Manager

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LONG TERM DOSE ASSESSMENT

1.0 Purpose

To provide the Technical Support Center and Emergency Operations Facility Staff guidance in performance of long term dose assessment, dose projection model validation and dose projection back calculation.

2.0 References

See Table 1.

3.0 General

3.1 The procedure for long term dose assessment is provided at step 4.

3.2 The procedure for validation of dose assessment model or detecting unmonitored releases is provided at step 5.

3.3 The procedure for back calculating source term and dose for unmonitored releases or non-conservative dose projections found when performing step 5 is provided at step 6.

CAUTION: LONG TERM DOSE PROJECTION PERFORMED USING STEP 4 OF THIS PROCEDURE SHOULD NOT BE USED TO CHANGE THE EMERGENCY CLASSIFICATION. THIS INFORMATION SHOULD ONLY BE USED FOR LONG TERM PLANNING.

4.0 Long Term Dose Projection

4.1 Long term dose assessment is required only if the Emergency Director (ED) or EOF Manager desire to have it performed.

4.2 If weather conditions are not expected to change drastically, the Meteorological Information and Dose Assessment System (MIDAS) program can display and print plume maps and reports for .25 hour, 4 hour, 24 hour and 96 hour projections. These projections can be used for long range planning, with no further action required in this procedure.

4.3 Perform the following steps if weather conditions are expected to change and long term dose assessment is desired.

- 4.4 Obtain current and long term weather forecasts.
 - 4.4.1 Obtain from the MIDAS dose projection that is in progress, the current meteorological data at the time for the projection to start, and record on Figure 1.
 - 4.4.2 Use FNP-0-EIP-9.2 as guidance for obtaining weather forecasts.
 - 4.4.3 Forecasts for a maximum of 24 hours into the future can be performed using MIDAS.
 - 4.4.4 Complete Figure 2 forecast information for wind speed, wind direction, temperature, cloud cover and rainfall for desired projection time.
 - 4.4.5 Use predicted cloud cover and FNP-0-EIP-9.2 as guidance to determine the predicted stability class. Record the predicted stability class (es) on Figure 2.
- 4.5 Obtain current and long term radiation release forecasts.
 - 4.5.1 Obtain current radiation monitor and flow data at the time for the projection to start from the MIDAS dose projection that is in progress, and record on Figure 1.
 - 4.5.2 Estimate the radiation monitor and flow readings for the release points that are expected to be releasing and record on Figure 1.
- 4.6 Using the guidance of FNP-0-EIP-9.3, determine the long range dose assessment. The following exceptions to the guidance should be used:
 - 4.6.1 When entering meteorological and radiation monitor data, use the information on Figure 1 to fill in all of the fifteen minute values from current time to the forecast time.
 - 4.6.2 Remaining duration should be set to zero.
 - 4.6.3 The printed EMERGENCY NOTIFICATION form should be ignored.
 - 4.6.4 The projection time for the Total Effective Dose Equivalent (TEDE) and Thyroid Committed Dose Equivalent (Thyroid CDE) plots should be set to 0.25 hours and printed.
 - 4.6.5 The TEDE and Thyroid CDE plots will provide the projected information necessary for long range planning.

5.0 Dose Assessment Model Validation

- 5.1 The dose assessment models used are a conservative projection and will typically overestimate the doserate compared to actual values.
- 5.2 The dose assessment models used assume a 100% ground release mode. The assumed ground release mode will typically overestimate the doserate if the release is an elevated or mixed mode release.
- 5.3 For an elevated release, the plume may not actually touch down until it is beyond the site boundary.
- 5.4 Compare for discrepancies the dose rate readings obtained in the field to the dose rate readings indicated on the Effective Dose Equivalent (EDE) or Field Monitor (FM) dose rate plot that is printed out when doing dose projection using the Meteorological Information and Dose Assessment System (MIDAS) program.
- 5.5 If the MIDAS program is not in use and dose projections are being performed using the Offsite Dose Calculation Method (ODCM) per FNP-0-EIP-9.5, then compare the dose rate readings obtained in the field to the projected FNP-0-EIP-9.5 dose rates for discrepancies.
- 5.6 If plant effluent monitors do not indicate that there is a radioactive release in progress and field monitoring teams are in the field taking readings, then compare the dose rate readings obtained in the field to normal background readings to determine if there is an unmonitored release in progress.
- 5.7 If the comparison of dose rates per steps 5.4, 5.5 and 5.6 show that there is an unmonitored release or that the dose projection model is projecting non-conservative values, then the problem must be resolved by the Dose Assessment Supervisor and EOF Manager or the Engineering Supervisor and Emergency Director (ED).
- 5.8 The Back Calculation of step 6.0 can be used to help evaluate dose projection.

CAUTION: THE Total Effective Dose Equivalent (TEDE) AND Thyroid Committed Does Equivalent (Thyroid CDE) PLOTS PRINTED BY THIS SECTION OF THE PROCEDURE SHOULD NOT BE USED AS THE SOLE SOURCE FOR MAKING EMERGENCY CLASSIFICATIONS. PLANT CONDITIONS AND ACTUAL RADIOLOGICAL CONDITIONS MUST ALSO BE CONSIDERED.

6.0 Dose Assessment Back Calculation

- 6.1 Dose Assessment Back Calculation is required if the Emergency Director (ED) or EOF Manager desire to have it performed, or step 5 has shown that there is a discrepancy between field readings and the dose assessment model or there are indications of an unmonitored release.

NOTE: GENERAL INSTRUCTIONS FOR USING the Meteorological Information and Dose Assessment System (MIDAS) ARE INCLUDED IN FNP-0-EIP-9.3, TABLES 2-6.

- 6.2 Start the MIDAS program on a computer that is not currently being used for dose calculations.
- 6.3 From the SITE SELECTION SCREEN, select PLANT FARLEY and CONFIRM.
- 6.4 From the FUNCTION SELECTION SCREEN, select ACCIDENT DOSE CALCULATIONS and CONFIRM.
- 6.5 From the ACCIDENT DOSE CALCULATIONS SCREEN, select BACK CALCULATION (MENU E) and CONFIRM.
- 6.6 From the SCENARIO DATA TABLE CONTROL SCREEN, select START NEW SCENARIO and CONFIRM.
- 6.7 From the MET DATA SCREEN, enter the meteorological (MET) data for all required inputs. Enter met data starting 15 minutes before the estimated time that the radioactive release started until the current time. After the data has been entered, press "x" to continue.
- 6.8 From the DBA ACCIDENT TYPE SELECTION SCREEN, select the same accident that is currently being used in dose projection, or select an accident type using FNP-0-EIP-9.3, Figure 2, and CONFIRM.

- 6.9 From the FIELD MONITOR PARAMETER SELECTION SCREEN perform the following:

Verify GROUND is selected with a white background.

Select FIELD MONITOR READINGS, then from the number pad, enter the closed window reading obtained by the Field Monitoring Teams (FMTs) in millirem/hour (mrem/hr) and select enter.

Select DISTANCE FROM PLANT (MILES), then from the number pad, enter the distance (in miles) from the plant at which the closed window reading was obtained by the RMTs, and select enter.

NOTE: The time selected in this next step must fall within the timeframe that met data was entered in step 6.7 above.

Select TIME FROM START TO CURRENT, then from the number pad select the number of minutes from the estimated start of the release to the current time and select enter.

Select DIRECTION OF FIELD SAMPLE, then from the number pad select the direction that the closed window reading was recorded in degrees or compass point and select enter

After all data has been input, select CONFIRM.

NOTE: If the remaining duration of the continuing release is not known, then enter 240 minutes for the remaining duration in the following step.

- 6.10 From the RELEASE TIMING SELECTION SCREEN, select REMAINING DURATION then from the number pad enter the number of minutes from now that the release is expected to continue and select CONFIRM.
- 6.11 After the calculation has been completed, RELEASE POINT DATA SCREEN 1 will be displayed. This screen can be printed (press SHIFT-PRINT SCREEN) if desired. Then select CONTINUE.
- 6.12 RELEASE POINT DATA SCREEN 2 will be displayed. This screen can be printed (press SHIFT-PRINT SCREEN) if desired. Then select CONTINUE.
- 6.13 RELEASE POINT DATA SCREEN 3 will be displayed. This screen can be printed (press SHIFT-PRINT SCREEN) if desired. Then select CONTINUE.

- 6.14 The printer will now print out an Emergency Notification Form with the Radioactive Release data printed on it. Refer to CAUTION prior to Step 6.0.
- 6.15 The screen will display the TEDE plume map with scenario information. Verify that the map is a four hour projection that has the 10 mile Emergency Planning Zones (EPZ) displayed and any other information that is desired by the Engineering Supervisor or the Dose Assessment Supervisor. Print this map by selecting SHIFT-PRINT SCREEN if it is desired by the Dose Assessment Supervisor or Engineering Supervisor.
- 6.16 From the TEDE plume map sequentially select CONTINUE, NEXT REPORT then THY CDE DOSE PLOT.
- 6.17 The screen will display the Thyroid CDE plume map with scenario information. Verify that the map is a four hour projection that has the 10 mile EPZ zones displayed and any other information that is desired by the Engineering Supervisor or the Dose Assessment Supervisor. Print this map by selecting SHIFT-PRINT SCREEN if it is desired by the Dose Assessment Supervisor or Engineering Supervisor.
- 6.18 From the Thyroid CDE map, sequentially select CONTINUE, NEXT REPORT, then MORE REPORTS.
- 6.19 From the MORE REPORTS SCREEN, select any additional reports desired then select confirm and follow screen directions to display or print desired reports.
- 6.20 The Dose Assessment Supervisor / EOF Manager in the EOF or the Engineering Supervisor /ED in the TSC should now evaluate the data from the Notification Form, TEDE map and the Thyroid CDE map for the following: (Refer to CAUTION prior to Step 6.0.)
 - 6.20.1 If the data indicates that the GENERAL EMERGENCY effluent criteria has been met and the plant is not in a GENERAL EMERGENCY, then evaluate and consider declaring a GENERAL EMERGENCY.
 - 6.20.2 If the data indicates that the SITE AREA EMERGENCY effluent criteria has been met and the plant is not in a SITE AREA EMERGENCY, then evaluate and consider declaring a SITE AREA EMERGENCY.
 - 6.20.3 If the data indicates that the current Protective Action Recommendations (PARs) may be inadequate then evaluate and consider upgrading the PARs.

- 6.20.4 If there is no GENERAL EMERGENCY or SITE AREA EMERGENCY declaration required, then evaluate the actual offsite dose rate to determine if there is a requirement to declare a NOUE or ALERT declaration.
- 6.20.5 If there is no change in emergency classification or PARS, the off-site agencies should be informed of any change in radiological conditions using a follow-up message and direct communication, if appropriate.
- 6.21 From the MORE REPORTS SCREEN, select EXIT twice.
- 6.22 If additional back calculations are required, return to step 6.4 with a new field monitor reading and repeat the back calculation procedure.

SHARED

REFERENCES

1. FNP-0-EIP-9.0, Emergency Classification and Actions
2. FNP-0-EIP-9.2, Obtaining Meteorological Information
3. FNP-0-EIP-9.3, Personnel Computers - Automated Dose Assessment Method
4. MIDAS Users Manual for Southern Nuclear Operating Company
5. MIDAS Users Guide for Accident Calculations

04/09/03 9:58:48

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FIGURE 1**FORECASTED MET AND RAD SUMMARY**

Time	WS mph	WD deg fm	Stab Class	Rain (0/L/M/H)	RMS reading	RMS reading	RMS reading	FLOW
:00								
:15								
:30								
:45								
:00								
:15								
:30								
:45								
:00								
:15								
:30								
:45								
:00								
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:00								
:15								
:30								
:45								

1. ENTER CURRENT TIME, MET DATA AND RAD DATA, TO THE NEAREST 15 MINUTE INCREMENT IN THE TOPMOST APPROPRIATE LINE OF THIS FORM.
2. ENTER FORECASTED MET AND RAD DATA BASED ON FIGURE 2 FORECAST DATA IN THE REMAINING LINES AT 15 MINUTE INCREMENTS, FOR AS LONG AS THE FORECAST INFORMATION IS AVAILABLE.
3. UP TO 24 HOURS OF FORECASTED INFORMATION CAN BE INPUT TO MIDAS.
4. IF NECESSARY, USE ADDITIONAL FIGURE 1 TO FORECAST BEYOND 8 HOURS

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FIGURE 2

WEATHER FORECAST INFORMATION

DATE OBTAINED _____ TIME OBTAINED _____ CENTRAL

Information source for ZONE 69 forecast obtained from

- ☐ TALLAHASSEE
☐ BIRMINGHAM
☐ MOBILE
☐ FLIGHT SERVICE

When completing this form, select time intervals based on a front passing through, changes from daylight hours to nighttime hours for the calculation of stability class, the availability of weather forecasts and the estimation of how much longer the release will be in progress. If a front is predicted to pass through the area, the times should be based on forecasts before the front, during the front's passage and after the front has passed through.

forecast time interval	_____ to _____	_____ to _____	_____ to _____
wind direction	_____ or _____ compass point degrees	_____ or _____ compass point degrees	_____ or _____ compass point degrees
wind speed	_____ x 1.15 = _____ knots mph	_____ x 1.15 = _____ knots mph	_____ x 1.15 = _____ knots mph
Clear = 0/10, Scattered = 1/10 to 5/10, Broken = 6/10 to 9/10 and overcast = 10/10			
cloud cover 1	_____ % or _____ /10	_____ % or _____ /10	_____ % or _____ /10
at ceiling height	_____ feet	_____ feet	_____ feet
cloud cover 2	_____ % or _____ /10	_____ % or _____ /10	_____ % or _____ /10
at ceiling height	_____ feet	_____ feet	_____ feet
Enter the sum of available cloud covers and lowest available ceiling height below for each forecast time interval			
total cloud cover	_____ /10	_____ /10	_____ /10
minimum ceiling height	_____ feet	_____ feet	_____ feet
Based on the above information, calculate the stability class using the guidance of FNP-0-EIP-9.2			
stability class	_____	_____	_____
Estimate the amount of precipitation for each time interval as being 0, L-light, M-medium or H-heavy			
precipitation	_____	_____	_____

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FNP-0-EIP-27.1
March 17, 2003
Version 6

FARLEY NUCLEAR PLANT
EMERGENCY PLAN IMPLEMENTING PROCEDURE
FNP-0-EIP-27.1

ALTERNATE EOF SETUP AND ACTIVATION

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D

PROCEDURE USAGE REQUIREMENTS per FNP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	GUIDELINES AND ATTACHMENTS
Information Use	ALL OTHER SECTIONS

Approved:



Nuclear Plant General Manager

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ALTERNATE EOF SETUP AND ACTIVATION

1.0 Purpose

The purpose of this procedure is to delineate the criteria and authority for initiating EOF relocation from the On-Site EOF to the Alternate EOF located at the APCo Office in Headland, Alabama. In addition, the steps necessary to set up the Alternate EOF are located in this procedure.

2.0 References

See Table 1

3.0 General

- 3.1 The EOF Manager is responsible for ordering the relocation of the on-site EOF to the Alternate EOF if any of the criteria established in step 4 are exceeded.
- 3.2 The EOF Support Coordinator is responsible for the coordination of the transfer to and the setup of the Alternate EOF.
- 3.3 The Dose Assessment Supervisor is responsible for the coordination of the dose assessment, FMT communication, and follow-up message generation transfer during the transfer to the Alternate EOF.
- 3.4 When the Alternate EOF has been setup, FNP-0-EIP-27.0 and FNP-0-EIP-26.0 will be used to guide the activities in the Alternate EOF.
- 3.5 Figure 5 is a map that shows the location of the Alternate EOF.

4.0 Criteria for Transfer to the Alternate EOF

Relocation to the Alternate EOF will be evaluated for the following:

- 4.1 Radiation levels are such that emergency workers could receive TEDE exposure in excess of the limits specified by 10CFR20 for radiation workers. Emergency worker working hours, expected release duration (or source duration, if exposure is from a confined source), and meteorology should be considered when making this evaluation.
- 4.2 On-site EOF becomes uninhabitable due to loss of electrical power, loss of HVAC system, or other necessary support equipment.

- 4.3 On-site EOF HVAC system has been lined up in the isolation mode. Oxygen deficiency can occur when lined up in the isolation mode. Refer to FNP-0-EIP-27.0 for specific criteria.
- 4.4 A security event makes use of the on site EOF unavailable. This decision may be made by the Emergency Director when activating the ERO callout system.

5.0 Relocation Procedure

When the decision has been made by the Recovery Manager to relocate to the Alternate EOF, the following steps should be taken:

- 5.1 The EOF Manager should establish radio or cellular phone communications with the TSC using whatever circuits are available.
- 5.2 The EOF Support Coordinator. should complete Guideline 1.
- 5.3 The Dose Assessment Supervisor should complete Guideline 2.

6.0 After the Alternate EOF is Set Up

- 6.1 The EOF Manager can break the communications established in step 5.1 after phone lines in the Alternate EOF have been verified operable.
- 6.2 The Dose Assessment Supervisor will resume Dose Assessment Supervisor functions in the Alternate EOF when directed by the EOF Manager.
- 6.3 The EOF Support Coordinator will resume EOF Support Coordinator functions in the Alternate EOF when directed by the EOF Manager.

7.0 Recommended Directions to the Alternate EOF (See Figure 5)

7.1 Primary (fastest) route

Out of gate 95 on 95 North, turn right. At Highway 52 (dead end), turn right (toward Columbia). The first road on the left is Houston County Road 22, which turns into Henry County 1012. Turn left on 22/1012 and follow 1012 across 431. In Headland, 22/1012 turns into Cleveland Street. Follow Cleveland past the traffic light to Grove Street. Turn left on Grove Street (the Headland APCo Office will be on the right). Additional parking is available in the rear of the office.

7.2 Secondary route

Turn right out of gate 95 on 95 North. At 52, turn right toward Columbia. In Columbia, turn left at the traffic light and follow 95 North. Just past the Port Authority, turn left on state route 134 and follow 134 past 431. In Headland, 134 turns into East Church Street. Follow East Church to the traffic light at Cleveland, then turn right on Cleveland. Follow Cleveland to Grove, then turn left on Grove Street (the Headland APCo Office will be on the right). Additional parking is available in the rear of the office.

8.0 Emergency Planning Contingencies

Procedure FNP-0-TCP-32.0 (Emergency Planning Contingencies) has been developed to provide instructions for performing some non-routine activities. Examples of these types of activities are providing portable generator power to the EOF and sirens, local activation of sirens, and Tone Alert Radios.

8.1 In the event that normal power to the alternate EOF is lost when the alternate EOF is required, the following activities may be implemented to restore power:

- Verify the 10kw, natural gas generator starts, and energizes appropriate loads.
- Provide a portable 208 volt generator to the alternate EOF and coordinate with Southeast Division electricians to provide power to necessary loads.

ALTERNATE EOF SETUP

1. EOF Manager, EOF Support Coordinator or the Dose Assessment Supervisor must obtain the key for the Alternate EOF from the Headland Police Department. Enter the Headland City Hall from the Park Street entrance and obtain the keys from the Dispatcher (334/693-2222). If it is after hours knock on the front door and the dispatcher will open the door. If the Alternate EOF key is unavailable, contact one of the individuals listed in step 2 to unlock the Alternate EOF and provide the equipment cabinet keys stored there.
2. Inform one of the individuals associated with the Headland APCo Office of the intention to move to the Alternate EOF.

<u>NAME</u>	<u>HOME</u>	<u>OFFICE</u>
Charles Bowers	334-671-0701	8-286-4044 (334-693-4044)
Jerry Grantham	334-693-0664	8-286-4028 (334-693-4028)
Customer Service Center - 24 Hour Manning: 1-800-245-2244		

3. Transport the procedures that are currently in use to the Alternate EOF.
4. If the EOF can be entered safely transfer the EOF spare portable Southern LINC RMT radio to the TSC or other location for use by the RMTs.
5. If the EOF can be entered safely transport the following EOF Southern LINC radios to Headland and set them up in room 111 as shown in Figure 2:
 - FMT communicator from room 118
 - EOF Support Coordinator from cabinet behind the EOF Manager
 - EOF ENN from cabinet in the communications area
 - Alabama fleet radio from cabinet in the communications area
 - Georgia fleet radio from cabinet in the communications area
6. At the Alternate EOF, locate all of the equipment needed to set-up and move it to the appropriate rooms. (See Figure 1 for storage locations.)
7. Arrange the furniture in Room 111 as indicated in Figure 2. See Figure 6 for the telephone connection scheme. (The phone numbers are located on the sides of the tables; the tables are located in the storage room.)
8. Relocate and connect the Cannon fax machine that is shown in Figure 1 to room 111 for use at the extension labeled for outbound faxes at the ENN/Fax Jack for table 7.
9. Relocate and connect the fax machine located in the ENN cabinet to extension 276-4993 (334-814-4993) for inbound faxes at the ENN/Fax Jack for table 7.

10. Connect the phone jack next to the FAX machine to table 5.
11. Connect the phone jack behind the RM Table to the RM table.
12. Locate the ERDS/ARDA computer and the Mimic SPDS computer as shown on Figure 2. Connect to the LAN connections next to the phone jacks on the east wall of room 111.
13. Relocate the phones and the ENN from the ENN cabinet and connect all of the phones and telecopier to the phone jacks located under the tables or to the wall jacks next to fax machine location.
14. Set up the radio and FMT communication in room 112 per Figure 2.
15. Verify phones and equipment in room 122 (NRC) and room 106 Engineering Support per Figures 3 and 4.
16. Distribute procedures, office supplies and other equipment as necessary.
17. Coordinate with the TSC or the EOC to set the clocks in the Alternate EOF to the correct time.

EOF SUPPORT COORDINATOR GUIDELINE FOR RELOCATION TO ALTERNATE EOF

INITIALS

- ____ 1. If the EOF staff is at the On Site EOF, inform them of the plan to move to the Alternate EOF. Potential briefing options include:
 - The key for the Alternate EOF can be picked up from the Headland Police Department by the EOF Manager, the EOF Support Coordinator or the Dose Assessment Supervisor.
 - If the on site EOF is habitable, consider leaving key personnel at the EOF performing necessary functions such as Dose assessment and FMT communication until the Alt EOF is available. This will require augmenting additional people to the Alt EOF.
 - Turn over all functions to the TSC or the CEOC and have all personnel report to the ALT EOF.
 - If the EOF can be entered, relocate Southern LINC radios from the EOF as described in attachment 1. It is not mandatory to transfer these radios if the EOF can not be entered safely.
 - Unless there is a security event in progress, the FMTs should be dispatched and controlled from the TSC or the EOF as described in Guideline 2.
 - If there is a security event in progress, the FMTs should be dispatched only if it is considered safe to dispatch them.
- ____ 2. Inform individuals/agencies listed in Table 2 of the intent to move to the Alternate EOF.
- ____ 3. Transfer EOF Support Coordinator duties to the TSC or EOC, as appropriate.
- ____ 4. Have personnel transfer duties to the TSC or EOC as appropriate.
- ____ 5. Have personnel hand-carry copies of procedures in use to the Alternate EOF.

NOTE: A MAP SHOWING HOW TO FIND THE ALTERNATE EOF IS LOCATED IN FIGURE 5.
--

- ____ 6. Arrange for transportation of all staff and necessary equipment to the Alternate EOF.
- ____ 7. Obtain the keys to the Alternate EOF from the Headland Police Department per Attachment 1. If the Alternate EOF key is unavailable, contact one of the individuals listed in step 2 of Attachment 1 to unlock the Alternate EOF.

NOTE: ALL OF THE EQUIPMENT NECESSARY TO SET UP THE ALTERNATE EOF IS LOCATED IN CABINETS IN ROOM 114, EQUIPMENT STORAGE ROOM, ROOM 112, OR PERMANENTLY LOCATED IN THE ROOMS.

STATUS BOARDS MAY HAVE COVERS OVER THEM WHICH MUST BE REMOVED TO DISPLAY THE BOARDS.

- _____ 8. After arrival at the Alternate EOF, assign personnel to set up rooms 111, 112, 106, and 122 per Attachment 1 and Figures 1, 2, 3, 4 and 6 of this procedure.
- _____ 9. Have communication circuits tested.
- _____ 10. When directed by the EOF Manager, resume the normal duties of the EOF Support Coordinator, and have staff under EOF Support Coordinator supervision resume normal duties.
- _____ 11. Inform individuals/agencies listed in Table 2 when the Alternate EOF is operational.

**DOSE ASSESSMENT SUPERVISOR GUIDELINE FOR
RELOCATION TO ALTERNATE EOF**

INITIALS

- _____ 1. Dose Assessment and follow-up message generation should be transferred to the TSC, or remain at the EOF until the Alternate EOF is ready to resume the function. The decision should be based on EOF conditions and available personnel.
- _____ 2. FMT communication should be transferred to the TSC, or remain at the EOF until the Alternate EOF is ready to resume the function. The decision should be based on EOF conditions and available personnel.
- _____ 3. Have other personnel under the supervision of the Dose Assessment Supervisor transfer functions to the TSC or EOF as appropriate.
- _____ 4. Have personnel hand-carry copies of the procedures in use to the Alternate EOF.
- _____ 5. Provide support for the EOF Support Coordinator in setting up the Alternate EOF.
- _____ 6. After the EOF has been set up, establish dose assessment and follow-up message generation at the Alternate EOF.
- _____ 7. After the EOF has been set up, establish FMT communication at the Alternate EOF.
- _____ 8. When the Alternate EOF has been set up, resume the normal duties of the Dose Assessment Supervisor, and have staff under Dose Assessment Supervisor resume normal duties.

REFERENCES

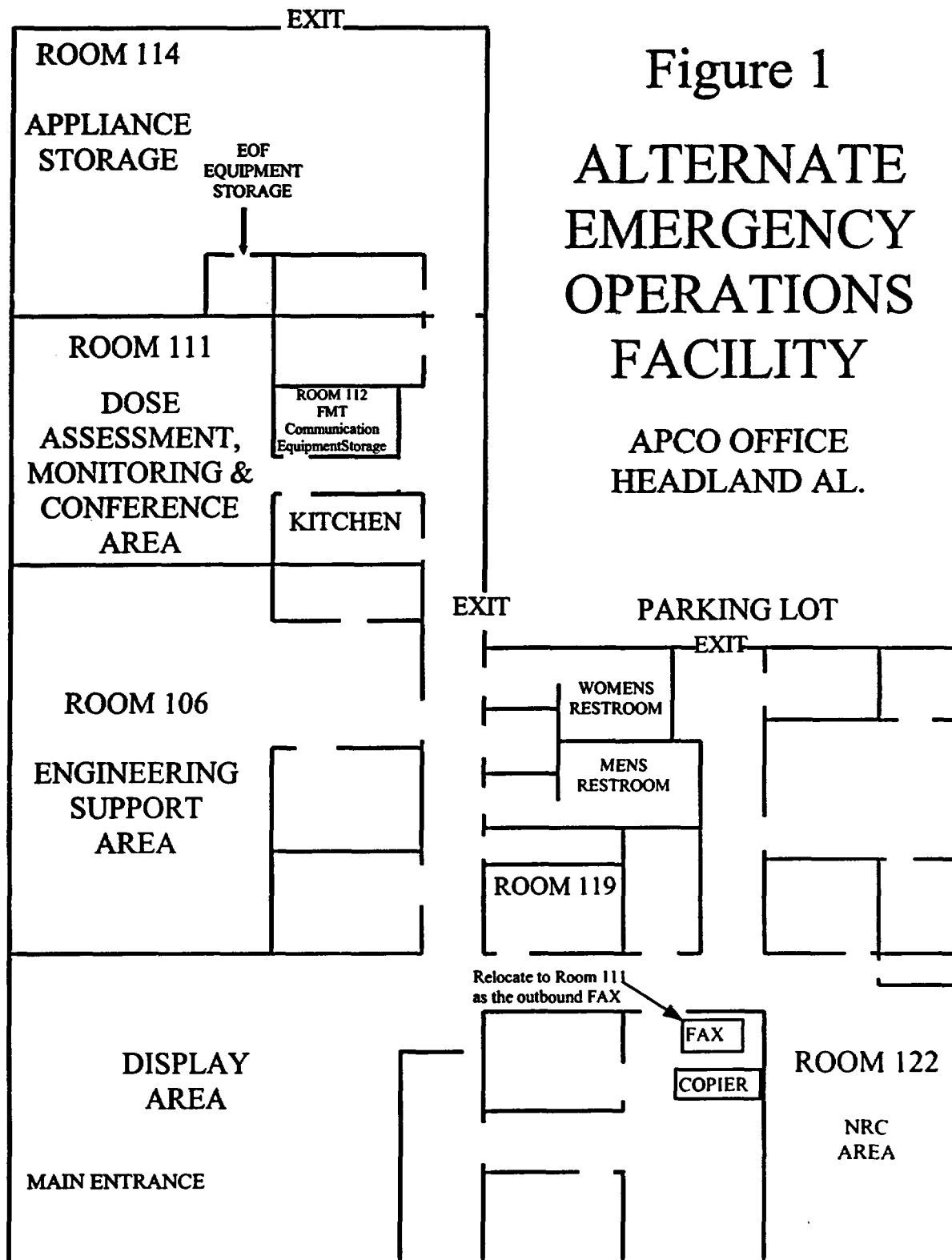
1. Joseph M. Farley Nuclear Plant Emergency Plan
2. FNP-0-EIP-27.0, EOF Setup and Activation
3. FNP-0-EIP-8.1, Emergency Phone Directory
4. 10CFR20
5. FNP-0-EIP-26.0, Recovery Manager's Duties and Responsibilities

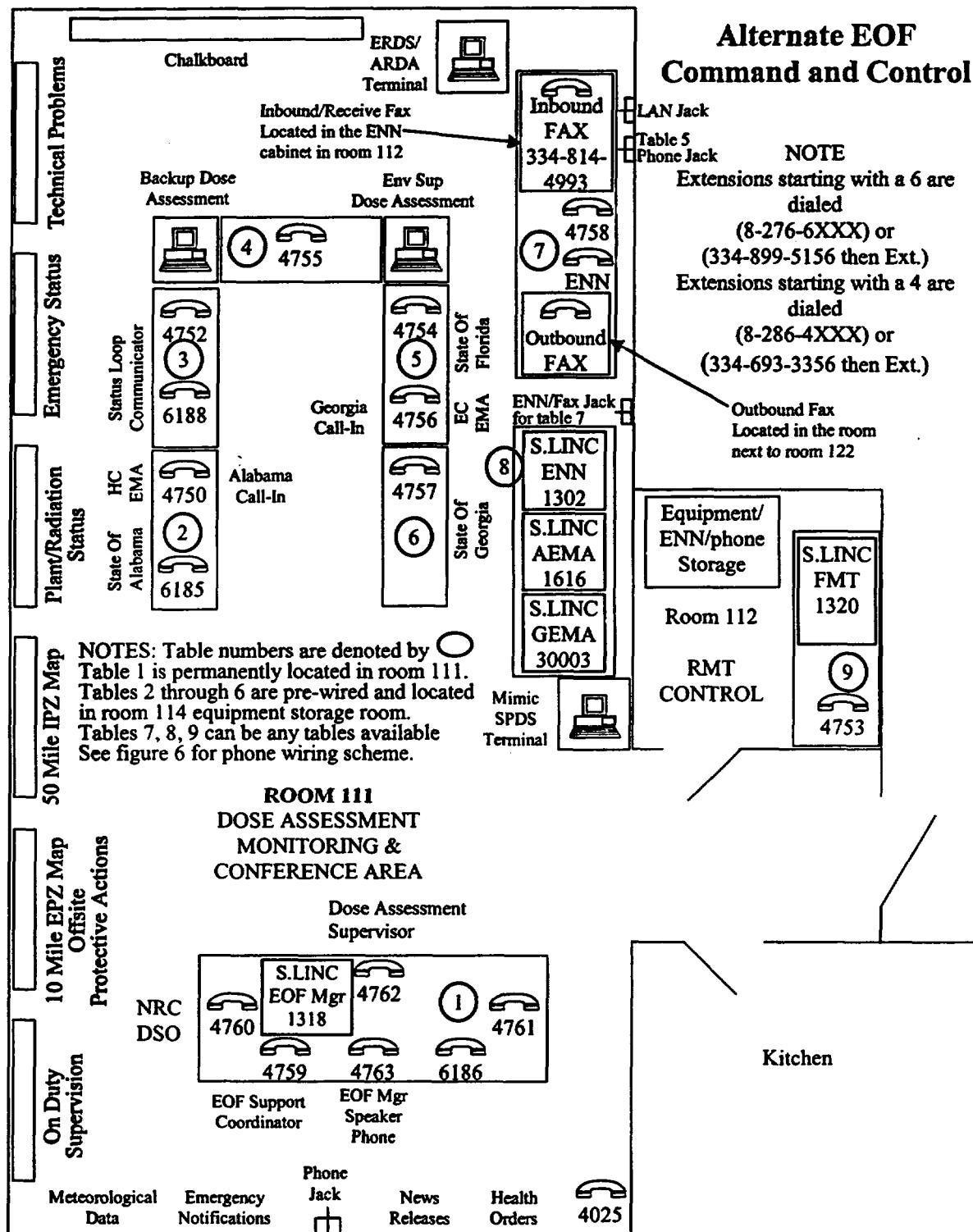
AGENCIES/INDIVIDUALS TO BE INFORMED OF EOF STATUS

This column is
NA if reporting
Directly to Alt EOF
Leaving
EOF

ALT EOF
Established

_____	_____	Southeast Division Client Services (Request SED Provide support to setup communications equipment at the Alternate EOF)
_____	_____	Alabama Radiation Control (EOC Montgomery or FEOC Houston County)
_____	_____	Alabama Emergency Management Agency EOC Clanton or FEOC Houston County)
_____	_____	Houston County Emergency Management Agency
_____	_____	Georgia Emergency Management Agency (EOC Atlanta or FEOC Early County)
_____	_____	Early County Emergency Management Agency
_____	_____	Florida Department of Emergency Management
_____	_____	Emergency Director
_____	_____	SNC Duty Manager
_____	_____	CEOC Birmingham
_____	_____	News Media Center
_____	_____	Headland APCo Office (Of Intent to use the Headland APCo Office)

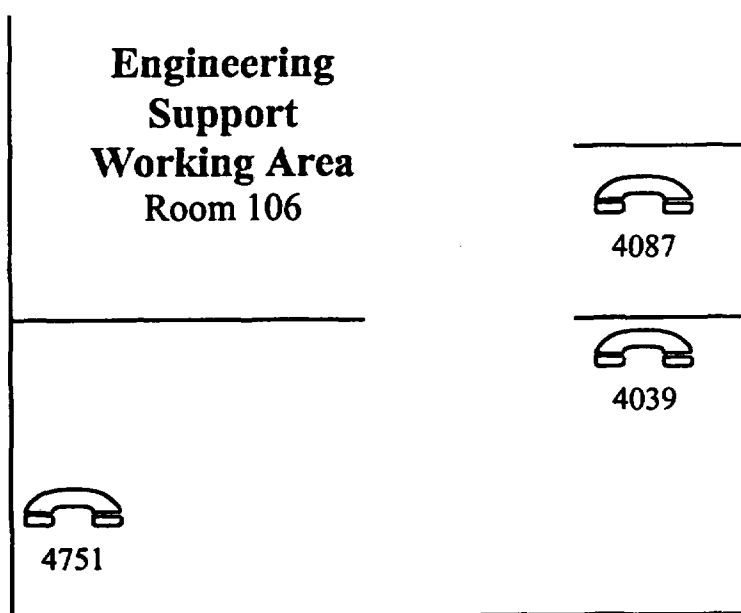




Alternate EOF

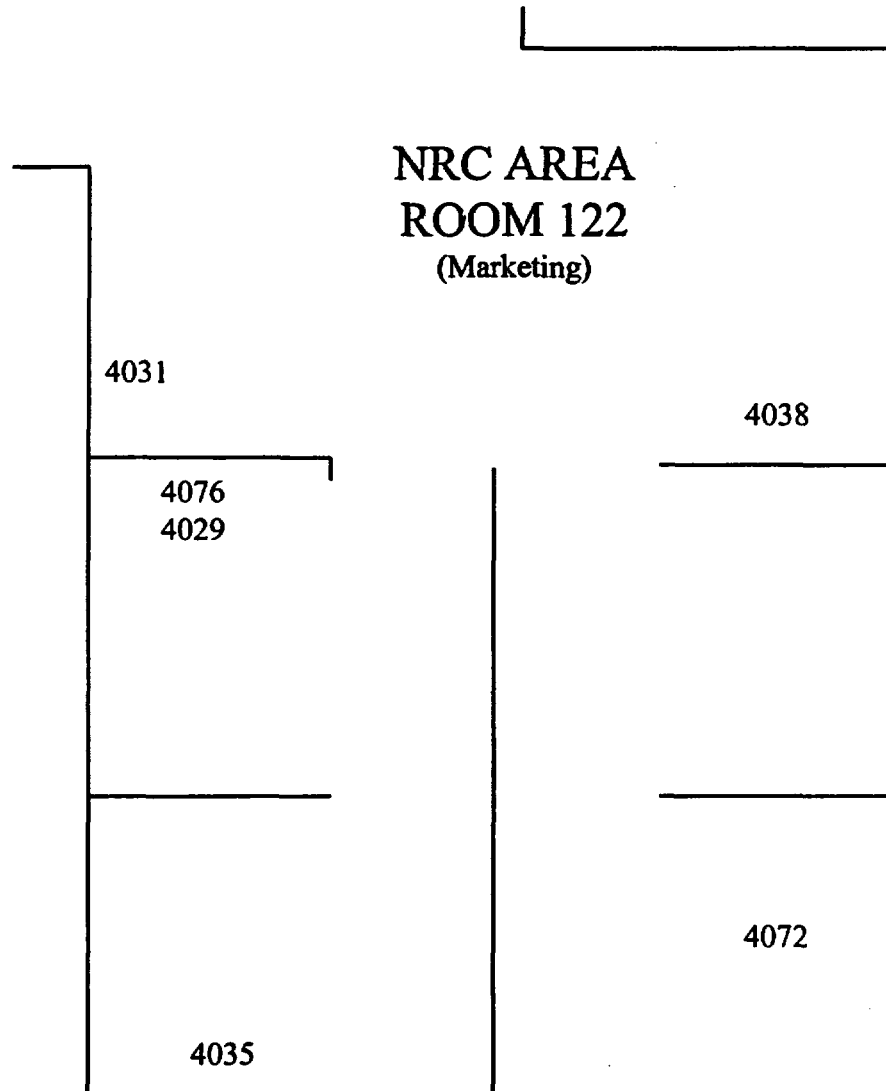
Engineering Support Area

NOTE
Dial (8-286-4XXX) or
(334-693-3356 then Ext.)

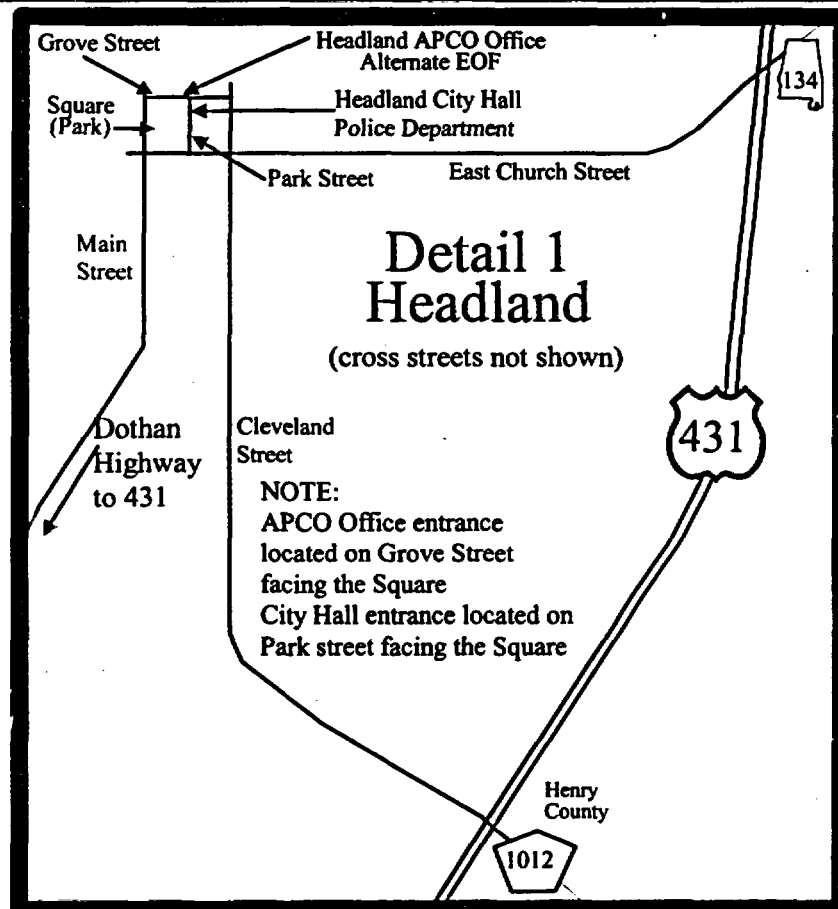
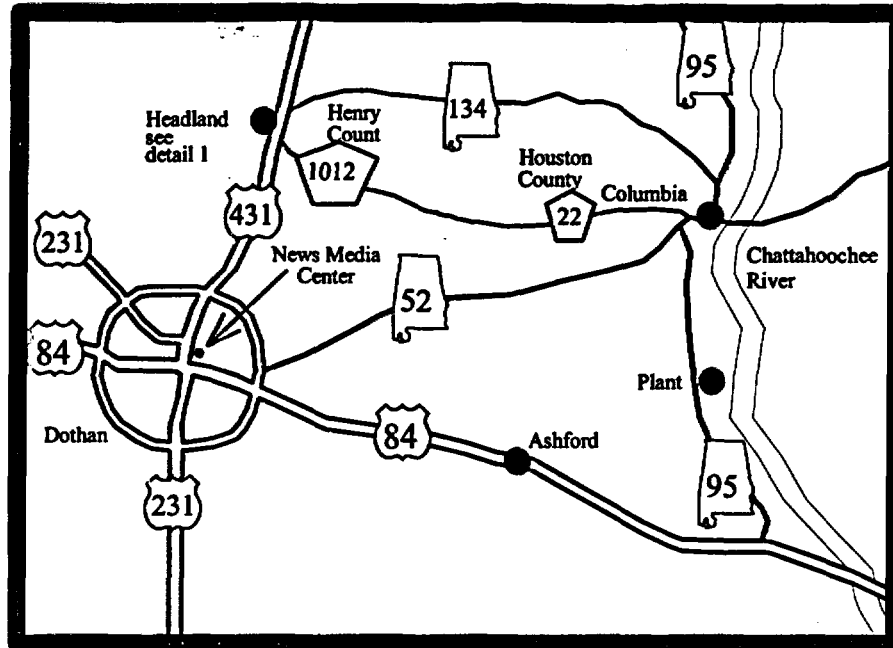


Alternate EOF NRC AREA

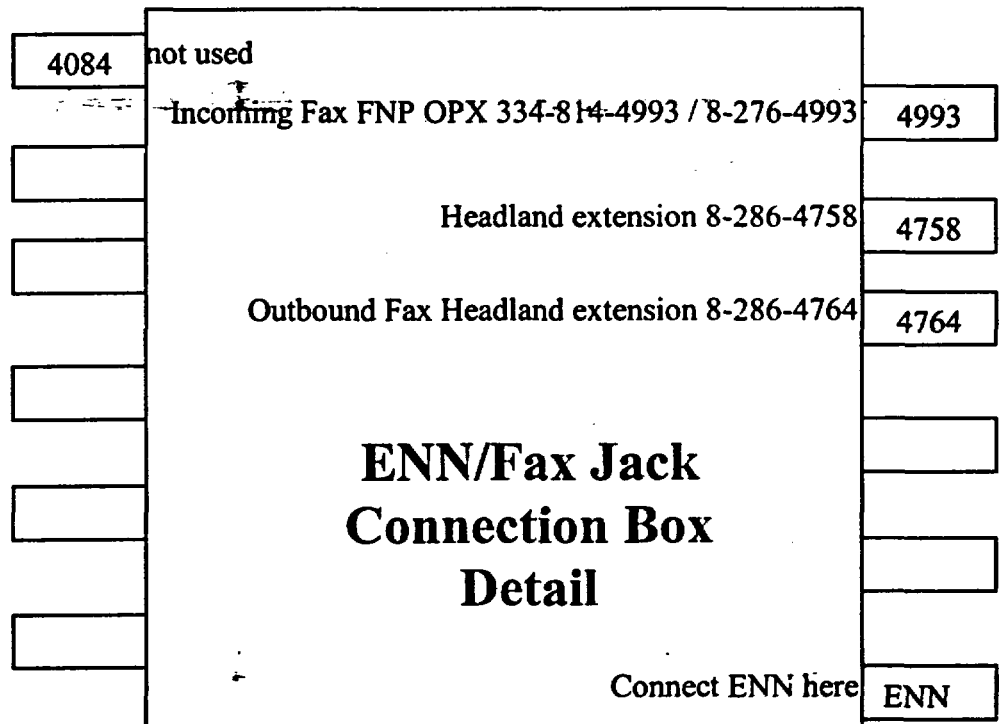
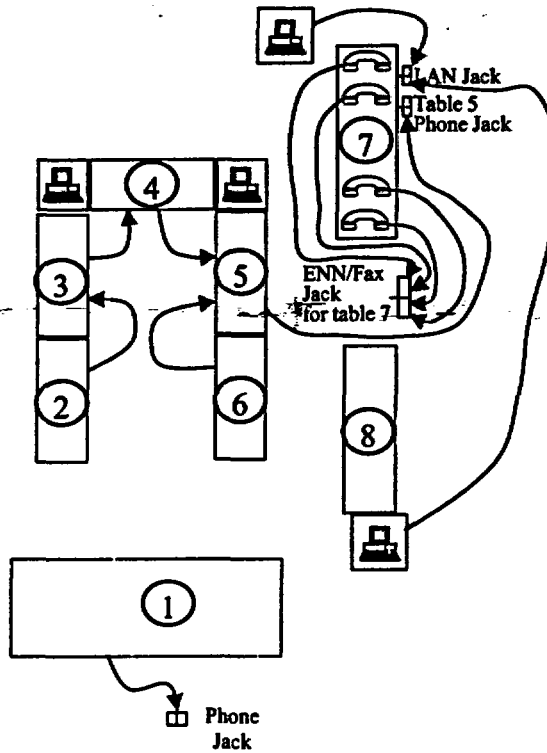
NOTE
Dial (8-286-4XXX) or
(334-693-3356 then Ext.)



MAP TO THE ALTERNATE EOF



Phone Line Connection Scheme



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FNP-0-EIP-27.0
February 18, 2003
Version 34

FARLEY NUCLEAR PLANT
EMERGENCY PLAN IMPLEMENTING PROCEDURE
FNP-0-EIP-27.0

EOF SETUP AND ACTIVATION

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PROCEDURE USAGE REQUIREMENTS per FNP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	GUIDELINES AND ATTACHMENTS
Information Use	ALL OTHER SECTIONS

Approved:



Nuclear Plant General Manager

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EOF SETUP AND ACTIVATION

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Figure 11	ON DUTY SUPERVISION
Figure 12	COMMUNICATIONS LOG

EOF SETUP AND ACTIVATION

1.0 Purpose

This procedure provides guidance for the activation and operation of the Emergency Operations Facility (EOF).

2.0 References

See Table 1

3.0 General

The EOF, when activated by the EOF Manager, will be operated for continued evaluation and coordination of activities performed as the result of an emergency having the potential for environmental consequences. The EOF accident recovery organization shall control all activities necessary to establish safe plant conditions and to limit exposure to the public.

The EOF, when activated by the EOF Manager, will be the point of interface for federal, state, and local authorities for implementation of offsite emergency plans, in addition to providing a centralized meeting location for key representatives from Off-Site agencies.

The EOF is defined as the basement of the FNP Training Center. The upstairs portion of the building does not meet the NUREG 0654 criteria for on-site EOFs (i.e., shielding factor of five or more).

4.0 EOF Setup

4.1 It is recommended that the EOF staff be called in at an alert declaration or higher and take all actions assuming that the EOF will be fully operational.

4.2 Determine the requirements for activating the EOF in accordance with Table 2.

- 4.3 The equipment and supplies necessary to set up the EOF are located in the installed cabinets in room 106 or in portable cabinets that are stored in room 118
- 4.4 The cables to provide temporary power to the portable NRC trailer, if it is brought on site, are located in a cabinet in room 113, located outside the south exit of room 106.
- 4.5 When an Alert or higher emergency is declared, a Security Force Member (SFM) will report to the EOF to start setup.
- 4.6 The SFM will ensure that the keys for the EOF that are located in the CSC are brought to the EOF.
- 4.7 All EOF staff will support EOF setup under the coordination of the Support Coordinator.
- 4.8 The Support Coordinator is responsible for overall coordination and assigning responsibilities for EOF setup.
- 4.9 Perform setup of the EOF per Attachment 1.

5.0 EOF Manager

The EOF Manager's duties and responsibilities are discussed in FNP-0-EIP-26.0.

6.0 Support Coordinator

The duties and responsibilities of the Support Coordinator are described in Guideline 1.

6.1 Report to the EOF or other location directed by the EOF Manager.

6.2 Perform the steps in Guideline 1.

7.0 Dose Assessment Supervisor

The duties and responsibilities of the Dose Assessment Supervisor are described in Guideline 2.

7.1 Report to the EOF or other location directed by the EOF Manager.

7.2 Perform the steps in Guideline 2.

8.0 Computer Services Support

- 8.1 Report to the EOF or other location directed by the EOF Manager.
- 8.2 To support Dose Assessment, operate the data acquisition systems as directed by the Dose Analyst or the Dose Assessment Supervisor.
- 8.3 Provide support for maintaining all computer systems on site, as directed by the EOF Manager.

9.0 Dose Analyst

- 9.1 Report to the EOF or other location directed by the EOF Manager.
- 9.2 Direct operation of the dose assessment effort in the EOF, at the direction of the Dose Assessment Supervisor.
- 9.3 If dose assessment has been started in the TSC, ensure a timely turnover of dose assessment functions to the EOF.
- 9.4 Operate the MIDAS computer as the primary method of dose assessment.
- 9.5 Coordinate HP support in the EOF. HP equipment and supplies are located in HP cabinets #1 and #2 in the hall outside the central stairwell.
- 9.6 Perform the duties of the Dose Analyst as described in FNP-0-EIP-20.0.

10.0 FMT Communicator

- 10.1 Report to the EOF or other location directed by the EOF Manager.
- 10.2 Perform a radio test of the radios in the FMT Communicator area.
- 10.3 Monitor state FMT radio communications and record results.
- 10.4 If FMT Communication has been started from the TSC, ensure a timely turnover of FMT Communication to the EOF.
- 10.5 Coordinate with the Dose Assessment Supervisor and the FMT HP Technicians to provide RWT/Respirator qualified individuals from plant staff as assistants for each of the FMTs.
- 10.6 Coordinate with the Dose Assessment Supervisor and the FMT HP Technicians to provide FMT vehicles for each of the FMTs.

10.7 Control the On-Site (out of plant) and off site FMTs per FNP-0-EIP-4.0.

10.8 If an FMT Communicator assistant is assigned have the individual review step 19.3.2.

11.0 EOF ENN Communicator

11.1 Report to the EOF or other location directed by the EOF Manager.

11.2 Act as the communicator in the EOF.

11.3 Verify operability of the EOF telecopiers.

11.4 When directed by the Dose Assessment Supervisor or EOF Manager, telecopy dose assessment/follow-up reports to state and local agencies.

11.5 Communicate with off site authorities on the ENN or other communication circuits as directed by the Dose Assessment Supervisor or EOF Manager.

11.6 Verify that all communications sent from the EOF have been received by the off site authorities.

11.7 Maintain a chronological log of EOF off site communications sent through the communications area, noting the organization contacted and a summary of the conversation. Figure 12, a similar form, or a log may be used.

11.8 Maintain a chronological log of all EOF off site telecopies. Figure 12, a similar form, or a log may be used.

11.9 If an EOF Communicator assistant is assigned, have the individual review step 19.3.3.

12.0 Alabama FEOC Liaison

12.1 Report to the Dose Assessment Room at the Houston County Courthouse (Alabama Forward EOC) or other location directed by the EOF Manager.

12.2 If the state or county agencies have not established operations at the forward EOC, contact the EOF Manager or the Emergency Director for further instructions.

12.3 Contact the Support Coordinator in the EOF or the TSC Manager in the TSC to determine current plant status.

12.4 Provide state and county agency personnel with explanations of plant terminology, hardware, and plant operations.

12.5 Contact the Support Coordinator in the EOF or the TSC Manager in the TSC if communication problems become evident, of concerns of the state and county agency personnel that need resolving, and of significant off-site actions (such as evacuations) that are pending or in progress.

12.6 Assist state and county agency personnel in resolving problems relating to communications or actions at the plant.

12.7 Refrain from providing information or comments to news media personnel.

13.0 Georgia FEOC Liaison

13.1 Report to the Dose Assessment Room at the Early County Jail (Georgia Forward EOC) or other location directed by the EOF Manager.

13.2 If the state or county agencies have not established operations at the forward EOC, contact the EOF Manager or the Emergency Director for further instructions.

13.3 Contact the Support Coordinator in the EOF or the Technical Manager in the TSC to determine current plant status.

13.4 Provide state and county agency personnel with explanations of plant terminology, hardware, and plant operations.

13.5 Contact the Support Coordinator in the EOF or the Technical Manager in the TSC if communication problems become evident, of concerns of the state and county agency personnel that need resolving, and of significant off-site actions (such as evacuations) that are pending or are in progress.

13.6 Assist state and county agency personnel in resolving problems relating to communications or actions at the plant.

13.7 Refrain from providing information or comments to news media personnel.

14.0 FMT 2

14.1 Report to the EOF or other location directed by the EOF Manager.

14.2 Coordinate with the Dose Assessment Supervisor and the EOF FMT Communicator to provide RWT/Respirator qualified individuals from plant staff as assistants for each of the FMTs.

14.3 Coordinate with the Dose Assessment Supervisor and the EOF FMT Communicator to provide FMT vehicles for each of the FMTs.

14.4 Perform the duties of an FMT per FNP-0-EIP-4.0.

15.0 FMT 3

- 15.1 Report to the EOF or other location directed by the EOF Manager.**
- 15.2 Coordinate with the Dose Assessment Supervisor and the EOF FMT Communicator to provide RWT/Respirator qualified individuals from plant staff as assistants for each of the FMTs.**
- 15.3 Coordinate with the Dose Assessment Supervisor and the EOF FMT Communicator to provide FMT vehicles for each of the FMTs.**
- 15.4 Perform the duties of an FMT per FNP-0-EIP-4.0.)**

16.0 Security Force Member (1) (On Shift)

- 16.1 Obtain the EOF keys from the CSC and report to the EOF or other location directed by the EOF Manager.**
- 16.2 One Security Force Member will start setup of the EOF per Attachment 1.**
- 16.3 Maintain EOF security and establish Access Control per FNP-0-EIP-7.0.**
- 16.4 If directed by the Dose Assessment Supervisor, issue dosimetry to all personnel in the EOF.**
- 16.5 Perform other duties as directed by the EOF Manager or his assistant.**

17.0 EOF Status Loop Communicator

- 17.1 Report to the EOF or other location directed by the EOF Manager.
- 17.2 Assist in setup of the EOF per Attachment 1.
- 17.3 Turn on the non-reg ERDS system in the EOF by turning on the monitors in the dose assessment area and by the status boards. If the screen is blank, move the cursor or press any key on the key board.
- 17.4 Start ERDS automatic printout per FNP-0-EIP-9.1.
- 17.5 Every 15 minutes, pick up the ERDS printout and transcribe the data onto the PLANT and RADIATION STATUS boards, erasing the oldest set of data. Maintain a file of all ERDS printouts.

NOTE: THE TSC STATUS LOOP COMMUNICATOR CAN BE CONTACTED AT EXTENSION 4988.

- 17.6 Request the TSC Status Loop Communicator provide the containment hydrogen concentration, when the hydrogen monitors have been placed in service, approximately every 30 minutes. Transcribe the value onto the status board and the appropriate ERDS printout.
- 17.7 Request the Dose Assessment Supervisor or individual performing dose assessment provide source term information if applicable every thirty minutes and post the source term information on the radiation status board.
- 17.8 If the ERDS computer is not functioning, request the EOF Status Loop Communicator fax filled out copies of Figures 6 and 7 approximately every 30 minutes. Transcribe the data onto the PLANT and RADIATION STATUS boards, erasing the oldest set of data. Maintain a file of all faxes.
- 17.9 Request the TSC Status Loop Communicator fax copies of Figure 8 through 11 whenever new information is posted on those status boards in the TSC. Transcribe the data onto the appropriate status boards. Maintain a file of all faxes.
- 17.10 Coordinate with the Support Coordinator to fill out the EOF and news media portions of Figure 11, including phone numbers of the individuals listed. Post the information on the EOF status board and fax the information to the TSC and CEOC.
- 17.11 Post the appropriate Emergency Class and Unit signs on the wall behind the EOF Manager, update these signs if the emergency class changes.

- 17.12 If there are any changes to the status boards initiated in the EOF, copy the new information onto the appropriate figure and fax to the TSC and CEOC.
- 17.13 If a EOF Status Loop Communicator Assistant is assigned, have the individual review step 18.3.4.
- 17.14 Maintain a running log of significant information that has been posted on the Emergency Status Board, Technical Problem Status Board, and the Protective Action Status Board.

18.0 Augmented EOF Staff

- 18.1 For more efficient operation of the EOF, it may be desirable to provide additional staff. It should be remembered that too many people can add confusion and be counter-productive.
- 18.2 Individuals who have been brought to the EOF as additional staff should remain in the EOF if the plant Emergency Alarm is activated. The senior individual in the Assembly area, where the additional staff would normally assemble, should be informed.
- 18.3 Preplanned EOF staff augmentation

18.3.1 Public Information Technical Advisor

An individual who is knowledgeable of plant operation (CAR 2329) should be made available to act as a technical advisor to the Public Information Staff at the News Media Center (NMC). When this individual is made available, from available plant staff, the duties would include:

- a. Report to the NMC in Dothan.
- b. Act as a liaison between the EOF Manager in the EOF and the NMC staff.
- c. Stay aware of plant status.
- d. Advise the PI staff as to the technical accuracy of the press releases and information.

18.3.2 FMT Communicator Assistant

An individual, preferably with previous FMT or HP experience, may be made available to assist the FMT Communicator. The duties of this individual are:

- a. Review step 10 and 18.2.
- b. Assist the FMT Communicator in the performance of duties.

18.3.3 EOF Communicator Assistant

An individual, preferably with previous operational experience, may be made available to assist the EOF ENN Communicator as the EOF communicator. The duties of this individual are:

- a. Review step 18.2.
- b. Assist the EOF ENN Communicator in the performance of EOF communicator duties.

18.3.4 EOF Status Loop Communicator Technical Support

An individual, preferably with previous operational experience, may be made available to technical support for maintaining the EOF status boards. The duties of this individual are:

- a. Review step 18.2.
- b. Assist in the performance of EOF Status Loop Communicator duties.

18.3.5 ENS EOF Communicator

An individual with previous operational experience may be made available in the EOF to communicate with the NRC on the ENS. The main ENS communicator should be in the TSC or the Control Room. If the NRC has questions about off site dose assessment, state/local protective actions or press releases, an individual in the EOF may be better able to handle these questions. The duties of this individual are:

- a. Review step 18.2.
- b. Communicate with the NRC on the ENS phone and keep plant staff aware of NRC concerns.
- c. The communicator should be careful about transmitting information until it has become official. For example if it is overhead that we are thinking about upgrading emergency classification, this should not be relayed to the NRC unless directed by management.

18.3.6 HPN EOF Communicator

An individual with previous HP/Dose Assessment experience may be made available in the EOF to communicate with the NRC on the HPN. The Main HPN communicator should be in the EOF. If the NRC has questions about off site dose assessment or state/local protective actions, the EOF HPN communicator would be the best person to handle these questions.

- a. Review step 18.2.
- b. Communicate with the NRC on the HPN phone and keep plant staff aware of NRC concerns.
- c. The communicator should be careful about transmitting information until it has become official. For example if it is overhead that we are thinking about upgrading emergency classification, this should not be relayed to the NRC unless directed by management.

18.3.7 IR Client Services

An individual from IR client Services should be made available in the EOF to respond to communication problems and computer problems that could impact the plants ability to combat the current emergency

- a. Review step 18.2.
- b. Coordinate with the Support coordinator to provide necessary support for plant operation.

18.3.8 Support Staff

The Support Staff position should normally be filled by an Administrative Assistant or other individual with administrative skills. The duties of this individual would be to maintain logs, provide logistic support such as arranging for meals or contacting and calling out additional staff, making public address announcements, or other duties as designated by the Support Coordinator or EOF Manager.

- a. Review step 18.2.
- b. Coordinate with the Support coordinator to provide necessary support for plant operation.

19.0 Partial EOF Staffing Recommendations

In the event that the EOF is partially staffed for a declared emergency for which full staffing is not required, the following items need to be taken into consideration:

- 19.1 EOF staffing requirements are described in the Emergency Facility Activation, Table 2. This table should be referred to any time partial staffing is being considered.
- 19.2 At a GENERAL EMERGENCY, the EOF is required to be fully staffed.
- 19.3 At the ALERT or SITE AREA level, the TSC may be staffed, but the level of staffing is "that level that is deemed necessary by the EOF Manager".
- 19.4 Below an ALERT level, the EOF is not required to be staffed. EOF staff can be placed in standby, partially activated, fully activated, or no action taken for the EOF staff -- at the EOF Manager's discretion.
- 19.5 Any time that the plant has declared an ALERT or above, it is recommended that the full EOF staff be activated initially. For classifications below GENERAL EMERGENCY (after emergency conditions have been assessed), then the EOF Manager may, at his discretion, allow those positions that are not needed to go into a standby mode.

20.0 Minimum EOF Staff

The following five positions constitute the minimum staff required to be called in to perform the required EOF functions:

- EOF Manager
- Support Coordinator
- Dose Assessment Supervisor
- On Call Staff Member for dose projection (any one of two listed not used for communication):
 - Dose Analyst
 - Computer Services
- On Call Staff Member for communication (any one of two listed not used for dose projection)
 - Dose Analyst
 - EOF ENN Communicator

21.0 Emergency Planning Contingencies

Procedure FNP-0-TCP-32.0 (Emergency Planning Contingencies) has been developed to provide instructions for performing some non-routine activities. Examples of these types of activities are providing portable generator power to the EOF and sirens, local activation of sirens, and Tone Alert Radios.

22.0 EOF Status Definitions

- 22.1 The following definitions apply to the status of the EOF to describe its state of readiness for assumption of EOF duties and responsibilities.

STANDBY: For ALERT declaration, minimum staff available, and able to perform required functions.

OPERATIONAL: For SITE AREA EMERGENCY or GENERAL EMERGENCY declaration, minimum staff available, and ready to turnover required functions.

- 22.2 Functions such as dose assessment, FMT control, communications, and engineering support may be accomplished in the EOF prior to being fully operational with the ED and EOF Managers permission.

23.0 Expectations for the Readiness of the EOF on a Day to Day Basis (OR# 2000188.1)

The EOF is an emergency facility and as such it has to be able to be set up and placed in operation within a short amount of time. To ensure that this can happen there is a minimum standard that must be met on a day to day basis. The following items will help to ensure that the EOF can be made operational in the required amount of time:

- 23.1 In general, emergency equipment should be left in its standby condition as evidenced by appropriate cabinets being sealed or locked.
- 23.2 The cleanliness and general condition of the EOF including rooms 106, 118, 103, 104, and 105 shall be maintained neat and orderly.
- 23.3 Additional equipment or supplies used in the EOF shall be capable of rapid removal from the operating areas of the EOF.
- 23.4 The tables that are assigned to room 106 shall be left in room 106.
- 23.5 Emergency phones utilized during non-emergencies should remain connected to their designated emergency jacks.
- 23.6 In the event that emergency equipment or facilities are needed for non-emergency use, the user of the equipment is responsible for ensuring EOF setup requirements in accordance with this procedure are not impacted.
- 23.7 Should questions arise regarding the setup capability of the EOF, it is the responsibility of the Training and Emergency Preparedness Manager to promptly resolve the issue.

EOF SETUP

INITIALS

- _____ 1. When the requirements of step 23.0 of the main body of the procedure have been met, then the EOF may be declared in standby or operational while continuing with the remainder of this attachment.
- _____ 2. When the EOF has been declared in standby or operational with the concurrence of the EOF Manager, announce to the EOF and inform the TSC that the EOF is in standby or operational.
- _____ 3. The Security Force Member reporting to the EOF will obtain the EOF keys from the CSC prior to reporting to the EOF.

NOTE: REFER TO FIGURES 1 THROUGH 5 FOR ROOM NUMBERS AND ROOM LOCATIONS.

THE KEY CABINET LOCATED IN ROOM 118 HAS MASTER KEYS AT LOCATIONS 1, 2 AND 3. THESE KEYS WILL UNLOCK ALL THE DOORS IN THE EOF EXCEPT FOR THE COMMUNICATIONS ROOM, ROOM 108 (KEY 12), THE STOREROOM OUTSIDE THE VISITORS CENTER ENTRANCE, ROOM 263 AND THE NURSES STATION/FITNESS FOR DUTY FACILITY. ACCESS TO THESE LAST TWO LOCATIONS IS NOT REQUIRED FOR OPERATION OF THE EOF.

A PAIR OF CUTTERS IS LOCATED ON THE SIDE OF THE KEY CABINET TO AID IN REMOVING SEALS.

- _____ 4. Unlock all interior doors in the lower level of the EOF, the main entrance doors in the lobby of the Visitors Center and the Room 106 exterior doors.
- _____ 5. Relocate the cabinets and carts in Room 118:
 Phone cart 1 and 2 in center of Room 106
 Room 105 phone cart to Room 105
- _____ 6. Break the seals and open all the cabinets located in room 106.
- _____ 7. Activate and post 5 area dosimeters per Attachment 4.
- _____ 8. Connect phone cart #1 to the phone line with walking strip from the center cabinet on the East wall of room 106. Connect phone cart #2 to phone cart #1.
- _____ 9. Arrange 10 tables as the EOF Managers conference table under the signs in the center of Room 106, as shown in Figure 1.

EOF SETUP

- ____ 10. Place the phones from phone carts #1 and #2 on the EOF Managers conference table, as shown in Figure 1.
- ____ 11. After steps 8, 9 and 10 have been completed, enable the deactivated EOF phones by turning on the EOF CARD A and CARD B switches in the communications Room 108.

NOTE: NO POSITION C EXISTS ON THE HOCS "T" SWITCHES.

- ____ 12. If directed by the Engineering Supervisor or the Dose Assessment Supervisor, reposition the HOCS "T" switches in the communications room 108.
- ____ 13. Remove the phones from the wall cabinets in the communications area and place on the counter.
- ____ 14. Position 2 tables in the dose assessment area as shown in Figure 1.
- ____ 15. Turn on the monitors for the EIP-9 and the EIP-29/30 computers. If the computers are not operating properly, refer to Attachment 2 (normally done by the Dose Analyst).
- ____ 16. Set up Room 118 as the RMT Control Area (normally done by the FMT Communicator).
- ____ 17. Obtain a master key (key 1, 2 or 3) and align the EOF ventilation system in the OUTSIDE AIR FILTRATION MODE, per Attachment 3, unless otherwise directed.
- ____ 18. Plug in the Tone Alert Radio in the center cabinet on the East wall. Test the radio by placing the switch in the MONITOR position and returning it to the ALERT position.
- ____ 19. Position the access control table and place the equipment on the table as shown in Figure 1. Phones are located in the cabinet below the FAX machines.
- ____ 20. Position chairs in Room 106 as appropriate.
- ____ 21. Move the coffee machines, microwaves, un-needed chairs and other equipment such as trash cans to the machine shop.
- ____ 22. Connect the phones in Room 105 as shown in Figure 2. The NRC can rearrange the room as they desire.

EOF SETUP

- _____23. Test as many phones in the EOF as possible by verifying that the phones can place a call, receive a call or ring when called.
- _____24. Pull down the shades on Room 112 and Room 106 exterior doors and on the door between the east stairwell and the machine shop.
- _____25. Normal access to the EOF should be from the central stairwell. Lock the doors from the following list. They should be unlocked for FMT access and as permitted by the EOF Manager/Support Coordinator for deliveries:

 - Room 112 Exterior Door
 - Room 106 Exterior Door
 - Room 121 Exterior Door
 - Machine Shop to East Stairwell
- _____26. Contact the TSC and determine the official Control Room time. Set the EOF wall clocks to the official Control Room time.
- _____27. One Security Force Member establish Access Control per FPN-0-EIP-7.0.

DOSE ASSESSMENT COMPUTERS

EIP-9 AND EIP-29/30 COMPUTERS

NOTE: THESE COMPUTERS ARE NORMALLY LEFT ON, WITH THE MONITORS TURNED OFF. THIS PROCEDURE MAY BE USED TO PLACE THE COMPUTERS ON LINE IF TURNING ON THE MONITOR DOES NOT SOLVE THE PROBLEM.

BOTH THE EIP-9 AND THE EIP-29/30 COMPUTERS HAVE THE SAME SOFTWARE LOADED ON THE HARD DRIVE.

INITIALS

- ____ 1.0 Verify that the power strip or UPS is plugged into the wall outlet.
- ____ 2.0 Verify that the power strip or UPS is turned on.
- ____ 3.0 Verify that the computer, monitor and printer are plugged into power strip or UPS.
- ____ 4.0 Verify that all components are turned on.
- ____ 5.0 Verify that the monitor is connected to the PC.
- ____ 6.0 Verify that the printer is connected to the PC.
- ____ 7.0 If the PC is now operating properly, further operation of the EIP-9 PC will be per FNP-0-EIP-9.3, and EIP-29/30 PC will be per FNP-0-EIP-29.0/30.0.
- ____ 8.0 If the PC is still not operating properly, contact the Support Coordinator to get the PC repaired.

EOF VENTILATION SYSTEMS

	CONTROL PANEL ROOM NUMBER	SWITCH LOCATION/ CONTROL PANEL	SWITCH ID	NORMAL OPERATING SWITCH POSITION	*OUTSIDE AIR FILTRATION MODE SWITCH POSITION	**ISOLATION MODE SWITCH POSITION	P & ID
1.	121 (Fig.5, VENT L/U ROOM A) (Mech Equip Rm)	CP-1	"EIVCS"***	OPEN	OPEN	CLOSE	D-180952
2.	121	CP-1	AH-2	START	START	START	D-180930
3.	121	CP-1	AH-5	START	STOP	STOP	D-180929
4.	121	HDDE2 DIST.PNL	BKR #1 (EF-1&2)	ON	OFF	OFF	D-174306
5.	121	HDDE2 DIST.PNL	BKR #2 (EF-4)	ON	OFF	OFF	D-174306
6.	121	CP-1	MD-2-1	CLOSE	OPEN	CLOSE	D-180930
7.	121	MCC1DD	BKR HDDE3 (AH-10)	ON	OFF	OFF	D-180958
8.	100(Fig.5, VENT L/U ROOM B) (Mach.Shop)	CP-2	AH-3	START	START	STOP	D-180930
9.	100	CP-2	MD-6-1	PRESS TO AUTO/ TURN TO MID	PULL TO MANUAL/ TURN TO MIN****	PULL TO MANUAL/ TURN TO MIN****	D-180925
10.	100	CP-2	AH-6	START	STOP****	STOP****	D-180931
11.	100	CP-2	MD-3-1	CLOSE	OPEN	CLOSE	D-180928
12.	100	CP-2	AH-4	START	START	START	D-180931 D-180925
13.	100	CP-2	EF-6	AUTO	OFF	OFF	D-180925
14.	100	CP-2	EF-7	AUTO	OFF	OFF	D-180925
15.	100	CP-2	EF-8	STOP	STOP	STOP	D-180925
16.	110(Fig.4, VENT L/U ROOM C)(Mech Equip Room outside simulator)	CP-8	AH-11	START	START	STOP for actual emergencies only START for drills or testing*****	D-180958
17.	110	CP-8	AH-12	AUTO	AUTO	STOP for actual emergencies only AUTO for drills or testing*****	D-180958
18.	113 (Fig.5, VENT L/U ROOM D)(Mech Equip Rm)	HEEE2 DIST.PANEL	BKR NO. 1 (EF-9)	ON	OFF	OFF	D-180924
19.	113	HEEE2 DIST.PANEL	BKR NO.3 (EF-12)	ON	OFF	OFF	D-180957
20.	113	CP-3	AH-1	START	START	START	D-180930
21.	113	CP-3	MD-1-1	CLOSE	OPEN	CLOSE	D-180930
22.	114 (Fig.5, VENT L/U ROOM E) (CHM Lab)	FUME HOOD	EF-10	OFF	OFF	OFF	D-180930

EOF VENTILATION SYSTEMS

	CONTROL PANEL ROOM NUMBER	SWITCH LOCATION/ CONTROL PANEL	SWITCH ID	NORMAL OPERATING SWITCH POSITION	*OUTSIDE AIR FILTRATION MODE SWITCH POSITION	**ISOLATION MODE SWITCH POSITION	P & ID
23	(Fig.4, VENT L/U ROOM I) TRN RECP. AREA	LP-2B SECTION 2	BKR NO.72 (EF-3)	ON	OFF	OFF	D-180922

NOTE 1: IF THE NURSES STATION IS STAFFED, INFORM THE NURSES STATION WHEN PERFORMING THIS ALIGNMENT. IF THEY DESIRE TO RESTORE AIR CONDITIONING AND HEATING, THEY MAY PERFORM THE STEPS FOLLOWING NOTE 2. IF THE NURSES STATION DOES NOT DESIRE TO REALIGN THE VENTILATION SYSTEM, THE LINE-UP IS COMPLETE WHEN THE FOLLOWING TWO STEPS ARE COMPLETED. THE FOLLOWING TWO STEPS WILL ALIGN EF-11, AH-10 AND EF-5. IN ADDITION, OTHER AIR CONDITIONING AND HEATING EQUIPMENT IN THE NURSES STATION WILL ALSO BE DEENERGIZED.

24	214(Fig.4, VENT L/U ROOM H) (Audio Visual Studio)	NSR19L550F-N, LP-2E	BKR 24	ON	OFF	OFF	N/A
25	214	NSR19L550F-N, LP-2E	BKR 33	ON	OFF	OFF	N/A

NOTE 2: THE FOLLOWING FOUR STEPS SHOULD NORMALLY BE PERFORMED BY SAFETY AND HEALTH STAFF WITH THE PERMISSION OF THE SUPPORT COORDINATOR, IF THE AIR CONDITIONING AND HEATING EQUIPMENT IS REQUIRED IN THE NURSES STATION AND THE FFD FACILITY.

26	260(Fig.4, VENT L/U ROOM G) Nurses Station FFD Toilet #1)	Light Switch	EF-11	ON	OFF	OFF	D-180955
27	272(Fig.4, VENT L/U ROOM H (NURSES STATION IAB)	NSR19L552-N Dist Cab	BKR EF-5	ON	OFF	OFF	D-174324
28.	214(Fig.4, VENT L/U ROOM H)(Aud.Vis. Stu)	NSR19L550F-N, LP-2E	BKR 24	ON	ON	ON	N/A
29.	214	NSR19L550F-N, LP-2E	BKR 33	ON	ON	ON	N/A
30.	Exterior Doors	In the outside air filtration mode and the isolation mode, verify that all exterior doors are closed to minimize the amount of non-filtered air that can enter the building. Entry and exit to the building should be minimized if there is a radiological or smoke hazard external to the building. Locking of the exterior doors is not required.					

- * Outside Air Filtration Mode - 10% outside air is combined with EOF return air and HEPA filtered to provide fresh air makeup and cleanup of the return air. This mode provides protection from particulate dose.
- ** Isolation Mode - No outside air is brought into the EOF ventilation system. While in this mode, protection from particulate, iodine, and noble gas is provided as consequence of outside air being isolated.
- *** "EIVCS" - Emergency Isolation Valves Control Switch (closes EIV-1, EIV-2, & EIV-3)
- **** Make adjustments to MD-6-1 prior to stopping AH-6
- ***** Damper VD-7 is specified to be closed in ES 91-2124, however, it is inaccessible. Damper SD 11-7 in series with VD 7 will close if smoke is in the intake and provide the required isolation in the filtration mode. Stopping AH-11 and 12 will provide the effect of isolation in the isolation mode due to no driving force to draw in air. AH-11 and 12 are left on during drills to prevent overheating the simulator equipment. (1998 EP Self Assessment Report)

**EOF DIGITAL ALARMING DOSIMETER
INSTRUCTIONS**

1. **IF** one of the area dosimeters in steps 2 and 3 goes into alarm, **THEN** perform the actions required, starting at step 4.
2. Activate five dosimeters as area dosimeters by pressing the button on the dosimeter one time.
 - 2.1 The EOF dosimeters are set up in the fast entry mode. Pressing the button one time will turn on the dosimeter; additional presses on the button will toggle the display from accumulated dose to current dose rate.
 - 2.2 In the fast entry mode, the dosimeters cannot be turned off by pressing the button on the dosimeter. Turn in activated dosimeters to the Emergency Planning group to have the dosimeters turned off.
 - 2.3 The dosimeters used in the EOF have default alarm setpoints for an accumulated dose of 100 mrem and dose rate alarm of 40 mrem/hr. The dosimeters for FMTs have default alarm setpoints for accumulated dose of 1 REM and dose rate alarm of 1 REM/hr. FMT dosimeters are labeled to indicate FMT use.
3. Locate the five area dosimeters activated in step 2 at the following locations, as shown in Figures 1 and 2:
 - 3.1 Adjacent to the outside door to room 106
 - 3.2 To the right of the FAX machines in room 106
 - 3.3 Below the emergency classification sign in room 106
 - 3.4 In the machine shop on the gauge rack located near the door to room 105
 - 3.5 In room 105 southwest corner
4. If one of the area dosimeters activated in steps 2 and 3 goes into a valid alarm based on dose or dose rate, perform the remaining steps.
 - 4.1 Inform the Dose Assessment Supervisor or EOF Manager, in that sequence to coordinate performing the remaining steps.
 - 4.2 Check the remaining dosimeters to determine their dose and dose rate.

- 4.3 Have the on-site FMT or other HP technicians dispatched to the EOF to evaluate radiological conditions.
- 4.4 Ensure that the EOF Manager, Dose Assessment Supervisor, Emergency Director, Engineering Supervisor and HP Supervisor are informed of the alarm and radiological conditions.
- 4.5 If radiological conditions warrant, have a Thermal Luminescent Dosimeter (TLD) issued to personnel at the EOF.
- 4.6 Consider other appropriate protective measures for the EOF staff, as necessary.
- 4.7 Issue a dosimeter, activated per step 2, to any individual assigned to the EOF that is being sent out of the area covered by the area dosimeters.
- 4.8 If there are other individuals in the Visitor/Training Center, consider relocation, issuing dosimetry or other appropriate measures.

SUPPORT COORDINATOR GUIDELINE**INITIALS**

- ____ 1. When accountability is required, the Support Coordinator or senior individual in the EOF will report the names of individuals who are known to be on site, who are assigned to the EOF and are missing to the CSC. Figure 4 of FNP-0-EIP-10.0 may be used as an aid.
- ____ 2. Assign personnel and supervise the set up of the EOF per step 4.
- ____ 3. Notify Southern Company Information Resources-Plant Farley Support per On-Call Memo, Attachment E, or Emergency Phone List, Step 18, to program EOF telephones per GO-EIP-138.

NOTE: THE PAX OPERATOR FUNCTION WILL NORMALLY BE MAINTAINED AT THE CSC. IN THE EVENT THAT THE CSC HAS TO BE EVACUATED, THE PAX OPERATOR FUNCTION CAN BE TRANSFERRED TO THE EOF.

- ____ 4. If it is desired to transfer the PAX operator function to the EOF, notify Southern Company Information Resources-Plant Farley Support per On-Call Memo, Attachment E, or Emergency Phone List, Step 18 to program the EOF Pax Console for Condition III per GO-EIP-138.
- ____ 5. Draw from available personnel in the OSCs and assembly areas to augment the EOF staff per step .
- ____ 6. Have preplanned augmented staff review appropriate portions of step 18 when they arrive in the EOF.
- ____ 7. Post press releases, initial Emergency Notification messages, and follow up Emergency Notifications.
- ____ 8. Act as a liaison between the EOF and the EOC staff in Birmingham to provide engineering and logistic support to the plant staff, TSC staff, and EOF staff.
- ____ 9. Telecopy the names and phone numbers of the EOF Manager, Support Coordinator, and Dose Assessment Supervisor to the state and local agencies.
- ____ 10. Contact the state and local agencies by phone to establish a working relationship.
- ____ 11. Provide information to state and county agency personnel at the direction of the EOF Manager.
- ____ 12. Provide information to the FEOC liaisons.

- _____ 13. Notify EOF Manager of communications problems or of significant offsite actions such as evacuations pending or in progress.
- _____ 14. Assist the state and county agencies in resolving problems with communications or other areas.
- _____ 15. ALERT DECLARATION. Announce in the EOF and inform the TSC and the EOC when the EOF is in STANDBY as described in step 23 of the main body of this procedure.
- _____ 16. SITE AREA or GENERAL EMERGENCY declaration. Announce in the EOF and inform the TSC and the EOC when the EOF is OPERATIONAL as described in step 23 of the main body of this procedure.
- _____ 17. Establish a link with the management bridge per the card in front of FNP-0-EIP-8.3. Contact the TSC and the CEOC to them to establish a link with the management bridge.

DOSE ASSESSMENT SUPERVISOR GUIDELINE

NOTE: STEPS MARKED WITH A "C" ARE CONTINUING ACTION STEPS.

INITIALS

- _____ 1. If the HOCS will be used to transmit followup messages, have the HOCS aligned for the system that will be transmitting messages.
- _____ 2C. Supervise transmission of followup messages to the state and local agencies at least hourly. Transmission every 30 minutes is the desired goal.
- _____ 3. If dose assessment/FMT control has been started from the TSC, turnover should occur in an expeditious manner.
- _____ 4. Establish and supervise dose assessment in the EOF per EIP-9.0.
- _____ 5. Assign and supervise an individual to coordinate FMT control from the EOF.
- _____ 6C. Review dose assessment and FMT data to ensure that the current emergency classification is at least as high as is required by radiological conditions. Provide input for classification and Protective Action Recommendation (PAR) changes to the EOF Manager.
- _____ 7. Assign a person to handle and log off site communications. (Normally EOF ENN Communicator.)
- _____ 8. Review previous initial and follow-up messages.
- _____ 9C. Review dose assessment/follow-up messages for accuracy prior to sending via HOCS or FAX.
- _____ 10. If the HOCS is not available, have the communicator fax the follow-up messages.
- _____ 11C. Establish personal communications and interface with state and local agencies for matters pertaining to radiological conditions, dose assessment and FMT control. Determine and report to the EOF Manager the names of the individuals in charge at these agencies.
- _____ 12C. Notify the EOF Manager of communications problems or of significant offsite actions such as evacuation pending or in progress.

NOTE: THERE ARE 5 AREA DIGITAL ALARMING DOSIMETERS POSTED IN THE EOF AREA AS AN AID IN EVALUATING RADIOLOGICAL CONDITIONS. ATTACHMENT 4 PROVIDES INSTRUCTIONS ON THE USE OF THE DOSIMETERS.

- _____ 13. If radiological conditions warrant, notify the HP Supervisor that Health Physics assistance is needed at the EOF. The on-site FMT may be used for this function.
- _____ 14. If radiological conditions warrant, have dosimetry issued by Access Control Security staff.
- _____ 15. If radiological conditions warrant, shift ventilation systems to the "isolation mode".
- _____ 16. If the EOF ventilation system is aligned to the ISOLATION MODE, then:
 - Notify the HP Supervisor to begin oxygen level monitoring of the air in the EOF.
 - Notify the EOF Manager that consideration should be given to relocating the EOF staff to the Alternate EOF within 24 hours.
- _____ 17. Determine if there is any additional staff required for Dose Assessment or FMT, and request that staff from the Support Coordinator.
- _____ 18. Ensure EOF Dose Assessment status boards are updated by assigning an individual to update them.
- _____ 19C. If there is a potential radiological hazard, ensure that all rooms on Figure 4 and Rooms on Figure 5 that are marked with an asterisk are evaluated by Health Physics, if the rooms are to be occupied continuously.

REFERENCES

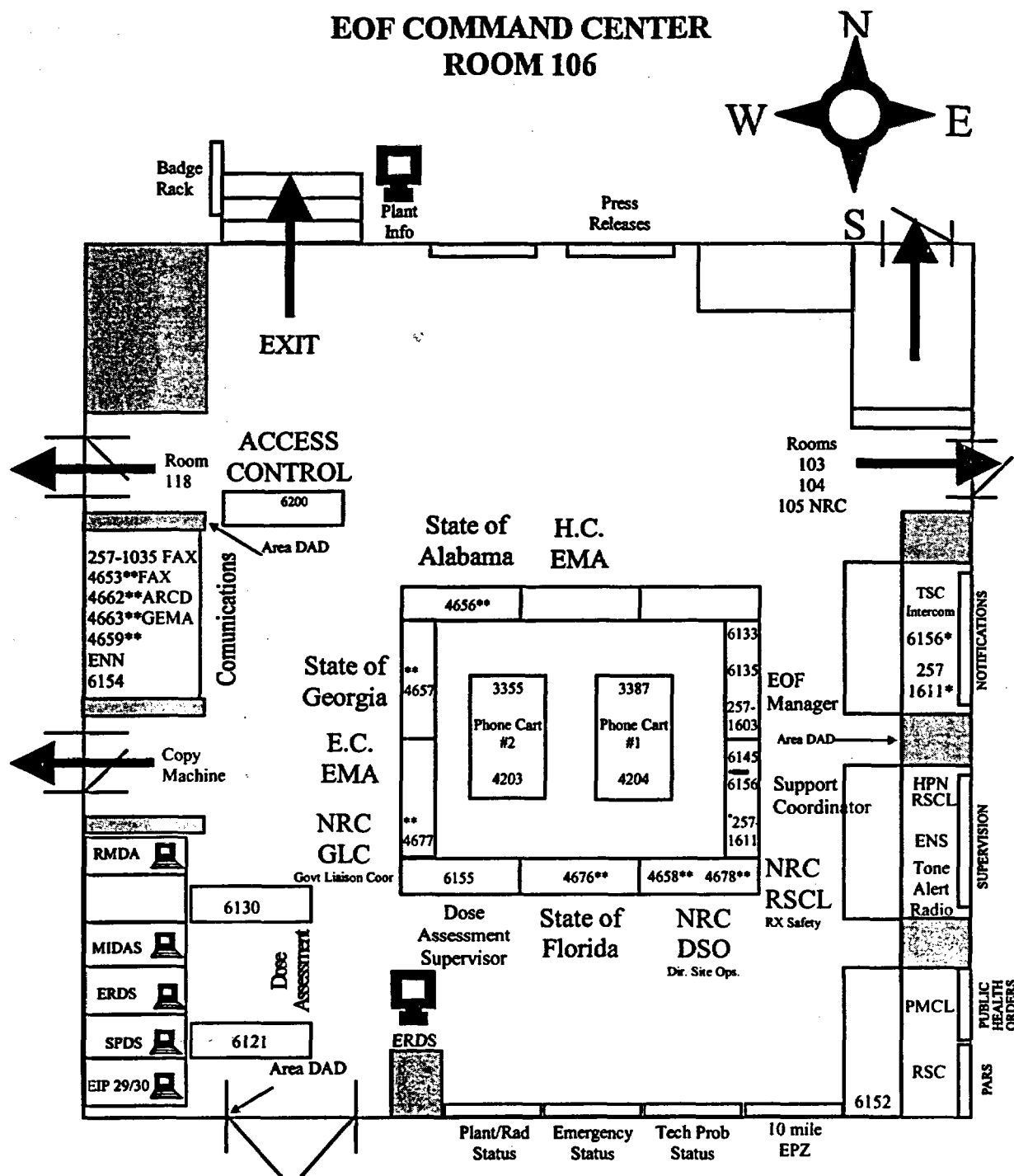
1. Joseph M. Farley Nuclear Plant Emergency Plan
2. GO-EIP-101, Nuclear Generation Department Corporate Emergency Organization
3. FNP-0-EIP-6.0, Technical Support to the Emergency Plan
4. D180876, D180881, D180884, D180897 EOF Communications Diagrams
5. Training and Emergency Operations Facility Drawings:
 - D-180930 HVAC - P&ID for AH-1 and AH-2
 - D-180928 HVAC - P&ID for AH-3 Sh. 2
 - D-180927 HVAC - P&ID for AH-3 Sh. 1
 - D-180931 HVAC - P&ID for AH-4 and AH-6
 - D-180925 HVAC - Air Flow Diagram for AH-4 and AH-6
 - D-180919 HVAC - General Arrangement Lower Floor El. 181.8
 - D-180950 HVAC - Control Sequence Sh. 1
 - D-180951 HVAC - Control Sequence Sh. 2
 - D-180952 HVAC - Control Sequence Sh. 3
 - D-180961 HVAC Det., LEG., NOTES, REF., and Control Sequence
6. U430867, Dictaphone Operation Manual
7. Emergency Operations Facility HVAC Evaluation Engineering Study, (ES) 91-2124

EMERGENCY FACILITY ACTIVATION

	<u>Unusual Event</u>	<u>Alert</u>	<u>Site Area Emergency</u>	<u>General Emergency</u>
Technical Support Center	*	Activate #	Activate #	Activate
Operations Support Center	*	Activate #	Activate #	Activate
Emergency Operations Facility	**	***	Activate #	Activate
Corporate Emergency Operations Center	**	***	Activate #	Activate
APC Emergency Response Center (ERC)	**	***	Activate #	Activate
Emergency News Center(ENC)##	N/A	****	***	Activate

NOTE: IT IS RECOMMENDED THAT THE FULL TSC AND EOF STAFFS BE CALLED IN AT THE "ALERT" LEVEL. AFTER EVALUATING PLANT CONDITIONS, STAFF MAY BE RELEASED BELOW A "GENERAL EMERGENCY" (AT THE DISCRETION OF THE EOF Manager/ED).

- * NO ACTION, STANDBY OR ACTIVATE AT THE DISCRETION OF THE EMERGENCY DIRECTOR.
- ** NO ACTION, STANDBY OR ACTIVATE AT THE DISCRETION OF THE EOF MANAGER.
- *** STANDBY OR ACTIVATE AT THE DISCRETION OF THE EOF MANAGER.
- **** ACTIVATION DEPENDENT ON LEVEL OF MEDIA INTEREST OR EOF ACTIVATION.
- # ACTIVATION WILL BE TO THE EXTENT DEEMED NECESSARY BY THE EMERGENCY DIRECTOR AND EOF MANAGER.
- ## AUTOMATICALLY ACTIVATED UPON EOF ACTIVATION.

SHARED**EOF COMMAND CENTER
ROOM 106**

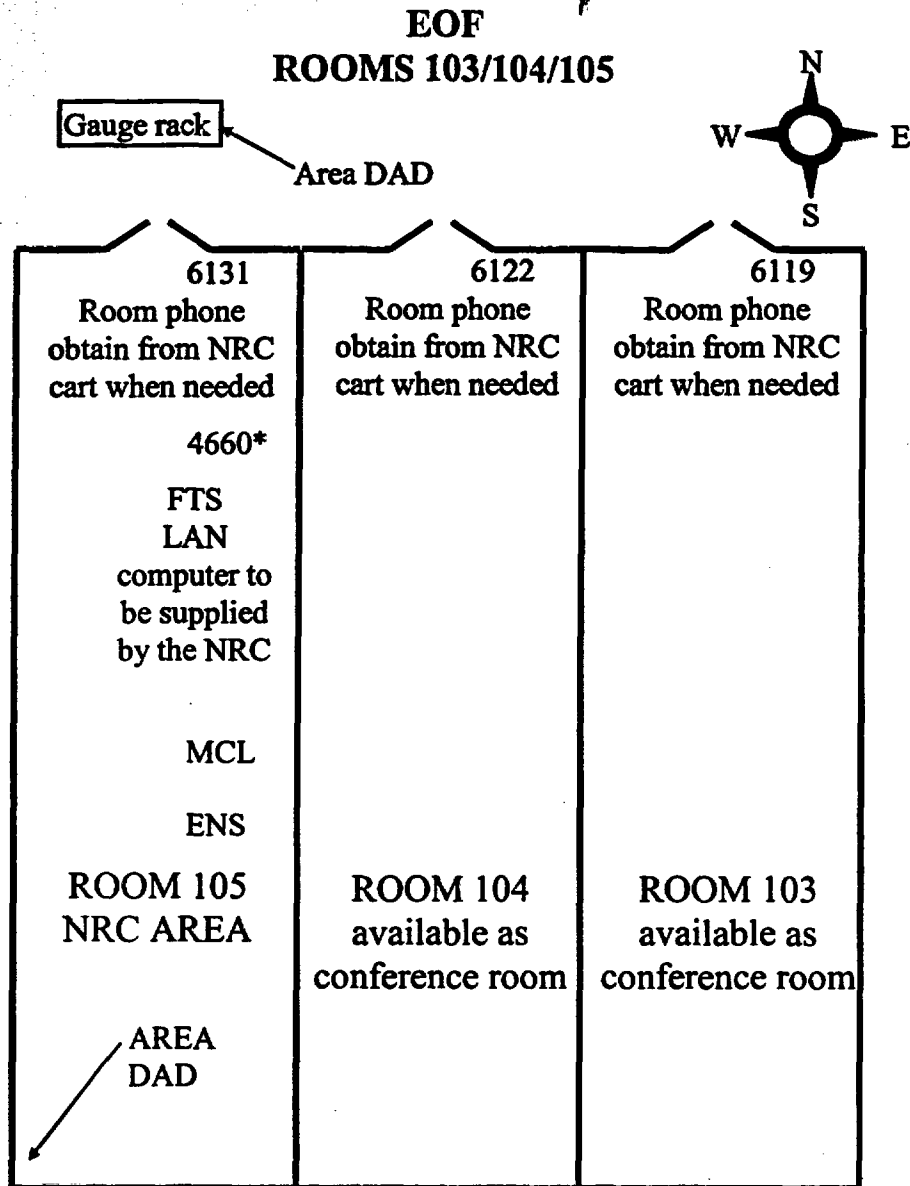
Note 1: Dial 4 digit PAX numbers directly from FNP phones or OPX. Use prefix 8-276 from other Southern Company phones. Dial 334-899-5156, ask for extension from commercial phones.

Note 2: Dial 257 prefix preceded by an 8 from Southern Company phones. Dial direct from commercial phones. Area Code 205 required outside Birmingham.

* Phones ring in two locations. Always active in Cabinet behind RM.

** Extensions are Direct Inward Dial numbers that can be dialed direct from an outside line with area code 334 and prefix 814, or Southern Company phones with 8-276 prefix or the 4 digit extension from PAX phones

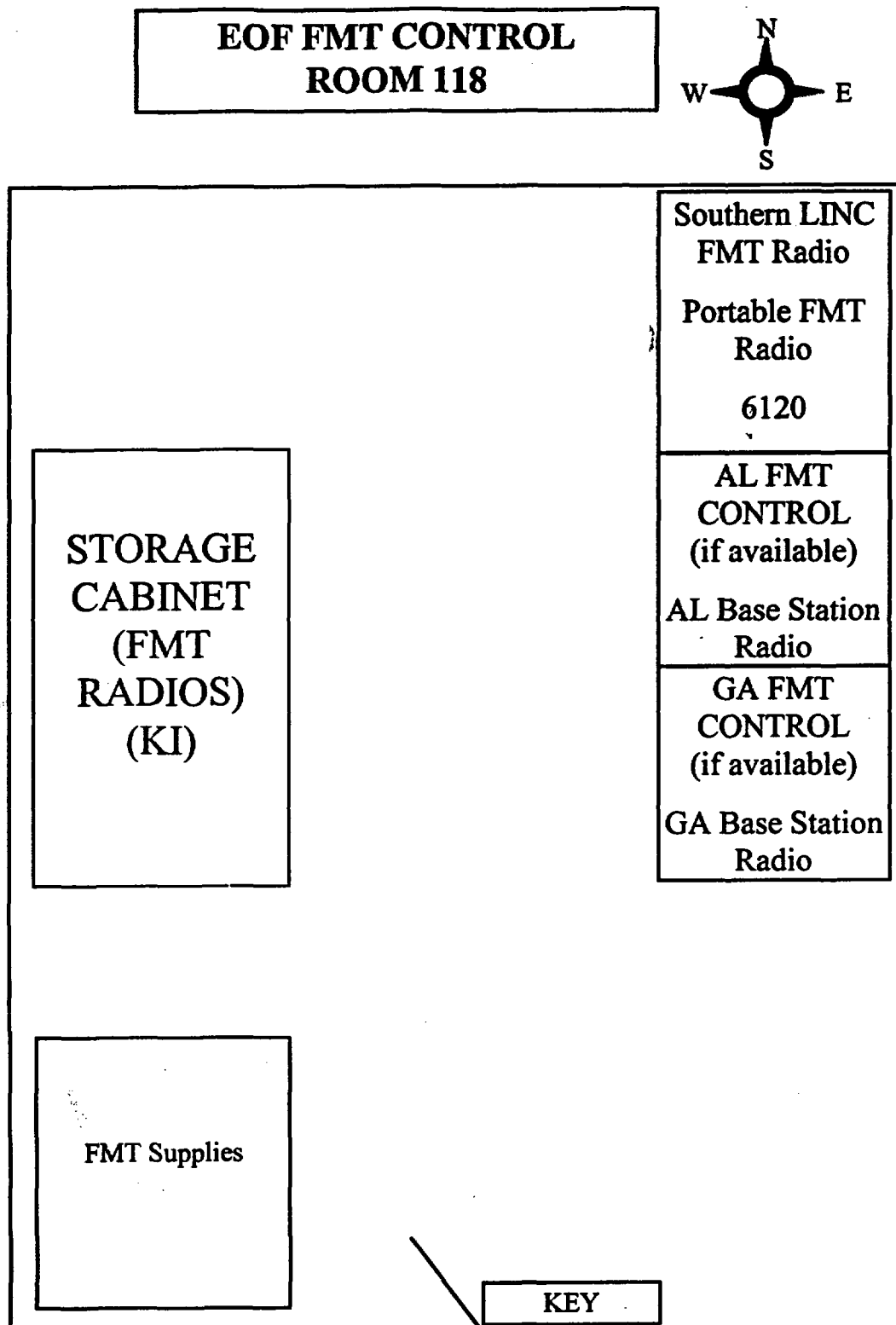
SHARED



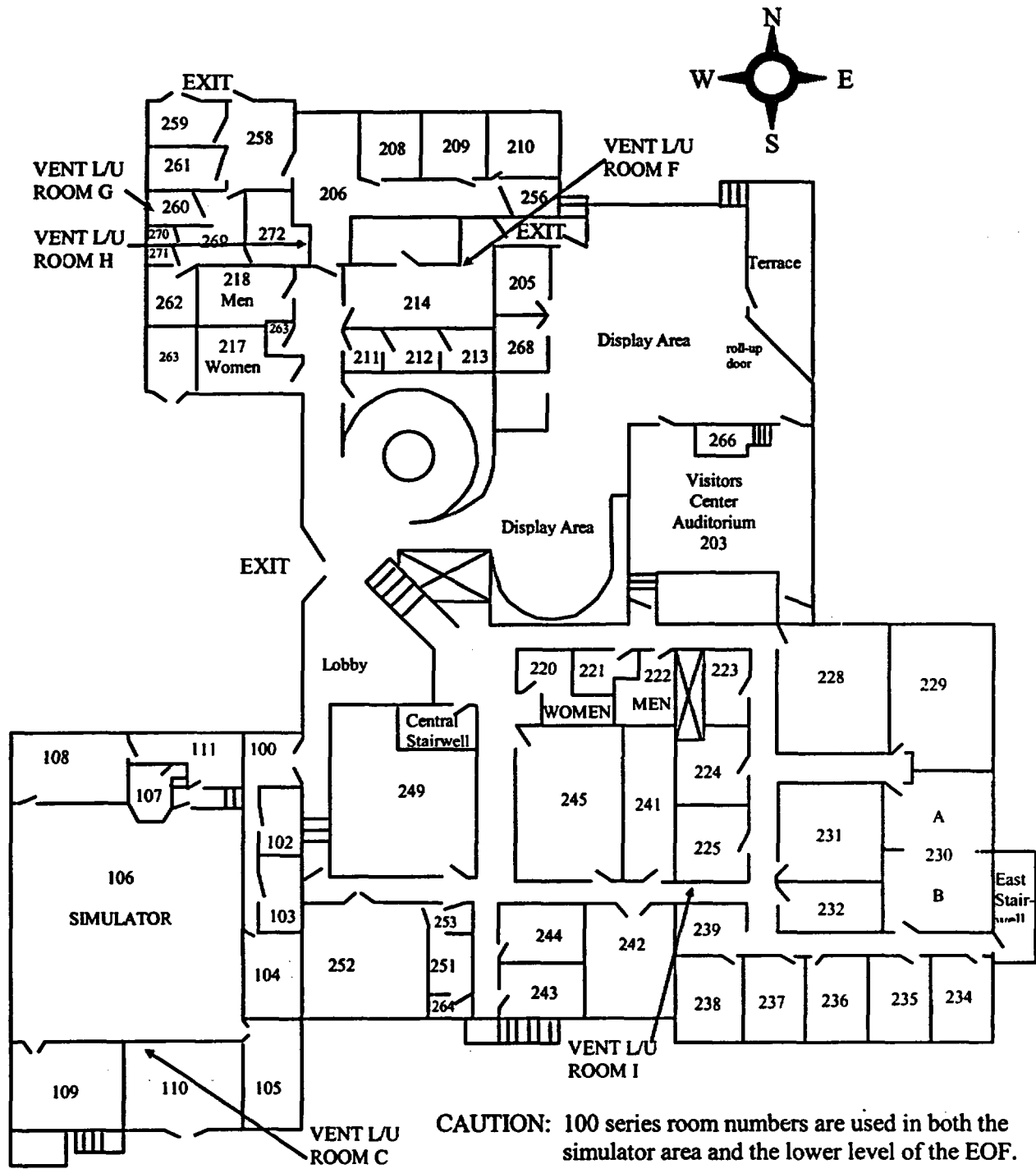
- * Ext. 4660 is a Direct Inward Dial extension that can be dialed directly from an outside line with area code 334 and prefix 814 or Southern Company phones with a 8-276 prefix or the four digit extension from PAX phones.

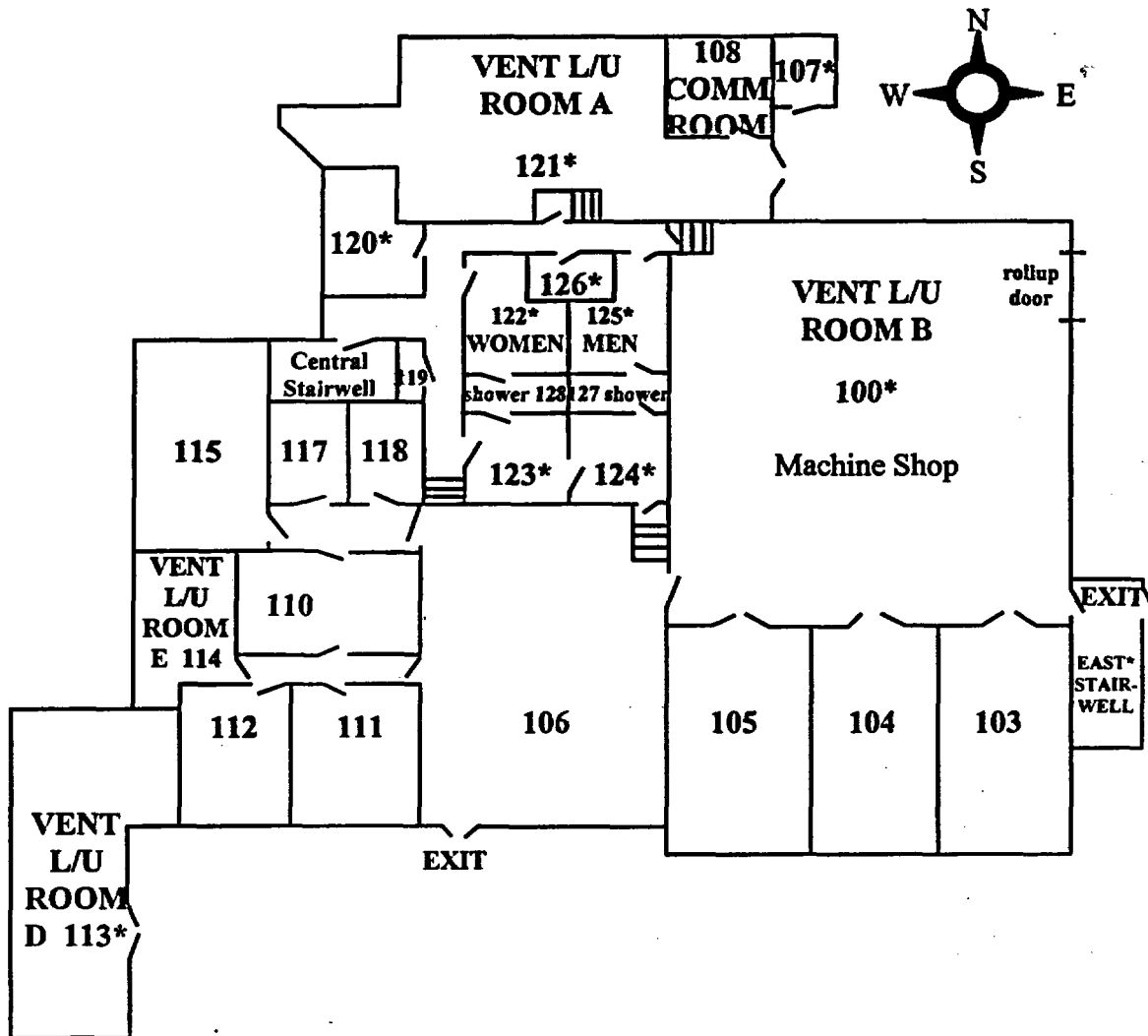
NOTE: The telephones in these rooms require an access code to access an outside line prior to reprogramming per GO-EIP-138.

SHARED



EOF
VISITOR/TRAINING CENTER
Upper Level and Visitor Center



SHARED**EOF
VISITORS/TRAINING CENTER
LOWER LEVEL**

CAUTION: Rooms that are marked with an asterisk (*) are not intended for continuous occupancy during an actual or potential radiological hazard. The required radiation shielding factor of 5 does not exist or has not been evaluated for these rooms.

CAUTION: 100 series room numbers are used in both the simulator area and the lower level of the EOF.

UNIT ____ PLANT STATUS

	TIME ____	TIME ____	TIME ____
CONTAINMENT			
DOME INSIDE AIR TEMP (DEG F)			
HIGHEST CTMT PRESSURE (PSIG)			
ECCS NR SUMP LEVEL (FT)			
HYDROGEN (H2 MANUAL) (%)			
RCS			
PRESSURE (PSIG)			
POWER RANGE FLUX (%)			
LOOP A FLOW (%)			
LOOP B FLOW (%)			
LOOP C FLOW (%)			
SUBCOOLING CHANNEL A (DEG F) (-) Indicates Superheat			
SUBCOOLING CHANNEL B (DEG F) (-) Indicates Superheat			
5th HOTTEST CETC (DEG F)			
PRESSURIZER LEVEL (%)			
LOWEST UPPER HEAD LEVEL (%)			
LOWEST UPPER PLENUM LEVEL (%)			
SG A WIDE RANGE LEVEL (%)			
SG B WIDE RANGE LEVEL (%)			
SG C WIDE RANGE LEVEL (%)			
ECCS			
RWST LEVEL (FT)			
RHR LOOP A FLOW (GPM)			
RHR LOOP B FLOW (GPM)			
CHARGING LINE FLOW (GPM)			
HHSI FLOW (FE0943) (GPM)			
AFW FLOW TO SG A (GPM)			
AFW FLOW TO SG B (GPM)			
AFW FLOW TO SG C (GPM)			

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UNIT ____ RADIATION STATUS

	TIME ____	TIME ____	TIME ____
CONTAINMENT			
HIGHEST HI LEVEL RAD R27(R/HR)			
RCS			
GROSS FAILED FUEL DET. R50 (CPM)			
METEOROLOGY			
WIND SPEED 35' ELEV (MPH)			
WIND SPEED 150' ELEV (MPH)			
WIND DIRECTION 35' (From-Degrees)			
WIND DIRECTION 150' (From-Degrees)			
STABILITY CLASS DELTA TEMP (DEG F) (35'-200')			
PLANT VENT EFFLUENT			
GAS MONITOR R14 (CPM)			
AIR PARTICLE MONITOR R21 (CPM)			
GAS MONITOR R22 (CPM)			
I-131 GAS-SPING4 R29B I2 (µc/ml)			
NOBLE GAS-SPING4 R29B NG (µc/ml)			
STACK FLOW FT2879 (CFM)			
SECONDARY EFFLUENT			
SJAE EXHAUST MONITOR R15C (R/HR)			
SG A ATMOS.RELIEF R60A (R/HR)			
SG B ATMOS.RELIEF R60B (R/HR)			
SG C ATMOS.RELIEF R60C (R/HR)			
AFW TURBINE EXHAUST R60D (R/HR)			
SG A N16 LEAK DETECTION R70A (GPD)			
SG B N16 LEAK DETECTION R70B (GPD)			
SG C N16 LEAK DETECTION R70C (GPD)			
VENT STACK SOURCE TERM			
NOBLE GAS(STERM-NG) (µcl/sec)			
IODINE-131 (STERM-12) (µcl/sec)			

EMERGENCY STATUS

CURRENT EMERGENCY CLASS: _____ DECLARED ON: _____ AT: _____
MMDDYY MEAN CENTRAL
TIME

REASON FOR CURRENT CLASS: _____

DATE/TIME

MMDDYY/MEAN

-MAJOR EVENT CHRONOLOGY EVENT DESCRIPTION-

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FNP-0-EIP-27.0
FIGURE 9

TECHNICAL PROBLEM STATUS

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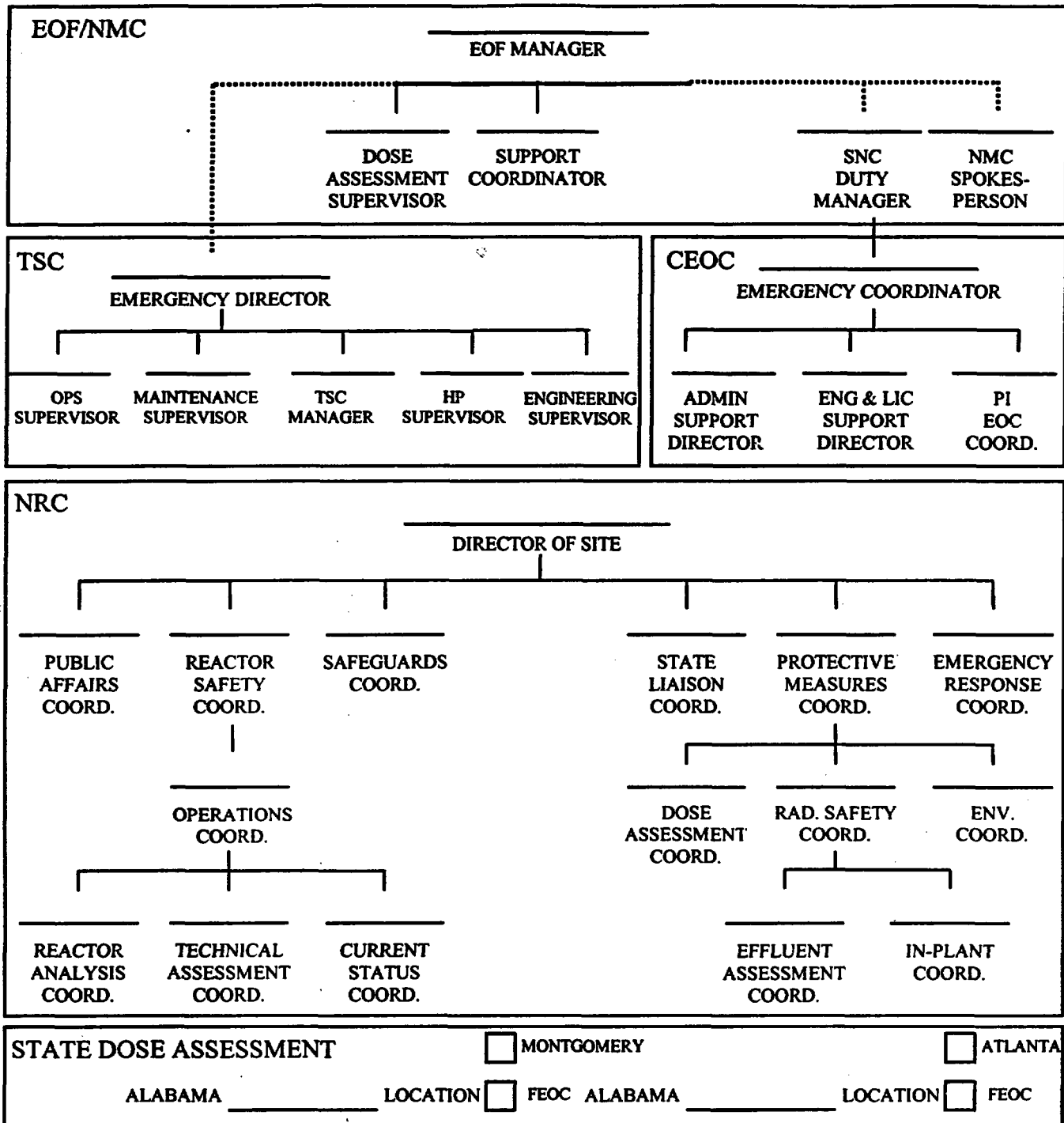
FNP-0-EIP-27.0

FIGURE 10

OFF-SITE PROTECTIVE ACTION

[illegible][illegible]

ON DUTY SUPERVISION



EOF/TSC COMMUNICATOR ACKNOWLEDGMENT/LOG

[illegible]

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SHARED

FNP-0-EIP-26.0
March 17, 2003
Version 45

FARLEY NUCLEAR PLANT

EMERGENCY PLAN IMPLEMENTING PROCEDURE 26.0

FNP-0-EIP-26.0

DUTIES OF THE EOF MANAGER

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PROCEDURE USAGE REQUIREMENTS per FNP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	GUIDELINES
Information Use	REMAINDER OF PROCEDURE

Approved:



Nuclear Plant General Manager

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8-29-03

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DUTIES OF THE EOF MANAGER

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5.0	Activation of the EOC in Birmingham	1
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7.0	EOF Manager's Movement in the EOF	2
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Guideline 2	EOF Managers EOF Operation Guideline	
Guideline 3	EOF Managers Press Release Guideline	
Table 1	References	

DUTIES OF THE EOF MANAGER

1.0 Purpose

This procedure provides guidance for the EOF Manager during emergency situations. Guideline 1, Guideline 2 and Guideline 3 of this procedure may be used to track completion of duties.

2.0 References

See Table 1.

NOTE: THIS PROCEDURE APPLIES TO THE EOF MANAGER'S ACTIVITIES DURING EMERGENCY CONDITIONS AT FARLEY NUCLEAR PLANT. CHANGES IN METHODS, AS SET FORTH IN THIS PROCEDURE, MAY BE MADE AT THE EOF MANAGER'S DISCRETION. THIS PROCEDURE IS NOT INTENDED TO BE ALL-INCLUSIVE, BUT IS A GUIDE WHICH MAY BE OF USE TO THE EOF MANAGER.

3.0 On-Call EOF Manager Assignment

The On-Call EOF Manager and alternates shall be assigned by the SNC Farley Project Vice President. These assignments will be documented in the FNP-0-EIP-8.1 On-Call Memo.

4.0 Required EOF Manager's Procedures

A copy of FNP-0-EIP-8.0, 8.1, 9.0, GO-EIP-114, and this procedure should be maintained with the On-Call EOF Manager at all times.

5.0 Activation of the CEOC in Birmingham

When the decision has been made to activate the Emergency Operations Facility (EOF), the Corporate Emergency Operations Center (CEOC) in Birmingham should also be activated. The EOF Manager will inform the SNC Duty Manager of the decision to activate the EOF. The ESM will inform the Emergency Coordinator (EC) to activate the CEOC. If the SNC Duty Manager is not available, the EOF Manager will directly inform the EC to activate the CEOC.

6.0 EOF Manager Responsibility

The EOF Manager shall have the responsibility for the overall direction of the SNC emergency activities, and with interfacing with offsite agencies. The EOF Manager shall, in addition to the specific actions delineated in applicable FNP EIPs and GO EIPs, follow the guidance provided in the RM Guideline 1, Guideline 2 and the EOF Managers Press Release Guideline, Guideline 3.

7.0 EOF Manager Movement in the EOF

- 7.1 The EOF Manager is free to move about the EOF within the limits described in FNP-0-EIP-27.0; however, he would normally be expected to remain in the Room 106/118 area.
- 7.2 In the event that the EOF Manager does leave the room 106/118 area, he should leave someone in administrative control of the command and control area. The individual left in charge will normally be the EOF Support Coordinator or the Dose Assessment Supervisor.
- 7.3 When the EOF Manager does leave someone else in charge of the command and control area, he is not relinquishing his responsibilities or authority.

EOF MANAGER INITIAL ACTIVATION GUIDELINE

Initials

- _____ 1. Confer with Control Room (CR) and/or Emergency Director (ED) and verify the correct emergency classification has been declared.
- _____ 2. Confer with CR and/or ED for current status/actions.
- _____ 3. Provide SNC Duty Manager with the following information:
 - DATE/TIME (of current declaration) _____
 - EMERGENCY DIRECTOR _____
 - EOF MANAGER _____
 - EMERGENCY CLASSIFICATION _____
 - DESCRIPTION _____
 - _____
 - OFFSITE DOSE/DOSE RATE _____
 - OFFSITE PROTECTIVE ACTION STATUS _____
 - _____
 - NOTIFICATIONS MADE _____
 - NOTIFICATIONS REQUESTS _____
 - TSC STATUS: ACTIVE STANDBY NONE
 - EOF STATUS: ACTIVE STANDBY NONE
 - CEOC STATUS: ACTIVE STANDBY NONE
 - ECO STATUS: ACTIVE STANDBY NONE
 - (EMERGENCY COMMUNICATIONS ORGANIZATION)
 - NMC STATUS: ACTIVE STANDBY NONE
 - (NEWS MEDIA CENTER)
 - REQUEST SPOKESPERSON NAME _____
- _____ 4. If the EOF is being staffed, contact the SNC Duty Manager or the EC and request that the CEOC be staffed per Step 5.

EOF MANAGER EOF OPERATION GUIDELINE

Initials

- _____ 1. Verify that the EOF Support Coordinator is performing duties per FNP-0-EIP-27.0.
- _____ 2. Verify that the Dose Assessment Supervisor is performing duties per FNP-0-EIP-27.0.

NOTE: STANDBY is defined as follows: For ALERT declaration, minimum staff available and able to perform required functions.

OPERATIONAL is defined as follows For SITE AREA EMERGENCY or GENERAL EMERGENCY declaration, minimum staff available and ready to turnover required functions.

- _____ 3. When the EOF is in STANDBY or OPERATIONAL, inform the following individuals and agencies: (The ENN may be used for those locations monitoring the ENN.)
 - a. EOF Staff
 - b. Emergency Director
 - c. SNC Duty Manager
 - d. Emergency Coordinator
 - e. Public Information Director
 - f. The agencies including the NRC that are listed on EIP-9.0 Figure 6 Side 2: (The ENN may be used for those locations monitoring the ENN.)
- _____ 4. Identify any federal, state or local agency personnel in the EOF and request they contact you (the EOF Manager) with information requests or concerns.
- _____ 5. Refer to FNP-0-EIP-9.0 periodically to verify current emergency classification is correct.
- _____ 6. Refer to FNP-0-EIP-28.0 as appropriate for down grade or close out of the emergency classification.
- _____ 7. Refer to FNP-0-EIP-28.1 as appropriate for down grade or close out of the emergency classification.
- _____ 8. Refer to EOF Manager's Press Release Guideline, Guideline 3 for providing information for making press releases.
- _____ 9. Conduct periodic EOF Staff and Agency personnel briefings and updates.

EOF MANAGER EOF OPERATION GUIDELINE

- _____ 10. Items for EOF Managers Consideration. Review these items periodically to help evaluate overall plant status.
- What is the current plant status? Based on the status, what actions are indicated?
 - Are TSC needs being met? Are any additional steps in support of the TSC necessary?
 - When was the last update message sent to the states?
 - Are any significant weather changes predicted? If so, what is their impact likely to be?
 - What are the current protective action recommendations, and do plant, radiological or weather conditions warrant a change in the recommendations?
 - What are the latest Radiation Monitoring Team survey results?
 - When was the last press release made and is another release needed?
 - Is/Are SNC/APCo spokesperson(s) available at the News Media Center?
 - When was the last briefing of SNC/APCo corporate management?
 - When was the last NRC briefing?
 - When was the last briefing of the EOF staff?
 - Are any additional notifications necessary?
 - Should additional assistance from any SNC/APCo departments or outside agencies be requested?
- _____ 11. Refer to the following list for responsibilities and duties of the EOF areas of responsibility to help evaluate EOF functional status.
- A. Administrative Support (EOF Support Coordinator)
- Ensure proper manning of the EOF.
 - Ensure communications are functioning properly.
 - Coordinate access control.
 - Assume responsibility for all EOF equipment and supplies.
 - Coordinate procurement.
 - Update on-duty status board.
 - Coordinate off-shift personnel needs.

EOF MANAGER EOF OPERATION GUIDELINE

- B. **Engineering and Licensing-Coordinate with CEOC (EOF Support Coordinator)**
 - Coordinate resolution of engineering and licensing problems.
 - Preparation of all written reports required by regulatory agencies.
 - Interface with offsite support companies.
 - Coordinate engineering manpower augmentation from offsite sources.

- C. **Dose Assessment (Dose Assessment Supervisor)**
 - Transmit follow-up messages to the states hourly (30 minutes is goal).
 - Interface with states as necessary.
 - Control Field Radiation Monitoring Teams (FMT).
 - Update EOF dose assessment status boards.
 - Ensure HP coverage of EOF.
 - Input as necessary for emergency classification changes.

EOF MANAGERS PRESS RELEASE GUIDELINE

Initials

- _____ 1. When requested, provide SNC Duty Manager /NMC Spokesperson/EC with the following information for press releases or news conferences:

Basic sequence of events and the times declared for each:

- Emergency classification _____
Declared at _____
Reason _____
- EOF activated at _____
Functional at _____
Current status _____
- Alert and notification system activated at _____
Siren activated at _____
Tone alert radios activated at _____
Current status _____
- State EMAs - emergency actions ordered by states; such as precautions, evacuations, ki (potassium iodide) use, etc.
Sheltering _____
Recommendations _____
Areas evacuated _____
Status of evacuation centers _____
Sampling of produce and milk _____
Evacuation of schools _____
- SNC/plant - plant information
What occurred _____
Status of repair efforts _____
Status of major plant equipment _____
Status of other unit _____
Status of off-site electrical systems _____
Plant casualties/injuries _____
Precautions taken at the plant ki (potassium iodide) use, etc _____

EOF MANAGERS PRESS RELEASE GUIDELINEInitials

- Radiation releases:

Source (SNC) _____

Amounts (State/SNC RMTs) _____

Offsite dose rate confirmation (State/SNC RMTs) _____

Identify affected areas (State/SNC RMTs) _____

Expected duration of release (SNC) _____

Future planned releases (SNC) _____

NOTE: Press releases will normally be made from the CEOC in Birmingham.

- _____ 2. Consider the items below if the RM is requested to approve a press release.

- The following positions are responsible for news release approvals:

Vice President - Farley Project or his designee (VP-FP)

SNC Duty Manager (DM) EOF Manager (EOFM)

Emergency Coordinator (EC)

Public Information Director (PID)

Public Information Emergency Coordinator (PIEC)

Corporate Media Coordinator (CMC)

NMC Coordinator (NMCC)

- All news releases must be approved by:

Prior to NMC activation -

VP-FP	_____	_____ and _____	_____	PID
or			or	PIEC
DM			or	CMC
or				
EC				

Following NMC activation -

VP-FP	_____	_____ and _____	_____	PID
or			or	NMCC
EOFM				
or				
EC				
or				
DM				

EOF MANAGERS PRESS RELEASE GUIDELINE

Initials

- _____ 3. Suggested News Release Contents
- Nature and classification of the accident and cause (if known)
 - Injuries (if none, say none); if injuries exist, then state injured employee or plant visitor, condition and location of treatment facility
 - Hazards to the public (if any) and their duration
 - Radioactive releases in progress (if none, say none)
 - Steps being taken to correct the situation and to protect the public
 - Damages (if any) and effect on the Alabama Power system
 - Likely extent and duration of any outages
 - Effects on other unit
- _____ 4. Refer to GO-EIP-114, "News Release Coordination and Distribution" for specific guidance for news releases.
- _____ 5. Refer to GO-EIP-118 for News Media Center activation.

DUTIES OF THE EOF MANAGER

REFERENCES

1. FNP EMERGENCY PLAN
2. GO-EIP-114, NEWS RELEASE COORDINATION AND DISTRIBUTION
3. FNP-0-EIP-8.0, NON-EMERGENCY REPORTS
4. FNP-0-EIP-8.1, EMERGENCY PHONE DIRECTORY
5. FNP-0-EIP-9.0, EMERGENCY CLASSIFICATION AND ACTIONS
6. FNP-0-EIP-28.0, RECOVERY
7. FNP-0-EIP-28.1, DE-ESCALATION
8. GO-EIP-118, EMERGENCY COMMUNICATION ORGANIZATION
CORPORATE ACTIVATION AND NOTIFICATION
9. FNP-0-EIP-27.0, EOF SETUP AND ACTIVATION

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SHARED

FNP-0-EIP-20.0
March 17, 2003
Revision 7

FARLEY NUCLEAR PLANT

EMERGENCY PLAN IMPLEMENTING PROCEDURE 20.0

FNP-0-EIP-20.0

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CHEMISTRY AND ENVIRONMENTAL SUPPORT TO THE EMERGENCY PLAN

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PROCEDURE USAGE REQUIREMENTS PER FNP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	ALL
Information Use	

Approved:



Nuclear Plant General Manager

Date Issued 8-29-03

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CHEMISTRY AND ENVIRONMENTAL SUPPORT TO THE EMERGENCY PLAN

1.0 Purpose

This procedure delineates the responsibilities of the Chemistry & Environmental group during emergency conditions.

2.0 References

See Table 1.

3.0 The EOF Dose Analyst:

3.1 Report to the Emergency Operations Facility (EOF), or other location directed by the EOF Manager.

3.2 Complete requirements of the EOF Dose Analyst Guideline (Guideline 1).

4.0 The On-Call Chemistry Supervisor Shall:

4.1 Report to the Technical Support Center (TSC), or other location designated by the Engineering Supervisor.

4.2 Complete requirements of the Chemistry Supervisor Guideline (Guideline 2).

5.0 The TSC Dose analyst Shall:

5.1 Report to the TSC, or location directed by the Emergency Director.

5.2 Complete requirements of the TSC Dose analyst Guideline (Guideline 3).

6.0 On-Shift Chemistry Technician-Sampling

The On-Shift Chemistry Technician assigned to sampling shall report to the OSC in the breakroom outside TSC and perform duties as instructed per FNP-0-EIP-0.0 and FNP-0-EIP-6.0.

7.0 On-Shift Chemistry Technician-Radiation Protection

The On-Shift Chemistry Technician assigned to radiation protection shall report to the OSC in the breakroom outside TSC and perform duties as instructed per FNP-0-EIP-0.0 and FNP-0-EIP-6.0. This position normally fills the On-Shift FMT Communicator position.

8.0 On-Shift FMT Communicator

8.1 The On-Shift individual who is assigned to FMT Communicator shall report to the TSC and perform the duties of FMT Communicator per FNP-0-EIP-0.0, 4.0, and 6.0.

8.2 A separate position On Shift for FMT Communicator is not required to be maintained. The On-Shift FMT Communicator position can be filled by the On-Shift Chemistry Technician-Radiation Protection, the On-Shift Chemistry Technician-Sampling, an HP Technician who is not assigned other duties by the EIPs, or other qualified RMT controller who is not assigned other duties by the EIPs.

9.0 On-Call Chemistry Technician

The On-Call Chemistry Technician shall report to the OSC in the breakroom outside TSC and perform duties as instructed per FNP-0-EIP-0.0 and FNP-0-EIP-6.0.

EOF Dose Analyst GuidelineInitials

- _____ A. Report to the EOF or other location directed by the EOF Manager.
- _____ B. Coordinate efforts to transfer dose assessment to the EOF as soon as possible.
- _____ C. Coordinate dose assessment in the EOF per FNP-0-EIP-9.0.
- _____ D. Operate the Meteorological Information and Dose Assessment System (MIDAS) computer as the primary method of dose assessment in the EOF.
- _____ E. In conjunction with the Dose Assessment Supervisor, coordinate the generation of follow-up notification messages for off-site dose assessment per FNP-0-EIP-9.0.
 - _____ E.1 Review dose assessment information for accuracy.
 - _____ E.2 Ensure that all pertinent information relative to plant conditions is obtained from the Dose Assessment Supervisor or EOF Manager.
 - _____ E.3 Obtain Dose Assessment Supervisor review and EOF Manager approval of the message as indicated by the Recovery Manager's initials or signature on the message and retain the signed copy.
 - _____ E.4 Ensure that the message is transmitted to all receiving locations and verify the EOF communicator ensures receipt of message at all receiving locations.

NOTE: Diversion of EOF lab drains is only required if radioactive samples are taken to the lab in the EOF.

- _____ F. Divert EOF lab drains to holding tank from sewage treatment plant if the EOF lab is to be used for analyzing radioactive samples. (CCP-1300 App. H & I)

Initials

- _____ G. Coordinate the issuance of personnel dosimetry in the EOF during accident conditions, as necessary.
- _____ G.1 Arrange for dosimetry issue by security group personnel.
- _____ G.2 Have security maintain log of all dosimetry devices issued.
- _____ G.3 Ensure personnel in the Visitors Center Assembly area are issued applicable dosimetry.
- _____ G.4 Ensure personnel in the EOF are issued applicable dosimetry.

NOTE: HP EQUIPMENT IS LOCATED IN THE CABINETS IN THE HALLWAY JUST OUTSIDE THE CENTRAL STAIRWELL AND ACROSS FROM THE JANITOR'S CLOSET.

- _____ H. If there are no HP personnel in the EOF, coordinate issuing of HP equipment as required.

CHEMISTRY SUPERVISOR GUIDELINEInitials

- _____ A. Report to the TSC or other location directed by the Engineering Supervisor or Emergency Director.
- _____ B. Determine from Operations Supervisor when the RCS sample, plant vent stack, and containment atmosphere samples are needed.

NOTE: ADEQUATE RCS PRESSURE MUST EXIST TO SAMPLE FROM THE HOT LEGS. IF ADEQUATE PRESSURE IS NOT AVAILABLE, THE RHR SYSTEM MUST BE TAKING A SUCTION FROM THE CONTAINMENT SUMP (RECIRCULATION PHASE) FOR THE SAMPLE TO BE ACCEPTABLE VIA THE RHR SAMPLE POINTS (REFERENCE CCP-1300, APP. F.).

- _____ C. Coordinate sampling and analysis of primary coolant and ECCS sump (via RHR) if required.
- _____ D. Coordinate, as needed, sampling and analysis of the plant vent stack for determination of effluent source term for use in FNP-0-EIP-9.0, Dose Assessment Calculations.
- _____ E. Coordinate sampling and analysis of containment atmosphere if required.
- _____ F. Coordinate sampling of site drinking water for radioactive contamination. If drinking water exceeds 10CFR20, App. B, Table 1, Column 2 limits, order quarantining and posting of affected outlets.
- _____ G. Coordinate sampling of sewage treatment plant as needed, isolating sources of excessive contamination.
- _____ H. Coordinate sampling of waste settling pond as needed, isolating sources of excessive contamination.
- _____ I. Coordinate dose assessment in the TSC or act as the Dose Analystt for dose assessment.

TSC Dose Analyst GuidelineInitials

- _____ A. Report to the TSC or other location directed by the Emergency Director.
- _____ B. Provide offsite dose information for the initial notification message if requested by the Engineering Supervisor or Emergency Director.
- _____ C. If the EOF has been activated, coordinate with the Engineering Supervisor and EOF staff to transfer dose assessment to the EOF. Once dose assessment has been transferred to the EOF, track dose assessment and keep TSC staff informed.
- _____ D. Perform dose assessment in the TSC per FNP-0-EIP-9.0.
- _____ E. Operate the MIDAS computer as the alternate method of dose assessment in the TSC and to follow the dose assessment done in the EOF.
- _____ F. If generating follow-up notification messages for off-site dose assessment per FNP-0-EIP-9.0, perform the following in conjunction with generating the message.
 - _____ F.1 Review dose assessment information for accuracy.
 - _____ F.2 Ensure that all pertinent information relative to plant conditions is obtained from the Engineering Supervisor or Operations Supervisor.
 - _____ F.3 Obtain Engineering Supervisor's review and Emergency Director's approval of the message, as indicated by the Emergency Director's initials or signature on the message. Retain the signed copy.
 - _____ F.4 Ensure that the message is transmitted to all receiving locations and verify the TSC communicator ensures receipt of message at all receiving locations.
- _____ G. If required to perform duties relative to radio chemistry, ensure that the dose assessment and follow-up message generation duties are transferred to the chemistry supervisor or other appropriate person.

TABLE 1

REFERENCES

1. Joseph M. Farley Nuclear Plant Emergency Plan
2. FNP-0-EIP-9.0, Radiation Exposure Estimation And Classification Of Emergencies
3. FNP-0-EIP-10.0, Evacuation And Personnel Accountability
4. FNP-0-EIP-14.0, Personnel Movement, Relocation, Re-Entry And Site Evacuation
5. FNP-0-CCP-1300, Chemistry And Environmental Activities During A Radiological Accident
6. FNP-0-EIP-0.0, Emergency Organization
7. FNP-0-EIP-4.0, Health Physics Support To The Emergency Plan
8. FNP-0-EIP-6.0, TSC Setup And Activation
9. FNP-0-EIP-9.1, ADMS - Automated Dose Assessment Method
10. FNP-0-EIP-9.3, Personal Computer - Automated Dose Assessment
11. FNP-0-EIP-27.0, EOF Setup And Activation

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FNP-0-EIP-14.0
March 17, 2003
Version 18

FARLEY NUCLEAR PLANT
EMERGENCY PLAN IMPLEMENTING PROCEDURE
FNP-0-EIP-14.0

PERSONNEL MOVEMENT, RELOCATION, RE-ENTRY AND
SITE EVACUATION

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PROCEDURE USAGE REQUIREMENTS per FNP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	ALL
Information Use	

Approved:



Nuclear Plant General Manager

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PERSONNEL MOVEMENT, RELOCATION, RE-ENTRY AND SITE EVACUATION

1.0 Purpose

This procedure provides the guidelines for movement, relocation, re-entry and evacuation of personnel after the initial sounding of the plant emergency alarm. A fire emergency shall be considered a special re-entry and procedures for re-entry during this type of emergency are addressed in FNP-0-EIP-13.0, Fire Emergencies.

2.0 References

See Table 1

3.0 Definitions

3.1 Movement

Personnel movement within the same assembly area building

e.g. Service Building Auditorium to Document Control

Visitors Center Auditorium to EOF

3.2 Relocation

- a. The transfer of an individual (includes re-assigning designated assembly area) from one assembly area to another.

e.g. Service Building Auditorium to Visitors Center Auditorium

Service Building Maintenance Shop to TSC

- b. Assignment of personnel from their assembly area to a location (non-assembly area) in the plant or on the plant site.

e.g. Control Room to Radiochemistry Labs

Service Building Auditorium to WTP

3.3 Re-entry

Entry into an area of elevated personnel hazards due to an emergency condition.

e.g. Entry into the piping penetration room to isolate an ECCS leak following a major loss of coolant accident

Entry into an area where the TEDE whole body annual dose limit of 5 rem can be exceeded during the expected time to perform the job (normally 30 minutes can be assumed to perform the job)

3.4 Site Evacuation

Organized withdrawal of personnel offsite.

4.0 Exemptions to the Guidance of This Procedure

- 4.1 Field Monitoring Teams (FMTs) are not required to be tracked or be authorized to move from one location to another by this procedure.
- 4.2 Personnel reporting to the site for duties in the TSC, EOF or the OCS are not required to be tracked or be authorized to report to their designated emergency response facility.
- 4.3 When the Emergency Director has Security informed of the emergency classification, he will inform Security of any access route restrictions, if necessary. Security will be issued a controlled copy of the on call memo for Security Post Gate 95 North and Security Post Gate 95 South. Personnel on the on-call memo, in any position or as an alternate, will be granted access to the site during an emergency unless specifically restricted by the Emergency Director.
- 4.4 Security will inform arriving personnel of access route restrictions, if necessary.
- 4.5 Personnel moving between the Control Room and the TSC are not required to be tracked or authorized by this procedure.
- 4.6 Personnel moving between the TSC and the breakroom outside the TSC are not required to be tracked or authorized by this procedure, unless the HP Supervisor has determined that a hazardous condition exists in the breakroom.
- 4.7 Personnel in the protected area delivering accountability logs to the PAP or the SAP are not required to be authorized by this procedure, but should be tracked by the senior individual in their assembly area.

5.0 Requirements for Movement

- 5.1 The senior individual in the assembly area shall authorize the movement of personnel.
- 5.2 The senior individual or his designee in the assembly area shall be responsible for tracking the movement of personnel.

6.0 Requirements for Relocation

- 6.1 The senior individual in the assembly area shall authorize the relocation of personnel, when requested to relocate individuals by the TSC, OSC or EOF.
- 6.2 The relocation guideline/log (Figure 1/2) may be used as a tracking mechanism for relocations.
- 6.3 The Control Room/TSC/OSC/EOF will inform the senior individual of any personnel hazards (toxic gas, radioactive release, etc.) that may exist. The senior individual in the assembly area shall provide a transit route for the relocation as appropriate.
- 6.4 Personnel who are being relocated from one designated assembly area to another designated assembly area will be tracked by the senior individual in the assembly area from which they are departing, until arrival at the new assembly area.
- 6.5 Personnel who are being relocated to an area outside of a designated assembly area will be tracked by the Control Room/TSC prior to the arrival of the OSC Manager, who will then have the responsibility.
- 6.6 Tracking and authorization for relocations involving personnel in the TSC/Control Room/OSC will be performed by the OSC Manager. If the OSC Manager is unavailable, the relocation responsibilities may be performed by the Maintenance Supervisor or Control Room.
- 6.7 Personnel who have been relocated outside of an assembly area will report to their assembly area, should the plant emergency alarm resound--unless specifically authorized to remain on station by the ED.
- 6.8 The following applies to security, regarding relocation of personnel:
 - 6.8.1 Remain on station until relocation is required.

6.8.2 When relocation becomes necessary, security supervision shall:

- Determine the route, with assistance from Control Room/TSC/OSC (Shift Supervisor/Maintenance Supervisor, or OSC Manager)
- Implement appropriate compensatory measures.
- Inform the ED of the compensatory measures taken.

6.9 Exposure limits for a re-location shall be limited to five rem TEDE, including the current dose to date.

6.9.1 Exposures in excess of one rem for a relocation, exclusive of current dose, shall be approved by the HP Supervisor, or the ED in his absence.

6.9.2 If an internal hazard is present, the limit for the relocation should be reduced by a factor of two.

7.0 Requirements for Re-entry - General Guidance

7.1 TLD badges of personnel who receive an emergency exposure in excess of the 10CFR20 limits of step 7.10 will be pulled and read prior to receiving further non-emergency exposure.

7.2 Re-entry personnel shall not deviate from a planned route unless unanticipated conditions such as rescue, performing an operation which would minimize the emergency condition, etc., require such a deviation.

7.3 If emergency dose rates observed during re-entry exceed the limits established by the re-entry guideline or other adverse conditions are encountered, re-entry personnel shall return to a safe area and contact the OSC/TSC/Control Room for further instructions.

7.4 If the Plant Emergency Alarm (PEA) is sounded while a re-entry team is involved in their assigned tasks, the re-entry team shall call the Control Room/TSC/OSC and request further instructions for assembly requirements.

7.5 The re-entry guideline/log (Figures 3/4) will serve as a tracking mechanism for re-entries. One copy of the guideline will remain with the OSC and, if desired, another copy will be given to the re-entry team leader. The guideline may be photocopied, or a two-part form may be used. The re-entry guideline will be sequentially numbered.

- 7.6 Individuals listed on the re-entry guideline as responsible for completion of guideline items are not required to personally initial the guideline, but are responsible for ensuring that each requirement is performed and initialed by the person performing or ensuring performance of the task.
- 7.7 Radiological monitoring will be established for each re-entry. The following parameters will be considered when determining the degree of radiological monitoring:
- Releases in progress
 - Dose rates, airborne and contamination levels
 - Stability of plant radiological conditions
- 7.8 Re-Entry is the responsibility of the Emergency Director, and requires verbal ED approval to execute a re-entry. Re-entries may be authorized and executed by the OSC Manager or Maintenance Supervisor, with ED approval. Approval to exceed 10CFR20 radiation exposure limits listed in step 7.10 must be approved by the Emergency Director. Approval to exceed plant administrative dose limits listed in step 7.10 must be approved by the HP Supervisor, or the Emergency Director in the HP Supervisor's absence.
- 7.9 An Emergency Repair Party which functions as a re-entry team shall consist of at least two (2) persons.
- 7.10 Farley Nuclear Plant personnel who have completed the onsite radiation protection training may be required to receive an exposure up to the following 10CFR20 limits:
- | | <u>10CFR20
limit</u> | <u>Administrative
limit</u> |
|------------------------|--------------------------|---------------------------------|
| Whole body (TEDE) | - 5 rem | - 2 rem |
| Lens of the eyes | - 15 rem | - 6 rem |
| Skin of the whole body | - 50 rem | - 20 rem |
| Extremities | - 50 rem | - 20 rem |
| Internal organs | - 50 rem | - 20 rem |
- 7.11 Dosimetry records for potential re-entry team members are available in the Dosimetry Lab.

CAUTION: EMERGENCY EXPOSURE LIMITS SHALL ONLY BE AUTHORIZED BY THE E.D.

- 7.12 Emergency situations may transcend the normal requirement of maintaining personnel exposures below 10CFR20 limits, as noted in step 7.10. Emergency exposures shall be minimized to every degree practicable. Farley Nuclear Plant personnel who have completed the onsite radiation protection training may be required to receive an exposure up to 25 rem TEDE for the activity and conditions described below. For those same personnel to receive in excess of 25 rem, they must voluntarily agree to receive an emergency dose in excess of 25 rem, but less than 100 rem. Persons volunteering to receive in excess of 25 rem must be made fully aware of the risks involved. Emergency exposure limits are as follows:

TEDE DOSE	ACTIVITY	CONDITION
10 REM	PROTECTING VALUABLE PROPERTY	LOWER DOSE NOT PRACTICAL
25 REM	LIFE SAVING OR PROTECTION OF LARGE POPULATIONS	LOWER DOSE NOT PRACTICAL
>25, <100 REM	LIFE SAVING OR PROTECTION OF LARGE POPULATIONS	VOLUNTEERS ONLY THAT ARE FULLY AWARE OF THE RISKS INVOLVED

Limit the dose to the lens of the eyes to 3 times the listed value. Limit the dose to other organs, including skin and extremities to 10 times the listed values.

NOTE: THERE IS CURRENTLY NO METHOD AVAILABLE TO ASSESS INTERNAL EXPOSURE ON A REAL TIME BASIS; THEREFORE, FARLEY NUCLEAR PLANT WILL UTILIZE AN ADMINISTRATIVE DEFAULT CORRECTION FACTOR OF TWO TO RELATE DEEP DOSE TO EXTERNAL EXPOSURE.

- 7.13 If an internal hazard is present, the limit for the re-entry should be reduced by a factor of two, unless a compensatory measure has been used to eliminate the internal hazard.

8.0 Requirements for Re-entry - Specific Guidance

- 8.1 The Emergency Director must verbally approve all re-entries.
- 8.2 The ED must approve doses that exceed the 10CFR20 limits of step 7.10.
- 8.3 The HP Supervisor or ED will complete the applicable portions of section II of the re-entry Guideline.
- 8.4 The HP Supervisor or designee will complete section IV of the Re-Entry Guideline.

- 8.5 The OSC Manager or Maintenance Supervisor (or ED, if OSC and Maintenance Manager are not available) will coordinate the re-entry and complete sections III and VI of the Re-Entry Guideline.
- 8.6 Re-Entry personnel shall:
- 8.6.1 Don necessary protective/emergency clothing and devices as prescribed in the re-entry guideline.
 - 8.6.2 Perform assigned duties in the emergency area as quickly and safely as possible.
 - 8.6.3 Report to the OSC Manager or TSC staff any unexpected conditions which may seriously affect their assigned duties.
 - 8.6.4 Without delaying the mission or causing unnecessary exposures, monitor the dose rate along the route followed to obtain radiological information, as appropriate.
 - 8.6.5 Frequently observe personal dosimeters and withdraw to a safe area prior to reaching an established dose limit, as applicable.
 - 8.6.6 Upon exiting, follow established self-monitoring and personnel decontamination procedures, as necessary, under the supervision of the individual charged with health physics monitoring.
 - 8.6.7 Record and report to the OSC Manager or TSC staff the radiological conditions, damage assessments, or any actions taken in the emergency area.
 - 8.6.8 Complete applicable sections of the re-entry guideline.

9.0 Site Evacuation

- 9.1 Site Evacuation can be accomplished in one of the following two ways:

- 9.1.1 Site dismissal of non-involved personnel without monitoring.

This method can be used to perform an organized withdrawal of personnel that are not involved in combating the casualty to an offsite location during an emergency condition when there is no emergency radioactive release in progress.

9.1.2 Site dismissal of non-involved personnel with monitoring.

This method can be used to perform an organized withdrawal of personnel that are not involved in combating the casualty to an offsite location during an emergency condition when there is an emergency radioactive release in progress.

9.2 Involved personnel that should not be routinely dismissed from the site include, but are not limited to:

- Personnel that are included on the on call memo as Emergency Response Organization staff. Individuals that will be required for relief and long term staffing should be released from the site as appropriate.
- Health Physics staff, including Health Physics contractors that are qualified to perform HP technician duties. Individuals that will be required for relief and long term staffing should be released from the site as appropriate.
- Chemistry staff that are qualified to perform Chemistry technician duties. Individuals that will be required for relief and long term staffing should be released from the site as appropriate.
- Operations staff that are required for safe operation of the plant and personnel necessary to mitigate the consequences of the event in progress. Individuals that will be required for relief and long term staffing should be released from the site as appropriate.
- Maintenance personnel necessary to mitigate the consequences of the event in progress. Individuals that will be required for relief and long term staffing should be released from the site as appropriate.
- Warehouse personnel necessary to support maintenance activities.
- Security staff required to maintain appropriate site security.
- Personnel that have been augmented to any of the emergency response facilities or other areas of the plant to help control the plant.

9.3 Non-involved personnel that should routinely be dismissed from the site include, but are not limited to:

- Personnel that are not currently qualified as radiation workers.
- Contractors and vendors except as noted above.
- Visitors
- Other personnel that are on site that do not meet the guidelines established in step 9.2.

9.4 Site dismissal should be considered any time there is a declared emergency with a potential for escalating to the point where there is hazard to on site personnel.**9.5 Emergency Director shall authorize site dismissals and appoint an individual to coordinate the dismissal.**

- 9.6 If the EOF is staffed when a site dismissal is required, the Emergency Director should normally request that the EOF coordinate the dismissal.
- 9.7 If there is an emergency radiological release in progress, go to step 9.9 to conduct a site dismissal of non-involved personnel with monitoring.
- 9.8 If there is no emergency radiological release in progress, conduct a site dismissal of non-involved personnel without monitoring as follows.
- 9.8.1 Confer with Houston County EMA and Early County EMA and determine if there are any off-site concerns with dismissing personnel from the site at this time.
- 9.8.2 Determine off site routes. Unless there have been restrictions placed on routes by the Houston County or Early County EMA, there should be no restrictions on off site routes.
- 9.8.3 Determine the on site exit routes using Figure 5. The roads leading to the north and south gates may have road blocks in place to prevent unauthorized access. Contact security to open access to these routes if they will be used. Access can be opened rapidly if it is required.
- 9.8.4 Contact management/supervision for groups or departments on site and determine who is considered to be non-involved in the current emergency, using the guidance of steps 9.2 and 9.3.
- 9.8.5 Inform security of the intent to dismiss personnel and authorize their exit from the site.
- 9.8.6 Inform the personnel that have been designated as non involved of:
- The on site exit routes
 - Off site route restrictions
 - Directions on what to do after exiting the site such as stand by home phones, call in at a specific time, etc
 - Provide any other specific guidance required based on plant conditions.
- 9.8.7 Inform designated non-involved personnel that they are dismissed with the instructions of step 9.8.6.
- 9.9 If there is an emergency radiological release in progress, conduct a site dismissal of non-involved personnel with monitoring as follows.

- 9.9.1 Prior to ordering a site dismissal of non-involved personnel with monitoring, the following items should be considered along with other extenuating circumstances:
- Will individual dose be greater for leaving the site or sheltering?
 - When will the off-site authorities be able to cope with the traffic flow?
 - When will the off-site authorities be able to cope with the influx of people to the de-contamination and reception centers?
 - What is the availability of food and supplies if no dismissal is ordered?
 - What will be the effect on plant personnel and families if no dismissal is ordered?
 - What personnel will be required on site for effective plant operation and recovery?
- 9.9.2 Confer with Houston County EMA and Early County EMA and determine the off-site concerns with dismissing personnel from the site at this time. Coordinate the dismissal time with Houston County EMA and Early County EMA.
- 9.9.3 Determine the off site routes and the reception center locations to send plant personnel to for monitoring and possible decontamination from Houston County EMA and Early County EMA. The reception center in Houston County will normally be the Wiregrass Recreation Center or the Houston County Farm Center. The reception center in Early County will normally be the Early County High School. Figures 6 and 7 can be used as aids.
- 9.9.4 Determine the on site exit routes using Figure 5. The roads leading to the north and south gates may have road blocks in place to prevent unauthorized access. Contact security to open access to these routes if they will be used. Access can be opened rapidly if it is required.
- 9.9.5 Contact management/supervision for groups or departments on site and determine who is considered to be non-involved in the current emergency, using the guidance of steps 9.2 and 9.3.
- 9.9.6 Confer with Health Physics to determine if on site monitoring prior to leaving the site is ALARA and appropriate, or if dispatch to the reception centers off site without on site monitoring is appropriate.
- 9.9.7 If on site monitoring is appropriate prior to being dismissed, designate locations for the on site monitoring to be accomplished for personnel from the protected area and personnel from outside the protected area.

9.9.8 Inform security of the intent to dismiss personnel and authorize their exit from the site.

9.9.9 Inform personnel that have been designated as non involved of:

- where to go to get monitored on site if appropriate.
- the on site exit routes.
- off site route and what reception center to go to.
- directions on what to do after leaving the reception center such as stand by home phones, call in at a specific time, etc.
- provide any other specific guidance required based on plant conditions.

9.9.10 Inform designated non-involved personnel that they are dismissed with the instructions of step 9.9.9.

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**PERSONNEL MOVEMENT, RELOCATION, RE-ENTRY
AND SITE EVACUATION**

REFERENCES

- Joseph M. Farley Nuclear Plant Emergency Plan
- EPA Emergency Worker and Lifesaving Activity Protective Action Guide
- IE Information Notice No. 84-40: Emergency Worker Doses
- NCRP No. 91
- SNC EPA 400 Manual Interpretation Document
J. D. Woodard to D. N. Morey, June 7, 1994

PERSONNEL MOVEMENT, RELOCATION, RE-ENTRY AND SITE EVACUATION**RISKS ASSOCIATED WITH ACUTE HIGH
LEVEL RADIATION EXPOSURE****HEALTH EFFECTS ASSOCIATED WITH WHOLE BODY ABSORBED DOSES
RECEIVED WITH A FEW HOURS***

WHOLE BODY ABSORBED DOSE (RAD)	Forewarning Symptoms of More Serious Health Effects Associated with Large Doses of Radiation (PERCENT AFFECTED)
50	2%
100	15%

**APPROXIMATE CANCER RISK TO AVERAGE INDIVIDUALS FROM
25 REM EFFECTIVE DOSE EQUIVALENT, DELIVERED PROMPTLY***

AGE AT EXPOSURE (YEARS)	APPROXIMATE RISK OF PREMATURE DEATH (DEATHS PER 1,000 PERSONS EXPOSED)	AVERAGE YEARS OF LIFE LOST IF PREMATURE DEATH OCCURS (YEARS)
20 TO 30	9.1 (.91%)	24
30 TO 40	7.2 (.72%)	19
40 TO 50	5.3 (.53%)	15
50 TO 60	3.5 (.35%)	11

PROMPT EFFECTS OF ACUTE RADIATION EXPOSURE**

ACUTE DOSE REM	PROBABLE CLINICAL EFFECT
0-25	No observable effects.
25-100	Slight blood changes, no other observable effects.
100-200	Vomiting may occur in 5 to 50% within three hours, with fatigue and loss of appetite. Moderate blood changes are likely. Except for the blood forming organs, recovery will occur in essentially all cases within a few weeks.

* REFERENCE EPA 400 MANUAL

** REFERENCE INPO GUIDANCE

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FNP-0-EIP-14.0
FIGURE 1

RELOCATION GUIDELINE

RELOCATION # _____

DESCRIPTION: _____

RELOCATION FROM: _____

RELOCATION TO: _____

DUTIES: _____

ESTIMATED TIME TO COMPLETE: _____

TRANSIT ROUTE: _____

HP REQUIREMENTS: THE MAXIMUM ALLOWED DOSE FOR THE RE-ENTRY IS FIVE REM, INCLUDING PREVIOUS EXPOSURE FOR THE YEAR.

IF AN EXTERNAL HAZARD IS INVOLVED, EXTERNAL DOSE LIMIT SHOULD BE REDUCED BY A FACTOR OF TWO.

DOSE LIMIT: _____ HP SUP/ED APPROVAL IF >1 REM: _____
(N/A IF NO RAD HAZARD)

OTHER HP REQ: _____

PERSONNEL: _____

☐ Continued on Additional Sheet

APPROVAL: _____

TIME OUT: _____ TIME IN: _____

CALL BACK PHONE NUMBER: _____

COMMENTS: _____

04/03/03 13:05:36

SHARED

FNP-0-EIP-14.0
FIGURE 2

RELOCATION LOG

SHEET # _____

TIME

[illegible]

SHARED**RE-ENTRY GUIDELINE**

RE-ENTRY FOR: _____ RE-ENTRY # _____
UNIT # _____ DATE/TIME _____ / _____

**NOTE: THE STEPS OF THIS GUIDELINE MAY BE DONE IN ANY ORDER PRIOR
TO DISPATCHING THE RE-ENTRY TEAM.**

OSC MANAGER SECTION I

____ A. Obtain ED verbal approval for the reentry.
OSC MGR

____ B. Select qualified personnel for the re-entry.
OSC MGR
*team leader * _____

____ C. Specify duties for re-entry:
OSC MGR (notes if desired) _____

____ D. Specify transit route (discuss):
OSC MGR (notes if desired) _____

____ E. Specify communications and actions
OSC MGR to take if communications cannot be established:
call back number 1. _____ 2. _____
gaitronics _____
radio _____

____ F. Dispatch re-entry team when HP requirements
OSC MGR per page 2 are met and brief per page 3 is complete.

HEALTH PHYSICS SECTION II

- A. Specify dose and dose rate limits.
HP
- APPROVED DOSE: _____
- APPROVED DOSE RATE: _____
- B. Authorize dose limits less than 10CFR20 limits and greater than admin limits per
HP SUP paragraph 7.10.
- C. Authorize dose limits greater than 10CFR20 limits in paragraph 7.10.
ED
- E. Have re-entry personnel complete appropriate sections of re-entry individual
HP exposure record including signing the form if 10CFR20 limits will be exceeded and complete Re-Entry Individual Exposure Record.
- D. Verify that the approved dose will not cause the individual(s) to exceed FNP HP
HP admin limits unless approved by HP Supervisor or 10CFR20 limits unless approved by the ED.
- E. For doses in excess of 25 rem, the following two steps must be performed:
1. Verify that the individual to receive the dose is a volunteer.
HP SUP
2. Ensure that the individual to receive the dose has been briefed and is fully
HP SUP aware of the risks involved. (Use table 2 as guidance for the brief.)
- F. Are Thyro Block (KI) tablets required? ☐ yes ☐ no (Ref FNP-0-EIP-4.0, Fig 3)
HP SUP
- G. Specify appropriate protective clothing and monitoring devices.
HP

- | | |
|--|---|
| <input type="checkbox"/> STREET CLOTHES | <input type="checkbox"/> SINGLE W/B TLD |
| <input type="checkbox"/> STD LABCOAT DRESSOUT | <input type="checkbox"/> MULTIBADGE |
| <input type="checkbox"/> STD CVRALL DRESSOUT | EXT TLD <input type="checkbox"/> HANDS <input type="checkbox"/> FEET |
| <input type="checkbox"/> CLOTH <input type="checkbox"/> PAPER <input type="checkbox"/> PLASTIC | PICS <input type="checkbox"/> 200MR <input type="checkbox"/> 2R <input type="checkbox"/> 5R |
| <input type="checkbox"/> SCBA | <input type="checkbox"/> DAD |
| <input type="checkbox"/> OTHER RESPIRATOR _____ | |
| <input type="checkbox"/> OTHER _____ | |

COMBINED BRIEF SECTION

Conduct a pre-job brief of the Re-Entry. The following information must be included:

- Duties for the re-entry including required procedures and safe work practices. Reference the OSC managers section and the re-entry duties section.
- Hazards associated with the assigned tasks (Radiological and Non Radiological)
- Dose and dose rate limits while performing the re-entry (per Health Physics section)
- Personnel protective equipment required (per Health Physics section if radiological)
- Isolation and control of energy sources (Clearance)
- Special support needs and precautions
- Transit route. It is acceptable for the team to modify the transit route based on the conditions encountered during the re-entry. If the route is modified, the OSC or control room should be notified as soon as possible if the change places the team in areas that are not on the route.
- Communications and actions to take if communications cannot be established

The following information may be considered in the pre-job briefing:

- Industry experience
- Plant or equipment conditions including potential radiological or industrial safety hazards and precautions
- Each person's job or task assignment
- Expected sequence of events and results
- Problems to be anticipated
- Criteria to be used to stop the evolution
- Contingencies if the evolution is stopped or the expected result is not achieved
- Potential distractions and how they will be minimized
- Housekeeping and fluid system cleanliness requirements
- Chemical control and disposal requirements
- Foreign Material Exclusion (FME) Controls

SHARED

RE-ENTRY TEAM SECTION

NOTE: IT IS NOT REQUIRED FOR THE RE-ENTRY TEAM TO KEEP A COPY OF THIS FORM WITH THEM DURING THE RE-ENTRY.

- ____ A. Monitor dose and dose rate and retreat to a safe area if limits are exceeded.
Team
- ____ B. Report unexpected conditions to the TSC or OSC as applicable.
Team
- ____ C. Monitor and record dose rates along re-entry route without delaying the mission or causing unnecessary exposure.
Team
- ____ D. Report to the TSC or OSC as applicable, radiological conditions, damage assessment or any actions taken during the re-entry.
Team
- ____ E. Perform the task assigned to the re-entry team.
Team

DEBRIEF SECTION

- ____ A. Debrief re-entry personnel.
OSC MGR
- ____ B. Report debriefed information to the appropriate staff in the TSC.
OSC MGR

04/03/03 13:05:36

SHARED

FNP-0-EIP-14.0
FIGURE 4

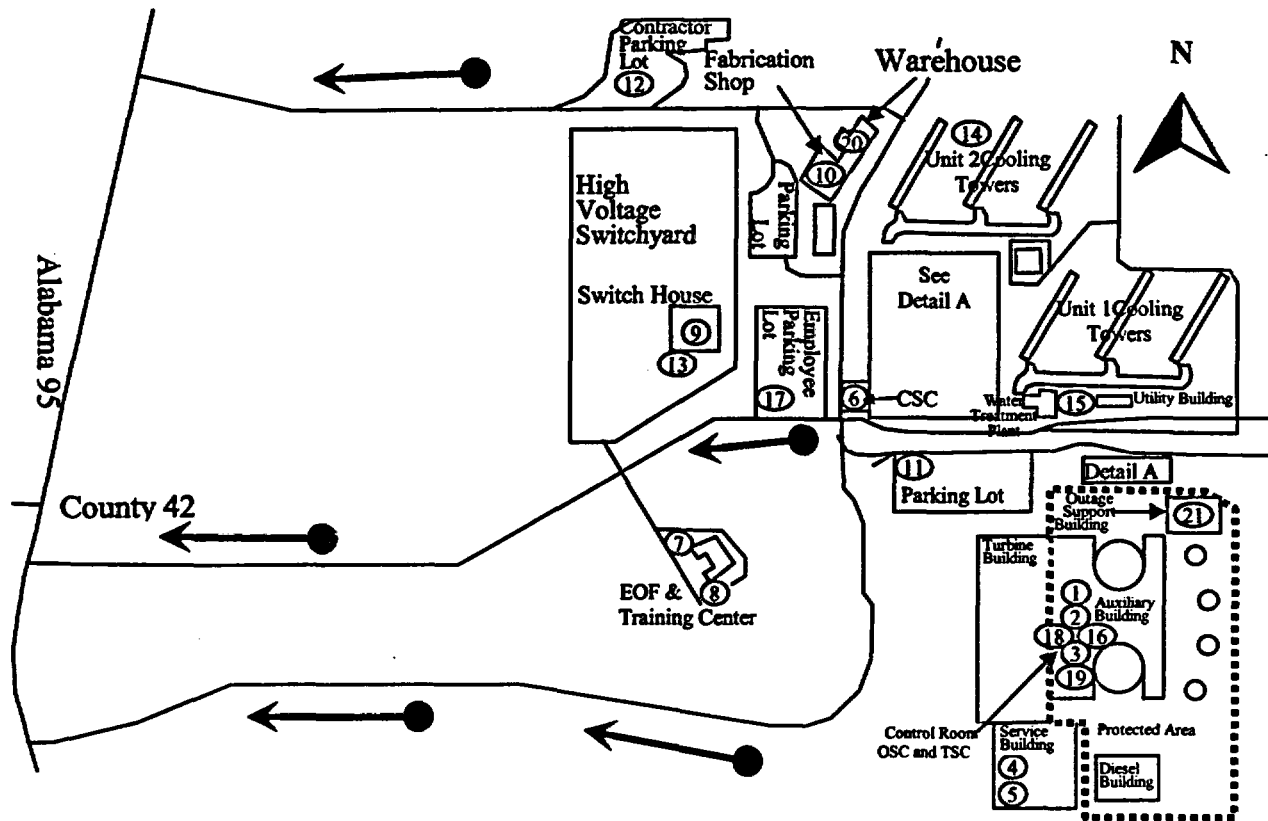
RE-ENTRY LOG

SHEET # _____

TIME

[illegible]

ON-SITE EVACUATION ROUTES, ASSEMBLY AREAS, AND OPERATIONS SUPPORT CENTER



LEGEND

OPERATION SUPPORT CENTER

1 BREAKROOM OUTSIDE TSC

ASSEMBLY AREAS

- 2 CONTROL ROOM
- 3 TSC
- 4 SERVICE BUILDING AUDITORIUM
- 5 MAINTENANCE SHOP
- 6 CSC
- 7 VISITORS CENTER AUDITORIUM
- 8 EOF
- 9 SWITCH HOUSE
- 10 FABRICATION SHOP
- 20 WAREHOUSE RECEIVING AREA
- 21 OUTAGE SUPPORT BUILDING

ALTERNATE ASSEMBLY AREAS

- 11 PARKING LOT SOUTH OF S.B.
- 12 CONTRACTOR PARKING LOT
- 13 SWITCHHOUSE PARKING LOT
- 14 BETWEEN 2A & 2B COOLING TOWERS
- 15 UTILITY BUILDING
- 16 SE CORNER OF CONTROL ROOM
- 17 EMPLOYEE PARKING LOT
- 18 BREAKROOM NEAR PAP
- 19 HP OFFICE AREA

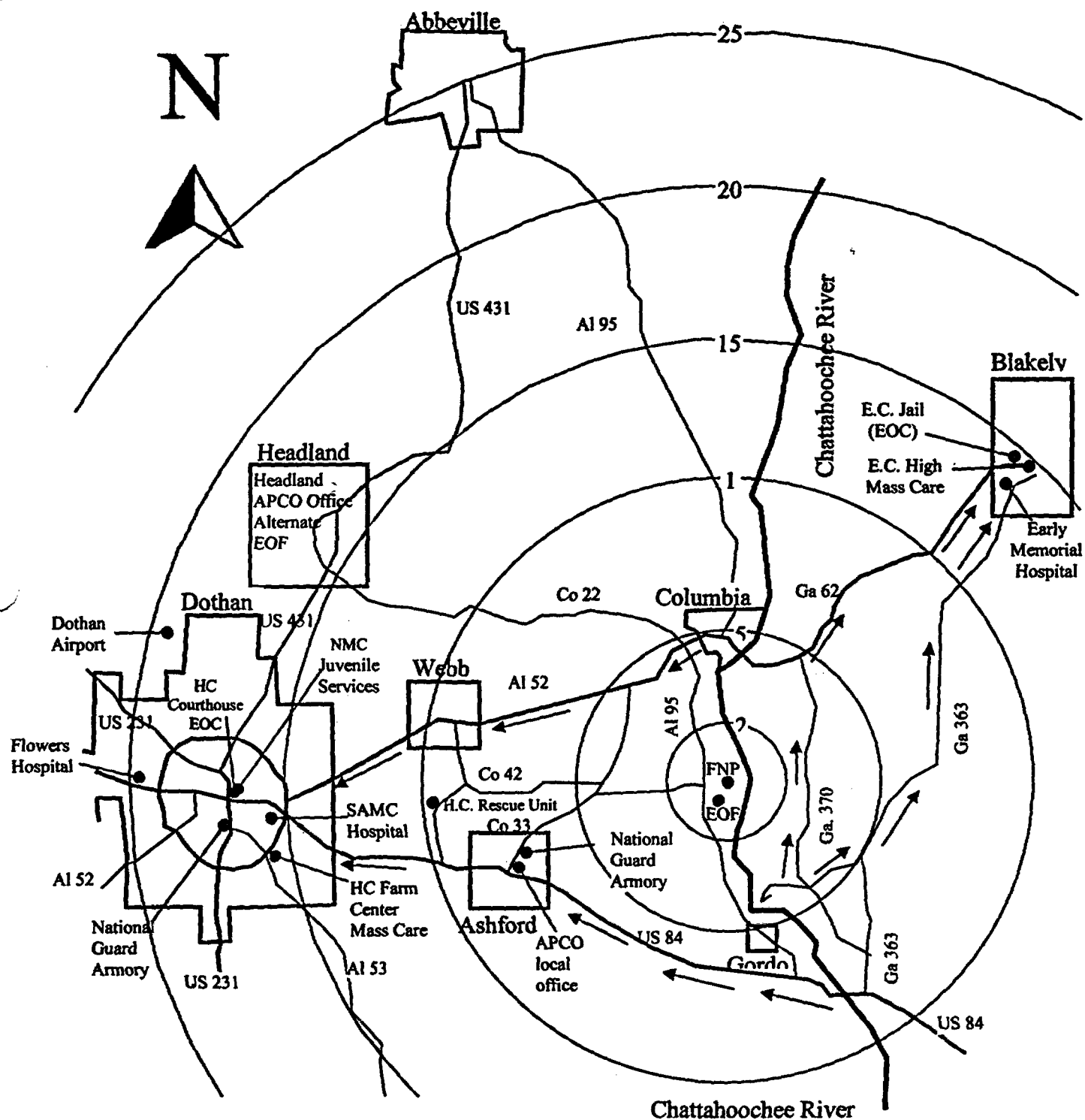
EVACUATION ROUTES



FIGURE 5
Page 1 of 1

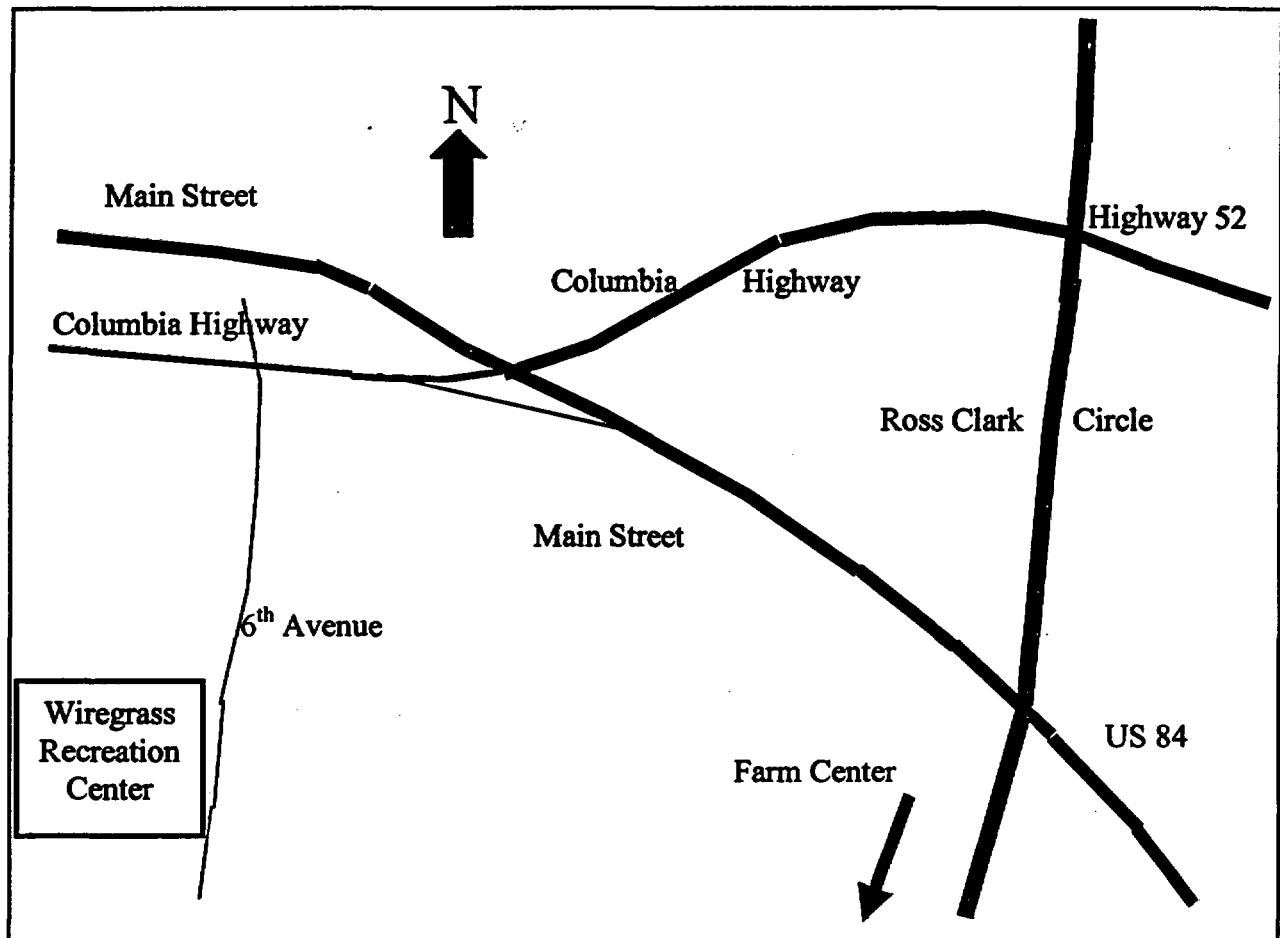
SHARED

OFF-SITE EVACUATION ROUTES

FIGURE 6
Page 1 of 1

Version 18

LOCATION OF RECEPTION CENTERS IN HOUSTON COUNTY



04/03/03 15:29:00

SHARED

FNP-0-EIP-10.0
March 17, 2003
Version 34

FARLEY NUCLEAR PLANT
EMERGENCY PLAN IMPLEMENTING PROCEDURE
FNP-0-EIP-10.0

EVACUATION AND PERSONNEL ACCOUNTABILITY

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D

PROCEDURE USAGE REQUIREMENTS PER FNP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	Sections 8.5, 8.7, 9.3, 9.4, 9.5, 9.6, 9.7, 9.8
Information Use	Remainder of Procedure

Approved:



Nuclear Plant General Manager

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CAUTION: This copy is not maintained
current. Do not use in a Safety Related Activity.

Date Issued

8-29-03

LIST OF EFFECTIVE PAGES

<u>Procedure Contains</u>	<u>Number of Pages</u>
Table of Contents	2
Body	13
Table 1	1
Figure 1	1
Figure 2	1
Figure 3	1
Figure 4	1
Figure 5	1
Figure 6	1

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9.0	Outside Protected Area Assembly Areas Accountability - Final Accountability	10
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11.0	Evacuation of Plant Site	12
12.0	Use of the Plant Emergency Alarm (PEA) Warble Tone, Siren, and Pulse Tone	12
13.0	Eating, Drinking, and Smoking Policy for Assembly Areas	13
Table 1	References	
Figure 1	Onsite Evacuation Routes/Operations Support Centers/ Assembly Areas	
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<u>Section</u>	<u>Title</u>	<u>Page</u>
Figure 3	Protected Area Accountability Contingency Plan	
Figure 4	Outside Protected Area Accountability	
Figure 5	CSC Final Accountability Log	
Figure 6	Accountability Flow Chart	

EVACUATION AND PERSONNEL ACCOUNTABILITY

1.0 Purpose

This procedure describes the action to be taken for the evacuation and accountability of all personnel onsite, in the event of an emergency at the Farley Nuclear Plant.

2.0 References

See Table 1

3.0 Designated Assembly Areas

For purposes of site evacuation and personnel accountability, the following areas (as shown in Figure 1) are designated as Assembly Areas:

OPERATIONS SUPPORT CENTERPAX EXT

1.	Break Room Outside TSC	2448
----	------------------------	------

PRIMARY ASSEMBLY AREAS

Protected Area Assembly Areas

2.	Control Room	2353
3.	Technical Support Center (TSC)	6018
21.	Outage Support Building (OSB)	4873

Outside Protected Area Assembly Areas

4.	Service Building Auditorium	2236
5.	Service Building Maintenance Shop	4357/2240
6.	Central Security Control (CSC)	2438
7.	Visitors Center Auditorium	6158
8.	Emergency Operations Facility	6156
9.	Switchhouse	2321
10.	Fabrication Shop	3578/3468
20.	Warehouse Receiving Area	4645

Alternate Assembly Areas

NOTE: EXTRA PERSONNEL FROM THE OSC MAY BE RELOCATED TO THE BREAKROOM NEAR THE PAP, THE SE CORNER OF THE CONTROL ROOM, AND THE HP OFFICE AREA BY THE OSC MANAGER. OTHER ALTERNATE ASSEMBLY AREAS MAY BE USED AT THE ED's DISCRETION.

- | | | |
|-----|---|------|
| 11. | Parking Lot South of Service Building | |
| 12. | Contractor Parking Lot | |
| 13. | Switchhouse Parking Lot | |
| 14. | Area between the 2A and 2B Cooling Towers | |
| 15. | Utility Building | 4502 |
| 16. | SE Corner of Control Room | 2306 |
| 17. | Employee Parking Lot | |
| 18. | Breakroom near PAP | 2405 |
| 19. | HP Office Area | 2334 |

4.0 Assembly Area Assignments

- 4.1 On-Call Personnel assigned to a position with the emergency response organization will report to their designated emergency response facility as described in FNP-0-EIP-6.0 or FNP-0-EIP-27.0, instead of assembling with their work group.
- 4.2 Personnel that are on site and assigned to a position with the emergency response organization on the on-call memo, but not currently in an on-call status will also report to their designated emergency response facility. After it has been determined if these individuals are needed to augment the facility staff or will be required to be available for a long term relief they may be relocated or evacuated offsite as necessary.
- 4.2.1 Accountability in the EOF shall include those personnel that are in the on-call memo assigned to the EOF that are known to be on-site at the time of accountability or other personnel that have been augmented or assigned to the EOF for duty.
- 4.3 On-shift personnel with a specific function assigned in FNP-0-EIP-0.0 will report to the area described, or perform the described function instead of assembling with their work groups.
- 4.4 Escorted visitors will remain with their escort and report to the same assembly area as their escort.
- 4.5 Other visitors/contractors on site will report to the assembly area designated for the on-site work group with whom they are working.

- 4.6 Personnel assigned to training at the Training Center or the Fire Training Facility will assemble in the Visitors Center Auditorium instead of with their work groups.
- 4.7 Operations, Chemistry, Environmental and Health Physics On Shift staff involved in training anywhere on site except the Training Center or Fire Training Facility will report to the breakroom outside the TSC.
- 4.8 Personnel who have been assigned to augment the TSC or EOF staffs will remain at that location for accountability. The senior individual in their normal assembly area should be notified as soon as possible, when time permits.
- 4.9 On shift security will remain on station until relocation is required. Relocation will normally be controlled by FNP-0-EIP-14.0. If, due to a personal hazard, security personnel must evacuate their station, security supervision will be notified as soon as possible to implement the guidance of FNP-0-EIP-14.0.
- 4.10 NRC Inspectors will assemble in any one of the assembly areas as appropriate for plant and emergency conditions.
- 4.11 Individuals in the following work groups who are not described above will report to the designated assembly area as indicated by the number that corresponds to the assembly area numbers assigned in step 3.

<u>Work Group</u>	<u>Assembly Area</u>
Administrative Staff assigned to Administrative Assistant	4
Administrative Staff assigned to Support Building	10
Cafeteria Staff	4
Chemistry and Env. Staff (Off Shift)	4
Chemistry and Env. Staff assigned to the EOF	7
Chemistry and Env. Staff (On Shift)	1
Contractor Personnel Assigned to Outage and Modification	10
Document Control Staff	4
Engineering Support Staff	10
EOF Staff (as assigned in FNP-0-EIP-27.0)	8

Facilities Staff	4
Financial Services	10
Fitness for Duty Facility Staff	7
Health Physics Staff	1
Health Physics Support Staff (On & Off Shift)	1
Information Management Systems Staff	4
Maintenance Contractor (not Williams)	5
Maintenance Supervision and Staff	5
Maintenance Teams 1 through 8	5
Maintenance Team 9	10
Material Dept. Personnel not assigned to the SB Cold Tool Room	20
Material Dept. Personnel assigned to the SB Cold Tool Room	5
NRC Administrative Staff	4
Operations Staff (off shift)	4
Operations-OATC (assigned post)	2
Operations-Shift Supervisors (assigned post)	2
Operations-Shift Technical Advisor	2
Operations-Unit Operators (assigned post)	2
Operations Staff (other on shift)	1
Outage and Modification - Outage Staff	4
Outage and Modification - Modification Staff	10
Quality Control Personnel	20

Safety Audit and Engineering Review	4
Satellite Document Control Staff	10
Security Staff Off Shift	6
Special Projects Staff	10
State and County Agency Personnel	7
Students participating in Training Activities at the Maintenance Training Area (SAER 2001-Q&T/14-2)	5
Students participating in Training Activities at the Training Center or Fire Training Facility	7
Switchboard Operator (assigned station)	9
Switchhouse Staff	9
Training Center Staff	7
Training Center Staff assigned to the Maintenance Training Area	5
TSC Staff (as assigned in FNP-0-EIP-6.0)	3
Visitors in the Visitors Center (Responsibility of the VC Staff)	7
Visitors Center Staff	7
Siemens-Westinghouse Turbine Group Personnel	5
Westinghouse NSSS Personnel	10
Williams Personnel	10
Personnel, in a non-work status, engaged in sporting or other recreational activities	
if the Visitor Center is open	7
if the Visitor Center is closed	4
Any other personnel on site and not previously listed in this procedure	4

5.0 Individual Responsibility

- 5.1 All personnel shall familiarize themselves with the location of their particular assembly area.
- 5.2 Personnel who report to an assembly area shall assemble according to groups to facilitate accurate and timely accountability.
- 5.3 When reporting to an assembly area, personnel should avoid any route or area of the plant which has been declared part of the emergency or which could result in excessive radiation exposure or personal injury.
- 5.4 Personnel who have been in the emergency area shall remain segregated from other personnel in the assembly area until they have been monitored for possible contamination, if applicable.
- 5.5 Each plant supervisor or senior individual onsite from each group shall be responsible for accounting for all persons working in or visiting his group.
- 5.6 When evacuating the Radiation Control Area (RCA), attempt to remove the outer layer of protective clothing before proceeding to the assembly area.
- 5.7 Personnel exiting the RCA wearing protective clothing during an evacuation should make every reasonable effort to avoid contaminating equipment, walls, floors and other personnel.
- 5.8 When accountability is required, personnel who enter the Fabrication Shop assembly area are not required to have on hard hat and safety glasses as long as they stay between the yellow lines of the walkway within the building. The senior individual in the assembly area can relax the requirements for hard hat and safety glasses within the rest of the assembly area after determining that there is no safety hazard present that would require their use.

6.0 General Evacuation

- 6.1 A general evacuation is initiated by sounding the plant emergency alarm (warble tone) for approximately 30 seconds and announcing for all personnel to report to their designated assembly areas.
- 6.2 If extenuating circumstances prohibit sounding the plant emergency alarm, a general evacuation can be initiated by announcing it over the public address system.
- 6.3 A general evacuation is required any time a Site Area Emergency or a General Emergency have been declared.

- 6.4 The Emergency Director can, at his discretion, initiate a general evacuation at any time as a precautionary measure.
- 6.5 When a general evacuation has been announced, all plant personnel will report to their designated assembly areas as described in steps 3, 4 and 5 of this procedure.
- 6.6 When a general evacuation has occurred, accountability will be performed per step 7.0 of this procedure.

7.0 Accountability

Accountability shall be performed whenever a general evacuation has been ordered by the Emergency Director and announced over the public address system. The announcement of a general evacuation will normally be followed by activating the Plant Emergency Alarm (warble tone) for a minimum of 30 seconds.

8.0 Operations Support Center and Protected Area Assembly Areas Accountability - Initial Accountability

Technical Support Center

Control Room

Breakroom Outside TSC

Outage Support Building

NOTE: DUE TO THE REQUIREMENT FOR REPORTING ACCOUNTABILITY IN 30 MINUTES AND THAT ANY MOVEMENT IN THE PLANT MUST BE GUIDED BY FNP-0-EIP-14.0, DO NOT DELAY REPORTING "ACCOUNTABILITY COMPLETE" TO THE EMERGENCY DIRECTOR WHILE SEARCHING FOR MISSING INDIVIDUALS.

- 8.1 Initial accountability will be considered complete when the individuals who are missing in the protected area are reported to the Emergency Director by the total number of missing personnel and their names.
- 8.2 Initial Accountability must be complete for the protected area within 30 minutes of announcing a general evacuation.
- 8.3 Individuals that are to assemble in a Protected Area Assembly Area shall swipe into the Biometrics card readers with their Protected Area Badge in their designated assembly area when the plant emergency alarm has been actuated or a general evacuation has been announced.

- 8.4 Personnel that are required to remain at a particular location that is within the Control Room, can have their Protected Area Badge swiped into the biometrics card reader by security when accountability is required. The Protected Area Badge should be returned to the owner as soon as possible.
- 8.5 Security shall:
- 8.5.1 Account for security at posts in the protected area by direct communication or messenger.
 - 8.5.2 Dispatch security to the Control Room to assist in accountability. Security may collect the protected area badges from those individuals in the control room that may not be able to leave their post, and swipe those individuals into the biometrics card reader in the control room. The badges should be returned to the individuals as soon as possible.
 - 8.5.3 Note the time that the plant emergency alarm was activated or when a general evacuation was announced to determine when the report of accountability should be sent to the TSC.
 - 8.5.4 Twenty minutes after the plant emergency alarm has been actuated or assembly and accountability have been announced, provide the Emergency Director with an emergency accountability report per FNP-0-SP-36 (PLANT SECURITY COMPUTER SYSTEM AND CENTRAL ALARM STATION/SECONDARY ALARM STATION OPERATIONS).
 - 8.5.5 In the event of a failure of the computer that would prevent performing accountability, inform the OSC, TSC, and outside the protected area assembly areas listed in step 3 that protected area accountability must be performed per the Protected Area Accountability Contingency Plan specified in step 8.7.
 - 8.5.6 Ensure that no one except those with emergency duties enters the protected area without the permission of the Emergency Director or designee.
 - 8.5.7 Take other actions for general evacuation and accountability as described in FNP-0-EIP-7.0.
- 8.6 The Emergency Director shall:
- 8.6.1 Upon receipt of the initial accountability report, have the location of the missing individuals determined.
 - 8.6.2 Have teams activated to search for the missing individuals, using the guidance of FNP-0-EIP-14.0.

8.7 Protected Area Accountability Contingency Plan

The following steps shall be performed when the Protected Area Accountability Contingency Plan is required:

- 8.7.1 The senior individual in each of the outside the protected area assembly areas will perform accountability in the normal manner, but will ensure the results are reported within approximately twenty minutes of the time that the plant emergency alarm was activated or the public address announcement was made.
- 8.7.2 The senior individual in the OSC will have the senior individuals for Operations, Chemistry, Health Physics, and Health Physics Support report the names of individuals that are on site for their group, should have assembled in the protected area and are missing. The senior individuals for these groups should take into account personnel that may be in the TSC or Control Room. (Figure 3 may be used as an aid.)
- 8.7.3 The senior individual in the OSC will determine from the TSC staff any individuals from the TSC staff that may not be accounted for. (Figure 3 may be used as an aid.)
- 8.7.4 The senior individual in the OSC will determine from the Control Room staff any individuals from the control room that may not be accounted for. (Figure 3 may be used as an aid.)
- 8.7.5 The senior individual in the OSB shall report to the OSC the names of individuals that are on site for their group, should have assembled in the protected area and are missing.
- 8.7.6 Within twenty minutes after the plant emergency alarm is activated or the public address announcement is made, the senior individual in the OSC will report to the ED or the Shift Supervisor the names of personnel that are missing. (Figure 3 may be used as an aid.)
- 8.7.7 Accountability for the protected area and outside the protected area, will be complete when names of the missing individuals from all assembly areas are reported to the Emergency Director.
- 8.7.8 The ED or Shift Supervisor will take steps to locate any missing individuals per step 8.6. All missing personnel may be in the protected area for this type of accountability.

9.0 Outside Protected Area Assembly Areas Accountability - Final Accountability

Service Building Auditorium

2236

Service Building Maintenance Shop	4357/2240
Central Security Control (CSC)	2438
Visitors Center Auditorium	6158
Emergency Operations Facility	6156
Switchhouse	2321
Fabrication Shop	3578/3468
Warehouse Receiving Area	4645

NOTE: DUE TO THE REQUIREMENT FOR FINAL ACCOUNTABILITY BEING REPORTED AS SOON AS POSSIBLE AND ANY MOVEMENT ON THE PLANT SITE MUST BE GUIDED BY FNP-0-EIP-14.0, DO NOT DELAY REPORTING "ACCOUNTABILITY COMPLETE" TO THE EMERGENCY DIRECTOR WHILE SEARCHING FOR MISSING INDIVIDUALS.

- 9.1 Final accountability will be considered complete when the individuals who were known to be on site and are missing are reported to the Emergency Director by the total number of missing personnel and their names.
- 9.2 Final accountability should be completed for the plant site as soon as possible after announcing a general evacuation.
- 9.3 The EOF Support Coordinator or senior individual in the EOF will report the names of individuals who are known to be on site, who are assigned to the EOF and are missing, to the CSC. Figure 4 may be used as an aid.
- 9.4 The senior Outage and Modification (O & M) individual in the Fabrication Shop will coordinate with the contractor supervision and management to identify all contractors on site who should have reported to the Fabrication Shop and are missing. Time cards, personnel knowledge or other documentation may be used. When the list of missing individuals has been compiled, the senior O & M individual will report the names of the missing to the CSC. Figure 4 may be used as an aid.
- 9.5 The senior individual in the Visitors Center Auditorium should contact the EOF or TSC staff to arrange evacuating visitors in the Visitors Center Auditorium as soon as possible.
- 9.6 The senior individual in an assembly area for each work group will determine the accountability of the individuals in their work group or visiting their work group, and report any missing individuals to the senior individual in the assembly area.
- 9.7 The senior individual in each assembly area will compile a list of names of individuals known to be on site who are missing from their assembly area, when the list of missing individuals has been compiled, the senior individual will report the names of the missing to the CSC. Figure 4 may be used as an aid.

9.8 Security shall:

9.8.1 Account for security on posts outside the protected area by direct communication or messenger.

9.8.2 Contact the following locations and inform them of assembly requirements.

Outage Modification - 3545

Westinghouse - 2479

Warehouse - 3391

9.8.3 Account for security assembled in the CSC.

9.8.4 Assemble the names of persons missing from all of the assembly areas outside the protected area. Figure 5 may be used as a guide.

9.8.5 Report that security has completed final accountability to the Emergency Director, providing the ED with the number of individuals who are missing on the plant site outside of the protected area, and the names of the missing individuals. Figure 5 may be used as an aid.

9.8.6 Take other actions for general evacuation and accountability as described in FNP-0-EIP-7.0.

9.9 The Emergency Director shall:

9.9.1 Upon receipt of the final accountability report, have the location of the missing individuals determined.

9.9.2 Have teams activated to search for the missing individuals using the guidance of FNP-0-EIP-14.0.

NOTE: INITIAL AND FINAL ACCOUNTABILITY DO NOT NEED TO BE COMPLETED PRIOR TO PERFORMING THE REMAINING SUBSTEPS OF STEP 9.9 BELOW.

9.9.3 Authorize release of News Media Center personnel.

9.9.4 Direct non-essential personnel (including children and casual visitors) to depart from the site. Limit the exposure of visitors and pregnant females consistent with the radiation exposure situation.

9.9.5 Provide for transportation for persons without vehicles.

9.9.6 Provide clothing for personnel found to be contaminated.

10.0 Local Evacuation

- 10.1 All personnel in the affected area shall stop work, render safe any hazardous equipment and leave the area by the most direct route to the assembly area unless otherwise instructed by the Control Room.
- 10.2 The Shift Supervisor or ED will activate emergency teams as required to locate and ensure the evacuation of personnel.
- 10.3 Accountability:
 - 10.3.1 For radiological side Auxiliary Building or Containment evacuation, Health Physics may utilize the Access Control section of the HIS 20 system to account for personnel if requested.
 - 10.3.2 In the event of local evacuations other than the containment or the radiological side Auxiliary Building, the senior SNC employee present will account for all personnel and notify the Shift Supervisor.
 - 10.3.3 For areas where the number of personnel who may be in the area is not known (e.g., a floor of the turbine building or entire turbine building), accountability may be affected by a systematic search of the affected area to ascertain all personnel have evacuated.

11.0 Evacuation of Plant Site

- 11.1 At the Emergency Director's discretion, he can have the plant site evacuated of all unnecessary personnel during an emergency condition.
- 11.2 If the plant site is to be evacuated, it will be done per FNP-0-EIP-14.0.

12.0 Use of Plant Emergency Alarm (PEA) Warble Tone, Siren, and Pulse Tone

- 12.1 The warble tone on the plant emergency alarm is reserved for announcing a general evacuation which requires all personnel to report to their designated assembly areas.
- 12.2 When announcing a general evacuation, a public address announcement should be made providing specific instructions; then, the warble tone should be actuated for approximately 30 seconds. The public address announcement should be repeated.
- 12.3 The siren on the plant emergency alarm may be used to muster the fire brigade for a drill or an actual fire. The siren may, at the discretion of the Shift Supervisor, be used in unusual circumstances when it is important to get the attention of all personnel on plant site.

- 12.4 When announcing a fire or other unusual circumstance, a public address announcement should be made providing specific instructions; then, the siren should be actuated for approximately 30 seconds (10 seconds for security events). The public address announcement should be repeated.
- 12.5 The pulse tone on the plant emergency alarm may be used to notify the security group of a specific contingency event or events. The pulse tone may be initiated from the Security Secondary Alarm station (SAS), or from the Control Room (at the direction of the Shift Supervisor) if SAS is unable to perform the function. Security personnel will respond to the pulse tone; no further actions are required by plant personnel unless directed by a public address announcement.

13.0 Eating Drinking and Smoking Policy for Assembly Areas

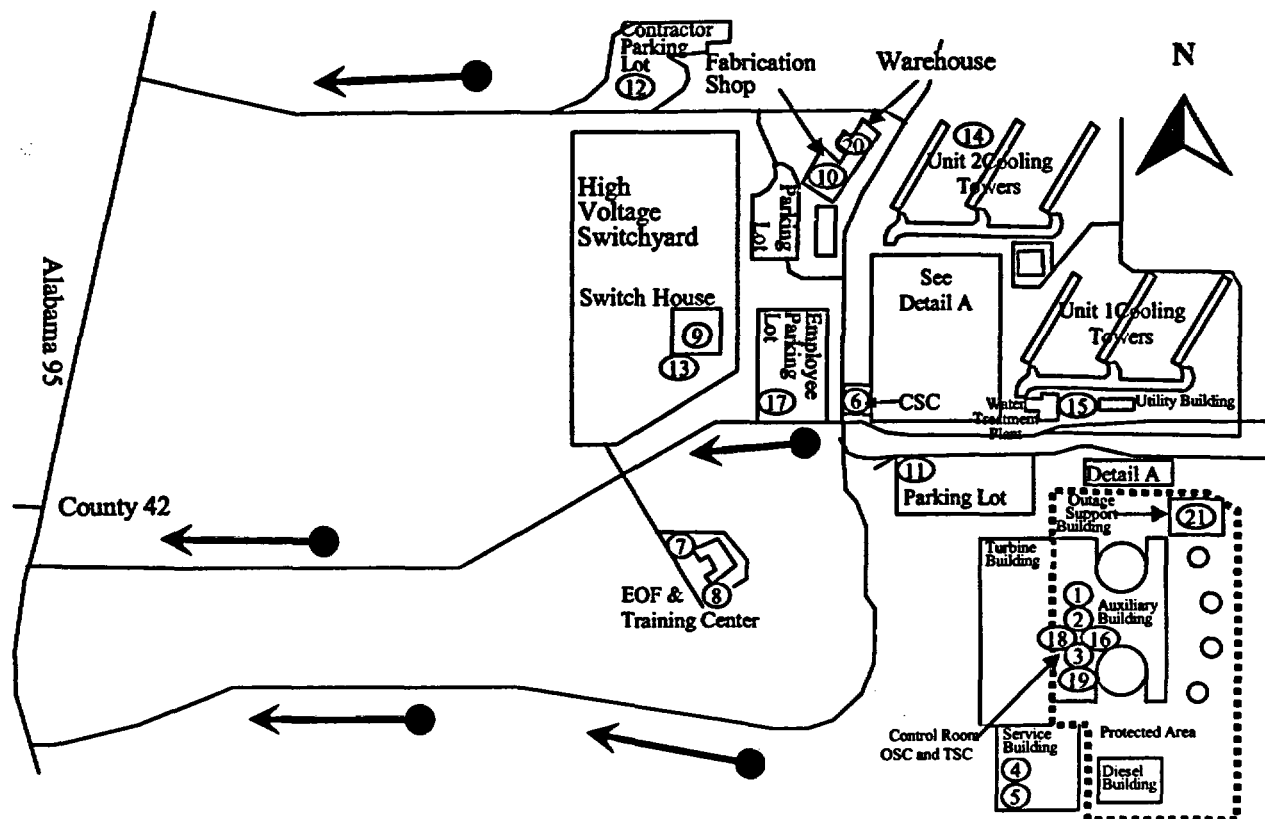
- 13.1 Any time the plant emergency alarm (PEA) has been actuated there is a possibility of a radioactive release being in progress. Due to the health risks involved when the PEA has been actuated, eating, drinking, and smoking should be terminated by all personnel on plant site as they go to their designated assembly areas.
- 13.2 Eating drinking and smoking should not be allowed in the assembly areas until there has been confirmation that there is no radiological release in progress.
- 13.3 When it has been confirmed that there is no radiological release in progress, eating and drinking in assembly areas may be allowed within normal plant policies and guidelines.
- 13.4 Personnel should not be allowed to leave the assembly area to go outside and smoke.

REFERENCES

1. Joseph M. Farley Nuclear Plant Emergency Plan
2. FNP-0-EIP-14.0 - Personnel Movement, Relocation, Re-Entry and Site Evacuation

FIGURE 1

**ON-SITE EVACUATION ROUTES,
ASSEMBLY AREAS, AND OPERATIONS SUPPORT CENTER**

**OPERATION SUPPORT CENTER**

1 BREAKROOM OUTSIDE TSC

ASSEMBLY AREAS

- 2 CONTROL ROOM
- 3 TSC
- 4 SERVICE BUILDING AUDITORIUM
- 5 MAINTENANCE SHOP
- 6 CSC
- 7 VISITORS CENTER AUDITORIUM
- 8 EOF
- 9 SWITCH HOUSE
- 10 FABRICATION SHOP
- 20 WAREHOUSE RECEIVING AREA
- 21 OUTAGE SUPPORT BUILDING

LEGEND**ALTERNATE ASSEMBLY AREAS**

- 11 PARKING LOT SOUTH OF S.B.
- 12 CONTRACTOR PARKING LOT
- 13 SWITCHHOUSE PARKING LOT
- 14 BETWEEN 2A & 2B COOLING TOWERS
- 15 UTILITY BUILDING
- 16 SE CORNER OF CONTROL ROOM
- 17 EMPLOYEE PARKING LOT
- 18 BREAKROOM NEAR PAP
- 19 HP OFFICE AREA

EVACUATION ROUTES

FIGURE 1
Page 1 of 1

FIGURE 3**PROTECTED AREA ACCOUNTABILITY
CONTINGENCY PLAN**

This figure should only be used for Protected Area Accountability in the event that the Biometrics System has failed and the Protected Area Accountability Contingency Plan is required.

The OSC Manager or senior individual in the OSC is responsible for completing Protected Area Accountability Contingency Plan per procedure step 8.7 if informed that it is required by security.

Determine from the senior individuals for each of the groups or locations listed below any individuals that are on site for their group, should have assembled in the protected area and are missing.

- ☐ Operations
- ☐ Chemistry
- ☐ Health Physics
- ☐ Health Physics Support
- ☐ TSC
- ☐ Control Room
- ☐ Outage Support Building (Steam Generator Replacement Group)

LIST THE NAMES OF MISSING PERSONNEL THAT SHOULD BE IN A PROTECTED AREA ASSEMBLY AREA AND ARE MISSING FROM THAT ASSEMBLY AREA.

PRINT LEGIBLY

BADGE NUMBER (if known)

LAST NAME

FIRST NAME

LIST THE NAMES OF MISSING PERSONNEL THAT SHOULD BE IN AN ASSEMBLY AREA OUTSIDE THE PROTECTED AREA AND ARE MISSING FROM THAT ASSEMBLY AREA PER THE SECURITY FINAL ACCOUNTABILITY REPORT. THESE INDIVIDUALS MAY BE CONSIDERED TO BE IN THE PROTECTED AREA UNTIL DETERMINED OTHERWISE.

PRINT LEGIBLY

BADGE NUMBER (if known)

LAST NAME

FIRST NAME

- ☐ Report to the Emergency Director that initial accountability has been completed using the contingency plan, and provide the Emergency Director with a list of any personnel who are missing and presumed to be in the Protected Area.

FIGURE 4

OUTSIDE PROTECTED AREA ACCOUNTABILITYService Building Auditorium

- ☐ Operations
☐ Chem & Env not on Shift
☐ SAER Personnel
☐ Cafeteria Staff
☐ IMS
☐ Document Control
☐ Admin Staff
☐ Facilities Staff
☐ O & M (Outage Staff)
☐ NRC Admin Staff
☐ Personnel Engaged In Non-Work Activities
☐ Other Personnel _____

Visitors Center Auditorium

- ☐ Training Staff
☐ Training Students
☐ Chem & Env Personnel
☐ Fitness For Duty Staff
☐ Visitors Center Staff
☐ Visitors In VC (VC Staff Responsibility)
☐ Personnel Engaged in Non-Work Activities
☐ Other _____

EOF

- ☐ EOF Assigned Staff (On Call Crew)
☐ Augmented EOF Staff (Includes Off Call in EOF)
☐ Alabama/Houston Co. Personnel
☐ Georgia/Early Co. Personnel
☐ Florida Personnel
☐ NRC Staff
☐ Other _____

Maintenance Shop

- ☐ Maintenance Teams Except Team 9
☐ Maintenance Supervision and Staff
☐ Maintenance Training Staff and Students
☐ Materials Dept Staff in Cold Tool Room
☐ Maintenance Contractors (not Williams)
☐ Siemens-Westinghouse Turbine Personnel

Fabrication Shop

- ☐ O & M (Modification Staff)
☐ Williams Personnel
☐ Contractors Assigned to O&M or Williams
☐ Support Building Admin Staff
☐ Engineering Support Staff
☐ Satellite Document Control staff
☐ Westinghouse NSSS Personnel
☐ Maintenance Team 9
☐ Financial Services
☐ Special Projects Staff
☐ Other _____

CSC

- ☐ Security Personnel Not On Shift
☐ Security Personnel assigned Outside PA

Warehouse Receiving Area

- ☐ Materials Dept Staff not in Cold Toolroom
☐ QC Personnel

Switch House

- ☐ Switchhouse Personnel

Missing Personnel

Accountability Reported To CSC(2438) by _____

FIGURE 4

FIGURE 5

CSC FINAL ACCOUNTABILITY LOG

ASSEMBLY AREAACCOUNTABILITY COMPLETED
DATE/TIME

SERVICE BUILDING AUDITORIUM

/

MAINTENANCE SHOP

/

CSC

/

VISITORS CENTER AUDITORIUM

/

SWITCHHOUSE

/

FABRICATION SHOP

/

EOF

/

WAREHOUSE RECEIVING AREA

/

ALTERNATE ASSEMBLY AREAS (IF USED)

/

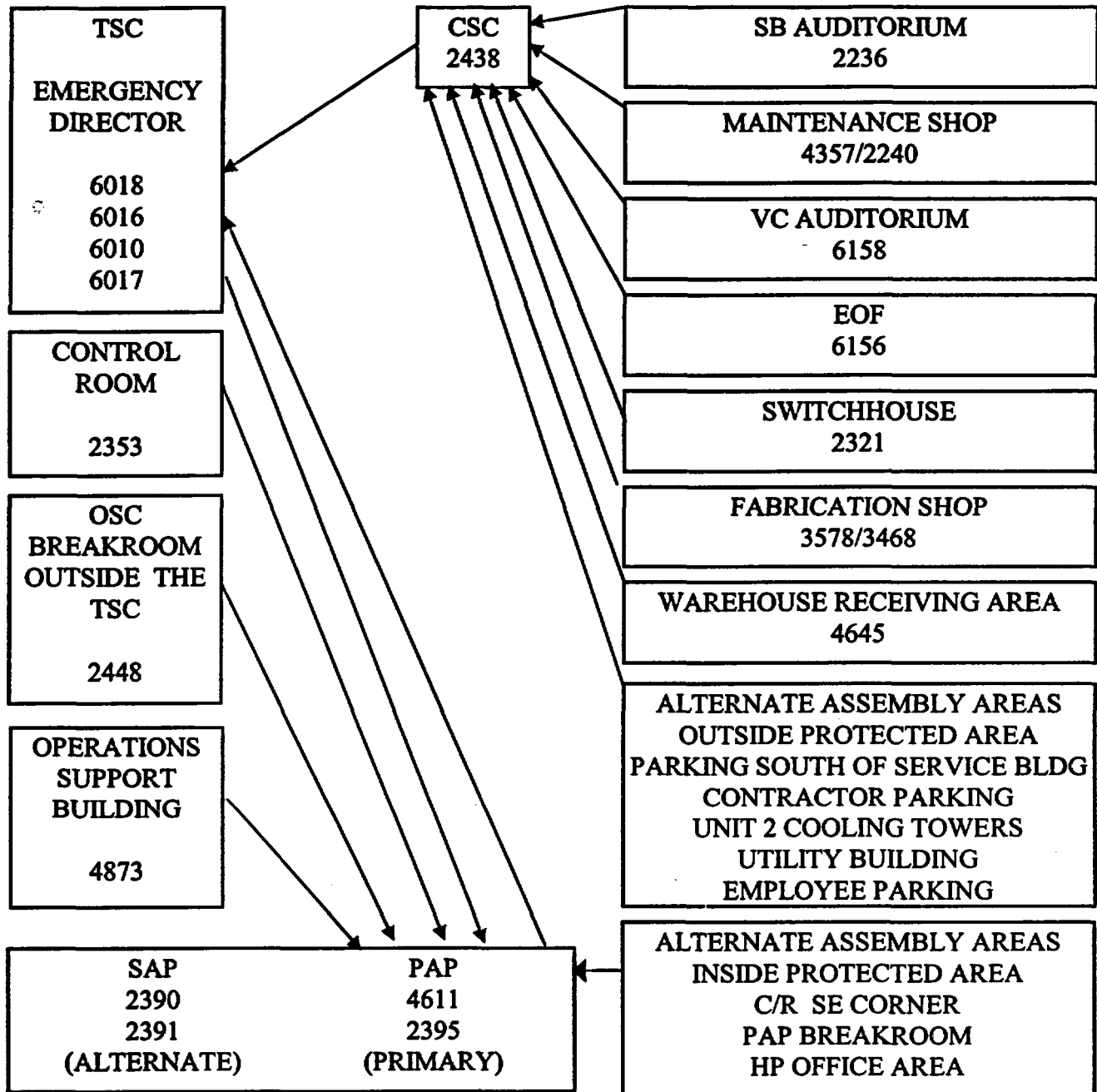
MISSING PERSONNEL

REPORT TO THE EMERGENCY DIRECTOR THAT SECURITY HAS COMPLETED FINAL ACCOUNTABILITY OUTSIDE THE PROTECTED AREA AND PROVIDE THE EMERGENCY DIRECTOR WITH A LIST OF ANY PERSONNEL THAT ARE MISSING. (ED PHONE # 6016, MM PHONE # 6018, TM PHONE # 6010 OR OM PHONE #6017. IF THE ED OR TSC CANNOT BE REACHED AT ANY OF THE FOUR LISTED NUMBERS, SEND A RUNNER, CONTACT THE PAP OR SAP TO SEND A RUNNER TO THE TSC TO MAKE THE REPORT.)

ACCOUNTABILITY REPORTED TO ED BY

/ DATE/TIME

ACCOUNTABILITY FLOW CHART



08/19/03 11:53:56

SHARED

FNP-0-EIP-9.5
August 19, 2003
Version 8

FARLEY NUCLEAR PLANT
EMERGENCY PLAN IMPLEMENTING PROCEDURE
FNP-0-EIP-9.5

EMERGENCY CLASSIFICATION BASED ON ODCM

S
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D

PROCEDURE USAGE REQUIREMENTS per FNP-0-AP-6	SECTIONS
Continuous Use	
Reference Use	ALL
Information Use	

Approved:



Nuclear Plant General Manager

Date Issued 8-29-03

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CAUTION: This copy is not maintained
current. Do not use in a Safety Related Activity

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TABLE 1 REFERENCES

TABLE 2 Steam Generator Relief/Safety Valves Open

FIGURE 1 Calculations Based On RE-29B Noble Gas Or Iodine Channel Values

FIGURE 2 Calculation Based On RE-14 Count Rate

FIGURE 3 Calculations Based On Plant Vent Stack Grab Sampling And Analyses

FIGURE 4 Calculations Based On SG Atmospheric Reliefs, Safeties and TDAFW Releases

FIGURE 5 Calculation Based On Steam Jet Air Ejector Release

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SHARED

FNP-0-EIP-9.5

FIGURE 6 RATIO OF TOTAL IODINES TO I-131

FIGURE 7 ODCM COMPUTER SCREEN

EMERGENCY CLASSIFICATION BASED ON ODCM

1.0 Purpose

To provide a method of emergency classification based on exceeding Technical Specification effluent criteria using the annual average mixed mode X/Q value, as defined in the ODCM.

2.0 References

See Table 1.

3.0 General

3.1 This procedure is entered to perform dose assessment using the Offsite Dose Calculation Method (ODCM) if Automated Dose Assessment was NOT required to automatically activated OR if a General, or Site Area Emergency is not indicated by FNP-0-EIP-9.1 or FNP-0-EIP-9.3 from the following:

- a. EIP-9.0, step 4.2.7
- b. EIP-9.0, step 4.2.10

3.2 When this procedure is entered continue to perform dose assessment periodically as required by FNP-0-EIP-9.0, not to exceed 30 minute intervals.

4.0 Determine the Methodology for Performing Dose Assessment

4.1 If the ODCM dose assessment using the NRERDS computer terminal is available and any of the following conditions exist, then dose assessment should be performed using step 8.0, ODCM dose assessment using the NRERDS computer terminal.

- There is no known tube leak or rupture and R-29B data is available on NRERDS.
- There is no known tube leak or rupture and R-14 data is available on NRERDS.
- A steam generator tube leak or rupture is in progress, R-15 has increased by a factor of 10 or more and R-15 data is available on NRERDS.
- A steam generator tube leak or rupture is in progress, R-60A, B, C or D has increased by a factor of 10 or more and R-60 data is available on NRERDS.

4.2 If any of the step 4.1 conditions are not met, then determine the release concentration or release count rate per steps 4.2.1, 4.2.2 or 4.2.3 and use steps 5, 6 or 7 to determine the ODCM Dose assessment.

- 4.2.1 For a release that takes less than 1 hour, normalize the release concentration or release count rate to 1 hour and add this value to the average background (e.g., if the release lasted 15 minutes, divide by 4; if the release lasted for 10 minutes, divide by 6, etc.). For releases that lasted more than 1 hour, use the method above to determine the highest average release for any continuous 60-minute period with the highest average release (i.e., comparable to a moving average).
- 4.2.2 If a single grab sample is the only method of determining release concentration, the value of that grab sample will be the release concentration.
- 4.2.3 If multiple grab samples are taken within a 60 minute period, they may be averaged to provide the release concentration.

5.0 ODCM Dose Assessment for the Plant Vent Stack

Steps 5.1, 5.2, and 5.3 should be used in sequence order any time that a plant vent stack ODCM calculation is performed. This is true even if the alarm that required doing the calculation was a different alarm.

5.1 Classification based on RE-29B noble gas or iodine channel values.

NOTE: Figure 1 may be used as an aid to calculate ODCM off site dose rates based on RE-29B noble gas or iodine channel values.

- 5.1.1 Obtain noble gas release concentration per step 4.0 from the Non-Regulatory Emergency Response Data System (NRERDS), the Radiological and Meteorological Data Acquisition System (RMDA), Plant Computer or directly from the low range gas channel. IF the noble gas channel of RE-29B is INOPERABLE, THEN go to 5.2.
- 5.1.2 Obtain iodine release concentration per step 4.0 from NRERDS, RMDA, Plant Computer or directly from the iodine channel. IF the iodine channel is inoperable, THEN go to 5.3.
- 5.1.3 Obtain Plant Vent Stack (PVS) flow rate from NRERDS, RMDA, Plant Computer or from the PVS flowrate recorder in the Control Room.

5.1.4 IF the PVS flow rate is NOT available from these sources, THEN use the following default values: (REA 98-1906)

- a. One Auxiliary Building Fan...94,000 scfm
- b. Two Auxiliary Building Fans...136,000 scfm
- c. Zero Auxiliary Building Exhaust Fans, one train PRF...5,000 scfm
- d. Zero Auxiliary Building Exhaust Fans, two trains PRF...10,000 scfm

5.1.5 Calculate the noble gas and iodine dose rates at the site boundary using the following ODCM equations:

noble gas release concentration in $\mu\text{ci/ml}$ from step 5.1.1	X	PVS flowrate (scfm) from step 5.1.3/4	X	conversion factor 8.54E-07	=	noble gas dose rate (rem/hr)
--	---	---------------------------------------	---	----------------------------	---	------------------------------

iodine release concentration in $\mu\text{ci/ml}$ from step 5.1.2	X	PVS flowrate (scfm) from step 5.1.3/4	X	conversion factor 9.41E-04	=	Iodine dose rate (rem/hr)
---	---	---------------------------------------	---	----------------------------	---	---------------------------

5.1.6 Go to EIP-9.0, step 4.2.11 for evaluation of emergency classification criteria.

5.2 Classification based on RE-14 count rate (cpm)

NOTE: Figure 2 may be used as an aid to calculate ODCM off site dose rates based on RE-14 count rate.

5.2.1 Obtain release count rate (cpm) per step 4.0 from NRERDS, RMDA, Plant Computer or directly from RE-14.

5.2.2 IF RE-14 count rate is less than 650,000 cpm, no emergency classification can be based on effluent noble gas dose rates. Go to step 5.2.7.

5.2.3 IF RE-14 release count rate (cpm) is greater than 1,000,000 cpm (full scale) OR NOT operable, THEN go to step 5.3.

5.2.4 Obtain Plant Vent Stack (PVS) flow rate from NRERDS, RMDA, Plant Computer or from the PVS flowrate recorder in the Control Room.

5.2.5 IF the PVS flow rate is NOT available from these sources, THEN use the following default values: (REA 98-1906)

- a. One Auxiliary Building Fan...94,000 scfm
- b. Two Auxiliary Building Fans...136,000 scfm
- c. Zero Auxiliary Building Exhaust Fans, one train PRF...5,000 scfm
- d. Zero Auxiliary Building Exhaust Fans, two trains PRF...10,000 scfm

5.2.6 Calculate the noble gas dose rate at the site boundary using the following ODCM equation:

RE-14 release count rate in cpm from step 5.2.1	X	PVS flowrate (scfm) from step 5.2.4/5	X	conversion factor 5.697E-16	=	noble gas dose rate (rem/hr)
---	---	---	---	-----------------------------------	---	---------------------------------

5.2.7 Go to step 5.1.2 to perform applicable steps if the iodine dose rate is required or go to EIP-9.0, step 4.2.11 for evaluation of emergency classification criteria.

5.3 Classification based on Plant Vent Stack grab sampling and analyses

NOTE: Figure 3 may be used as an aid to calculate ODCM off site dose rates based on Plant Vent Stack grab sampling and analyses.

5.3.1 IF RE-14 is reading full scale or inoperable AND the low range noble gas channel of RE-29B is NOT operable, THEN request that the Shift Radiochemist obtain a plant vent stack noble gas grab sample AND analyze it for DOSE EQUIVALENT Xe-133.

5.3.2 IF the RE-29B iodine channel is NOT operable, THEN request that the Shift Radiochemist obtain a plant vent stack iodine grab sample AND analyze it for DOSE EQUIVALENT I-131.

5.3.3 Obtain Plant Vent Stack (PVS) flow rate from NRERDS, RMDA, Plant Computer or from the PVS flowrate recorder in the Control Room.

5.3.4 IF the PVS flow rate is NOT available from these sources, THEN use the following default values: (REA 98-1906)

- a. One Auxiliary Building Fan...94,000 scfm
- b. Two Auxiliary Building Fans...136,000 scfm
- c. Zero Auxiliary Building Exhaust Fans, one train PRF...5,000 scfm
- d. Zero Auxiliary Building Exhaust Fans, two trains PRF...10,000 scfm

5.3.5 Calculate the dose rates at the site boundary using the following ODCM equations:

DE Xe-133 release concentration in $\mu\text{ci/ml}$ from step 5.3.1	X	PVS flowrate (scfm) from step 5.3.3/4	X	conversion factor 8.54E-07	=	Noble gas dose rate (rem/hr)
--	---	---------------------------------------	---	----------------------------	---	------------------------------

DE I-131 release concentration in $\mu\text{ci/ml}$ from step 5.3.2	X	PVS flowrate (scfm) from step 5.3.3/4	X	conversion factor 9.41E-04	=	Iodine dose rate (rem/hr)
---	---	---------------------------------------	---	----------------------------	---	---------------------------

5.3.6 Go to EIP-9.0, step 4.2.11 for evaluation of emergency classification criteria.

6.0 ODCM Dose Assessment for Steam Generator Atmospheric Relief and/or Safety Release and/or Turbine Driven Auxiliary Feedwater (TDAFW) Release.

NOTE: Figure 4 may be used as an aid to calculate ODCM off site dose rates based on Steam Generator Atmospheric Reliefs, Safeties and TDAFW.

- 6.1 Using the values from the R-60 series of instruments, HP surveys or other criteria, determine which steam generators have a radioactive release in progress.
- 6.2 Obtain the current pressure (psig) for each steam generator of the affected unit that has a radioactive release in progress.
- 6.3 Determine the number of valves that are open for each steam generator that has a radioactive release in progress by consulting Operations staff. Table 2 may be used as an aid in determining this.
- 6.4 Determine the total flow in cfm from each generator that has a radioactive release in progress by using the following equation:

steam pressure psig from step 6.2	X	# valves open from step 6.3	X	conversion factor 359.12	=	steam flow cfm
-----------------------------------	---	-----------------------------	---	--------------------------	---	----------------

- 6.5 Using the values from the R-60D, HP surveys or other criteria, determine if the TDAFW exhaust has a radioactive release in progress. If the TDAFW is determined to have a radioactive release the flow for that release point is 11175 cfm.
- 6.6 Add the flows determined in step 6.4 and 6.5 for all steam generators and the TDAFW that have a radioactive release in progress.

- 6.7 Obtain noble gas release concentration per FNP-0-CCP-1300, Appendix P.
- 6.8 Obtain total iodine release concentration per FNP-0-CCP-1300, Appendix P.
- 6.9 If the reactor is at power, convert from total iodine concentration to I-131 concentration by dividing the total iodine concentration from step 6.8 by 8.4.
- 6.10 If the reactor is shutdown, convert from total iodine concentration to I-131 concentration by dividing the total iodine concentration from step 6.8 by the ratio obtained from Figure 6.
- 6.11 Calculate the noble gas and iodine dose rates at the site boundary using the following ODCM equations:

noble gas release concentration ($\mu\text{ci/ml}$) from step 6.7	X	steam flow rate (cfm) from step 6.6	X	conversion factor 3.85E-05	=	noble gas dose rate (rem/hr)
---	---	-------------------------------------	---	-------------------------------	---	------------------------------

iodine 131 release concentration in $\mu\text{ci/ml}$ from step 6.9/10	X	steam flow rate (cfm) from step 6.6	X	conversion factor 4.24E-02	=	iodine dose rate (rem/hr)
--	---	-------------------------------------	---	-------------------------------	---	---------------------------

- 6.12 Go to EIP-9.0, step 4.2.11 for evaluation of emergency classification criteria.

7.0 ODCM Dose Assessment for Steam Jet Air Ejector Release

NOTE: Figure 5 may be used as an aid to calculate ODCM off site dose rates based on Steam Jet Air Ejector.

- 7.1 Obtain noble gas concentration per FNP-0-CCP-1300, Appendix O.
- 7.2 Obtain total iodine concentration per FNP-0-CCP-1300, Appendix O.
- 7.3 If the reactor is at power, convert from total iodine concentration to I-131 concentration by dividing the total iodine concentration from step 7.2 by 8.4.
- 7.4 If the reactor is shutdown, convert from total iodine concentration to I-131 concentration by dividing the total iodine concentration from step 7.2 by the ratio obtained from Figure 6.

- 7.5 Calculate the noble gas and iodine dose rates at the site boundary using the following ODCM equations:

noble gas release concentration in $\mu\text{Ci/ml}$ from step 7.1.	X	conversion factor, includes 1050 scfm flow 4.0425 E-2	=	noble gas dose rate (rem/hr)
---	---	---	---	------------------------------

iodine 131 release concentration in $\mu\text{Ci/ml}$ from step 7.3/4 (block 3)	X	conversion factor, includes 1050 scfm flow 44.52	=	iodine dose rate (rem/hr)
---	---	--	---	---------------------------

- 7.6 Go to EIP-9.0, step 4.2.11 for evaluation of emergency classification criteria.

8.0 ODCM Dose Assessment Using the NRERDS Computer Terminal

NOTE Figure 7 may be used as an aid when performing ODCM Dose Assessment using the NRERDS Computer Terminal.

- 8.1 To view the ODCM screen click on the "ODCM Dose Assessment" option on the top menu of the display.
- 8.2 When it is desired to return to the ERDS / ARDA screens, then click on the "Group Display" option on the top menu of the display.
- 8.3 Select the desired unit by clicking on the "Unit" option on the top menu of the display, then click on the desired unit.
- 8.4 To print the ODCM information, click on the "Print Screen" button" when the ODCM page is displayed.
- 8.5 The peak projected column can NOT be used to classify a radiological release as an Alert or NOUE. The peak projected column can be used by plant personnel to determine what the ODCM dose rate would be if peak release conditions stayed the same for the next hour.
- 8.6 The instruments used to calculate the total noble gas and the total iodine dose rate are indicated with an "X" in the left hand column (USE IN TOTAL column). If there are any instruments that are not operating properly, they should not have the X in the brackets and a manual dose assessment per steps 5.0 through 7.0 should be performed.
- 8.7 The default state for instruments used is R29B and all others with no "X" in the brackets.

- 8.8 To change the instruments used for the total calculation, click on the "ODCM Options" option on the second menu of the display, then click on the particular unit and instrument to toggle the status of that instrument.
- 8.9 Overriding the met information, radiation monitors, plant parameters, previous history or inventory will have no effect on ODCM readings.
- 8.10 Manually selecting R60A, B or C as a release point will cause an ODCM release rate to be calculated for that steam generator even if actual steam pressure is below the PORV setpoint.
- 8.11 The ODCM dose rate is calculated using a one hour rolling average for its calculation as compared to the EDCM which uses a 15 minute average.
- 8.12 Plant Vent Stack Release

A plant vent stack release is always in progress from the plant as long as there are ventilation fans running. Any ODCM dose assessment should include either R29B or R14. If neither instrument is operating properly or plant vent stack flow is not indicating properly, the ODCM should be calculated using the appropriate portions of step 5.0 and adding the value to any steam generator dose assessment that is being done.

- 8.12.1 If plant vent stack flow is not reading correctly on the NRERDS system, then proceed to step 5.0 to calculate the plant vent stack ODCM dose assessment. Add the results to any SG Dose assessment subsequently performed.
- 8.12.2 If R29 is operating properly, then verify that R29B is selected to be used in the total and R14 is not selected to be used in the total.
- 8.12.3 If R29B is not operating properly and R-14 is operating properly, then verify that R14 is selected to be used in the total and R29B is not selected to be used in the total.
- 8.12.4 If R29B and R14 are not operating properly, then proceed to step 5.3 to calculate the plant vent stack ODCM dose assessment using a grab sample. Add the results to any SG Dose assessment subsequently performed.
- 8.12.5 If there is an indication of a SG tube leak or rupture, then proceed to step 8.13 to complete ODCM dose assessment.
- 8.12.6 If there is no indication of a SG tube leak or rupture, then verify that R15 and R60A, B, C, D are not selected for use and print this screen. Proceed to step 9.0 to evaluate the data.

8.13 Steam Jet Air Ejector (SJAE) Release

The ODCM calculation will use the instrument that is closest to midscale of R15, R15B, or R15 C to calculate the ODCM release rate from the SJAE. If there is no release from the SJAE then this calculation should not be performed. If there is an indication that the R15 series instruments are not working properly then SJAE ODCM calculation should be done using a manual calculation.

8.13.1 If the SG that has the tube leak or rupture has been isolated from the main steam system by closing the associated MSIVs, then verify that R15 is not selected to be used in the total and proceed to step 8.14.

8.13.2 If the SG that has the tube leak or rupture is not isolated from the main steam system, the SJAE are operating, and the R15 series instrument are working properly then verify that R15 is selected to be used in the total and proceed to step 8.14.

8.13.3 If the R15 instruments are not operating properly then proceed to step 7.0 to calculate the SJAE ODCM dose assessment using a grab sample. Add the results to any other dose assessment subsequently performed and proceed to step 8.14.

8.14 Turbine Driven AFW (TDAFW) Release

The TDAFW ODCM calculation is not performed unless the dose assessment for ARDA has been started automatically as a result of R-60D going into alarm or ARDA has been started manually for that release point. Once started, TDAFW ODCM is calculated based on an assumed flow from the TDAFW pump. If the TDAFW pump is not actually running using steam from a ruptured SG, the ODCM calculation is meaningless. The R60D instrument is a fairly high range instrument. The actual background reading may be lower than the minimum reading on the instrument. R60D should not be used for the calculation unless it has shown an observable increase.

8.14.1 If the TDAFW pump is not running using steam from the SG with a rupture or tube leak, verify that R60D is not marked for use in the total and proceed to step 8.15.

8.14.2 If the TDAFW pump is running using steam from the SG with a rupture or tube leak and R60D has shown an increase above normal background, verify that ARDA dose assessment has been started for R60D and it is marked for use in the total and proceed to step 8.15.

8.14.3 If the TDAFW pump is running using steam from the SG with a rupture or tube leak and R60D has not shown an increase above normal background, verify that R60D is not marked for use in the total. Calculate TDAFW ODCM using step 6.0. Add the results to any other dose assessment subsequently performed and proceed to step 8.15.

8.15 Steam Generator Atmospheric Relief or Safety Release

The R60A, B and C ODCM calculations are not performed unless the dose assessment for ARDA has been started automatically as a result of R60A, B or C going into alarm or ARDA has been started manually for that release point. The ODCM source term will be zero unless there is adequate steam pressure to cause a SG safety or relief to be open providing a flow term. By selecting a specific SG and valve(s) to be open, the operator can force an ODCM source term to be calculated. The R60 instruments are fairly high range instruments. The actual background reading may be lower than the minimum reading on the instrument. R60s should not be used for the calculation unless it has shown an observable increase.

8.15.1 If there is no flow out of any SG open relief or safety valve in a SG with a tube rupture or leak, then verify that R60A, B and C are not marked for use and print this screen.

8.15.2 If there is flow out of a SG open relief or safety valve due to elevated pressure in a SG with a tube rupture or leak and R60A, B or C for that SG has shown an increase above normal background, then verify that ARDA has started for that release point. Mark the specific SG for use and proceed to the next step.

8.15.3 If there is flow out of a SG open relief or safety valve due to a failed open valve in a SG with a tube rupture or leak and R60A, B or C for that SG has shown an increase above normal background, then verify an override for the specific valve as a release point, verify ARDA started for the release point, and select the specific R60 for use. Print the screen and proceed to step 9.0 to evaluate the data.

8.15.4 If there is flow out of a SG open relief or safety valve in a SG with a tube rupture or leak and R60A, B or C for that SG have not shown an increase above normal background, then verify that the specific R60 instruments are not selected for use. Calculate relief/safety ODCM using step 6.0. Add the results to any other dose assessment subsequently performed and proceed to step 9.0 to evaluate the data.

9.0 Evaluation of ODCM Dose Assessment Using the NRERDS Computer Terminal

- 9.1 After all of the appropriate instruments have been selected for use in the total or removed from the selection as appropriate and any required relief or safety valves have been selected as a release point, then print the ODCM screen.
- 9.2 The USE IN TOTAL column is selected by the operator to have the program use only those effluent instruments that are appropriate for this particular ODCM calculation.
- 9.3 There are some ODCM calculations that cannot be performed by the computer. If these are performed manually, the results should be added to the totals for evaluation. For example if plant vent stack has been calculated by using a grab sample add those results to the total for evaluation.
- 9.4 The total noble gas and total iodine from the PREVIOUS HOUR column plus any manual calculations required should be used to evaluate if the effluent criteria for the Alert or NOUE declarations has been exceeded.
- 9.5 If the NOUE criteria for noble gas or iodine has been exceeded, the word "EXCEEDED" will be printed next to the value that has exceeded the NOUE criteria.
- 9.6 The NOUE and ALERT effluent limits have been listed at the bottom of the page for easy reference.
- 9.7 The PEAK PROJECTED column can be used to evaluate what would happen to the ODCM calculation if the PEAK value from the previous hour is maintained for the next hour. This is a planning tool only and should not be used to make a declaration.

TABLE 1

REFERENCES

1. FNP-0-CCP-1300, Chemistry and Environmental Activities During a Radiological Accident
2. FNP-0-EIP-9.0, Emergency Classification and Actions
3. FNP-0-EIP-20.0, Chemistry and Environmental Support to the Emergency Plan
4. NT-86-0014, Gaseous Releases, Emergency Classifications
5. FNP-0-M-011, Off-site Dose Calculation Manual

TABLE 2

Steam Generator Relief/Safety Valves Open

This table may be used to help determine the number of valves that are open for each steam generator assuming there are no failed open or stuck valves.

Steam Generator Pressure Range in PSIG	Total Number Of Valves Open Per SG	Valves That Are Open For Each SG
<1035	0	NONE
1035-1074	1	1 SG PORV
1075-1087	2	1 SG PORV / 1 SG SAFETY
1088-1101	3	1 SG PORV / 2 SG SAFETIES
1102-1114	4	1 SG PORV / 3 SG SAFETIES
1115-1128	5	1 SG PORV / 4 SG SAFETIES
≥1129	6	1 SG PORV / 5 SG SAFETIES

FIGURE 1

**Calculations Based On RE-29B Noble Gas Or Iodine Channel Values (STEP 5.1)
WORKSHEET**

1. Enter the noble gas release concentration in $\mu\text{Ci/ml}$ from step 5.1.1 in block 1.
2. Enter the RE-29B iodine release Concentration in $\mu\text{Ci/ml}$ from step 5.1.2 in block 5.
3. Enter the PVS flowrate in scfm from step 5.1.3 or 5.1.4 in blocks 2 and 6.
4. Obtain the noble gas dose rate by multiplying block 1 times block 2 times block 3 and enter the results in block 4.
5. Obtain the iodine dose rate by multiplying block 5 times block 6 times block 7 and enter the results in block 8.
6. Return to step 5.1.6.

noble gas release concentration in $\mu\text{Ci/ml}$ from step 5.1.1	X	PVS flowrate (scfm) from step 5.1.3/4	X	conversion factor	=	noble gas dose rate (rem/hr)
BLOCK 1		BLOCK 2		BLOCK 3		BLOCK 4
	X		X	8.54E-07	=	

iodine release concentration in $\mu\text{Ci/ml}$ from step 5.1.2	X	PVS flowrate (scfm) from step 5.1.3/4	X	conversion factor	=	Iodine dose rate (rem/hr)
BLOCK 5		BLOCK 6		BLOCK 7		BLOCK 8
	X		X	9.41E-04	=	

FIGURE 2

Calculation Based On RE-14 Count Rate (STEP 5.2)
WORKSHEET

1. Enter the RE-14 release count rate in cpm from step 5.2.1 in block 1.
2. Enter the PVS flowrate in scfm from step 5.2.4 or 5.2.5 in block 2.
3. Obtain the noble gas dose rate by multiplying block 1 times block 2 times block 3 and enter the results in block 4.
4. Iodine dose rate must be calculated using step 5.1.2 (figure 1) or step 5.3.2 (figure 3).
5. Return to step 5.2.7.

RE-14 release count rate in cpm from step 5.2.1	X	PVS flowrate (scfm) from step 5.2.3/4	X	conversion factor	=	noble gas dose rate (rem/hr)
BLOCK 1		BLOCK 2		BLOCK 3		BLOCK 4
	X		X	5.697E-16	=	

FIGURE 3

**Calculations Based On Plant Vent Stack Grab Sampling And Analyses (STEP 5.3)
WORKSHEET**

1. Enter the dose equivalent Xe-133 release concentration in $\mu\text{ci/ml}$ from step 5.3.1 in block 1.
2. Enter the dose equivalent I-133 release concentration in $\mu\text{ci/ml}$ from step 5.3.2 in block 5.
3. Enter the PVS flowrate in scfm from step 5.3.3 or 5.3.4 in blocks 2 and 6.
4. Obtain the noble gas dose rate by multiplying block 1 times block 2 times block 3 and enter the results in block 4.
5. Obtain the iodine dose rate by multiplying block 5 times block 6 times block 7 and enter the results in block 8.
6. Return to step 5.3.6.

DE Xe-133 release concentration in $\mu\text{ci/ml}$ from step 5.3.1	X	PVS flowrate (scfm) from step 5.3.3/4	X	conversion factor	=	Noble gas dose rate (rem/hr)
BLOCK 1		BLOCK 2		BLOCK 3		BLOCK 4
	X		X	8.54E-07	=	

DE I-131 release concentration in $\mu\text{ci/ml}$ from step 5.3.2	X	PVS flowrate (scfm) from step 5.3.3/4	X	conversion factor	=	Iodine dose rate (rem/hr)
BLOCK 5		BLOCK 6		BLOCK 7		BLOCK 8
	X		X	9.41E-04	=	

FIGURE 4

**Calculations Based On SG Atmospheric Reliefs, Safeties and TDAFW Releases (STEP 6.0)
WORKSHEET**

1. Enter the steam pressure in psig from step 6.2 for each SG with a radioactive release, determined in step 6.1, in blocks 1, 5 and 9 respectively.
2. Enter the number of valves open from step 6.3 for each SG with a radioactive release, determined in step 6.1, in blocks 2, 6 and 10 respectively.
3. Obtain the steam flow for each SG with a radioactive release by multiplying the steam pressure in blocks 1, 5 and 9 times the number of valves open in blocks 2, 6 and 10 times the conversion factor in blocks 3, 7 and 11. Enter the results in blocks 4, 8 and 12.
4. If there is a radioactive release, determined in step 6.5, from the TDAFW then enter a value of 11175 in block 13 otherwise enter a value of ZERO there.
5. Obtain total steam flow by adding the values in blocks 4, 8, 12 and 13. Enter the total steam flow value in block 14 and in blocks 19 and 23 on the following page.

Steam Generator A					
steam pressure psig from step 6.2	X	# valves open from step 6.3	X	conversion factor	= steam flow cfm
BLOCK 1		BLOCK 2		BLOCK 3	BLOCK 4
	X		X	359.12	=
Steam Generator B					
steam pressure psig from step 6.2	X	# valves open from step 6.3	X	conversion factor	= steam flow cfm
BLOCK 5		BLOCK 6		BLOCK 7	BLOCK 8
	X		X	359.12	=
Steam Generator C					
steam pressure psig from step 6.2	X	# valves open from step 6.3	X	conversion factor	= steam flow cfm
BLOCK 9		BLOCK 10		BLOCK 11	BLOCK 12
	X		X	359.12	=
TDAFW					
If there is a radioactive release from the TDAFW then enter a value of 11175 in block 13 otherwise enter a value of ZERO					BLOCK 13
Total Steam Flow					
Add the Steam flow from all three SGs and the TDAFW and enter the value in block 14 and in blocks 19 and 23 on the following page					BLOCK 14

FIGURE 4

6. Enter the total iodine release concentration from step 6.8 in block 15.
7. Enter the ratio of total iodine to I-131 from step 6.9, step 6.10 or figure 6 in block 16.
8. Determine the Iodine 131 release concentration by dividing block 15 by block 16 enter the value in block 17 and block 22.
9. Enter the noble gas release concentration from step 6.7 in block 18.
10. Enter the steam release flow rate (cfm) from step 6.6 (Block 14 from the previous page).
11. Obtain the noble gas dose rate by multiplying block 18 times block 19 times block 20 and enter the results in block 21.
12. Obtain the iodine dose rate by multiplying block 22 times block 23 times block 24 and enter the results in block 25.
13. Return to step 6.12.

total iodine release concentration in $\mu\text{ci/ml}$ from step 6.8	/	ratio of total iodine to I-131 from step 6.9/10	=	iodine 131 release concentration in $\mu\text{ci/ml}$
BLOCK 15		BLOCK 16		BLOCK 17
	/		=	

noble gas release concentration ($\mu\text{ci/ml}$) from step 6.7	X	steam flow rate (cfm) from step 6.6 (block 14)	X	conversion factor	=	noble gas dose rate (rem/hr)
BLOCK 18		BLOCK 19		BLOCK 20		BLOCK 21
	X		X	3.85E-05	=	

iodine 131 release concentration in $\mu\text{ci/ml}$ from step 6.9/10 block 17	X	steam flow rate (cfm) from step 6.6 (block 14)	X	conversion factor	=	iodine dose rate (rem/hr)
BLOCK 22		BLOCK 23		BLOCK 24		BLOCK 25
	X		X	4.24E-02	=	

FIGURE 5

**Calculation Based On Steam Jet Air Ejector Release (STEP 7.0)
WORKSHEET**

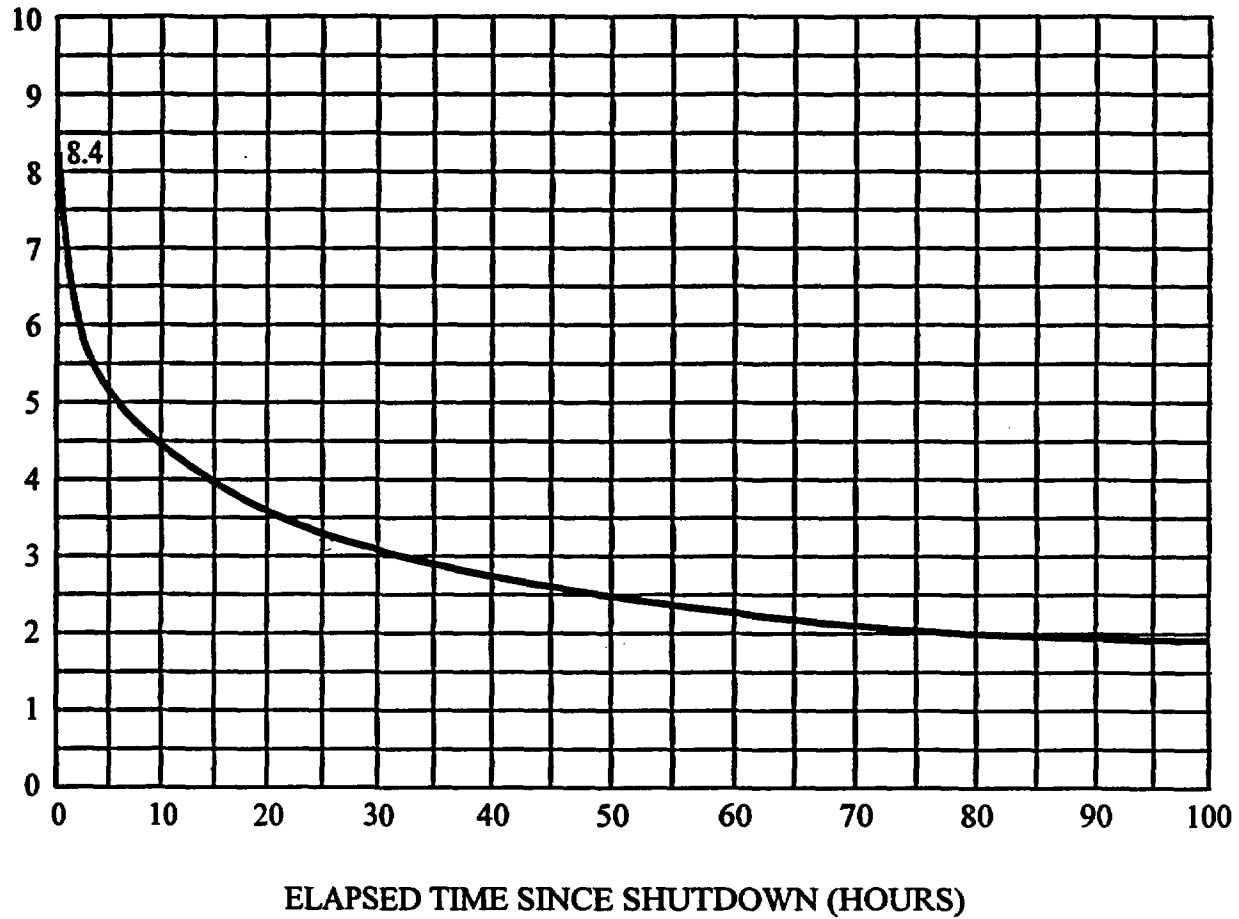
1. Enter the total iodine release concentration from step 7.2 in block 1.
2. Enter the ratio of total iodine to I-131 from step 7.3, step 7.4 or figure 6 in block 2.
3. Determine the Iodine 131 release concentration by dividing block 1 by block 2. Enter the value in block 3 and block 7.
4. Enter the noble gas release concentration from step 7.1 in block 4.
5. Obtain the noble gas dose rate by multiplying block 4 times block 5 and enter the results in block 6.
6. Obtain the iodine dose rate by multiplying block 7 times block 8 and enter the results in block 9.
7. Return to step 7.6.

total iodine release concentration in $\mu\text{Ci/ml}$ from step 7.2	/	ratio of total iodine to I-131 from step 7.3/4	=	iodine 131 release concentration in $\mu\text{Ci/ml}$
BLOCK 1		BLOCK 2		BLOCK 3
	/		=	

noble gas release concentration in $\mu\text{Ci/ml}$ from step 7.1	X	conversion factor, includes 1050 scfm flow	=	noble gas dose rate (rem/hr)
BLOCK 4		BLOCK 5		BLOCK 6
	X	4.0425 E-02	=	

iodine 131 release concentration in $\mu\text{Ci/ml}$ from step 7.3/4 (block 3)	X	conversion factor, includes 1050 scfm flow	=	iodine dose rate (rem/hr)
BLOCK 7		BLOCK 8		BLOCK 9
	X	44.52	=	

Figure 6
RATIO OF TOTAL IODINES TO IODINE - 131



SHARED

Figure 7

Unit X ODCM		NON REGULATORY ERDS/ARDA				Date Time
Unit Group Display Print Start Dose Assessment ODCM Dose Assessment					Print Screen	
Release Point		Met Information	Radiation Monitors	Plant Parameters	Previous History Inventory ODCM Options	
USE IN TOTAL			Previous Hour	EXCEEDED	Peak Projected	
[X]	RE 29B	Noble Gas Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
		Iodine Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
[]	RE 14	Noble Gas Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
[]	RE 15	Noble Gas Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
		Iodine Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
[]	RE 60A	Noble Gas Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
		Iodine Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
[]	RE 60B	Noble Gas Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
		Iodine Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
[]	RE 60C	Noble Gas Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
		Iodine Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
[]	RE 60D	Noble Gas Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
		Iodine Dose Rate	X.XXE-XX		X.XXE-XX R/HR	
		Total Noble Gas	X.XXE-XX		X.XXE-XX R/HR	
		Iodine	X.XXE-XX		X.XXE-XX R/HR	
Noble Gas NOUE Limit = 5.7E-05 R/HR			Noble Gas ALERT Limit = 5.7E-04 R/HR			
Iodine NOUE Limit = 1.7E-04 R/HR			Iodine ALERT Limit = 1.7E-03 R/HR			

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SOUTHERN NUCLEAR OPERATING COMPANY

JOSEPH M. FARLEY NUCLEAR PLANT

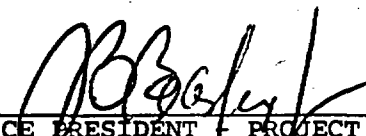
UNITS 1 & 2

EMERGENCY PLAN


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APPROVED:


VICE PRESIDENT - PROJECT (FARLEY)

4/7/03
Date Approved


NUCLEAR PLANT GENERAL MANAGER

8-22-03
Date Approved



Date Issued: 8-29-03

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JOSEPH M. FARLEY NUCLEAR PLANT
EMERGENCY PLAN

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The second phase includes immediate and planned action directed toward termination of the incident, containment of the effluent, establishment of incident boundaries, establishment of control, channeling of information and protection of the facility and equipment. The third phase is to restore the facility to its normal operating condition. To respond effectively utilizing these phases, emergencies are classified according to increasing severity as Notification of Unusual Event, Alert, Site Area Emergency or General Emergency.

C. GENERAL INFORMATION

1. Definitions

a. Affected Persons

Individuals who have been radiologically exposed or physically injured as a result of an accident to a degree requiring special attention, e.g., decontamination, first aid, or medical services.

b. Assessment Actions

Those actions taken during or after an accident which are collectively necessary to make decisions to implement specific emergency measures.

c. Controlled Area

The Controlled Area is the fenced area immediately surrounding the nuclear plant, access to which is controlled for industrial security purposes.

d. Corrective Action

Those emergency measures taken to terminate an emergency situation at or near the source of the problem.

e. Emergency Action Levels

Radiological dose rates; specific contamination levels of airborne, waterborne or surface deposited concentrations of radioactive materials; or specific instrument indications (including their rates of change) that may be used as thresholds for initiating such specific emergency measures as designating a particular class of emergency, initiating a notification procedure or initiating a particular protective action.

f. CEOC Manager

The Vice President - Project (Farley) or his designated alternate as

the CEOC Manager is responsible for the activation of the corporate emergency organization and for providing corporate emergency support prior to and following Emergency Operations Facility activation.

g. Emergency Director

The Nuclear Plant General Manager or his designated alternate as the Emergency Director is charged with the responsibility of overall direction of the plant emergency activity and with initial interfacing with offsite groups.

h. SNC Corporate Duty Manager

The Vice President - Project (Farley) or his designated alternate will serve as the Duty Manager. The Duty Manager is responsible for the overall management of emergency support at FNP. The Duty Manager will serve as the corporate spokesperson until such time as an alternate Duty Manager or alternate EOF Manager is available to assume the role of spokesperson.

i. Offsite

All land and water areas outside the site property lines are considered to be offsite.

j. Onsite

All land and water areas inside the site property lines, use of which must be authorized by SNC, is referred to as onsite.

k. Population at Risk

Those persons for whom protective actions are being or would be taken.

l. Protected Area

The Protected Area is the fenced area immediately surrounding the plant Vital Areas, access to which is limited to those individuals with good cause for entry.

m. Protective Action Guides

Projected radiological dose or dose commitment values to individuals in the general population that warrant protective action following a release of radioactive material.

n. Protective Actions

Those emergency measures taken after an uncontrolled release of radioactive material has occurred for the purpose of preventing or minimizing radiological exposures that would be likely to occur to persons if the actions were not taken.

o. Radiation Controlled Area

The containment and the potentially contaminated portion of the Auxiliary Building and other areas onsite such as High Radiation Area, Radiation Area, Radioactive Materials Area, Airborne Radioactivity Area or Contaminated Area.

p. Recovery Actions

Those actions taken after the emergency to restore the plant as nearly as possible to its pre-emergency condition.

q. EOF Manager

The Vice President - Project (Farley) or his designated alternate as the EOF Manager is responsible for the overall emergency response and recovery effort.

r. Vital Area

The Vital Areas are those plant areas which enclose major systems, equipment and components necessary to prevent or mitigate the consequences of an accident.

2. Emergency Ingress and Egress

a. Emergency Ingress

Ingress to any area of the plant can be obtained by the use of keys which are maintained by the Shift Supervisor and the Senior Security Force Member. In the case of electrically locked doors, keys will override the locking device. The necessary keys will be issued as required to combat the emergency.

b. Emergency Egress

Egress from any area of the plant is assured without keys, electrical power or other devices.

II. ORGANIZATION

The organization, responsibilities and functions of Southern Nuclear Operating Company onsite and offsite resources are individually discussed below. The onsite and offsite organizations provide emergency response during the activation, emergency, and recovery phases of accident response. Principal federal, state, local and private agencies are also discussed. Figures 12 and 13 illustrate the interrelationships of these organizations before and after Emergency Operations Facility activation respectively.

A. ONSITE

The normal onsite organization for Farley Nuclear Plant is shown on Figure 1. Management positions in the onsite organization meet the qualification requirements of ANSI N18.1-1971.

The qualifications for the professional-technical level positions also meet the requirements of ANSI N18.1-1971.

1. Technical Support Center (TSC)

The emergency onsite organization implemented for events requiring activation of the TSC is described in FNP-0-EIP-0 and is shown in Figure 2. Responsibilities and authorities of personnel in the TSC emergency organization are as follows:

a. Emergency Director (ED)

The ED is charged with the responsibility of overall direction of onsite emergency activity including near-site field monitoring team dispatch and control and interfacing with offsite organizations and agencies until the Emergency Operations Facility (EOF) is functional. After the EOF is functional, the ED is responsible for overall direction of all in-plant emergency activity. The ED shall supervise the TSC and manage the inplant recovery efforts and the inplant recovery organization. The ED shall communicate directly with the EOF Manager when the EOF is activated and shall have full authority to direct the onsite recovery efforts without further consultation when the situation demands such action. Following EOF activation when time permits the ED will consult with EOF personnel prior to initiating major evolutions or changes in plant configuration. The ED's general responsibilities include:

- 1) Staffing the TSC. The TSC will be staffed by plant supervisory personnel supplemented by plant engineering, technical and administrative personnel as necessary to staff the TSC 24 hours a day and discharge the responsibilities discussed below.
- 2) Evaluating the classification of the emergency and amending as appropriate. Downgrading an emergency level will not be delegated to other elements of the emergency organization and will be concurrently agreed upon by the EOF Manager.

- 3) Verifying correct control room response to the emergency classification.
- 4) Determining radiological status and initiating notifications to state agencies (and local agencies for General Emergencies). The decision to notify offsite government agencies may not be delegated to any other element of the emergency organization.
- 5) Initiating, prior to EOF activation and on initial or upgrade emergency notifications, recommendations to state agencies on advisability of evacuations. Recommendations to local agencies when state authorities cannot be contacted for immediate evacuation may not be delegated to any other element of the emergency organization.
- 6) Initiating rescue or emergency repair operations as appropriate.
- 7) Maintaining plant security.
- 8) Establishing communications with and providing information to the EOF Manager.

In fulfilling the above listed responsibilities the Emergency Director (ED) is guided by the procedures listed below:

- FNPP-0-EIP-3 Duties of the Emergency Director
- FNPP-0-EIP-8.1 Emergency Phone Directory
- FNPP-0-EIP-8.3 Communication Equipment Operating Procedures
- FNPP-0-EIP-9.0 Emergency Classification and Actions
- FNPP-0-EIP-29 Long Term Dose Assessment

The ED position is initially filled by the Shift Superintendent until relieved by the on-call ED.

The line of succession of individuals who may serve as the ED is as follows:

Nuclear Plant General Manager

Plant Operations Assistant General Manager

Plant Support Assistant General Manager

Operations Manager

On-call Operations Supervisor

Shift Superintendent

Shift Supervisor

Other Managers or staff designated by the Nuclear Plant General Manager

The above line of succession does not preclude higher level management from assuming the role of Emergency Director (ED) in any circumstance which, in the judgment of the manager, is appropriate or necessary to protect the health and safety of the public. This designation also does not relieve higher level management from the responsibility to be aware of those circumstances that may initiate this action. These individuals will be trained as ED's.

b. TSC Manager

The on-call TSC Manager reports to the Technical Support Center (TSC) and is responsible for implementing FNP-0-EIP-6, "TSC Setup and Activation", which includes coordination of communications between the TSC and other locations and coordination of engineering support and log maintenance.

c. Operations Supervisor

The on-call Operations Supervisor reports to the Technical Support Center (TSC) and is responsible for coordinating the efforts of the operating crew, advising the ED on emergency operations and facilitating communications between the ED and Shift Supervisor. Supervisory personnel designated by the Nuclear Plant General Manager and holding a Senior Reactor Operator License rotate as the on-call Operations Supervisor.

d. Maintenance Supervisor

The on-call Maintenance Supervisor reports to the TSC and is responsible for implementing FNP-0-EIP-5, "Maintenance Support to the Emergency Plan", including coordination of the efforts of Emergency Repair Parties and advising the ED on proposed modifications, alterations or repair to plant systems and on specifics of plant systems and equipment. Supervisory I&C and Maintenance personnel designated by the Nuclear Plant General Manager rotate as the on-call Maintenance Supervisor.

e. Health Physics Supervisor

The Health Physics Supervisor reports to the TSC and is responsible for implementation of FNP-0-EIP-4, "Health Physics Support to the Emergency Plan", including coordination of the efforts of in-plant Field Monitoring Teams, decontamination activities, Health Physics and ALARA support, and advising the ED on the status of onsite and offsite radiation protection activities. This individual is also responsible for coordination of out-of-plant and SNC offsite Field Monitoring Teams until relieved by the Emergency Operations Facility (EOF) staff. The Health Physics Supervisor and other supervisory personnel designated by the Nuclear Plant General Manager and, to the maximum extent possible, meeting the requirements of Regulatory Guide 1.8, September 1975, rotate as the on-call Health Physics Supervisor.

f. Security

Security supervision is responsible for implementing FNP-0-EIP-7, "Security Support to the Emergency Plan", maintaining site security and advising the ED.

g. Engineering Supervisor

The on-call Engineering Supervisor coordinates offsite dose projections until relieved by the Emergency Operations Facility (EOF) staff. Until relieved by the EOF staff, the Technical Support Center (TSC) is responsible for communications with the Nuclear Regulatory Commission (NRC), state and local government agencies and other offsite authorities. During that period, the Engineering Supervisor is responsible for supervising collection of pre-identified information for transmittal to offsite authorities, responding to requests from offsite agencies for information, coordinating approval by the Emergency Director or his designee of all information transmitted to offsite authorities and for routing incoming information to the appropriate personnel. This position will be filled by a qualified individual designated by the Nuclear Plant General Manager.

h. Shift Superintendent

The Shift Superintendent is responsible for directing operational activities to classify and combat the emergency as delineated in FNP-0-EIP-3, "Duties of the Emergency Director". The Shift Superintendent acts as the Emergency Director (ED) until relieved by the on-call ED and until relieved has the authority and responsibility to immediately and unilaterally initiate any necessary emergency actions, including providing protective action recommendations to authorities responsible for implementing offsite emergency measures.

i. Emergency Repair Party

The Emergency Repair Party, as shown in Figure 2, is a group of personnel competent in operations and repair work who will be used during an emergency situation to make temporary repairs to systems/components in order to mitigate the effects of the emergency. An Emergency Repair Party for initial re-entry and repair will consist of individuals as required from the following personnel groups:

- Operations Personnel
- Maintenance Personnel
- Instrumentation and Control Personnel
- Health Physics Personnel
- Chemistry Personnel

j. Field Monitoring Team (FMT)

The Field Monitoring Teams, as shown in Figure 2, consisting of permanent plant employees and/or qualified vendor personnel, will perform onsite and offsite monitoring. They

will provide radiation protection support at the Southeast Alabama Medical Center, during transport of potentially irradiated and/or contaminated casualties, and at the Assembly areas, and at any other location onsite or offsite as instructed by the Emergency Director (ED) or EOF Manager.

To perform these functions a number of teams will be designated consisting of a Team Leader and an Assistant.

Team Leader - A Health Physics Technician
or qualified vendor technician.
Assistant - Any qualified plant employee or
vendor personnel.

k. Dose Assessment Staff

9 The Shift Supervisor is responsible for offsite dose projections until relieved by the Technical Support Center (TSC) staff. Personnel reporting to the Engineering Supervisor are responsible for making dose projections until the Emergency Operations Facility (EOF) is functional, at which time EOF dose assessment personnel become responsible for making offsite dose projections. These projections may initially be made automatically by a computerized dose projection program described in FNP-0-M-007, "Emergency Dose Calculation Manual". A manual personal computer methodology is provided in FNP-0-EIP-9.3, "Personal Computer-Automated Dose Assessment Method" for long term dose assessment or in the event that the automatic computerized system is inoperable. Normally, dose projections are transmitted to appropriate state authorities by computer link to printers, telecopy, commercial telephone or the Emergency Notification Network (ENN). The Emergency Notification System (ENS), Health Physics Network (HPN), and commercial telephone lines are available for transmission of dose assessment data to the NRC. Data will be provided as directed by the NRC at the time of need.

1. Additional Plant Staff Assignments

1). Operations Support Center (OSC) Manager

The OSC Manager will be considered to be the senior individual in the OSC and will report to the Maintenance Supervisor. The OSC Manager will take the lead in coordinating the activities of the OSC or other location directed by the Emergency Director per FNP-0-EIP-5.0. The senior individual at each of the Assembly Areas will become the supervisor at that location. The Assembly Area senior individual will take the lead in coordinating the activities of the Assembly Area in support of OSC operations as directed by the OSC Manager.

2) Radiological monitoring

The Health Physics Group is responsible for all aspects of applied health physics. Emergency monitoring will be provided by a Health Physics Technician on shift, a qualified/trained vendor technician, or qualified member of the plant staff. Health Physics supervision will be responsible for relocation of access control to both units as necessary, and for implementing procedures for handling highly radioactive samples. -

3) Fire Fighting and Rescue

The plant fire brigade and rescue team on all shifts will be composed of personnel described in FNP-0-AP-37. The fire brigade will be directed by the Fire Brigade Chief with the aid of FNP-0-EIP-13.

4) First Aid

At least one person on each shift will be qualified to perform first aid.

5) Decontamination

Personnel decontamination is the responsibility of the Health Physics Group and during an emergency the responsibility of the Field Monitoring Team.

Area and equipment decontamination onsite as the result of an accident will be a joint effort of personnel from the Operations, Maintenance, Chemistry and Health Physics Groups.

6) Personnel Accountability

Personnel accountability is the responsibility of each plant supervisor or senior individual onsite in the group. That is, each supervisor is responsible for accounting for each person onsite in his group or visiting his group. Details for personnel accountability are provided by FNP-0-EIP-10, "Evacuation and Personnel Accountability". Information pertinent to personnel accountability will be kept by security guards at each access control point.

7) Record Keeping

A record of all significant events that occur will be kept by the operating crew in the Plant Operator's Logbook. A log will be kept by a designated plant staff member who will be responsible for maintaining communications with the corporate headquarters, and offsite authorities as directed by the Emergency Director. Radiological information such as radiological survey data, personnel exposures, decontamination activities and information from onsite groups will be maintained by the Health Physics Supervisor.

8) Communications

Responsibility for initial offsite communications will be handled by the Shift Supervisor or Emergency Director. After the emergency organization is functional, a designated plant staff member(s) will maintain communications with the corporate headquarters and with offsite authorities. If the Emergency Director is not located in the control room he will maintain communications with the control room through an assigned individual. When the Emergency Operations Facility (EOF) is functional, the EOF staff will handle offsite communicating. Communications interfaces are shown in Figure 3.

2. Emergency Operations Facility (EOF)

The Emergency Operations Facility (EOF) Emergency Response Organization (ERO) and its relationship to the Technical Support Center (TSC) emergency organization is given in Figure 4. The EOF Staff will be supplemented by plant personnel, vendor personnel and other corporate personnel as necessary to staff the EOF 24 hours a day and discharge the responsibilities designated below. The EOF Manager has authority to modify the organization as deemed necessary.

a. EOF Manager

The EOF Manager has overall responsibility and authority for management of SNC emergency resources, coordination of SNC emergency response activities with those of local, state, and federal organizations, and execution of EOF functions described below. The line of succession of individuals who may serve as the EOF Manager is as follows:

1. Vice President - Project (Farley)
2. Nuclear Plant General Manager
3. Nuclear Support General Manager
4. Plant Operations Assistant General Manager
5. Plant Support Assistant General Manager
6. Other managers as designated by Vice President - Project (Farley)

b. Dose Assessment Supervisor

The Dose Assessment Supervisor is responsible for coordinating company environmental monitoring activities, for evaluating the magnitude and effect of actual or potential radioactive releases, maintaining dose assessment status boards, for providing to the EOF Manager recommendations regarding offsite protective measures and for providing applied health physics support to the EOF. Reporting to this position are SNC offsite/onsite Field Monitoring Teams (FMTs), personnel for operating FMT EOF radio communications and for performing calculations necessary to evaluate and predict offsite dose rates including Dose Assessment Support and other personnel as necessary to accomplish the duties assigned to the Dose Assessment Supervisor. Dose assessment activities are delineated in FNP-0-EIP-9.0, "Emergency Classification and Actions", and FNP-0-EIP-29, "Long Term Dose Assessment". Prior to activation of the EOF, this activity is performed by the Technical Support Center Staff. This position will be filled by a qualified individual designated by the Nuclear Plant General Manager.

c. Support Coordinator

The Support Coordinator is responsible for coordinating overall administrative, logistics and engineering support for the EOF and plant. This position will be filled by a qualified individual as designated by the Nuclear Plant General Manager. Reporting to the Support Coordinator are personnel as necessary to perform the following:

- 1) Staffing of emergency communications and telephone switchboard equipment at the EOF (except dose assessment communication equipment and NRC dedicated communications) and maintaining communications logs.
- 2) Monitoring of plant status and offsite protective action status and maintaining logs and emergency status boards not maintained by the Dose Assessment Supervisor.
- 3) Providing liaison with the CEOC for administrative, logistics, and engineering support from within the Company and from outside vendors as required. Obtaining and scheduling manpower support using company personnel and outside vendors.
- 4) Coordinating personnel affairs (to include assisting temporary personnel obtain lodging and transportation).

d. Technical Advisor

The technical advisor serves as a liaison between the EOF Manager and the Public Information Director. The Technical Advisor reports directly to the EOF Manager and will normally be located at the Emergency News Center (ENC) once the ENC is activated. This position is staffed by available plant personnel.

B. OFFSITE

The normal Alabama Power Company (APC) offsite company organization is shown in Figure 5. The normal Southern Nuclear Company Farley Project organization and its relationship to the onsite organization is shown in Figure 6 and the Emergency Communication Organization is shown in Figure 7.

The normal functions, responsibilities and authorities of the Senior Southern Nuclear-Farley Project Management are as follows:

- President

Provides for upper level management of the Farley Project.

- Executive Vice President

The Executive Vice President provides upper level management for the Farley Project.

- Vice President - Project (Farley)

The Vice President shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

- Nuclear Support General Manager

Provides managerial guidance and direction for all offsite support activities involved in safe and efficient operation of Farley Nuclear Plant. These activities include engineering, licensing, maintenance, technical and administrative support.

- Quality Assurance Manager

Provides overall management and guidance for implementation of Quality Assurance review activities which include independent verification and evaluation of performance, plant procedures, activities, operations, and documentation from a nuclear safety perspective.

The offsite emergency organizations and their duties and responsibilities are described below:

1. Corporate Emergency Operations Center (CEOC)

In the event of an emergency condition at FNP that requires activation of the CEOC Emergency Response Organization (ERO) the organization shown in Figure 8 will be activated to notify Emergency Organization personnel and to provide corporate support from SNC. The CEOC Manager has authority to modify this organization as deemed necessary.

a. SNC Duty Manager

The Duty Manager is responsible for the overall management of emergency support at FNP. The Duty Manager is the primary contact for support from off-site agencies, and provides assistance, and advice to the EOF Manager and Emergency Director in decisions involving the overall effect of the event. The Duty Manager directs the efforts of the CEOC staff as it provides the necessary support to the EOF and TSC. During the activation phase, the Duty Manager is responsible for initiating the corporate response. The Duty Manager will serve as the corporate spokesperson until such time as an alternate Duty Manager or EOF is available to assume the role of spokesperson. This position will be filled by a qualified individual designated by the Vice President Project (Farley)

b. CEOC Manager

The CEOC Manager is responsible for supporting Emergency Organization activation and for supervising corporate emergency support. The CEOC Manager directs the CEOC staff to provide any necessary support as required. At all times a qualified individual, as designated by the Vice President - Project (Farley), is on-call or available as the CEOC Manager.

c. Administrative Logistics Manager (ALM)

The ALM is responsible for supporting Emergency Organization activation, notifying insurance agencies, other Company Departments potentially involved in emergency support, offsite support agencies and Corporate Office staff members who will augment or relieve the on-call CEOC or EOF staff. The ALM is also responsible for arranging for logistics needs. The Vice President - Project (Farley) shall designate individuals qualified to fill this position.

d. Engineering Support Manager (ESM)

The Engineering Support Manager is responsible for overall coordination of offsite technical and engineering support, and for maintaining appropriate status boards. Reporting to the ESM will be engineering and technical personnel assigned to the CEOC. The Vice President - Project (Farley) shall designate individuals qualified to fill this position.

e. Licensing Support Manager (LSM)

The Licensing Support Manager is responsible for overall coordination of all written requests required by regulatory agencies and for licensing related activities. Reporting to the LSM will be licensing personnel assigned to the CEOC. The Vice President - Project (Farley) shall designate individuals qualified to fill this position.

f. SNC News Writer

The SNC News Writer is responsible for monitoring emergency activities and keeping the Emergency Communication Organization management informed. The News Writer obtains information necessary for preparing news releases, coordinates all statements about an emergency with the CEOC Manager and the Corporate Media Coordinator, prepares news release drafts and obtains approval from the CEOC Manager.

g. Public Information Emergency Coordinator (PIEC)

The PIEC is responsible for activation of the Emergency Communication Organization and for supervising corporate Public Information activities until the Public Information Director arrives at the Emergency News Center.

h. Activation Assistant

The Activation Assistant is responsible for activating the Emergency Communication Organization as directed by the PIEC.

2. Emergency Communication Organization (ECO) Staff

During the emergency support phase the Emergency Communication Organization is implemented as shown in Figure 9. The Emergency Communication Organization Staff will be supplemented as necessary by personnel from other Southern Company system companies. This organization may be modified as deemed necessary by direction of the PID or his designee. The PID will notify the SNC Corporate Communications Manager and the CEOC Manager of modifications as appropriate. Responsibilities and authorities of personnel in this organization are as follows:

a. Vice President Administrative Services

The Vice President Administrative Services or his designee is responsible for overall corporate public and employee information support in the event of an emergency at FNP. In fulfilling this responsibility he: advises the company emergency organization of corporate public information activities and matters of public concern, communicates with the Southern Company regarding public information activities and any additional personnel needed to handle the public information activities, and communicates with state agencies, select groups and local/national trade associations.

b. Corporate Media Coordinator

The Corporate Media Coordinator is responsible for Emergency Communication Organization activities conducted from the APC corporate offices during an emergency at FNP. In fulfilling this responsibility, the Corporate Media Coordinator is responsible for coordinating the activities of the Division Public Information Representatives, maintaining and updating an emergency telephone actuality system and a rumor control system for the media, monitoring state and national news media reports and evaluating consistency of information and effectiveness of public information releases. Media Relations Representatives will normally

report to this position and assist in performing this function. Also reporting to the Corporate Media Coordinator are the Political Liaison, Financial Response Officer, Corporate Public Inquiry Coordinator and the Corporate Media Monitoring Staff.

c. Political Liaison

The Political Liaison is responsible for providing information to pre-identified organizations and public officials requiring information during an emergency at FNP.

This position will normally be filled by an APC Governmental Relations staff member.

d. Employee Communication Coordinator

The Employee Communication Coordinator is responsible for providing information on the emergency to company employees and manning the telephone actuality system at the APC Corporate Headquarters Office for use by media.

This position will normally be filled by an APC Public Relations staff member.

e. Public Information Director (PID)

The PID is responsible for all public information support activities conducted at the Emergency News Center (ENC), the Corporate Emergency Operations Center (CEOC), and the APC Headquarters Corporate Media Center (CMC). Reporting to this position are the SNC News Writer, the ENC Coordinator, and the Corporate Media Center Coordinator. Following activation of the Emergency Communication Organization, this individual advises the EOF Manager and SNC Corporate Communications Manager on communication activities and matters of public concern; directs the activities of the Emergency Communication Organization; ensures coordination of all public statements about an emergency; and coordinates acquisition of additional public information support personnel from other system companies as necessary.

This position will normally be filled by APC's Corporate Information Manager or a member of APC's Public Relations staff trained to perform the role of PID. This position usually operates from the Emergency News Center.

f. Emergency News Center Coordinator (ENC)

The ENC Coordinator is responsible for coordinating activities at the ENC, monitoring local news media reports to evaluate public information effectiveness, and coordinating the security pass system for media representatives working at the ENC. After ENC activation, the ENC Coordinator is responsible for serving as liaison between APC/SNC and agency public information contacts and the news media. This position will normally be filled by an APC Public Relations staff member. Reporting to this position will also be an assistant who will aid the ENC Coordinator in performing his function. Also reporting to this position are ENC Media Monitoring Staff, ENC Media Relations Representatives, ENC Security Coordinator, and the ENC Public Inquiry Coordinator.

3. Recovery Phase Organization

Upon termination of the emergency condition and at the discretion of the EOF Manager, the SNC Emergency Organization will shift to the Recovery Phase Organization shown in Figure 10. The Recovery Manager has authority to modify the organization as deemed necessary.

Responsibilities and authorities are:

a. Recovery Manager

The Recovery Manager shall direct the overall recovery effort. He has the full authority and responsibility to make decisions regarding plant recovery and return to operation. The line of succession for the Recovery Manager position will be as described previously in section II.A.2.

actions, responsibilities, functions and authorities during an emergency. Written agreements have been reached with the other offsite agencies listed below with regard to the type of support that will be furnished to the Joseph M. Farley Nuclear Plant in the event of an emergency. These agreements have been developed to ensure that there is a clear understanding of assigned responsibilities and that there will be proper coordination of activities in the event of an emergency. Letters of Agreement with offsite support groups are given in Part I, Appendix B.

Corporate and/or plant personnel will be dispatched to principal government agencies on an as needed basis.

Anticipated offsite federal assistance is discussed in the individual state plans.

a. Department of Energy Savannah River Operations Office

In the event of a General Emergency, the DOE Savannah River Operations Office has agreed to provide a DOE Radiological Assistance Team. This assistance team will be limited to advisory assistance in handling radiological emergencies. The Emergency Director or EOF Manager is authorized to request this assistance.

b. Nuclear Regulatory Commission

Upon notification of an emergency condition, the NRC will implement the incident response plans described in NUREG-0845 and NRC Region II will implement the incident response plans described in Supplement 2 to NUREG-0845. In addition to fulfilling its regulatory responsibilities, it is expected that the NRC will provide technical assistance and recommendations. For Site Area and General Emergencies, dispatch to the site of a NRC Region II site team is anticipated with arrival expected 2 to 6 hours following notification. As described in Section III, office space, telephones, etc. have been provided for NRC personnel at the Technical Support Center and Emergency Operations Facility.

c. State of Alabama

The Alabama Radiation Control Division of the State of Alabama Department of Public Health is responsible for initiating the "Alabama Radiological Response Plan for Nuclear Power Plants" in support of an emergency at the Farley Nuclear Plant. This plan provides a detailed description of the notification procedures and responsibilities and duties of the local and state agencies involved. Since the primary concern of the Alabama Radiation Control Division is for the welfare and safety of the general public, they will have primary responsibility and authority for handling the offsite aspects of the emergency in Alabama.

h. City of Dothan, Alabama - Fire Department

In the event an emergency (Section IV) is declared as a result of a fire at Farley Nuclear Plant, the Dothan Fire Department has agreed to provide support to help combat the fire. The Dothan Fire Department resources are listed in FNP-0-EIP-13, "Fire Emergencies". The estimated response time to Farley Nuclear Plant is 30 minutes. The Emergency Director is authorized to request this assistance.

2. Contractor and Private Offsite Organizations

a. Southern Nuclear/Southern Company Services

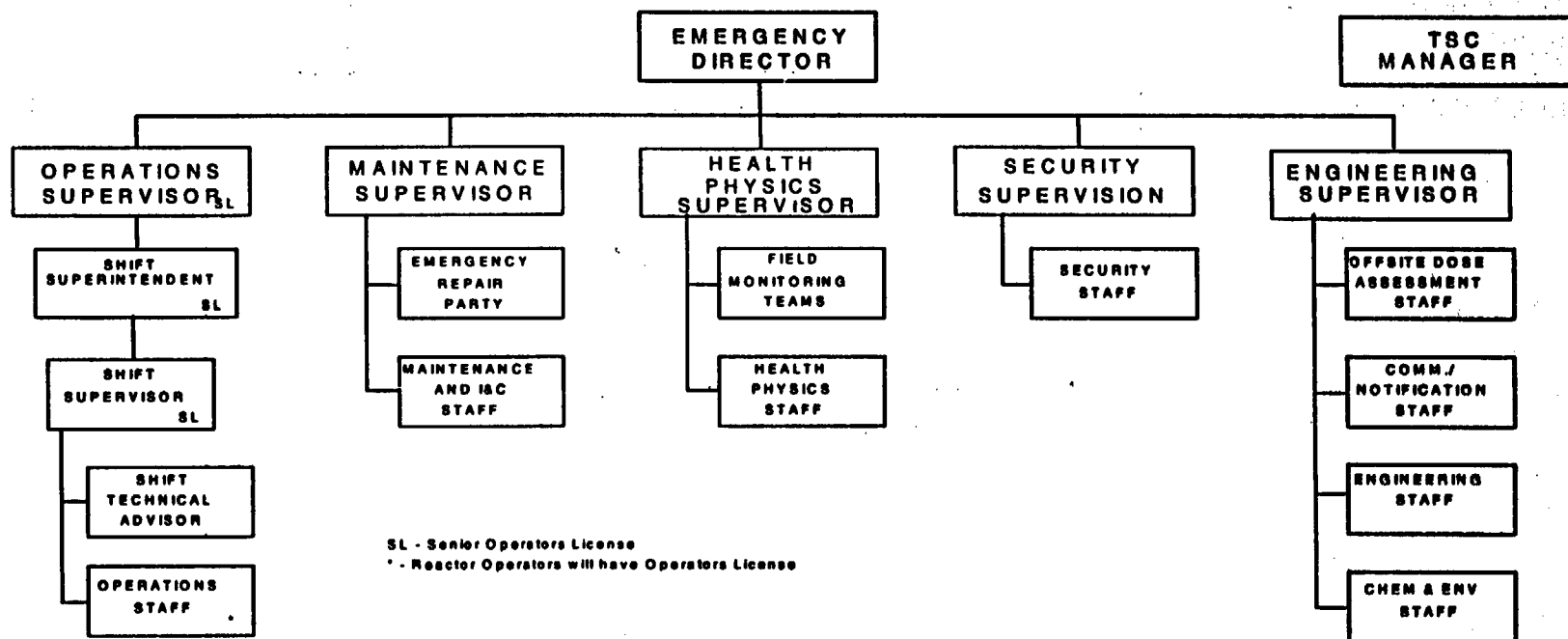
Southern Company Services, Incorporated (SCS), an affiliated service company, served as the original architect-engineer. As a result of the consolidation of SCS and SNC nuclear expertise, and in addition to being the licensee, SNC also serves as its own architect-engineer and performs functions previously performed by SCS to include design, licensing, and fuel management support during normal operation.

b. Bechtel Power Corporation

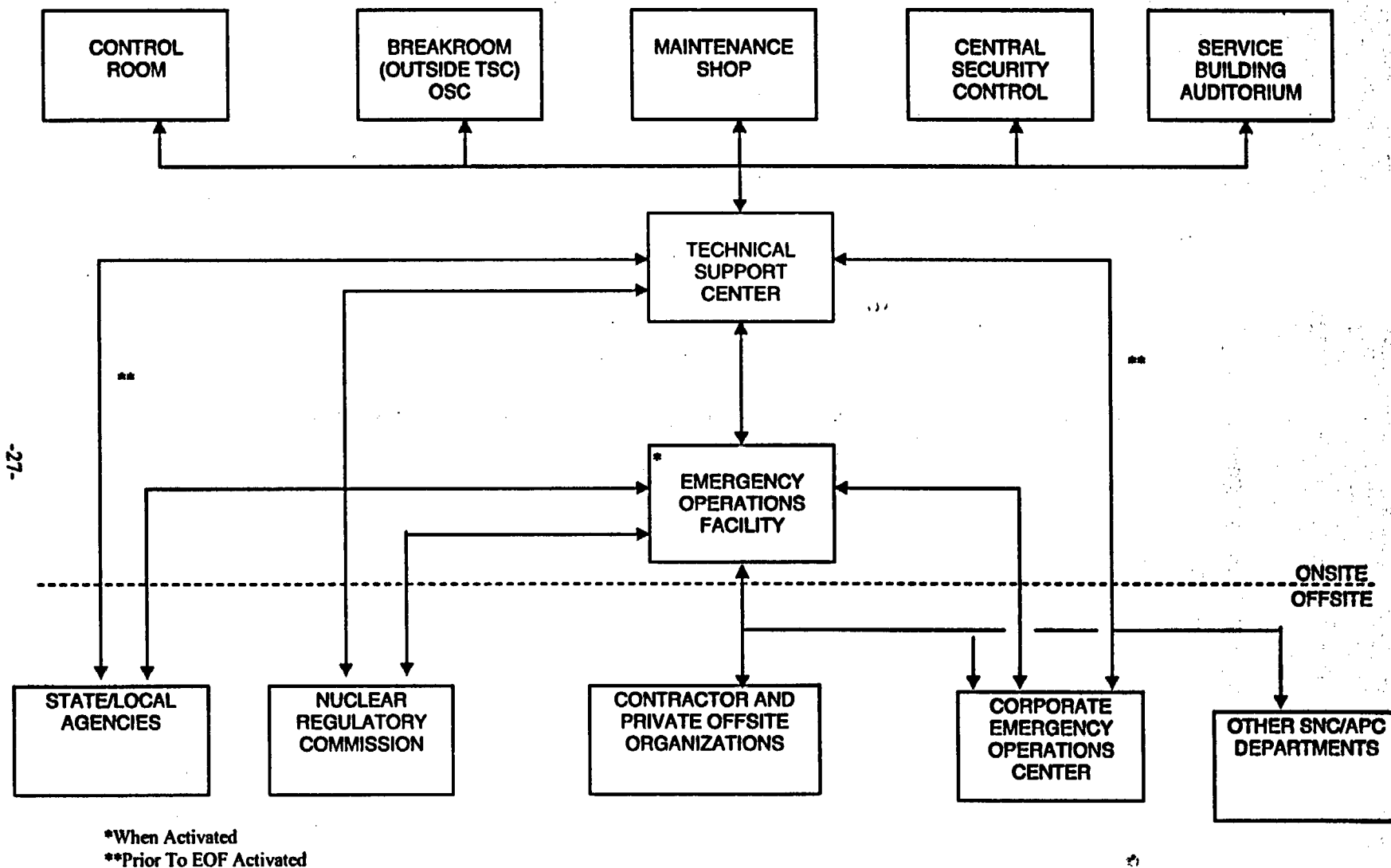
Bechtel is the architect/engineer for portions of Unit 1 and for Unit 2. Bechtel provides support in the areas of new concept design (including drawings, specifications, safety reviews, etc.); modification design; engineering support for licensing issues; and as advisor on component and system operation. The Engineering Support Manager (Emergency Support Phase) and the Technical Support Director (Recovery Phase) interface directly with Bechtel.

c. Westinghouse

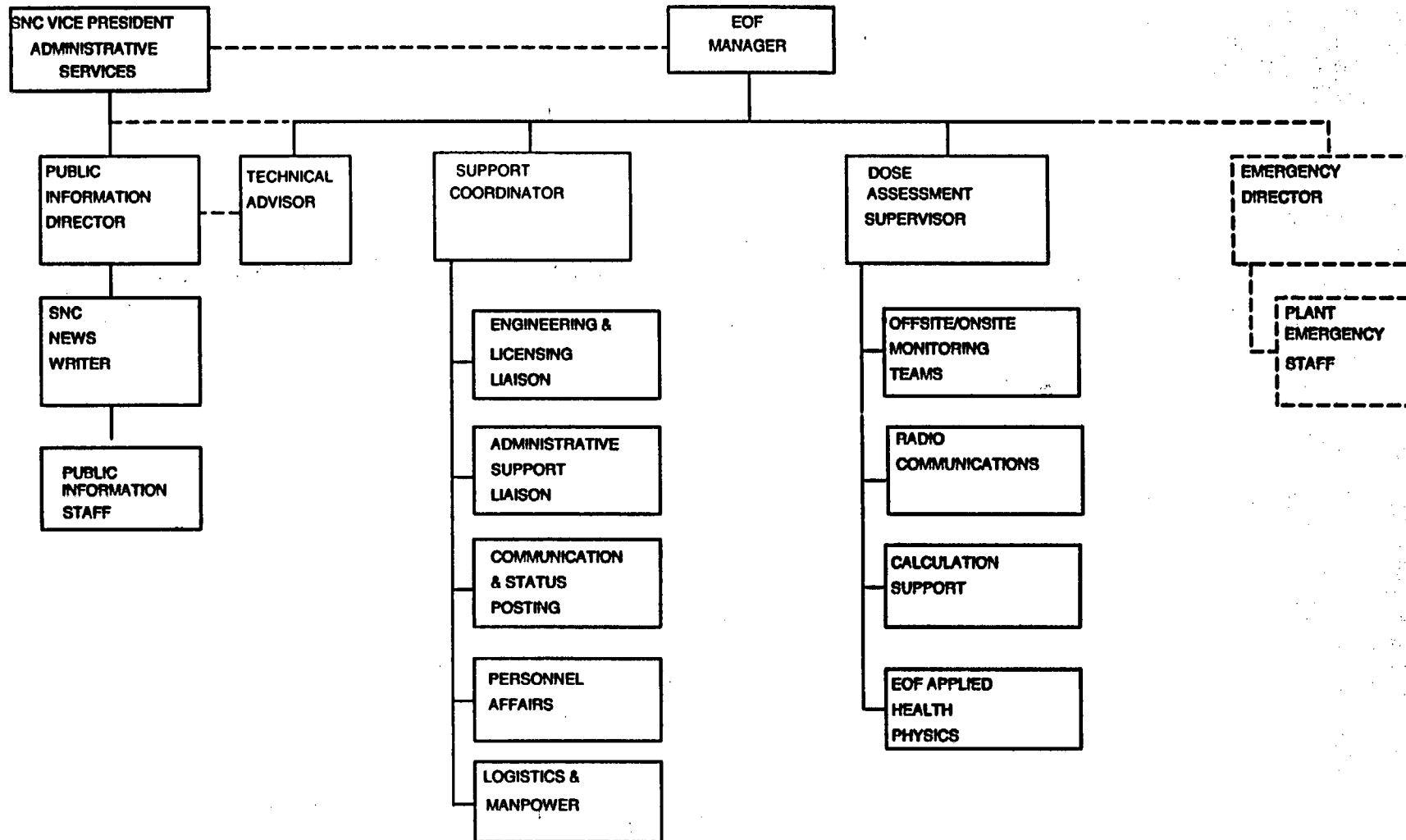
Westinghouse is the NSSS supplier for both Farley units. Their support activities associated with the NSSS include installation, testing, and corrective action assistance in their scope of supply; engineering support for licensing issues; new concepts design and modification design; advisor on components and systems; and engineering support related to operation, maintenance, and corrective action. The Engineering Support Manager (Emergency Support Phase) and the Technical Support Director (Recovery Phase) provide interface with Westinghouse either directly or through SCS.



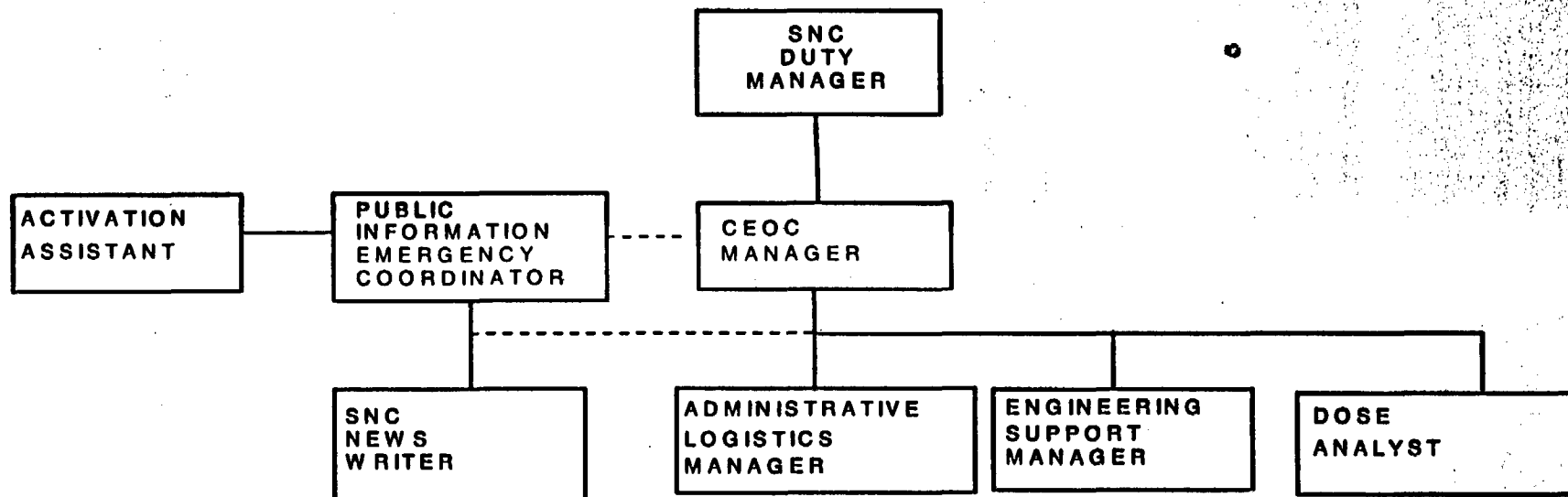
TECHNICAL SUPPORT CENTER EMERGENCY RESPONSE ORGANIZATION
Figure 2



COMMUNICATIONS INTERFACES
Figure 3

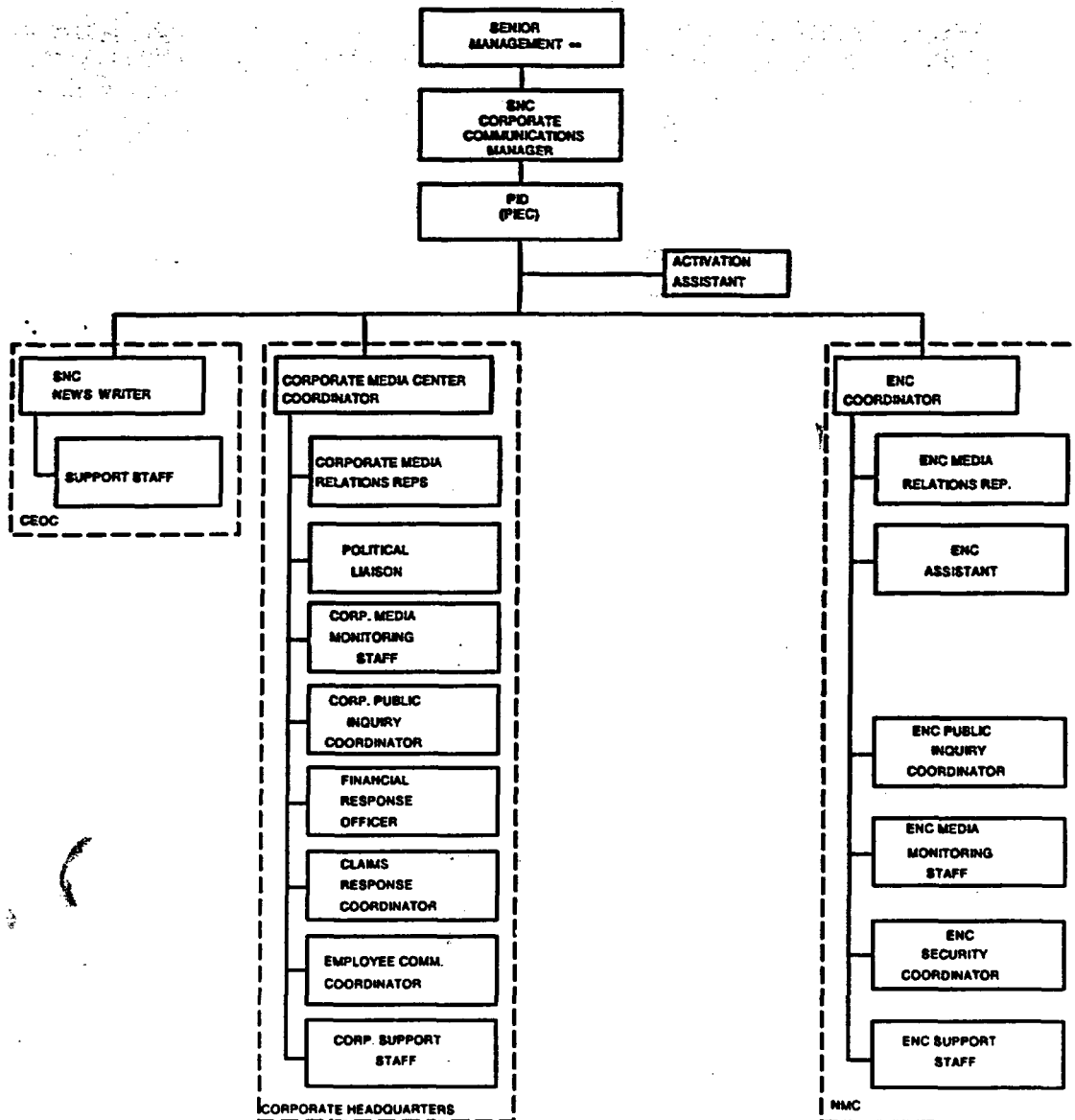


EMERGENCY OPERATIONS FACILITY EMERGENCY RESPONSE ORGANIZATION
Figure 4



CORPORATE EMERGENCY OPERATIONS CENTER EMERGENCY RESPONSE ORGANIZATION

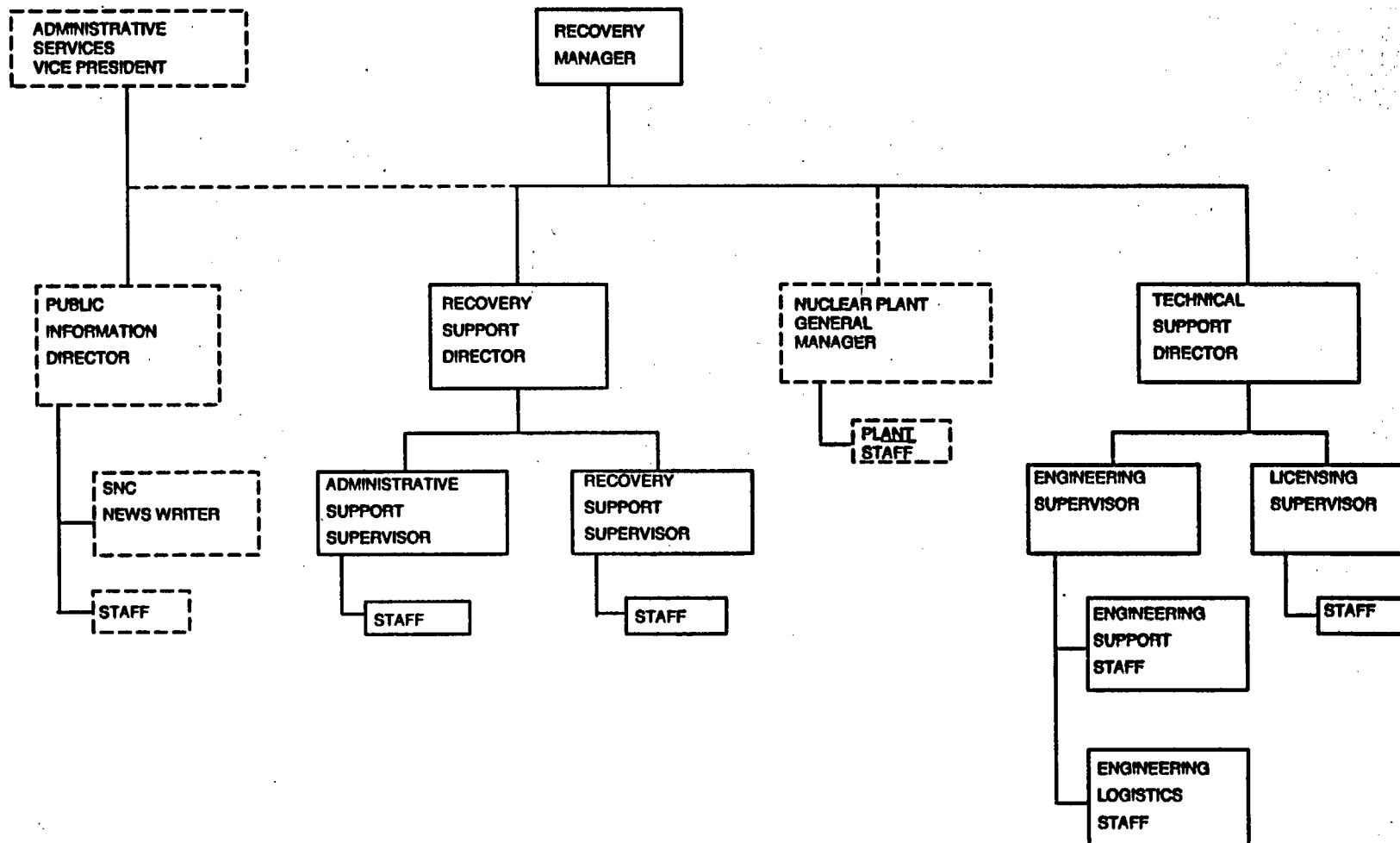
FIGURE 8



** TO INCLUDE APC EXEC. VP, APC SENIOR VP, SNC VP-ADMIN SERVICES, APC VP-PUBLIC RELATIONS

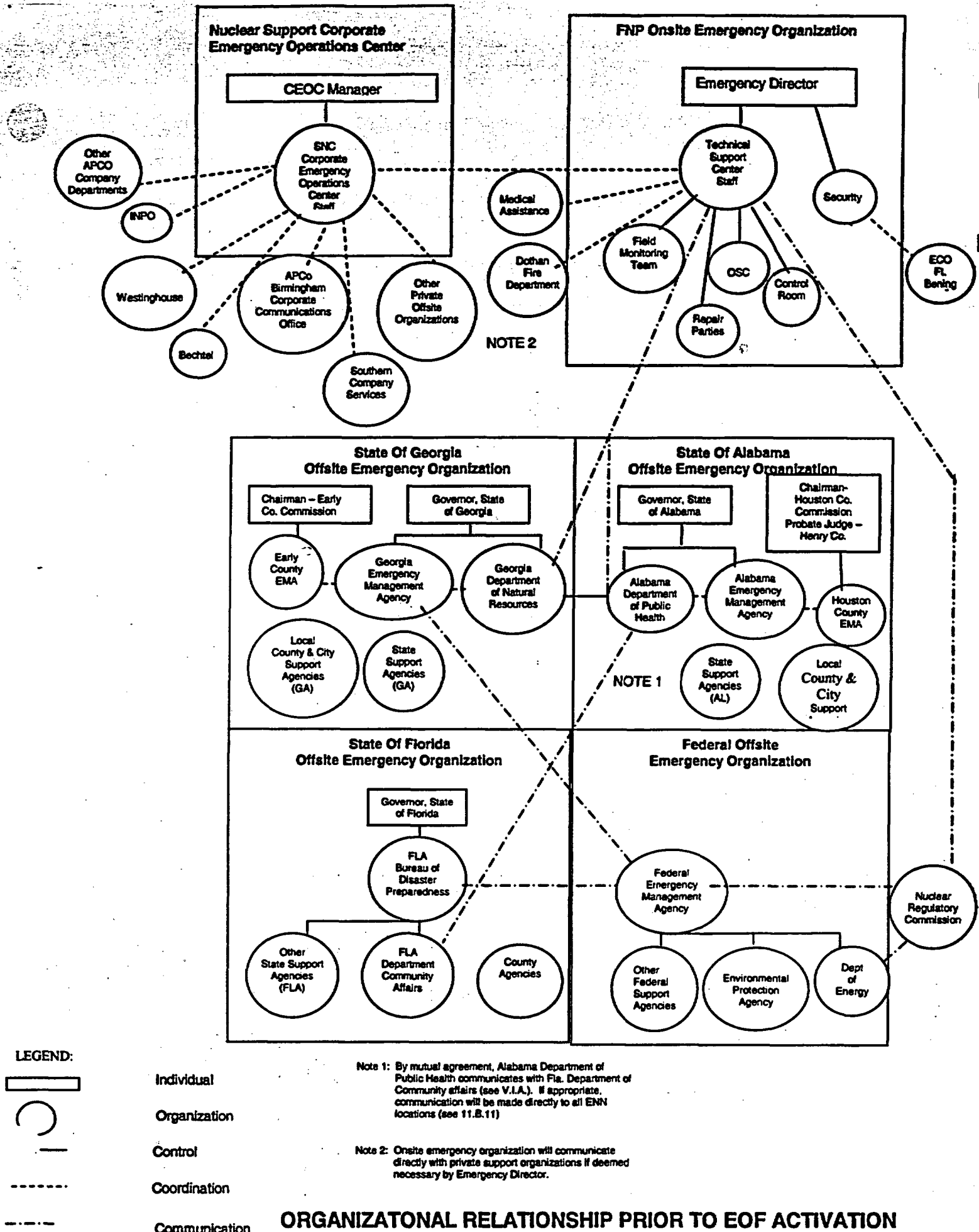
EMERGENCY COMMUNICATION ORGANIZATION

Figure 9



EMERGENCY OPERATIONS FACILITY RECOVERY ORGANIZATION

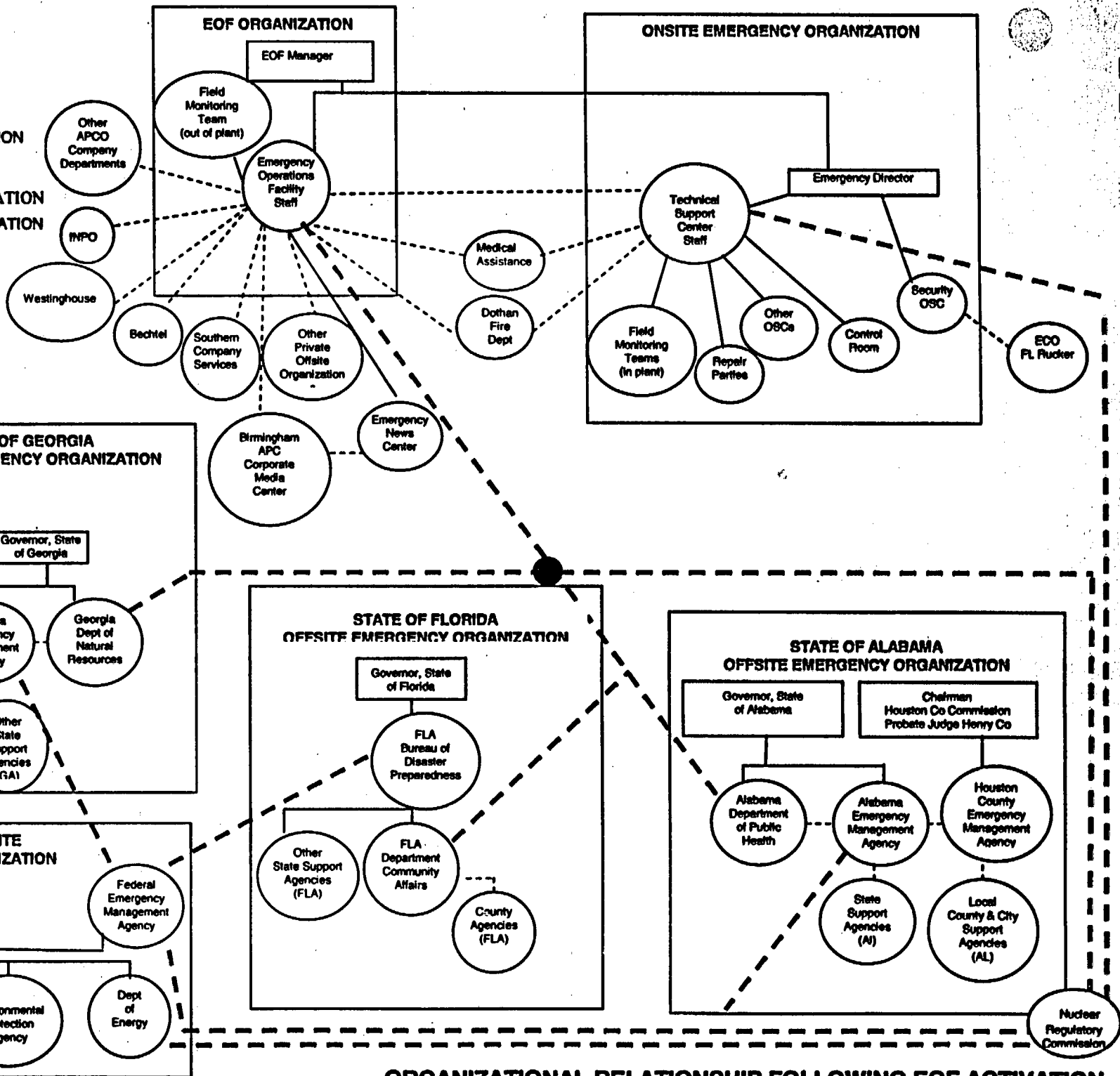
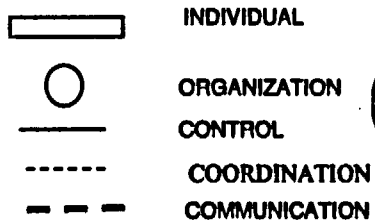
Figure 10



ORGANIZATIONAL RELATIONSHIP PRIOR TO EOF ACTIVATION

Figure 12

LEGEND



ORGANIZATIONAL RELATIONSHIP FOLLOWING EOF ACTIVATION

Figure 13

c. Document Room - Includes files, drawings, data sheets, and indexes.

d. Conference Area - Includes a conference table and chalk board/projection screen. Communications cabinets contain two-way radio, telephone, Emergency Notification Network and NRC Emergency Notification System/NRC HPN phone facilities. An intercom, sound powered headphones and a telecopier are also provided.

Figure 14 shows the above layout.

The TSC is designed to be habitable to the same extent as the control room for postulated radiological accidents. Its ventilation system includes a deep-bed charcoal filter to remove airborne contamination, and it has the capability of pressurizing the TSC area and recirculating the room air through the charcoal filter. A permanent radiation monitor is provided to continuously indicate radiation dose rates and airborne activity. A radiation alarm in the main control room make-up air supply duct automatically initiates room pressurization and recirculation. Electrical power sources are such that the HVAC, wall outlets and lighting can be powered from the diesel generators if offsite power is lost.

The TSC contains a set of piping and instrumentation drawings for each unit and technical manuals on selected major equipment. Other technical data are readily available from the document control facility in the plant Service Building which may be reached by intraplant phone from the TSC. Also available in the TSC are the Emergency Plan, Emergency Plan Implementing Procedures, Abnormal Operating Procedures, Emergency Response Procedures, Severe Accident Management Guidelines, and Unit Operating Procedures along with other general reference material. Should the emergency situation so dictate, the Emergency Director may shift the staff to other locations as designated by the Emergency Director.

2. Emergency Operations Facility (EOF)

The downstairs portion of Farley Nuclear Plant Training Center (Figure 16) will serve as the EOF where continued evaluation and coordination of all site activities related to an emergency can be carried out. The EOF will be staffed by the SNC onsite emergency response organization and will be activated as described in Section V.B. The EOF will be the location at which offsite communication will be controlled and from which the Company will provide information to federal, state, and local authorities responding to radiological emergencies subsequent to facility activation. The EOF Manager will direct all emergency support and recovery operations from this facility. The EOF will also serve as the central point for the receipt and

analysis of all SNC field monitoring data and coordination of sample media.

Located approximately 0.5 miles southwest of the main power block, the EOF will readily accommodate up to 60 people for performance of post-accident monitoring, evaluation of plant status, coordination of damage assessment, technical support, interface with state, federal, NRC, TSC, OSC and onsite/offsite communications. Separate areas have been provided for each group of officials and support organizations. See Figure 16 for EOF floor layout and room numbers identified below.

- a. Communications, Dose Assessment, Monitoring, Public Information, Conference and Command Area (Room 106) - Coordination of damage assessment and offsite communications will be conducted from this area. This area is equipped with status boards, maps, logs and communications equipment (two-way radios, telephones, ENN handset, telecopier, NRC Emergency Notification System and TSC-EOF-OSC intercom). Company, federal and state liaison officials will be located in this area. This area is also equipped with emergency dose assessment terminals for providing meteorological data and offsite dose rate estimates and projections.
- b. Nuclear Regulatory Commission Staff Area (Room 105) - Area allocated for the NRC. This area is equipped with a telephone, tables, blackboard, projection screen, and NRC Emergency Notification System.
- c. Field Monitoring Room (Room 118) - Coordination of Field Monitoring Teams will be conducted from this area. A two-way radio and telephone for communication are provided along with necessary maps, status boards, and logs.
- d. Public Information Area (Room 103) - Area where public information activities may be conducted. Public Information activities will normally be conducted at the Corporate Headquarters buildings in Birmingham, Alabama.

e. Radiation Control Area - This area which consists of the following rooms:

- 1) Chemistry Lab (Room 114)
- 2) Environmental Monitoring and Storage (Room 112)
- 3) Respirator Repair and Storage (Room 111)
- 4) Instrument Repair and Calibration Shop (Room 110)
- 5) Radiochemistry Counting Room (Room 117)
- 6) Training Instructor Offices (Room 115)
- 7) Emergency Equipment Storage and Radiation Monitoring Room (Room 118)
- 8) Hallway (Room 109)

Limited space is available on the second floor of the Training Facility to accommodate limited press briefings and visits to the site as conditions permit as determined by the EOF Manager.

The EOF has been designed for a radiation protection factor greater than 5.0. The EOF ventilation system is composed of three separate systems: 1) the upper level system which is an isolable variable air volume system with a prefilter, and a HEPA filter; 2) the lower level system (EOF) portion is an isolable constant volume system with HEPA filters; and 3) the RCA system which consists of a draw through air handling unit with HEPA filters.

Support facilities for the NRC Mobile Laboratory have been provided at the EOF parking area including three 115 volt, 20 amp power receptacles (Hubbell twist-lock) and one telephone jack connected to the plant's private automatic exchange.

3. Alternate Emergency Operations Facility

Should the EOF become untenable or the emergency situation so dictate, the EOF Manager may relocate the EOF staff to the Alabama Power Company Office in Headland, Alabama (Figure 17) located approximately 16 miles from the Plant (approximately 20 miles driving distance). This facility has adequate space available for conducting EOF functions (Figure 18). Commercial telephone, company microwave, and an Emergency Notification Network unit are provided for communication activities. In addition an emergency dose calculation terminal is available to allow dose projection activities.

4. Operations Support Center (OSC)

The Breakroom (outside the Technical Support Center)

will serve as the Operations Support Center (Figure 19), from which emergency operations support will be provided. In the event that the Operations Support Center becomes untenable, it will be relocated onsite or to offsite company facilities in Headland, Webb or Ashford, Alabama (Figure 17) at the discretion of the Emergency Director.

5. Emergency News Center

The Houston County Juvenile Court Services Building (Figure 17) will serve as a working and briefing center for local, state and national news media (Lease Agreement - Appendix B). All official information released by SNC and APC regarding the emergency will be released from the Emergency News Center once it has been activated.

6. Corporate EOC (CEOC)

The CEOC located in the Corporate Office will be staffed by the CEOC Manager and his staff and will be used to coordinate corporate emergency organization activation and corporate emergency support. Commercial telephone, company microwave and Emergency Notification Network facilities are provided along with necessary maps, status boards, logs and reference materials.

7. APC Corporate Media Center (CMC)

The APC Corporate Media Center will be staffed by the Emergency Communication Organization and serve as the coordination point for APC corporate public information support.

B. COMMUNICATIONS SYSTEMS

Several modes of communication are available, during both normal and emergency conditions, to transmit and receive information within the plant and at locations onsite and offsite.

1. Commercial Telephones

The commercial telephone lines consist of several lines connected through local exchanges to the Bell Telephone system. Access to these lines is available through selected private automatic exchange (PAX) telephones located throughout the plant, including the control room and TSC, and at the EOF, alternate EOF and News Media Center. Commercial telephone lines servicing the General Office may be accessed through off-premises extensions (OPX) of the General Office PAX which are located in selected plant locations, including the TSC, and in the EOF. Availability of OPX and PAX telephones capable of accessing commercial telephone lines is tabulated in Table 1. Commercial telephone

lines are also available at the Farley Project Corporate Emergency Operations Center, Birmingham Corporate Communication Offices and the Alternate EOF. A minimum of thirty (30) commercial telephone lines are available at the Emergency News Center.

2. Private Automatic Exchange

The plant private automatic exchange (PAX) consists of a network of telephones located strategically throughout the plant, at various stations on the site (including the Control Room, EOF, TSC and OSC) and at selected offsite locations Emergency News Center, Alternate EOF, the State of Alabama Forward Emergency Operations Center in Houston County, the State of Georgia Forward Emergency Operations Center in Early County, the AEMA EOC in Clanton, Alabama, the ARCD EOC in Montgomery, Alabama, and the GEMA EOC in Atlanta, Georgia. Selected PAX phones are capable of communication with similar private automatic exchanges at the General Office, other APC/SNC facilities, and facilities of Southern Company affiliates (e.g. Southern Company Services and Georgia Power Company).

In addition to the PAX network, selected phones operate as off-premises extensions (OPX) of the APC Corporate Headquarters private automatic exchange and operate independently of the plant PAX system. These phones may be used for calling APC/SNC facilities and offices connected to the APC Corporate Headquarters PAX system or for accessing Bell Telephone System commercial lines in Birmingham. OPX and PAX phone availability is tabulated in Table 1.

3. Microwave

APC's microwave system provides telephone circuits to all of the company's power plants and major offices including the Alabama Power Office in Headland, Alabama. All primary microwave routes are provided with standby RF equipment with automatic switchover. The microwave telephone circuits available may be used by dialing through the PAX system.

The microwave system will also provide data grade communication between the onsite computer and the backup EOF to allow dose assessment should the permanent EOF become untenable.

4. Alabama Control Center (ACC) Link

A computer link to the Alabama Control Center network is located in the Switchhouse. The link provides for communications with the ACC for purposes of load dispatch and coordination with the Southern Company Power Control Center and other APC plants.

5. Two-Way Radio

Two-way radios and base stations are available at the site as follows:

a. Plant operations frequency -

This radio frequency provides communication between the TSC, EOF, mobile units in selected company owned vehicles and personnel in the plant equipped with operations frequency radios. This frequency may be used as a backup frequency for communicating with offsite/onsite field monitoring teams.

b. Plant security frequency -

This radio frequency provides communication between the TSC, EOF, Security Offices, mobile units in security vehicles and other selected company owned vehicles and personnel equipped with security frequency walkie-talkies.

c. Plant field monitoring frequency -

This radio frequency provides communication between the TSC, EOF, mobile units in selected company owned vehicles and personnel in the plant equipped with FMT frequency radios. This frequency may be used as a backup for communicating with offsite/onsite field monitoring teams.

d. Digital radio system (multi frequency) -

This radio system provides both onsite/offsite group and private radio communication. This radio system will be the normal communication system for the field monitoring teams.

6. Public Address and Party Lines

A plant wide public address system consisting of six separate and independent communications lines (one page and five party lines) exists to provide quick communications between two or more locations, even in high noise level areas. The page channel is used to call personnel over the speakers, issue plant-wide instructions or to communicate between two or more hand-sets. The party lines are used to carry on communication after the paged party has answered. One of the party lines will be dedicated as an emergency communications channel during emergency conditions.

7. Sound Powered Telephone

Sound powered telephone lines are located between critical points in the plant and are normally used primarily for communications during maintenance and refueling.

8. Plant Emergency Alarm

The Plant Emergency Alarm is a variable tone alarm. The warble tone may be used to alert plant personnel, contractors, and visitors onsite in the event of a Site Area Emergency or General Emergency or other condition requiring all personnel to report to their emergency assembly areas. The siren tone may be used to muster the fire brigade or at the discretion of the Shift Supervisor in order to alert personnel. Blue beacon lights located in high noise areas or other locations where the alarms may not be audible are also activated to provide a visual emergency notification.

14. Management Counterpart Link (MCL)

This dedicated FTS communications line provides a dialup communications link for any NRC internal discussions between the NRC Executive Team Director or Executive Team members and the NRC Director of Site Operations or top level licensee management at the site. Phones are located in the TSC and EOF.

15. Local Area Network (LAN) Access

This dedicated FTS communications line provides the NRC site team with access to the NRC Operations Center's LAN. Connections are provided in the TSC and EOF.

16. Telecopier

Telecopiers are located at the TSC, EOF, EOC, Alabama Radiation Control Division, Alabama Emergency Management Agency, GEMA, Houston County Emergency Management Agency, and Early County Emergency Management Agency.

17. WebEOC Communication System

This system provides a direct data link via the internet between Farley Nuclear Plant and at Houston County Emergency Management Agency (EMA), Alabama Radiation Control Division in Montgomery, Georgia EMA in Atlanta, Early County EMA, and the CEOC. It may be used to rapidly transmit information on current emergency classification, radiological conditions, and meteorological conditions.

18. Emergency Response Data System (ERDS)

These dedicated FTS communications lines provide channels by which raw reactor parametric data is transmitted from the site to the NRC. The affected Unit ERDS will be activated within one hour following the declaration of an Alert emergency or above.

19. Other Communication Systems

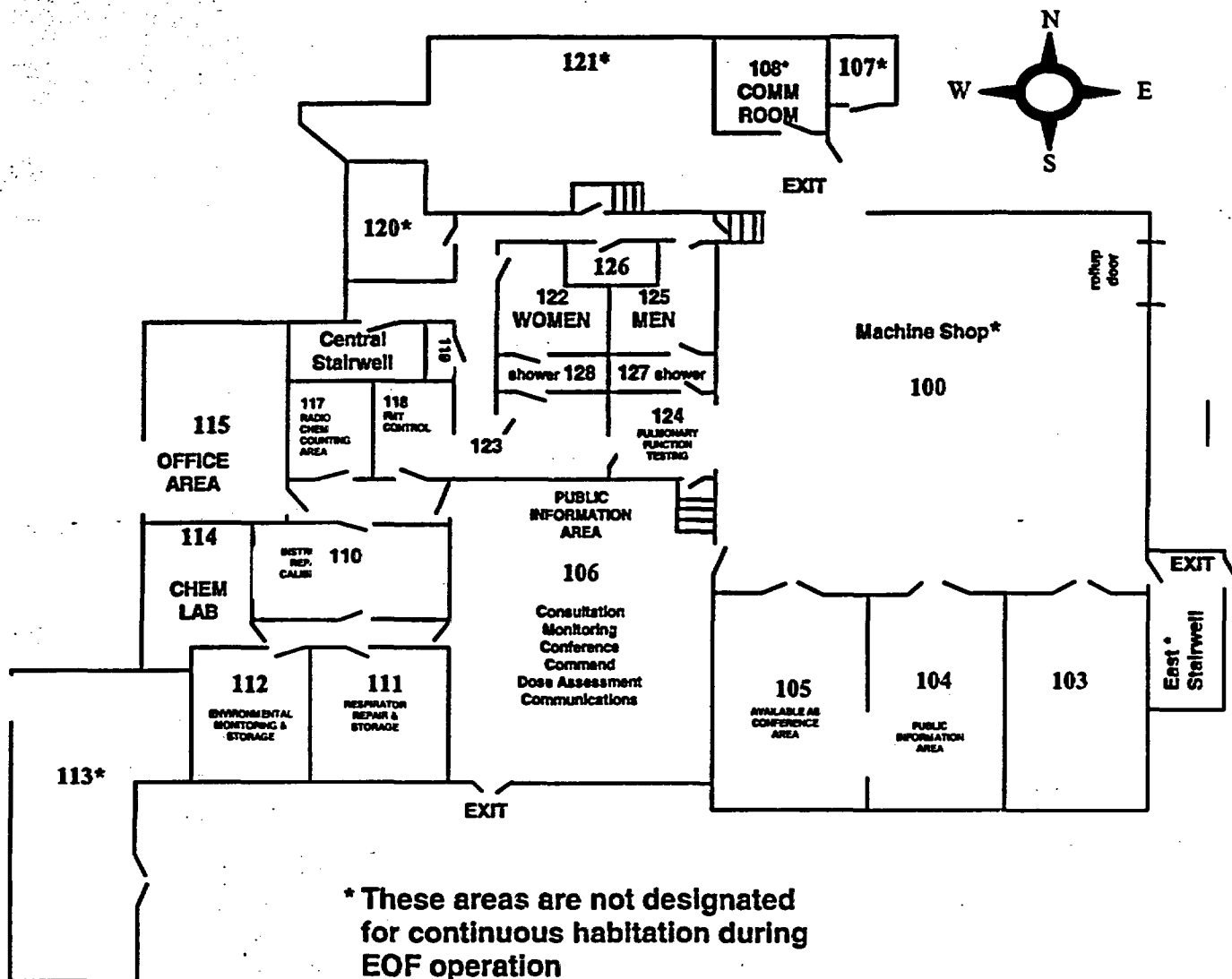
A cellular phone is provided for use by the EOF Manager while in transit to the EOF.

C. ASSESSMENT FACILITIES

In order to carry out the assessment actions described in Section IV, facilities must be available for initial as well as continuous evaluation of emergency conditions.

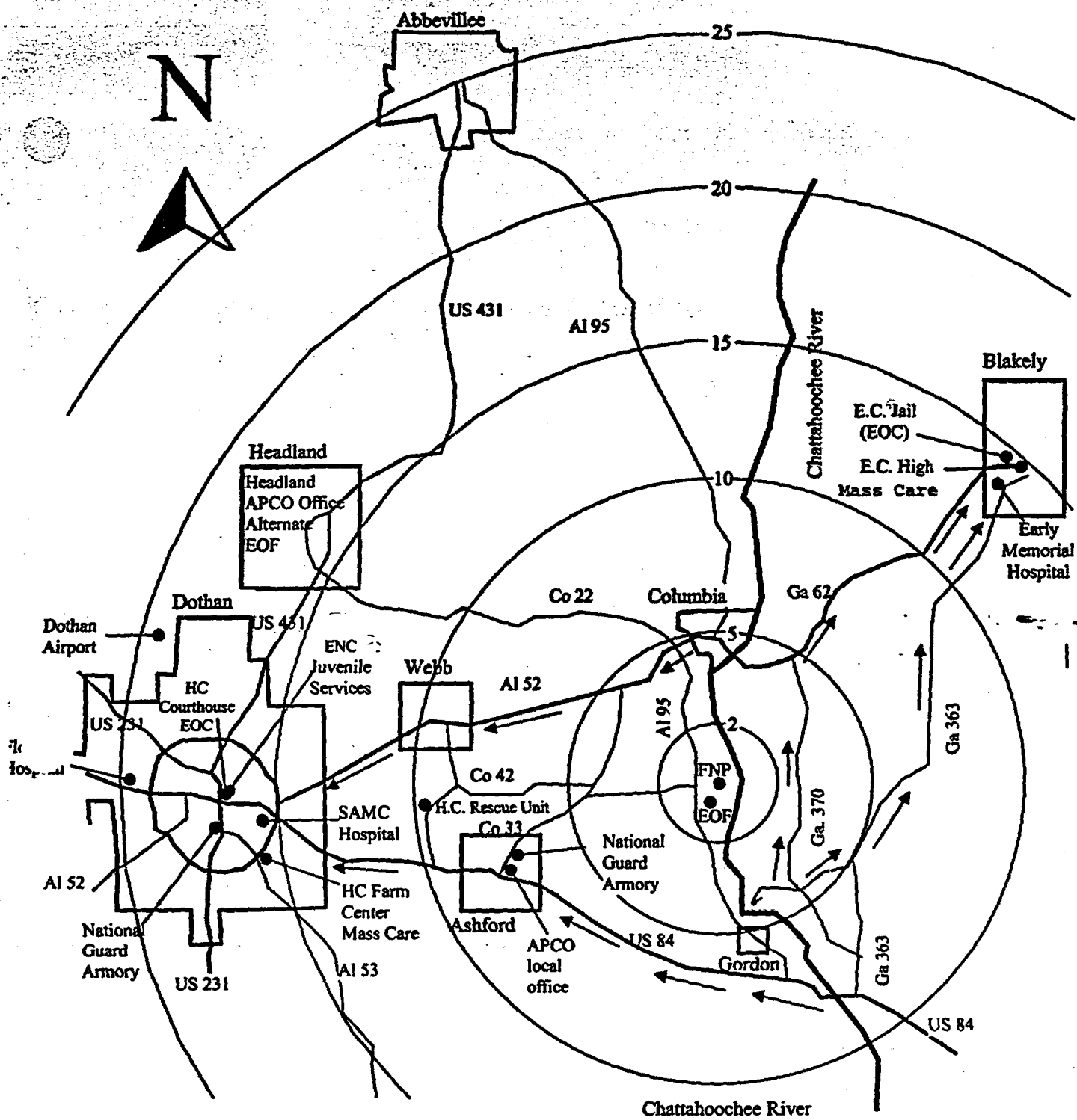
1. Onsite Systems and Equipment

a. Natural Phenomena Monitors



EMERGENCY OPERATIONS FACILITY

Figure 16



SITE AREA EMERGENCY FACILITIES, GENERAL POPULATION SUPPORT
SERVICES AND EVACUATION ROUTES

Figure 17

COMMUNICATIONS EQUIPMENT AVAILABILITY

LOCATION	Commercial Telephone Lines	PAX Telephone	General Office OPX Lines	Plant Intercom	TSC - EOF- OSC Intercom	Security Two-Way Radio	Operations Two-Way Radio	ENN - Emergency Notification Network	NRC(ENS) Emergency Notification System	NRC(HPN) HP Network	NRC - RSCL, PMCL, MCL, LAN	RMT Two-Way Radio
Technical Support Center	*	X	X *	X	X	X	X	X	X	X	X	X
Shift Foremans Office	*	X	*	X			X	X	X			
Control Room	*	X	*	X	X	X	X		X			
Emergency Operations Facility	*	X	X *	*	X	X	X	X	X	X	X	X
Alternate EOF	+ *	X	+ *	*		X	X	X				X
Emergency News Center	X * +	X	* +	*								
Assembly Areas	*	X	*	*								
Operations Support Center	*	X	*	X								

X Directly available

* Accessible through the FNP PAX system

+ Accessible through the District Office PAX system

TABLE 1

EMERGENCY FACILITY COMMUNICATIONS CAPABILITY

- 11) Loss of secondary coolant outside containment concurrent with ECCS activation.
- 12) Complete loss of forced RCS flow as indicated by RCS flow indicators on all three RCS loops.
- 13) Inadvertent loading of a fuel assembly into an improper position which causes F_q to be greater than the technical specification limit.

c. Response

In the event of a Notification of Unusual Event, the Shift Superintendent assesses the conditions and implements FNP-0-EIP-9, "Emergency Classification and Actions". He will immediately notify the Emergency Director.

The Emergency Organization will perform the following:

- 1) Notify the EOF Manager.
- 2) Inform state authorities of the nature of the Notification of Unusual Event status. This notification will be initiated within one hour of the declaration of the Notification of Unusual Event.
- 3) Notify NRC of the occurrence.
- 4) Close out with verbal notification to notified agencies followed by a written report as required by technical specifications or escalate to a more severe class.

2. Alert

a. Description

The classification of Alert applies to situations in which events are in process or have occurred which involve an, actual or potential substantial degradation of the level of safety of the plant. Any releases of radioactive material for the Alert classification are expected to be limited to small fractions of the EPA Protective Action Guideline exposure levels. The purpose of offsite alert is to assure that emergency personnel are readily available to respond if the situation becomes more serious or to perform confirmatory radiation monitoring if required and to provide offsite authorities current status information.

b. Criteria

An Alert would be declared for plant conditions that warrant precautionary activation of the Technical Support Center, Operations Support Center, and the Emergency Operations

Facility (at the discretion of the EOF Manager). Specifically, an Alert would be declared for any of the following:

- 1) Severe loss of fuel cladding as indicated by a reactor coolant activity equal to or greater than 300 uCi/gram equivalent I-131.
- 2) Steam generator tube rupture indicated by:
 - (a) ECCS actuation, AND
 - (b) High secondary coolant activity (R-15, R-19, R-23A, or R-23B reaches full scale).
- 3) Greater than 10 gpm primary to secondary leak AND a steam line break outside containment indicated by:
 - (a) Abnormally low steam pressure on one or all steam generators with one or more of the following:
 - ° Steam line high flow, OR
 - ° Steam line high differential pressure, OR
 - ° Steam flow greater than feedwater flow
 - AND
 - (b) No abnormal temperature or humidity increase in containment.
- 4) A primary coolant leak greater than 50 gpm. Indications of such a leak will include high charging flow AND
 - (a) High containment radiation (R-2, R-22, and R-12)
AND
 - (b) High containment humidity,
OR
 - (c) Pressurizer relief or safety valve discharge line temperature high AND
 - (d) Pressurizer relief tank level, pressure or temperature increasing or above normal.
- 5) High radiation levels or high airborne contamination

c. Response

In the event of an Alert the Shift Superintendent will assess the conditions and respond per FNP-0-EIP-9, "Emergency Classification and Actions". He will then immediately notify the Emergency Director.

The Emergency Organization will then perform the following:

- 1) Evacuate areas of the plant site as necessary and/or all non-essential personnel as directed by emergency implementing procedures and as conditions warrant.
- 2) Notify the EOF Manager.
- 3) Inform state authorities of the Alert status and reason for the status. This notification will be initiated within 15 minutes of the declaration of the Alert.
- 4) Notify the NRC within one hour.
- 5) Augment resources by activating the Technical Support Center and Operations Support Center. Activate or place on standby the Emergency Operations Facility to the extent required to respond to conditions precipitating the Alert.
- 6) Dispatch onsite monitoring teams and associated communications if release is occurring or imminent.
- 7) Provide periodic plant status updates to offsite authorities.
- 8) Provide periodic meteorological assessments to offsite authorities and, if any releases are occurring, dose estimates for actual releases.
- 9) Close out by verbal summary to offsite authorities followed by report as required by technical specifications.
- 10) Escalate to a more severe class, if necessary.
- 11) Activate the Emergency Response Data System for the affected Unit within 1 hour following declaration of the Alert emergency.

3. Site Area Emergency

a. Description

The classification of Site Area Emergency applies to those events which are in progress or have occurred that involve actual or likely major failures of plant functions needed for protection of the public from radiation or contamination. Any releases of radioactive material for the Site Area Emergency classification are not expected to exceed EPA Protective Action Guideline exposure levels except near the site boundary. The purpose of the declaration of a Site Area Emergency is to:

(c) Loss or potential loss of containment integrity as indicated by:

- o Containment pressure greater than 54 psig, OR
- o A rapid decrease in containment pressure, OR
- o Failure of the containment isolation system resulting in a direct path from containment to the environment.

- 3) Loss of physical control of the facility.
- 4) Other plant conditions exist, from whatever source, that make release of large amounts of radioactivity in a short time period possible, e.g. any core melt situation.

c. Response

In the event of a General Emergency the Shift Superintendent will assess the situation and respond per FNP-0-EIP-9, "Emergency Classification and Actions." The Shift Superintendent will immediately notify the Emergency Director.

The Emergency Organization will then perform the following:

- 1) Evacuate areas of the plant site as conditions warrant unless safety to personnel is a concern. Evacuate non-essential personnel from the plant site as conditions warrant unless safety to personnel is a concern.
- 2) Inform state and local authorities of General Emergency Status and reason for General Emergency. This notification will be initiated within 15 minutes of the declaration of the General Emergency. Make protective action recommendations in accordance with those stated in paragraph IV.C.2, "Offsite Protective Actions." Notify the EOF Manager and NRC.
- 3) Augment resources by activating the Technical Support Center, Operations Support Center and Emergency Operations Facility.
- 4) Dispatch onsite and offsite monitoring teams and associated communications.
- 5) Provide plant status updates to offsite authorities and periodic press briefings (perhaps joint with offsite authorities).
- 6) Provide meteorological and dose estimates to offsite authorities and periodic press briefings (perhaps joint with offsite authorities).

- 7) Provide release and dose projections based on available plant condition information and foreseeable contingencies.
- 8) Close out or recommend reduction of emergency class by verbal briefing of offsite authorities followed by written report as required by technical specifications.
- 9) Activate the Emergency Response Data System for the affected Unit within 1 hour following declaration of the General Emergency.

B. POST ACCIDENT ASSESSMENT ACTIONS

Effective coordination and direction of all elements of the emergency organization require continuing assessment through the duration of the emergency situation. In addition to continued monitoring of control room instrumentation and plant parameters, some special assessment actions are, initiated if appropriate. These assessment functions are identified below:

1. Reactor Coolant Sampling, Containment Atmosphere Sampling and Plant Vent Stack Sampling.

RCS, containment atmosphere and/or plant vent stack samples will be taken and analyzed to assess the severity of core damage and the potential radiological consequences. The detailed sampling procedures for each area are provided in FNP-0-CCP-1300, "Chemistry and Environmental Activities During a Radiological Accident". FNP-0-EIP-30 "Post Accident Core Damage Assessment" provides a method to estimate the extent of core damage utilizing various samples and plant monitor readings.

2. Surveillance of Control Room Monitors

Surveillance of radiological and meteorological monitors in the control room is primarily the responsibility of operations personnel. However, in the event that offsite assessments based on this data are necessary, an individual designated by the Emergency Director will maintain surveillance over effluent monitor readings and dose projections, periodically reporting them to those designated in FNP-0-EIP-6, "TSC Setup and Activation". If the dose assessment computer is inoperable, the designated individual will periodically log the monitor values.

3. In-Plant and Site Surveys

During emergency conditions in-plant, site and site boundary surveys will be performed as appropriate by the Field Monitoring Team as described in FNP-0-EIP-4, "Health Physics Support to the Emergency Plan". The Field Monitoring Team will be capable of sampling under field conditions and will be capable of measuring radioiodine in the presence of noble gas and background radiation to as low as 5×10^{-8} uCi/cc.

4. Population Exposure

Exposure projections shall be periodically estimated in the affected sectors utilizing projected dose and measured dose rates. The Dose Assessment Supervisor will work with the state/local agency representatives to determine the Total Effective Dose Equivalent (TEDE) exposure (resulting from external exposure and inhalation of the plume and external exposure from deposition) and thyroid Committed Dose Equivalent (CDE) exposure (resulting from the inhalation of radioiodines).

5. Environs Surveys and Monitoring

a. Short Term Assessment

Short term assessment will involve the use of the estimates and projections provided by the emergency dose calculation computer programs performed in accordance with FNP-0-EIP-9, "Emergency Classification and Actions". Onsite and offsite surveys will be performed as necessary to verify release information or as a backup assessment method should the instrumentation used for dose assessment go off-scale or become inoperable. Monitoring teams will normally be dispatched in vehicles and will be equipped with two-way radios for communication with the TSC or EOF. Teams will be equipped with liquid sampling equipment, a GM instrument, an ion chamber instrument, and/or an air sampler as deemed appropriate by the Emergency Director, Health Physics Supervisor or Dose Assessment Supervisor. An environs survey team could be in the field within one hour. Radiological survey and sampling points will be identified by sector as shown in Figure 20. The correlation of various measured parameters (contamination levels, water and air activities) to dose rates for key isotopes and gross radioactivity levels is provided by FNP-0-RCP-25.

b. Long Term Assessment

The long term aspects involving offsite assessments of contamination involving analysis of soil, vegetation, food, milk and water will be primarily handled by the states of Alabama, Georgia, and Florida as discussed in their Radiological Emergency Plans. The response of SNC would be to increase the sampling frequency of its established environmental monitoring program.

C. PROTECTIVE ACTIONS AND EMERGENCY ACTION LEVELS

The nature of protective actions to be implemented, the criteria for application, and the area involved or groups of persons for whom the protective actions would be taken are given here.

1. Onsite Protective Action

a. Evacuation

In the event of a Notification of Unusual Event or an Alert, an area of the turbine building, auxiliary building or containment may have to be evacuated. Personnel would be notified to evacuate the affected area via the public address system as directed by the Emergency Plan Implementing Procedures.

Should a Site Area Emergency, General Emergency or in some cases an Alert be declared, immediate notification of all persons onsite may be accomplished by sounding the Plant Emergency Alarm and announcing the condition over the plant public address system. Personnel onsite would report to their preassigned assembly area and preparations for evacuation of nonessential persons from the site would begin. Depending on the severity of the emergency condition, these individuals will be held in an assembly area, evacuated, or returned to work.

If a site evacuation is warranted, personnel will be advised as to which routes should be used. The normal routes are State Highway 95 North or South and County Road 42 West (Figures 17 and 19). All personnel being evacuated from the site will be monitored before being released. Personnel leaving the site would then proceed, in their own vehicles, on one of these major routes, to their residences. Transportation for persons without vehicles will be arranged.

The details for evacuation and personnel accountability of all categories of personnel listed above are given in FNP-0-EIP-9.0, "Emergency Classification and actions" and FNP-0-EIP-10, "Evacuation and Personnel Accountability".

b. Personnel Accountability

Each plant supervisor or the senior individual onsite from his group is responsible for accounting for all persons working in or visiting his group. Accountability within the Protected Area will be determined by the senior individual at the assembly area coordinating with the Primary Access Point (PAP) and then will be reported to the Emergency Director by the senior plant security force member at the PAP. Accountability within the Controlled Area will be determined by the senior individual at each assembly area coordinating with the Central Security Control (CSC) Building Staff and then will be reported to the Emergency Director by the senior individual in the CSC. Contractor personnel assigned to Plant Modification and Maintenance Support (PMMS) report to the Fabrication Shop and will assemble by individual craft. Fitness for Duty Facility personnel outside the Protected Area report to the Visitor's Center Auditorium. No public access areas

Dosimetry, respiratory protection equipment and protective clothing will be issued for use in accordance with established Radiological Control Procedures. A supply of radio protective drugs (potassium iodide) is available onsite and, if necessary, will be issued at the direction of the Emergency Director to emergency personnel remaining or arriving onsite.

The levels of permissible radioactive contamination for personnel and equipment to be released from an RCA during an emergency are as follows:

<u>Personnel</u>	<u>Equipment</u>	
<5000 dpm/100 cm ²	ND GMT/100 cm ²	(smearable)
	and	
	< .25 mr/hr	(fixed)

However, the Emergency Director may authorize higher levels based on plant conditions and recommendations from the HP Supervisor. When levels above these values are encountered, decontamination will be initiated. Facilities, supplies and waste disposal capability exists to provide for both personnel (emergency or onsite relocated) and equipment decontamination. Methods for equipment decontamination are discussed in FNP-0-RCP-862, "Area and Equipment Decontamination Guidelines" and for personnel in FNP-0-RCP-29.1, "Guidelines for Personnel Decon and Response to Personnel Contamination Events" with particular attention being given to radioiodine contamination of the skin. Extra clothing for personnel will be provided in the event personal clothing is confiscated. Information on personnel decontamination facilities is contained in Section III.

To prevent or minimize direct or subsequent ingestion exposure to radioactive materials deposited on the ground or other surfaces, access into the exclusion area will be controlled by security personnel. Additionally, if conditions warrant, the site drinking water will be sampled and analyzed for radioactivity and quarantined, if necessary. If a quarantine is placed on the water, it will not be returned to use until the activity has returned to within acceptable limits as dictated by the State of Alabama Board of Health "Regulations Governing Public Water Supplies".

Personnel which are found to be contaminated when monitored during evacuation will be returned to the plant for deconning if possible. If the plant is not accessible, the personnel will be transported to the nearest decontamination facility.

Offsite contamination controls are described in the states of Alabama, Georgia, and Florida plans.

2. Offsite Protective Action

The states of Alabama, Georgia, and Florida are responsible, in

Classification

Protective Actions to be Recommended
to State Authorities

SITE AREA

Recommendations are to be made at the discretion of the ED based on plant conditions or projected dose.

ALERT Recommendations are to be made at the discretion of the ED based on plant conditions or projected dose.

The authority for recommending initiation or relaxation of protective measures is vested solely with the Emergency Director and EOF Manager and may not be delegated to any other member of the emergency organization.

V. ACTIVATION OF EMERGENCY ORGANIZATION

A. DECLARATION OF AN EMERGENCY

The Shift Superintendent shall have the authority and responsibility to immediately and unilaterally declare an emergency and initiate emergency response. Section IV of this plan delineates criteria for declaring emergency conditions.

Upon declaration of an emergency the Shift Superintendent will immediately notify the on-call Emergency Director (ED). Until the on-call ED arrives onsite and relieves the Shift Superintendent, the Shift Superintendent shall complete the duties of the ED prior to the on-call ED taking full responsibility for implementation of the Emergency Plan. Duties of the Shift Superintendent as an Emergency Director are addressed in FNP-0-EIP-3, "Duties of the Emergency Director". Additional notification responsibilities are discussed in Section VI and described in FNP-0-EIP-9, "Emergency Classification and Actions."

The ED, Shift Clerk, or designee will notify the EOF Manager of the emergency condition. The EOF Manager will decide on the appropriate level of onsite and offsite activation utilizing the criteria shown in Table 4.

B. ONSITE ORGANIZATION ACTIVATION

The minimum quantity of personnel available on shift and the quantity of additional personnel available within 75 minutes following declaration of the emergency to staff the emergency organization are shown in Table 3.

Each shift shall have a Shift Technical Advisor (STA). The STA will have no duties or responsibilities for manipulation of controls or command of operations during an emergency.

The normal shift crew will consist of at least those positions listed as "on shift" in Table 3. There will be a licensed operator in each unit's control room at all times when fuel is in the core of the respective unit. There will be a Senior Reactor Operator (SRO) in the control room (shared) at all times when fuel is in either core. Shift staffing for core alterations will also include either a SRO limited to fuel handling or a SRO not assigned any duties concurrent with core alterations.

Upon receiving notification of an emergency, the Emergency Director will proceed to the site. The Shift Clerk will implement the plant call list to notify those individuals of the Emergency Organization needed to meet initial activation requirements. The EOF Manager will be notified in accordance with FNP-0-EIP-9, "Emergency Classification and Actions."

Criteria for the activation of the Technical Support Center, Operations Support Center and Emergency Operations Facility are shown in Table 4.

1. Technical Support Center Activation

The onsite emergency response organization which will be directed from the TSC is described in Section II. The TSC will be staffed and ready to receive emergency response functions by the following on-call individuals within 75 minutes following declaration of an emergency requiring TSC activation:

Emergency Director

Operations Supervisor

Health Physics Supervisor

Engineering Supervisor

Maintenance Supervisor

Within eight hours of the declaration of the emergency one full complement of personnel as designated above will be available to relieve the TSC personnel.

There shall be sufficient personnel available within 16 hours of the declaration of the emergency to ensure that the TSC can be staffed on a 24-hour-a-day basis for at least one week.

2. Operations Support Center (OSC) Activation

The Operations Support Center will be staffed and ready to provide support to the emergency response effort as directed by the Emergency Director within 75 minutes following declaration of an emergency requiring OSC activation.

Other members of the Emergency Organization arriving at the plant will report to their assigned assembly areas.

Within 8 hours after declaration of an emergency, sufficient personnel shall be available in the OSC to ensure that shift personnel can be relieved by qualified individuals.

Within 16 hours after declaration of an emergency, sufficient personnel shall have been notified and placed on-call to ensure that the OSC can be staffed on a 24-hour-a-day basis for at least one week.

3. Emergency Operations Facility (EOF) Activation

The onsite emergency response organization which will be directed from the EOF is described in Section II. The EOF will be staffed and ready to receive emergency response functions by the following on-call individuals within 75 minutes following declaration of an emergency requiring EOF activation:

EOF Manager

Support Coordinator

Dose Assessment Supervisor

Dose Analyst

ENN Communicator

The staff is augmented as necessary with plant personnel. Within 8 hours after declaration of an emergency, sufficient personnel shall be available in the EOF to ensure that shift personnel can be believed by qualified individuals. Within 16 hours after declaration of an emergency, sufficient personnel shall have been notified and placed on-call to ensure that the EOF can be staffed on a 24-hour-a-day basis for at least one week.

C. OFFSITE CORPORATE ORGANIZATION ACTIVATION

The CEOC will normally be activated concurrently with the plant organization using the automated recall system. If a decision is made to activate manually then the offsite organization and facilities are activated as follows:

1. The EOF Manager will contact the SNC Duty Manager. The Duty Manager will contact the CEOC Manager.
2. The CEOC Manager will notify the Administrative Logistics Manager (ALM) and the Public Information Emergency Coordinator (PIEC). The PIEC has responsibility for notifying the remaining on-call Public Information Emergency Communication Organization staff, and the SNC News Writer.
3. The SNC News Writer proceeds to the CEOC.
4. The ALM notifies the on-call CEOC staff members who proceed to the CEOC.

At the CEOC, the CEOC Manager and his staff (Section II) monitor plant status, maintain contact with the EOF Manager, provide initial logistics support to the plant, provide periodic briefings to senior company management and the APC Corporate Headquarters Office, and make

notifications as necessary to augment the EOF staff, activate EOF relief crews, and notify offsite vendor support organizations.

In Birmingham the Vice President-Administrative Services, the PID, and his staff monitor media reports, receive periodic and timely briefings from the CEOC, make initial and follow-up notifications to public information personnel in selected local, state and federal agencies and make notifications as necessary to augment the ENC Emergency Communication staff and the Birmingham Emergency Communication staff.

Anytime the EOF is activated, or if actual or potential media interest justifies activation of the Emergency News Center, at the direction of the Public Information Director or designee, Houston County Emergency Management Agency authorities will be contacted to implement agreements for use of the facility and telecommunications personnel will be contacted to arrange for activation of the commercial phone facilities. Arrangements for placing other necessary equipment at the facility will also be implemented.

The communications sequence associated with offsite Emergency Communication Organization activation is illustrated in Figure 23.

D. OFFSITE LOCAL, STATE AND FEDERAL AGENCIES

Notification of offsite governmental agencies is discussed in Section VI. Activation of the state agencies is discussed in their respective plans.

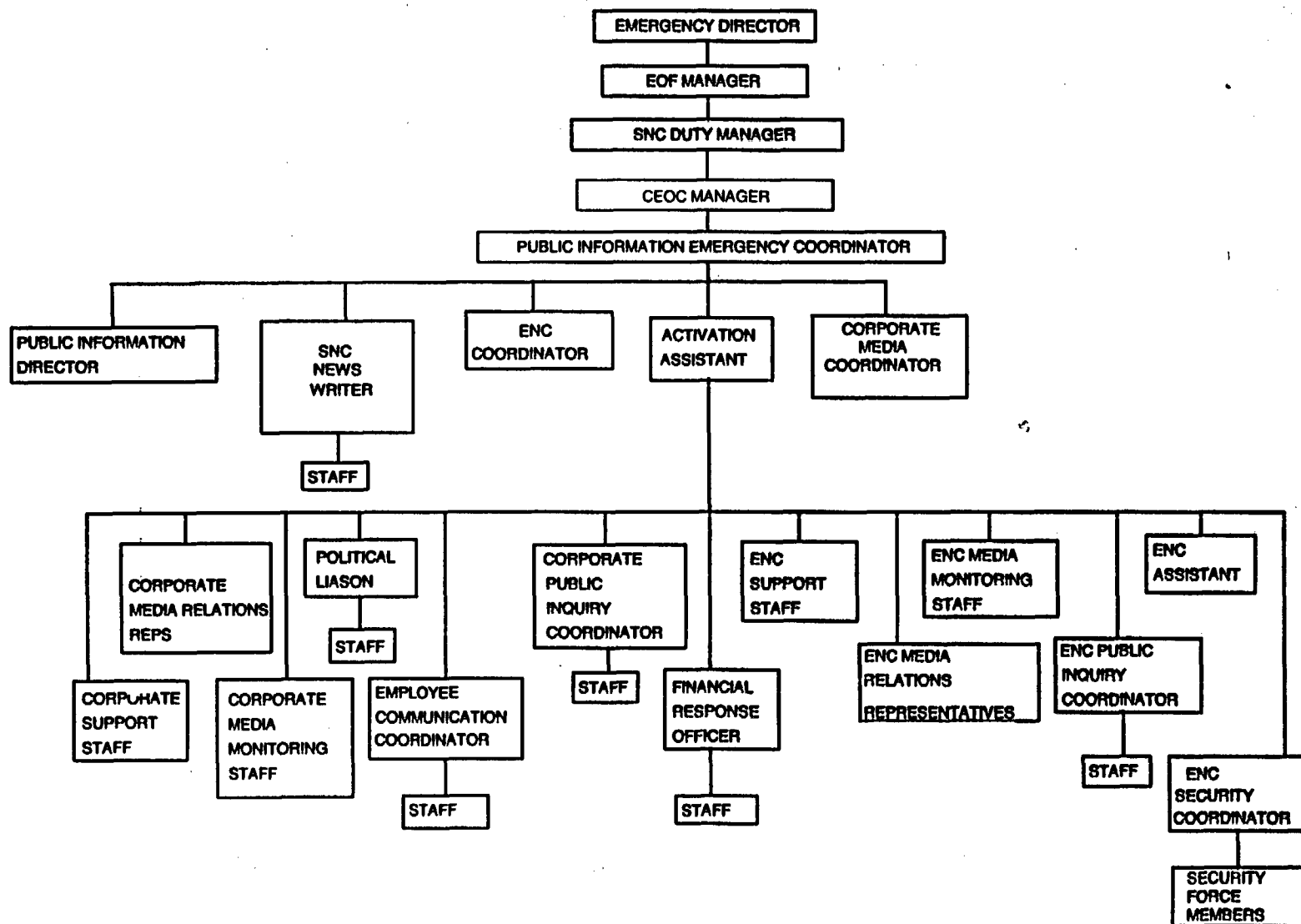
TABLE 4

EMERGENCY FACILITY ACTIVATION

	<u>Unusual Event</u>	<u>Alert</u>	<u>Site Area Emergency</u>	<u>General Emergency</u>
Technical Support Center	*	Activate#	Activate#	Activate
Operations Support Center	*	Activate#	Activate#	Activate
Emergency Operations Facility	**	***	Activate#	Activate
Corporate Emergency Operations Center	**	***	Activate#	Activate
APC Corporate Headquarters	**	***	Activate#	Activate
Emergency News Center##	N/A	****	****	Activate

NOTE:

- * No action, standby or activation at the discretion of the Emergency Director
- ** No action, standby or activation at the discretion of the EOF Manager
- *** Standby or activation at the discretion of the EOF Manager
- **** Activation dependent on level of media interest or EOF activation
- # Activation will be to the extent deemed necessary by the Emergency Director and EOF Manager
- ## Automatically activated upon EOF activation



EMERGENCY COMMUNICATION ORGANIZATION ACTIVATION SEQUENCE

Figure 23

Effort will be concentrated upon providing information to the public by written material that is likely to be available in the residence and in locations frequented by transients. The information will also provide instructions as to what local media (radio and television stations) will be providing additional information in the event of an emergency.

b. Local News Media

Alabama Power Company and Southern Nuclear Operating Company will conduct coordinated programs annually to acquaint the local news media with the emergency plans, information on radiation and contamination, points of contact for release of public information in an emergency, and facilities which may be used by the media during an emergency.

3. News Release Coordination and Rumor Control

During emergency conditions, the Birmingham APC Corporate Media Center will monitor national and state media reports for accuracy. The ENC public relations staff will monitor local media reports. All news releases will be coordinated with state and NRC officials. The CEOC Manager, EOF Manager or Vice President and Public Information Emergency Coordinator, Public Information Director or Corporate Media Center Coordinator or Vice President-Administrative Services must approve all news releases. One of the individuals designated to fill the SNC Duty Manager position or a EOF Manager who is off duty will be available at the Emergency News Center to serve as the APC/SNC spokesperson.

The APC Corporate Media Center will maintain a telephone actuality system to aid the media in dealing with rumors.

C. NRC OFFICE OF INSPECTION AND ENFORCEMENT

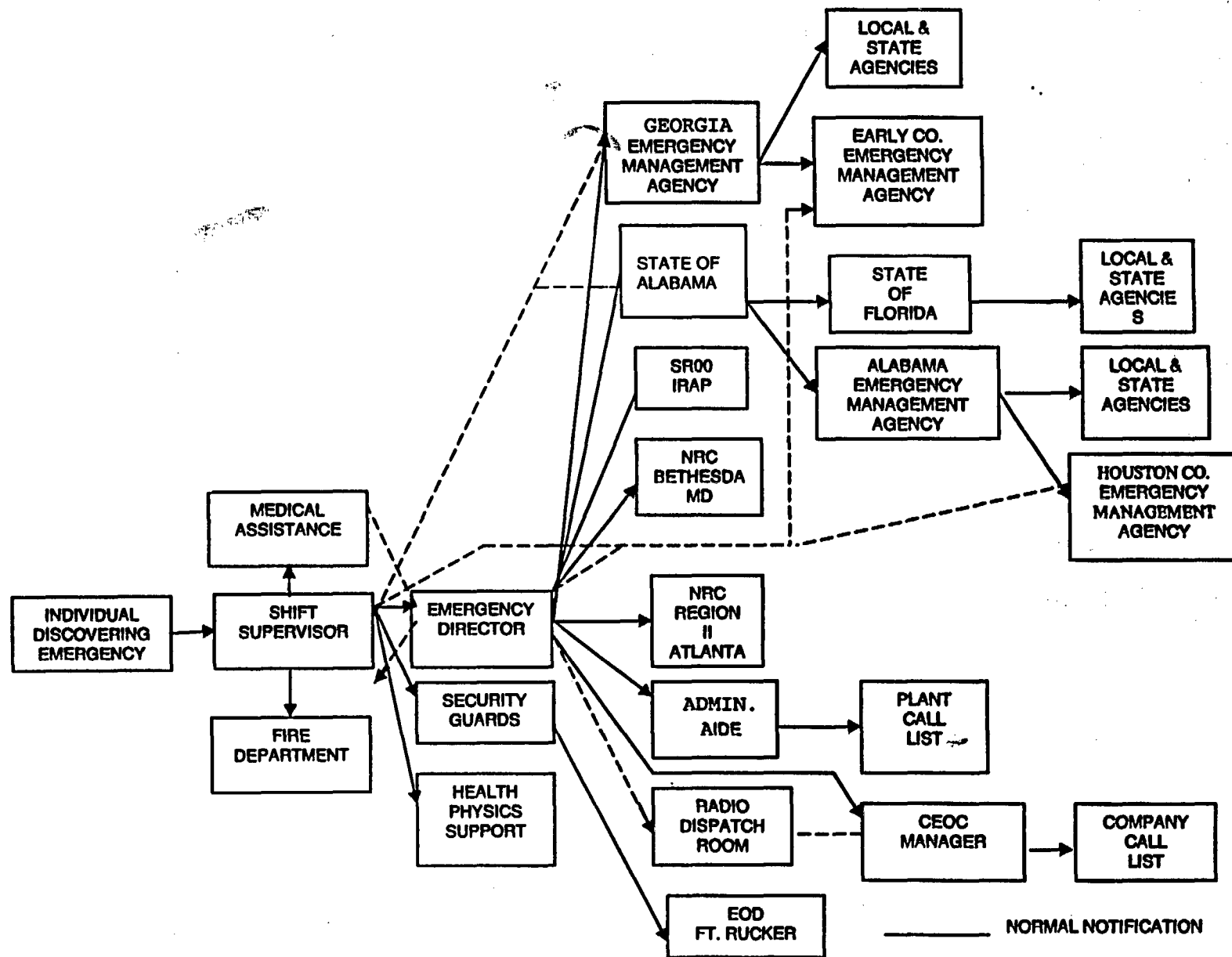
The Emergency Director or his designee will notify the Nuclear Regulatory Commission of any emergency condition utilizing the Emergency Notification System.

D. SAVANNAH RIVER OPERATIONS OFFICE

If conditions warrant, immediate assistance will be requested by the Emergency Director or Recovery Support Director from the DOE Savannah River Operations Office if their assistance is required to protect the health and safety of the general public.

E. MEDICAL

As the situation merits, the Shift Supervisor, Emergency Director and/or CEOC Manager/EOF Manager will notify one or more of the following:



NOTIFICATION ORDER
Figure 24

VII. RECOVERY

A. METHODOLOGY

Due to the unforeseeable conditions that would exist in an emergency condition, specific recovery criteria and procedures will be developed when required, considering maximum protection for plant personnel and the general public consistent with reasonable efforts to restore the affected Unit and continuing operation of the unaffected unit.

The decision to relax protective measures will be based upon a comprehensive review of plant system parameters. These shall include but not be limited to the following:

1. Stability of the reactor shutdown condition i.e., successful movement toward a cold shutdown condition.
2. Integrity of the reactor containment building.
3. Operability of radioactive waste systems and decontamination facilities.
4. The availability and operability of a heat sink.
5. The integrity of power supplies and electrical equipment.
6. The operability and integrity of instrumentation including radiation monitoring equipment. In the latter instance this shall include portable equipment assigned to the emergency.
7. Availability of trained personnel and support services.

The EOF Manager will analyze the input from his advisors in the areas listed above to determine if plant restoration efforts can begin. The following criteria shall be considered appropriate for the consideration of relaxation of protective measures:

1. Plant parameters of operation no longer indicate a potential or actual emergency exists.
2. The release of radioactivity from the plant is controllable and no longer exceeds permissible levels and no danger to the public from this source is credible.
3. The plant is capable of sustaining itself in a long term shutdown condition.
4. Plant entry and clean-up is possible without workers receiving in excess of their permissible exposures.

B. ORGANIZATION

The recovery organization which will conduct the activities of returning the plant toward its pre-emergency condition to the extent reasonable is described in Section II.B.3.

C. NOTIFICATION

The EOF Manager shall notify the Nuclear Plant General Manager and company management that a decision has been reached to initiate a recovery operation. He shall then notify offsite agencies' representatives ensuring the NRC, and state and local authorities are provided with the same information. He shall also inform these agencies if any change in the structure of the recovery organization is to occur.

Continuing training will be provided to all personnel as described below:

Specific training that will be conducted is listed below:

- a. Emergency Director Training (annually)
members of the plant staff who may serve as Emergency Director will receive training in:

- 1) Supervision of emergency teams,
- 2) Emergency assessment including interpretation of data and estimation of radiation exposure,
- 3) Coordination and communication with offsite groups.

- b. Field Monitoring Team Training
(annually)

This training will be given to plant and vendor personnel that may be required to perform surveys in-plant, on the environment, or at SAMC. It will include instruction in the selection and use of survey instruments and air sampling equipment and in re-entry criteria.

- c. First Aid Training (triennially)

Plant personnel will be considered first aid qualified upon successful completion of the Company's - First Aid and CPR Course.

- d. Fire Control (per Technical Specification requirements)

A training program for the plant employees that serve on fire fighting teams is conducted under the direction of the Plant Training and EP Manager. This course covers methods and equipment for fighting all types of fires that could occur on the site. Appropriate emphasis is placed on the radiological aspects of fire fighting. Drills and critiques are conducted periodically to train Fire Brigade personnel and to maintain their efficiency.

- e. Emergency Repair Party Training (annually)

Maintenance and I and C personnel who may be assigned to the Emergency Repair Party receive training in Radiation Control Procedures as part of their normal plant training. Personnel selected for Emergency Repair Party work will possess the required journeyman skills for the particular activity.

- f. Security Personnel (annually)

Security personnel will receive training on FNP-0-EIP-7, "Security Support to the Emergency Plan", including personnel evacuation and accountability, access control, vehicle escort and bomb search activities. Personnel will also receive training on Contingency Implementing Procedure 13 covering security activities during fire, explosion or other catastrophe.

g. Communications Personnel (annually)

Personnel responsible for the transmission of emergency information and instructions will receive training on emergency communication procedures and communication equipment operation.

2. Training of the Corporate Emergency Organization

a. EOF Managers/SNC Duty Managers

Members of the Plant Staff and Nuclear Support staff who may serve as EOF Manager or SNC Duty Manager will receive annual training in:

- 1) Company, local, state, NRC and vendor support agency emergency plans.
- 2) Farley Nuclear Plant Emergency Implementing Procedures including emergency assessment.
- 3) Southern Nuclear Operating Company Corporate Emergency Implementing Procedures.
- 4) Interpretation of radiological data and estimates and use of protective action guides.
- 5) Coordination and communication with onsite and offsite groups.

The Farley Project - Emergency Planning Coordinator is responsible for coordinating this training.

b. SNC Corporate Personnel

All designated corporate SNC personnel will receive on an annual basis a thorough orientation on the FNP Emergency Plan with particular emphasis on the SNC Corporate Emergency Organization, its activation and emergency role. Changes in the Emergency Plan applicable to SNC corporate personnel will be reviewed at scheduled staff meetings. Personnel with specific pre-designated emergency duties will receive training on those duties and the emergency implementing procedures, facilities, equipment and supplies

involved. Personnel who may be dispatched to the site under emergency conditions will receive Radiation Worker Training. The Farley Project - Emergency Planning Coordinator is responsible for coordinating this training.

c. Emergency Communication Staff

All personnel designated to perform emergency communication functions will receive annual training on the emergency communication organization, its activation, emergency facilities, their responsibilities within the organization and Corporate Emergency Implementing Procedures applicable to the emergency communication staff. The SNC Corporate Communications Manager is responsible for coordinating training.

3. Training of Local Services Groups

Offsite groups, such as fire departments, police and sheriff's departments, and ambulance services, that may participate in onsite activity will be encouraged to attend a training course to ensure that they are familiar with the plant layout and their actions in the event of radiological and non-radiological incidents. The Plant Training and Emergency Preparedness Manager is responsible for coordinating this training.

4. Training of SNC Emergency Planners

SNC shall provide training for those individuals responsible for radiological emergency planning.

C. INSPECTION, CALIBRATION AND TESTING OF EMERGENCY EQUIPMENT AND SUPPLIES

To insure the operational readiness of emergency supplies and equipment the following will be performed:

1. Periodic calibration using manufacturers' recommendations as guidelines on all portable emergency instrumentation designated for emergency use. This includes both onsite equipment and offsite equipment at SAMC supplied by APC/SNC.
2. Inspection quarterly of all onsite and SAMC emergency equipment and supplies designated for emergency use and supplied by APC/SNC. The purpose of the inspection is to ensure that the inventory is correct, that the supplies are functional and that instrument calibration is current.
3. An adequate reserve of emergency equipment will be maintained to allow for equipment taken out of service for repair, calibration, or replacement.

4. Communications Checks

- a. Communications checks will be performed monthly with all locations which are part of the Emergency Notification Network.
- b. The Emergency Notification System shall be tested at least monthly.
- c. The telephone numbers of organizations listed in FNP-0-EIP-8.1 will be updated quarterly and verified annually.
- d. The EOF/TSC/OSC intercom system will be tested at least annually.
- e. Radio communication equipment for Field Monitoring Team communications will be tested at least annually.
- f. The public Alert and Notification System will undergo a full activation test at least annually.

D. REVIEW AND UPDATING OF THE PLAN AND PROCEDURES

Review and updating of the Emergency Plan and EIP's will be performed at least annually. This review and updating will be based on information received from drills, exercises, and training sessions.

The Letters of Agreement with all offsite agencies and support services will be reviewed at least every three years and updated as necessary to maintain current the provisions of the agreements.

Any changes to the Emergency Plan and EIP's will be prepared, reviewed and approved and distributed according to established administrative procedures.

Employees will be informed of changes to applicable EIP's periodically and during annual continuing training.

Independent audits of the emergency preparedness program will be conducted at least once every twelve (12) months by the Quality Assurance Group. The audit will include an evaluation for adequacy of interfaces with state and local governments and of emergency drills, exercises, capabilities, procedures, training, records and facility and equipment preparedness.

Deficiencies discovered as a result of the audits and corrective actions implemented will be reported to the Vice President-Project (Farley) Records of such audits and corrective actions will be maintained for five (5) years.

APPENDIX A

EMERGENCY SUPPLIES AND EQUIPMENT

I. TECHNICAL SUPPORT CENTER

Emergency Plan

Emergency Plan Implementing Procedures

Drawings of Facility and Plant Site

Records Material

First Aid Kit

II. CONTROL ROOM

Emergency Plan

Emergency Plan Implementing Procedures

Records Material

Tools and Other Hardware

Stretcher

First Aid Kit

Respiratory Protection Equipment

Survey Instruments

Self Contained Breathing Apparatus

III. OPERATIONS SUPPORT CENTER

Survey Instruments

Dosimetry Devices

Respiratory Protection Equipment

Protective Clothing

IV. CENTRAL SECURITY CONTROL (CSC) BUILDING

Ambulance Kit

Respiratory Protection Equipment

V. CORPORATE HEADQUARTERS

Emergency Plan

Emergency Plan Implementing Procedures

Drawings of Facilities and Plant Site

Records Material

VI. AUXILIARY BUILDING

Protective Clothing

Decontamination Supplies

Signs and Labels

Respiratory Protection Equipment

First Aid Kit and Supplies

Stretchers

Fire Rescue Suit

Fire Brigade Equipment

VII. EMERGENCY OPERATIONS FACILITY

Emergency Plan

Emergency Plan Implementing Procedures

Drawings of Facilities and Plant Site

Records Material

Tools and Other Hardware

Stretcher

Field Monitoring Team Kit

First Aid Kit

Respiratory Protection Equipment

Protective Clothing

Survey Instruments

Dosimetry Devices

I. EMERGENCY PLAN PROCEDURES

A. Emergency Plan Implementing Procedures (EIP's) Listing

FNP-0-EIP-0	Emergency Organization
FNP-0-EIP-1	Duties of An Individual Who Discovers an Emergency Condition
FNP-0-EIP-2	Handling of Incoming Calls During Emergencies or Emergency Exercises
FNP-0-EIP-3	Duties of the Emergency Director
FNP-0-EIP-4	Health Physics Support to the Emergency Plan
FNP-0-EIP-5	Maintenance Support to the Emergency Plan
FNP-0-EIP-6	TSC Setup and Activation
FNP-0-EIP-7	Security Support to the Emergency Plan
FNP-0-EIP-8.0	Non-Emergency Notifications
FNP-0-EIP-8.1	Emergency Phone Directory
FNP-0-EIP-8.2	Plant Personnel Home Telephone Directory
FNP-0-EIP-8.3	Communications Equipment Operating Procedures
FNP-0-EIP-9	Emergency Classification and Actions
FNP-0-EIP-9.1	Automated Dose Assessment Method
FNP-0-EIP-9.2	Obtaining Meteorological Information
FNP-0-EIP-9.3	Personal Computer - Automated Dose Assessment Method
FNP-0-EIP-9.5	Emergency Classification Based on ODCM
FNP-0-EIP-10	Evacuation and Personnel Accountability
FNP-0-EIP-11	Handling of Injured Personnel
FNP-0-EIP-13	Fire Emergencies
FNP-0-EIP-14	Personnel Movement, Relocation, Re-Entry and Site Evacuation
FNP-0-EIP-15	Emergency Drills
FNP-0-EIP-16	Emergency Equipment and Supplies
FNP-0-EIP-20	Chemistry and Environmental Support to the Emergency Plan
FNP-0-EIP-26	Duties of the EOF Manager
FNP-0-EIP-27.0	EOF Setup and Activation
FNP-0-EIP-27.1	Alternate EOF Setup and Activation
FNP-0-EIP-28.0	De-Escalation
FNP-0-EIP-28.1	Recovery
FNP-0-EIP-29	Long Term Dose Assessment
FNP-0-EIP-30	Post Accident Core Damage Assessment

Emergency Plan Implementing Procedures (EIP's) – Continued...

NMP-EP-001	Corporate Emergency Response Organization (ERO)	
NMP-EP-002	Corporate Duty Manager	
NMP-EP-101	CEOC Activation	
NMP-EP-102	CEOC Manager	
NMP-EP-103	Licensing Support Manager	
NMP-EP-104	Dose Analyst	
NMP-EP-105	Engineering Support Manager	
NMP-EP-106	Administrative Logistics Manager	
GO-EIP-102	Emergency Coordination Organization and Facility	
GO-EIP-114	News Release Coordination and Distribution	
GO-EIP-118	Emergency Communication Organization Corporate Activation and Notification Procedures	
GO-EIP-131	Emergency Operations Center-Corporate Headquarters Emergency Equipment and Supplies	
GO-EIP-132	Emergency Plan Drills and Exercises	
GO-EIP-134	Corporate Emergency Plan Training	
GO-EIP-135	Emergency Plan Review and Revision	
GO-EIP-136	Alert Radio Distribution & Maintenance	
GO-EIP-137	ANS Siren System Testing and Maintenance	
GO-EIP-138	Reprogramming of ROLM Phone System	

B. Radiation Control Procedures (RCP's)

FNP-0-RCP-7	Coordinated Exposure Reduction Program	
FNP-0-RCP-13.1	Use of the HIS-20 RWP Section	
FNP-0-RCP-25	Health Physics Activities During a Radiological Accident	
FNP-0-RCP-29.1	Guidelines for Personnel Decon and Response to Personnel Contamination Events	

C. Chemistry-Radiochemistry Control Procedures (CCP's)

FNP-0-CCP-1300 Chemistry and Environmental Activities during a Radiological Accident

D. Administrative Procedures (AP's)

FNP-0-AP-45 Farley Nuclear Plant Training Plan

E. General Office Nuclear Generation Procedures (GO-NG's)

GO-NG 47 Farley Nuclear Support Reporting of Unusual Nuclear Operating Events to Upper Management

II. EMERGENCY PLAN/IMPLEMENTING PROCEDURE CROSS REFERENCE

The following listing indicates for each plan section the procedures that implement actions required by Southern Nuclear Operating Company.

<u>PLAN SECTION</u>	<u>APPLICABLE IMPLEMENTING PROCEDURES</u>
I. <u>Introduction</u>	N/A
II. <u>Organization</u>	
A. Onsite	
1. TSC	
a. Emergency Director	EIP-0 EIP-3
b. Operations Supervisor	EIP-0, 6
c. Maintenance Supervisor	EIP-0, 6 EIP-5, 6
d. Health Physics Supervisor	EIP-0, 6 EIP-4
e. Security Supervision	EIP-7
f. Engineering Supervisor	EIP-0, 6 EIP-6
g. Shift Supervisor (Emergency Director).....	EIP-0, 3
h. Emergency Repair Party	EIP-5 EIP-14
i. Field Monitoring Team.....	EIP-4
j. Dose Assessment Staff	EIP-6 EIP-9 Series EIP-29
k. Additional Plant Staff Assignments	
1) Operations Support Center (OSC) Manager	EIP-0
2) Radiological Monitoring	EIP-4
3) Fire Fighting and Rescue	EIP-13
4) First Aid	EIP-11
5) Decontamination	EIP-4
6) Personnel Accountability	EIP-10
7) Record Keeping	EIP-6
8) Communications	EIP-2, 8 Series EIP-3 EIP-9.0

PLAN SECTION

APPLICABLE IMPLEMENTING PROCEDURES

2. Emergency Operations Facility (EOF)

a. EOF Manager.....	EIP-0
	EIP-26
b. Dose Assessment Supervisor	EIP-0
	EIP-9
	EIP-29
c. Support Coordinator.....	EIP-0
	EIP-27.0
d. Technical Advisor.....	EIP-0, 27.0

B. Offsite

1. Corporate Emergency Operations Center (CEOC)

a. SNC Duty Manager.....	NMP-EP-001, 002
b. CEOC Manager.....	NMP-EP-101, 102
c. Administrative Logistics Manager.....	NMP-EP-001
	NMP-EP-106
d. Engineering Support Manager.....	NMP-EP-001
	NMP-EP-105
e. Licensing Support Manager.....	NMP-EP-001, 103
f. SNC News Writer.....	EIP-102
g. Public Information Emergency Coordinator.....	EIP-102

2. Emergency Communication Organization Staff.....

a. Vice President-Administrative Services.....	EIP-102
b. Corporate Media Center Coordinator.....	EIP-102
c. Political Liaison	EIP-102
d. Employee Communication Coordinator.....	EIP-102
e. Public Information (PI) Director.....	EIP-102
f. PI Emergency EOF Coordinator.....	EIP-102
g. PI Emergency Staff Office Coordinator.....	EIP-102
h. News Emergency Center Coordinator.....	EIP-102

3. Recovery Phase Organization.....

a. Recovery Manager.....	EIP-26.0
	EIP-28.0
b. Recovery Support Director.....	EIP-28.0
c. Technical Support Director.....	EIP-28.0
d. Recovery Support Supervisor.....	EIP-28.0
e. Administrative Support Supervisor.....	EIP-28.0
	EIP-27.0
f. Engineering Supervisor.....	EIP-28.0
g. Licensing Supervisor.....	EIP-28.0

PLAN SECTION**APPLICABLE IMPLEMENTING PROCEDURES****C. Outside Organizations**

- | | |
|---|--------------|
| 1. Government Agencies | EIP-8.1, 9.0 |
| a. Department of Energy Savannah River Operations Office..... | EIP-9.0 |
| b. Nuclear Regulatory Commission | N/A |
| c. State of Alabama | N/A |
| d. State of Georgia..... | N/A |
| e. State of Florida | N/A |
| f. Houston County, Alabama..... | N/A |
| g. Early County, Georgia..... | N/A |
| h. City of Dothan, Alabama Fire Department..... | EIP-13 |
| | EIP-9.0 |
| 2. Contractor and Private Offsite Organizations | |
| a. Southern Company Services..... | N/A |
| b. Bechtel Power Corporation..... | N/A |
| c. Westinghouse..... | N/A |
| d. INPO, NEI, EPRI..... | N/A |
| e. Maintenance Assistance..... | N/A |
| f. Radiological Monitoring Assistance..... | N/A |
| g. Other Utilities..... | N/A |

III. Facilities and Equipment**A. Control Centers**

- | | |
|---|------------|
| 1. Technical Support..... | EIP-0 |
| | EIP-6 |
| 2. Emergency Operations Facility..... | EIP-27.0 |
| 3. Alternate Emergency Operations Facility..... | EIP-27.1 |
| 4. Operations Support Center..... | EIP-0 |
| | EIP-10 |
| 5. Emergency News Center..... | EIP-102 |
| 6. Corporate EOC..... | NMP-EP-101 |
| 7. APC Corporate Media Center..... | EIP-102 |

B. Communications Systems

- | | |
|---|---------|
| 1. Commercial Telephones..... | N/A |
| 2. Private Automatic Exchange..... | N/A |
| 3. Microwave..... | N/A |
| 4. APC Load Dispatch Computer Link..... | N/A |
| 5. Two-Way Radio..... | EIP-8.3 |
| 6. Public Address and Party Lines..... | EIP-8.3 |
| 7. Sound Powered Telephone..... | N/A |
| 8. Plant Emergency Alarm..... | N/A |
| 9. NRC Emergency Notification System..... | EIP-8.3 |
| 10. NRC Health Physics Network..... | EIP-8.3 |
| 11. State/Local Agency Emergency Notification Network.... | EIP-8.3 |
| 12. RSCL..... | N/A |
| 13. PMCL..... | N/A |
| 14. MCL..... | N/A |
| 15. LAN..... | N/A |
| 16. Telecopier..... | EIP-8.3 |
| 17. WebEOC System..... | N/A |
| 18. ERDS..... | EIP-8.3 |
| 19. Other Communication Systems..... | EIP-8.3 |

PLAN SECTION

APPLICABLE IMPLEMENTING PROCEDURES

C. Protective Actions and Emergency Action Levels

1. Onsite Protective Action

a. Evacuation	EIP-10
b. Personnel Accountability	EIP-7
	EIP-10
c. Contamination and Exposure Control Measures.....	EIP-4
	EIP-7
	EIP-10
	EIP-11
	EIP-14
	RCP-6
	RCP-7
	RCP-13.1
2. Offsite Protective Action.....	EIP-9.0
a. Classification of Offsite Incidents.....	EIP-9.0
b. Response	EIP-3
	EIP-9.0

V. Activation of Emergency Organization

A. Declaration of an Emergency	EIP-9.0
B. Onsite Organization Activation	EIP-9.0
1. Technical Support Center Activation	EIP-6
	EIP-8.1
2. Operations Support Center Activation	EIP-0, 6
	EIP-7
	EIP-10
	EIP-27.0
3. Emergency Operations Facility	
C. Offsite Corporate Organization Activation	NMP-EP-101
	EIP-118
D. Offsite Local, State and Federal Agencies	N/A

VI. Notification Procedures

A. State and Local Agency Notification	EIP-8.1
	EIP-9.0
B. Plume Exposure Pathway Planning Zone Public Notification and Information	
1. Notification	N/A
2. Information	N/A
3. News Release Coordination and Rumor Control.	EIP-102
	EIP-114
C. NRC Office of Inspection and Enforcement	EIP-9.0
D. Savannah River Operations Office	EIP-8.1
E. Medical	EIP-8.1
	EIP-9.0
F. Fire	EIP-8.1
	EIP-9.0

EMERGENCY PLAN
Part II

MEDICAL PLAN

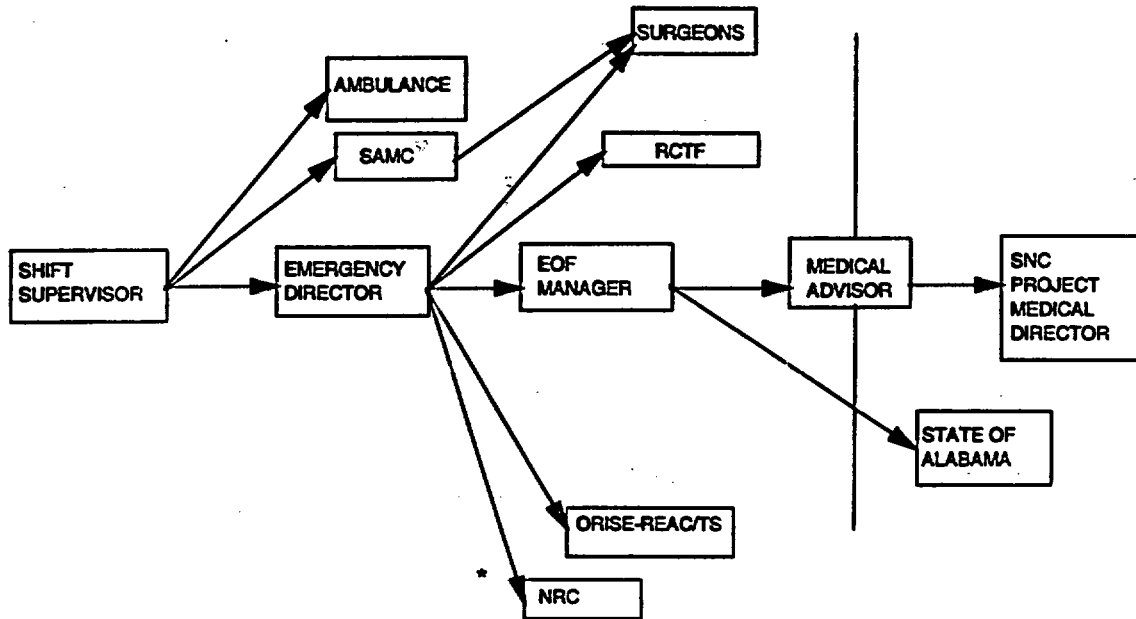
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B-10b	29	C-22	33	D-7	DEL
B-10c	29	C-23	33	D-8	DEL
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B-12	29	C-25	33	D-10	DEL
B-13	29	C-26	33	D-11	DEL
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B-15	14	C-28	33	D-13	DEL
B-16	14	C-29	33	D-14	DEL
B-17	14	C-30	33	D-15	DEL
B-18	14	C-31	33	D-16	DEL
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MEDICAL NOTIFICATION ORDER



* If injury involves contamination, excessive exposure or if it is anticipated that injured will be admitted to the hospital for observation or treatment in excess of 48 hours duration.

FIGURE 6 - MEDICAL NOTIFICATION ORDER