Dominion Nuclear Connecticut, Inc.
Millstone Power Station
Rope Ferry Road
Waterford, CT 06385

Dominion

SEP 11 2003

Docket Nos. 50-245
50-336
50-423
B18982

RE: 10 CFR 50, Appendix E
10 CFR 50.47(b)(5)

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Millstone Power Station, Unit Nos. 1, 2 and 3
Revised Emergency Plan Procedure

In accordance with 10 CFR 50, Appendix E, Dominion Nuclear Connecticut, Inc. hereby notifies the U.S. Nuclear Regulatory Commission that the following Emergency Plan procedure has been implemented:

MP-26-EPI-FAP10-005, "Unit 1 Dose Calculation for Fuel Handling Accident," Major Revision 0, Minor Revision 1, transmitted via Attachment 1.

There are no regulatory commitments contained within this letter.

If you should have any questions concerning this submittal, please contact Mr. David W. Dodson at (860) 447-1791, extension 2346.

Very truly yours,

DOMINION NUCLEAR CONNECTICUT, INC.

J. Alan Price
Site Vice President - Millstone

cc: See next page

A045

Attachment (1)

cc: H. J. Miller, Region I Administrator (2 copies)
R. J. Conte, Chief, Operational Safety Branch, Region I

cc: w/o attachment

D. G. Holland, NRC Project Manager, Millstone Unit No. 1
J. R. Wray, NRC Inspector, Region I, Millstone Unit No. 1
R. B. Ennis, NRC Senior Project Manager, Millstone Unit No. 2
V. Nerses, NRC Senior Project Manager, Millstone Unit No. 3
Millstone Senior Resident Inspector

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Attachment 1

Millstone Power Station, Unit Nos. 1, 2 and 3

Emergency Procedures Implementing (EPI) Functional Administrative Procedure (FAP)
MP-26-EPI-FAP10-005, "Unit 1 Dose Calculation for Fuel Handling Accident"
Major Revision 0, Minor Revision 1

8/6/03	8/12/03
Approval Date	Effective Date

Unit 1 Dose Calculation for Fuel Handling Accident

Control Room Data

Obtain the following:

RM-SFPI-01 Channel 1 (mR/hr)	RM-SFPI-01 Channel 2 (mR/hr)	RM-SFPI-01 Channel 3 (mR/hr)

Calculation

Perform the following:

1)	_____ RM-SFPI-01 Channel 3 (mR/hr)*	X	2 Factor A	=	_____ # Fuel Assemblies Damaged

*Note: If Channel 3 is inoperative, the higher value between Channels 1 and 2 must be used and Factor A in step 1 changed from 2 to 0.25.

2)	_____ # Fuel Assemblies Damaged	X	0.2	=	_____ TEDE	mRem
3)	_____ # Fuel Assemblies Damaged	X	10	=	_____ Skin	mRem

4) Notify CR-DSEO of results.

NOTE

The basis for the rad monitor factors is for airborne radioactivity released from damaged fuel assemblies, not shine from exposed components.

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Prepared by:	Signature	Print	Date/Time
Reviewed by:	Signature	Print	Date/Time