

September 18, 2003

Mr. Mark Peifer
Site Vice-President
Duane Arnold Energy Center
Nuclear Management Company, LLC
3277 DAEC Road
Palo, IA 52324

SUBJECT: DUANE ARNOLD ENERGY CENTER
NRC EVALUATIONS OF CHANGES, EXPERIMENTS, OR TESTS AND
PERMANENT PLANT MODIFICATIONS INSPECTION
REPORT 50-331/03-09(DRS)

Dear Mr. Peifer:

On August 29, 2003, the U.S. Nuclear Regulatory Commission (NRC) completed a routine baseline inspection at your Duane Arnold Energy Center. The enclosed report documents the inspection findings, which were discussed on August 29, 2003, with you and other members of your staff.

The inspection examined activities conducted under your license as they relate to safety and to compliance with the Commission's rules and regulations and with the conditions of your license. Specifically, this inspection focused on the baseline biennial inspections for evaluations of changes, tests, or experiments (10 CFR 50.59) and permanent plant modifications. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection, no findings of significance were identified.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component

of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Julio F. Lara, Chief
Electrical Engineering Branch
Division of Reactor Safety

Docket No. 50-331
License No. DPR-49

Enclosure: Inspection Report 50-331/03-09(DRS)
w/Attachment: Supplemental Information

cc w/encl: E. Protsch, Executive Vice President -
Energy Delivery, Alliant;
President, IES Utilities, Inc.
J. Cowan, Chief Nuclear Officer
J. Forbes, Senior Vice President
J. Bjorseth, Plant Manager
S. Catron, Manager, Regulatory Affairs
J. Rogoff, Esquire General Counsel
B. Lacy, Nuclear Asset Manager
D. McGhee, Iowa Department of Commerce
Chairman, Linn County Board of Supervisors
State Liaison Officer
Chairperson, Iowa Utilities Board
The Honorable Charles W. Larson, Jr.
Iowa State Representative

M. Peifer

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The Honorable Charles W. Larson, Jr.
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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No: 50-331

License No: DPR-49

Report No: 50-331/03-09(DRS)

Licensee: Alliant, IES Utilities, Inc.

Facility: Duane Arnold Energy Center

Location: 3277 DAEC Road
Palo, Iowa 52324-9785

Dates: August 25, 2003 through August 29, 2003

Inspectors: G. Hausman, Senior Reactor Inspector, Lead
T. Bilik, Reactor Inspector
D. Schrum, Reactor Inspector
J. Panchison, Mechanical Engineering Contractor

Approved by: Julio F. Lara, Chief
Electrical Engineering Branch
Division of Reactor Safety

Enclosure

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Enclosure

SUMMARY OF FINDINGS

IR 05000331/2003-009(DRS); IES Utilities, Inc.; 08/25/03 - 08/29/03; Duane Arnold Energy Center; Routine Baseline Inspection Report.

This report covers a five day period of announced baseline inspection on evaluations of changes, tests, or experiments and permanent plant modifications. The inspection was conducted by Region III inspectors and an engineering contractor. No findings of significance were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process" (SDP). Findings for which the SDP does not apply may be "Green" or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

A. Inspector-Identified and Self-Revealed Findings

Cornerstone: Initiating Events

No findings of significance were identified.

Cornerstone: Mitigating Systems

No findings of significance were identified.

Cornerstone: Barrier Integrity

No findings of significance were identified.

B. Licensee-Identified Violations

No findings of significance were identified.

REPORT DETAILS

Summary of Plant Status

Unit 1 operated at or near full power throughout the inspection period.

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

1R02 Evaluations of Changes, Tests, or Experiments (71111.02)

.1 Review of 50.59 Evaluations and Screenings

a. Inspection Scope

The inspectors reviewed 10 evaluations performed pursuant to 10 CFR 50.59. The evaluations related to permanent plant modifications, setpoint changes, procedure changes, conditions adverse to quality, and changes to the updated final safety analysis report. The inspectors confirmed that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspectors also reviewed 19 screenings where the licensee had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. These evaluations and screenings were chosen based on risk significance of samples from the different cornerstones.

b. Findings

No findings of significance were identified.

1R17 Permanent Plant Modifications (71111.17)

.1 Review of Recent Modifications

a. Inspection Scope

The inspectors reviewed 10 permanent plant modifications that were installed in the last two years. The modifications were chosen based upon their affecting systems that had high probabilistic risk analysis (PRA) significance in the licensee's Individual Plant Evaluation (IPE) or high maintenance rule safety significance. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the changes did not affect any systems' safety function. Design and post-modification testing aspects were verified to ensure the functionality of the modification, its associated system, and any support systems. The inspectors also verified that the modifications performed did not place the plant in an increased risk configuration.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES (OA)

4OA2 Identification and Resolution of Problems (71152)

.1 Routine Review of Condition Reports

a. Inspection Scope

The inspectors reviewed a selected sample of condition reports associated with Duane Arnold Energy Center's permanent plant modifications and concerning 10 CFR 50.59 evaluations and screenings. The inspectors reviewed these issues to verify an appropriate threshold for identifying issues and to evaluate the effectiveness of corrective actions. In addition, condition reports written on issues identified during the inspection were reviewed to verify adequate problem identification and incorporation of the problem into the corrective action system. The specific corrective action documents that were sampled and reviewed by the team are listed in the attachment to this report.

b. Findings

No findings of significance were identified.

4OA6 Meetings

.1 Exit Meeting

The inspectors presented the inspection results to Mr. Peifer and other members of licensee management at the conclusion of the inspection on August 29, 2003. The inspectors asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

.2 Interim Exit Meetings

No interim exits were conducted.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee

T. Anderson, Maintenance
J. Bjorseth, Plant Manger
C. Bleau, Design Engineering (Electrical/I&C Team Supervisor)
D. Brigl, Systems Engineering
S. Catron, Regulatory Affairs Manager
E. Christopher, Program Engineering
P. Collingsworth, Systems Engineering
D. Curtland, Engineering Director
S. Haller, Systems Engineering Manager
R. Harter, Performance Improvement Manager
B. Hopkins, Program Engineering
D. Miller, Employee Concerns Manager
K. Morgan, Nuclear Assessment Manager
B. Murrell, Regulatory Affairs/Licensing
A. Nebola, Training
M. Peifer, Site Vice President
J. Probst, Program Engineering
W. Simmons, Maintenance Manager
J. Stutzman, Training
L. Swenzinski, Regulatory Affairs/Licensing
M. Wood, Design Engineering (Mechanical Team Supervisor)

Nuclear Regulatory Commission

S. Caudill, Resident Inspector
D. Hills, Region III Mechanical Engineering Branch Chief
G. Wilson, Senior Resident Inspector

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None.

Closed

None.

Discussed

None.

LIST OF DOCUMENTS REVIEWED

The following is a list of documents reviewed during the inspection. Inclusion on this list does not imply that the NRC inspectors reviewed the documents in their entirety but rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document on this list does not imply NRC acceptance of the document or any part of it, unless this is stated in the body of the inspection report.

CALCULATIONS

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
CAL-VC8-009	Drywell Cooling Load Calculation	0
MC-25E	Develop FW System Curve	January 12, 1974

CORRECTIVE ACTION DOCUMENTS GENERATED DURING THE INSPECTION

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
CAP 028802	10CFR50.59 Safety Evaluation 99-041, RHR Pump Seal Water Coolers Not Required at DAEC, Should Be Reviewed for Possible Revision	August 27, 2003
CAP 028847	50.59 Screening Records Were Not Stand Alone Documents	August 28, 2003
CAP 028848	OI-416, Section 10 May Need Precaution Statement	August 28, 2003
CAP 028849	The Corrective Action Program's Method of Links and Daughter Documents Made it Very Difficult to Assess What Activities Had Been Completed	August 28, 2003
CAP 028851	Evaluate Replacement of KY4201 and KY4202	August 29, 2003

CORRECTIVE ACTION DOCUMENTS GENERATED PRIOR TO INSPECTION

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
AR 3025	RHR SW Strainer High dp	August 6, 2002
CAP 011318	UFSAR Change Requests for Plant Modifications	August 13, 2001
CAP 011327	NRC Inspection Found Overlap of ECP Scopes	August 16, 2001
CAP 011328	ESW Flow to RHR Pump Seal Coolers	August 16, 2001
CAP 011329	SE for Removal of HPCI/RCIC Auto Open Logic	December 14, 2002
CAP 011330	Inadequate Engineering Evaluations Identified for the Majority of Field Variances Written for ECP 1622 and ECP 1604	August 16, 2001
CAP 011464	Reactor Building Sump Maintenance Rule 10 CFR 50.59	August 29, 2001
CAP 011468	Form an Engineering Team to Identify Design Basis Calculations, Modifications, and Programs That Pertain to RHR, RHR SW, ESW, and EDG Systems	August 24, 2001
CAP 011478	Review of Revised 50.59 Process Implementation	September 4, 2001
CAP 012068	Inadequate Followup Action for 10 CFR 50.59 Evaluation 01-023	November 14, 2001

CORRECTIVE ACTION DOCUMENTS GENERATED PRIOR TO INSPECTION

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
CAP 012462	10 CFR 50.59 Evaluation 01-026 (spTP 202 PUP Power Ascension Testing) Requires Revision	January 10, 2002
CAP 013469	Insure That 10CFR50.59 Screening and a 10 CFR 50.59 Evaluation Are Completed	May 20, 2002
CAP 014503	Temporary Modification 02-059 Did Not Address the Seismic Qualification of a Jumper Cable Installation	October 1, 2002
CAP 014525	Error in Developing Temporary Modification 02-061 Delayed Exit from a Core Spray LCO	October 4, 2002
CAP 014710	Inadequate 10 CFR 50.59 Screening to Change Modes of Closed Operation in the Prerequisites	November 14, 2002
CAP 020061	RHRSW Strainer High dp While Running A and C RHRSW Pumps	August 6, 2002
CAP 025226	Administrative Clarification on 50.59 Screen Questions w/Pre-existing NRC SER	January 16, 2003
CAP 025315	Using 10 CFR 50.59 Screenings Written for EMAs/DDCs for Procedure Changes	January 28, 2003
CAP 026474	DCF Approved by Owner and OSS (OI 149) Without a 10 CFR 50.59 Screening	March 28, 2003
CAP 027142	Focused Area Self-Assessment of 10 CFR 50.59 Process	April 22, 2003
CAP 027189	10 CFR 50.59 Screening Course Exam Administered with Invalid Exam Questions	April 24, 2003
CAP 027440	AOV Reliability Improvement Mod to Install Flexible Air Lines at Valve	May 16, 2003
CAP 027457	Trend of Improperly Performed Mods to the DAEC	May 17, 2003
CAP 027594	No Show for 10 CFR 50.59 Screening Class (60030)	May 28, 2003
CAP 028265	Applicability of 10 CFR 50.59 and NMC/DAEC Supply Chain Procedures	July 17, 2003

DRAWINGS

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
BECH-M129	River Water Supply System	July 14, 2003

EVALUATIONS

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
SE 97-113	Install Flush Connections on the Piping to and from the Reactor Building Equipment Drain Sump Heat Exchanger and Install an Isolation Valve on the Drain Line from this Unit	March 17, 1998

EVALUATIONS

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
SE 98-014	UFSAR Change Request 98-122 revised the RWCU System Supply Isolation Valves MO2700 & MO2701 Design Closure Times from 20 Seconds to 22 Seconds	December 18, 1998
SE 98-075	BECH-M133 Updated to Remove HS3301A From the Control Circuit of 1P048	July 31, 1998
SE 99-041	ESW to RHR Pump Seal Coolers	January 1, 1900
SE 99-058	SE for Extending Fire Pump LCO Time	December 21, 1999
SE 01-025	ECP 1628, Reactor Building Vent Shaft and Control Building Air Intake Radiation Monitor Replacement	November 20, 2001
SE 01-026	Cycle 18 Power Ascension Test to 1790 Mw _t	January 29, 2002
SE 02-001	Revise Qual SC-100 & SC-101, EQ Documentation	May 28, 2002
SE 02-002	Operation at 100 percent Power with RHRSW Strainer Bypassed	August 10, 2002
SE 03-001	River H ₂ O Supply Valves, Safety Related Seismic Boundary (V-29-42 & 43 to Become Normally Open)	June 17, 2003

MODIFICATIONS

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
ECP 1616	MO2046 & MO1947 Control Logic Modification	April 24, 2000
ECP 1640	Circulating Water Pumps	
ECP 1648	Replace Reactor Building Vent Shaft/Control Building Air Intake Radiation Monitoring System with Digital Radiation Monitoring System	November 28, 2001
ECP 1650	Drywell Cooler Replacement	April 25, 2003
ECP 1661	Install Oil Lie off of Compressor 1K003	
EMA A36191	Add a Connection with an Isolation Valve to the Inlet Line from the Reactor Building Equipment Drain Sump to the 1E033 Heat Exchanger	April 21, 2000
EMA A45051	Decommission the Relative Humidity Indication Instrument Loops for the a SBTG Train	July 30, 2001
EMA A45791	Replace 64D10 Ground Replay with Magnacraft	
EMA A53462	Provide Permanent Power Supply to the Heat Tracing on the off Gas after Filter Vent Line	October 24, 2002
EMA A57819	Butterfly Valve Installed Will Provide Better Sealing than the Sluice Gate Being Replaced	June 5, 2002

PROCEDURES

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
ACP 103.0	Design Control Program	14
ACP 103.2	Safety Evaluation Applicability Review Process	14
ACP 103.2	10 CFR 50.59 Screening Process	19
ACP 103.3	Safety Evaluation Process	10

PROCEDURES

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
ACP 103.3	10 CFR 50.59 Evaluation Process	16
ACP 109.0	Engineering Change Packages	28
ACP 109.1	Engineered Maintenance Action	26
ACP 1203.07	External Design Interfaces	6
ACP 1203.11	Design Input Requirements	12
ACP 1203.21	Engineering Calculations	13
ACP 1203.23	Engineering Drawings/Documents	15
ACP 1203.25	Engineering Specifications	5
ACP 1203.26	Engineering Acceptance Requirements	8
ACP 1203.31	Design Verification	10
ACP 1203.32	Second Level Review and Acceptance of Procured Modifications	9
ACP 1203.42	Field Variances	10
ACP 1203.43	Field Change Notice	9
ACP 1203.45	ECP Closure Activities	18
ACP 1203.50	Engineering Programs/Checklists	0
ACP 1203.51	R Stamp Applicability	1
ACP 1203.52	Assessment of Potential Impact on EOPs/SAGs	0
ACP 1203.53	Fire Protection	4
ACP 1203.54	ALARA Design Considerations	1
ACP 1203.55	Control of Heavy Loads Analysis	1
ACP 1203.56	Environmental Qualification Assessment	3
ACP 1203.57	Inservice Inspection Checklist	0
ACP 1203.58	Human Factors Considerations	1
ACP 1203.59	Power System Configuration and Analysis	2
ACP 1203.60	Masonry Wall Analysis	0
ACP 1203.61	Inservice Testing Checklist	0
ACP 1203.62	Setpoint Control	3
ACP 1203.63	PRA Design Change Notification	0
ACP 1203.64	Regulatory Guide (Reg Guide) 1.97	0
ACP 1203.66	Station Blackout Applicability	0
ACP 1203.80	Capacitor Replacement Evaluation	1
ACP 1203.81	Abandoned in Place Equipment	0
OI-416	Operating Instruction - RHR Service Water System	31

REFERENCES

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
-----	System Importance Ranking Results and Important Human Actions from the DAEC Living PRA Training Summary Printout by Course from 01/01/1990 Through 08/21/2003 for Course: PST1.14, Perf/Rev 10 CFR 50.59 Screening	June 2001
-----		August 22, 2003

REFERENCES

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
-----	Training Summary Printout by Course from 01/01/1990 Through 08/21/2003 for Course: PST9.27, Perf/Verify 10 CFR 50.59 Eval	August 22, 2003
Amendment No. 243	DAEC - Issuance of Amendment Regarding Extended Power Uprate (TAC No. MB0543)	November 6, 2001
CENPD-397-P	Improved Flow Measurement Accuracy Using Crossflow Ultrasonic Flow Measurement Technology January 2000 (Proprietary Information)	1
GE-NE-A22-00100- 23-01-R2	Project Task Report Duane Arnold Energy Center Asset Enhancement Program Task T0400: Containment System Response December 2002 (Proprietary Information)	January 2, 1900
NG-00-1900	DAEC Docket No: 50-331 Op. License No: DPR-49 Technical Specification Change Request (TSCR-042): Extended Power Uprate	November 16, 2000

SCREENINGS

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
4	Change APRM Flow Biased Setpoints for Power Uprate per EMA A52240	December 4, 2001
33	Revise Document APED-A61-089, SBO Compliance Assessments & Issue APED-A61-091 to Incorporate Power Uprate Analysis Results into the SBO Program Documentation	August 8, 2001
83	Add Air Assist to CV2851-O in the Closing Direction to Increase the Closing Force Applied to CV2851	October 25, 2001
84	Add Air Assist to CV2850-O in the Closing Direction to Increase the Closing Force Applied to CV2850	October 25, 2001
229	Change Terminology in the UFSAR Associated with 10CFR50.59 to be Consistent with the Revised Rule	September 25, 2001
243	Trip of Main Turbine on Loss of 125 Vdc	September 30, 2001
284	Revise ACP 103.1 for PUP, Clarification of 50.59 Screening Applicability, and SLO Safety Limit MCPR	November 20, 2001
385	Revise Stroke Times for CV2315 in STP 3.5.1-05, 3.5.1-06386	October 22, 2001
484	Added Steps to Start/stop RHR Pump Room Cooling Units, Modified Steps Changing RHR Injection Points During SDC and Added Subsection to Return System to Original Injection Point. Added Information That Total FPC Flow May Be Increased up to 2400 gpm	November 9, 2001
496	Revise BECH-M175 to Change Max dp Across the Outside Air Intake Filter from 1.8 in Wg to 2.0 in Wg	November 14, 2001

SCREENINGS

<u>Document Number</u>	<u>Title or Description</u>	<u>Revision or Date</u>
532	Change Feedpump Low Pressure Suction Trip Setpoints from 250 psi to 225 psi	November 20, 2001
598	Change Required As Left Diesel Cooler dp to Greater Than or Equal to 69.3 Inches of Water	December 12, 2001
605	Cancel STPs, Revision 16 of the Offsite Dose Assessment Manual, No Longer Has a Requirement to Do Surveillances on the 1T-269 Level Instruments	December 13, 2001
649	Allow For Operation of Three RBCCW Pumps in All Reactor Modes	December 27, 2001
1685	Revision to IPOI-3	October 2, 2002
1839	Adds P&LS for Pump Flows and Strainer Venting. Also Changes SW Flow Rates for Pump Protection	November 16, 2002
2012	Install 0.109 Orifices in the P Port of the FASV's for Main Steam Valves	January 15, 2003
2159	Address Issues Identified During NRC Inspection on Smoke in Control Room	February 16, 2003
2514	Drywell Cooling Trends Indicate That All Drywell Coolers May Not Be Needed (Isolation of Non-essential Coolers)	May 27, 2003

LIST OF ACRONYMS USED

ACP	Administrative Control Procedure
ADAMS	Agency-Wide Document Access and Management System
ALARA	As Low As is Reasonably Achievable
AOV	Air Operated Valve
APED	Administrative Procedure Engineering Directive
APRM	Average Power Range Monitor
AR	Action Request
BECH	Bechtel
CAP	Corrective Action Program
CFR	Code of Federal Regulations
CV	Control Valve
DAEC	Duane Arnold Energy Center
DPR	Demonstration Power Reactor
DRS	Division of Reactor Safety
ECP	Engineering Change Package
DCF	Document Change Form
DDC	Design Document Change
EDG	Emergency Diesel Generator
EMA	Engineered Maintenance Action
EOP	Emergency Operating Procedure
ESW	Essential Service Water
FASV	Fast Acting Solenoid Valve
FPC	Fuel Pool Cooling
FW	Feedwater
gpm	gallons per minute
gov	Government
HPCI	High-Pressure Coolant Injection
HS	Hand Switch
html	Hypertext Markup Language
http	Hypertext Transfer Protocol
I&C	Instrumentation and Control
IMC	Inspection Manual Chapter
IPE	Individual Plant Evaluation
IR	Inspection Report
LCO	Limiting Condition of Operation
LLC	Limited Liability Company
MCPR	Minimum Critical Power Ratio
NMC	Nuclear Management Company, LLC
NRC	Nuclear Regulatory Commission
NRR	Office of Nuclear Reactor Regulation
NUREG	NRC Technical Report Designation
OA	Other Activities
OSS	Operations Shift Supervisor
PARS	Publically Available Records System
PRA	Probabilistic Risk Assessment

LIST OF ACRONYMS USED

PUP	Power Uprate
RBCCW	Reactor Building Closed Cooling Water
RHR	Residual Heat Removal
RHRSW	Residual Heat Removal Service Water
RCIC	Reactor Core Isolation Cooling
RWCU	Reactor Water Clean-Up
SAG	Severe Accident Guideline
SBGT	Standby Gas Treatment
SBO	Station Black Out
SDC	Shutdown Cooling
SDP	Significance Determination Process
SE	Safety Evaluation
SER	Safety Evaluation Report
SLO	Single-Loop Operation
STPs	Surveillance Test Procedures
SW	Service Water
TSCR	Technical Specification Change Request
UFSAR	Updated Final Safety Analysis Report
V	Valve or Volt
wpd	WordPerfect Document
www	World Wide Web